Southern California Edison Company

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August 24, 1982

U. S. Nuclear Regulatory Commission
Office of Inspection and Enforcement
Region V
1450 Maria Lane, Suite 210
Walnut Creek, California 94596-5368

Attention: Mr. R. H. Engelken, Regional Administrator

Dear Sir:

Subject: Docket No. 50-361
30-Day Interim Report
Licensee Event Report No. 82-076
San Onofre Nuclear Generating Station, Unit 2

This submittal is in accordance with the reporting requirements of Section 6.9.1.13b of Appendix A to Facility Operating Licence NPF-10. It describes a reportable condition involving Limiting Condition for Operation (LCO) 3.3.1 associated with the Reactor Protection System (RPS). A preliminary copy of LER 82-076 is enclosed.

While in Mode 4, at 1011 on July 25, 1982, the once a shift surveillance in accordance with procedure SO23-3-2.25 was in progress. During this surveillance, steam generator ΔP indicator 2PDI-0979-1 failed to meet the channel check test. Although the Action Statement associated with LCO 3.3.1 is not applicable in Mode 4, the steam generator #2 low flow trip for channel A was immediately bypassed, since Mode 2 entry was anticipated within the following 36 hours. LCO 3.3.1 does not prohibit change of modes.

The plant entered Mode 2 at 1349 on July 26, 1982. Upon entering Mode 2, the Action Statement associated with LCO 3.3.1 became applicable. The Action Statement associated with LCO 3.3.1 (Technical Specification Table 3.3-1, Action 2) requires that with the number of operable RPS channels one less than the total number of channels, startup (Mode 2) may continue provided the inoperable channel is placed in the bypassed or tripped condition within 1 hour. Additionally, the Action Statememt requires that the inoperable channel be returned to operable status no later than during the next cold shutdown.

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The first requirement of the Action Statement was satisfied when the plant was operating in Mode 4. The remaining requirement was satisfied on July 31, 1982 when the indicator was valved back in, tested in accordance with SO23-3-3.25 and returned to service.

Upon investigation, it was determined that the indicator had been valved out. The valve line-up was corrected and the indicator restored to an operable status. Investigation into the cause of the incorrect valve line-up is ongoing. A completed LER and final report will be provided as soon as the cause is determined, and in any event, on or about September 30, 1982.

Since the RPS contains 4 channels for reactor coolant low flow indication for each steam generator and only 3 channels (all of which remained functional throughout the event) are required for operability, there was no impact on health and safety of plant personnel or the public.

If there are any questions regarding the above, please contact me.

Sincerely,

cc: U. S. Nuclear Regulatory Commission
Office of Inspection and Enforcement

U. S. Nuclear Regulatory Commission
Office of Management Information & Program Control

Institute of Nuclear Power Operations

A. E. Chaffee (USNRC Resident Inspector, San Onofre Unit 2)