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TSC 93TSB05

United States Nuclear Regulatory Commission
ATTENTION: Document Control Desk
Washington, DC 20555

BRUNSWICK STEAM ELECTRIC PLANT, UNIT NOS. 1 AND 2
DOCKET NOS. 50-325& 50-324/LICENSE NOS. DPR 71 & DPR-62
REQUEST FOR LICENSE AMENDMENT
ADMINISTRATIVE CHANGES

Gentlemen:

In accordance with the Code of Federal Regulations, Title 10, Parts 50.90 and 2.101, Carolina Power & Light Company hereby requests a revision to the Technical Specifications for the Brunswick Steam Electric Plant (BSEP), Units 1 and 2.

The proposed amendment revises the Brunswick Technical Specifications by correcting several typographical errors, incorporating material implicitly contained in a footnote to an applicability statement, providing detailed labels for items listed in a table, correcting the citation of references, and removing references to the Rod Sequence Control System (RSCS) not included in a previous change request. These changes are considered administrative in nature.

Enclosure 1 provides a detailed description of the proposed changes and the basis for the changes.

Enclosure 2 details, in accordance with 10 CFR 50.91(a), the basis for the Company's determination that the proposed changes do not involve a significant hazards consideration.

Enclosure 3 provides an environmental evaluation which demonstrates that the proposed amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Therefore, pursuant to 10 CFR 51.22(b), no environmental assessment needs to be prepared with issuance of the amendment.

Enclosure 4 provides page change instructions for incorporating the proposed revisions.

Enclosure 5 provides the proposed Technical Specification pages for Unit 1.

Enclosure 6 provides the proposed Technical Specification pages for Unit 2.

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ADD 1

In order to allow time for procedure revision and orderly incorporation into copies of the Technical Specifications, CP&L requests that the proposed amendments, once approved by the NRC be issued with an effective date to be no later than 60 days from the issuance of the amendment.

Please refer any questions regarding this submittal to Mr. R. P. Lopriore at (910) 457-2212.

Very truly yours,



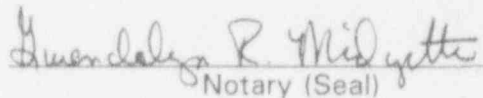
R. A. Anderson

SHC/shc (Admin.chg)

Enclosures:

1. Basis for Change Request
2. 10 CFR 50.91(a) Evaluation
3. Environmental Considerations
4. Page Change Instructions
5. Technical Specification Pages - Unit 1
6. Technical Specification Pages - Unit 2

R. A. Anderson, having been first duly sworn, did depose and say that the information contained herein is true and correct to the best of his information, knowledge and belief; and the sources of his information are officers, employees, and agents of Carolina Power & Light Company.


Notary (Seal)

My commission expires: August 12, 1996

cc: Mr. Dayne H. Brown, Director - Division of Radiation Protection, State of NC
Mr. S. D. Ebnetter, NRC Region II - Regional Administrator
Mr. P. D. Miano, NRC/NRR Senior Project Manager - Brunswick
Mr. R. E. Prevatte, NRC Senior Resident Inspector - Brunswick

ENCLOSURE 1

BRUNSWICK STEAM ELECTRIC PLANT, UNITS 1 AND 2
NRC DOCKET NOS. 50-325 & 50-324
OPERATING LICENSE NOS. DPR-71 & DPR-62
REQUEST FOR LICENSE AMENDMENT
ADMINISTRATIVE CHANGES

BASIS FOR CHANGE REQUEST

Proposed Change:

The proposed change will revise the Brunswick Technical Specifications by correcting several typographical errors, incorporating material implicitly contained in a footnote to an applicability statement, providing detailed labels for items listed in a table, correcting the citation of references, and removing references to the Rod Sequence Control System (RSCS) not included in a previous change request. These changes are considered administrative in nature and are summarized below.

SUMMARY OF ADMINISTRATIVE CHANGES

Brunswick Unit 1

Bases Section 2.2.1:	Remove references to the Rod Sequence Control System (RSCS) in item 2 on page B 2-4
Bases Section 2.2.1:	Correct typographical error in acronym for hydrogen water chemistry in item 6 on page B 2-6
TS 3.1.4.1:	Correct typographical errors in action d, misspelling of preset, and action d.1, misspelling of BPWS acronym, on page 3/4 1-14
TS Table 4.3.4-1:	Remove references to the RSCS in item g of the Notes on page 3/4 3-52
TS Table 3.3.5.5-1:	Label each item to permit identification consistent with the scheduling system used for surveillance testing on page 3/4 3-64a
TS Table 4.3.5.5-1	Label each item to permit identification consistent with the scheduling system used for surveillance testing on page 3/4 3-64c

- TS 4.3.6.1.1: Correct typographical error that references non-existent Table 4.3.6.1.1-1 to provide correct reference of Table 4.3.6.1-1 on page 3/4 3-88
- TS 3.4.2: Correct typographical error indicating extraneous second footnote on page 3/4 4-4

Brunswick Unit 2

- TS Table 2.2.1-1: Correct typographical error in item 2.b under allowable values by changing 115% to 115.5% on page 2-4
- Bases Section 2.2.1: Remove references to the Rod Sequence Control System (RSCS) in item 2 on page B 2-4
- Bases Section 2.2.1: Remove references to the Rod Sequence Control System (RSCS) in item 10 and revise bases description of the Select Rod Insertion consistent with removal of the RSCS on pages B 2-7 and B 2-8
- TS 3.1.4.1: Correct typographical error in action d.1 to correct misspelling of BPWS acronym on page 3/4 1-14
- TS Table 4.3.1-1: Correct grammatical omission of the word "is" in item e of the Notes on page 3/4 3-9
- TS Table 4.3.4-1: Remove references to the RSCS in item g of the Notes on page 3/4 3-52
- TS Table 3.3.5.5-1: Label each item to permit identification consistent with the scheduling system used for surveillance testing on page 3/4 3-64a
- TS Table 4.3.5.5-1: Label each item to permit identification consistent with the scheduling system used for surveillance testing on page 3/4 3-64c
- TS 3.3.6.2: Eliminate footnote, revise applicability statement and correct typographical errors in actions d and e that references non-existent Specification on page 3/4 3-93
- Bases Section 3/4.1.4: Correct identification of Reference cited to reference 6 on page B 3/4 1-4

Basis

Brunswick Unit 1

Item 2 Bases Section 2.2.1: The proposed change removes references to the Rod Sequence Control System (RSCS) in Item 2 of Bases Section 2.2.1 on page B 2-4.

By letter dated March 14, 1990 CP&L submitted a change request that would permit removal of the RSCS and reduce the RWM cutoff setpoint from 20% of rated thermal power to 10% rated thermal power. That change overlook the reference to RSCS in Item 2 of Bases Section 2.2.1. This proposed change corrects the previous oversight.

Item 6 Bases Section 2.2.1: The proposed change corrects the abbreviation for hydrogen water chemistry to HWC instead of WHC in item 6 of Bases Section 2.2.1 on page B 2-6.

TS 3.1.4.1: The proposed change corrects the spelling of 1) preset, instead of present, in action d, and 2) BPWS, instead of BWS, in action d.1. Both of these spelling errors are in TS 3.1.4.1 on page 3/4 1-14.

On March 14, 1990, CP&L requested a change to Technical Specifications for BSEP Units 1 and 2. These proposed changes were to revise the Specifications to permit the removal of the Rod Sequence Control System (RSCS) and reduce the Rod Worth Minimizer (RWM) cutoff setpoint from 20% rated thermal power to 10% rated thermal power. On August 9, 1990 CP&L submitted the typed Technical Specification change request pages to the NRC staff with the typographical errors to Specification 3.1.4.1. Therefore Amendment 144 was issued with typographical errors.

TS Table 4.3.4-1: The proposed change removes references to the RSCS in item g of the Notes on page 3/4 3-52.

On March 14, 1990, CP&L requested a change to Technical Specifications for BSEP Units 1 and 2. These proposed changes were to revise the Specifications to permit the removal of the Rod Sequence Control System (RSCS) and reduce the Rod Worth Minimizer (RWM) cutoff setpoint from 20% rated thermal power to 10% rated thermal power. That submittal failed to identify references to the RSCS in Note g of Table 4.3.4-1.

TS Table 3.3.5.5-1: The proposed change revises the labeling of items included in Technical Specification Table 3.3.5.5-1, on page 3/4 3-64a, to permit identification consistent with other Technical Specifications and the surveillance test scheduling system.

The proposed change allows the items listed in Table 3.3.5.5-1 to be labeled in a manner consistent with other Technical Specifications and the surveillance test scheduling system being used. As such, the proposed change is editorial in nature.

TS Table 4.3.5.5-1: The proposed change revises the labeling of items included in Technical Specification Table 4.3.5.5-1, on page 3/4 3-64c, to permit identification consistent with other Technical Specifications and the surveillance test scheduling system.

The proposed change allows the items listed in Table 3.3.5.5-1 to be labeled in a manner consistent with other Technical Specifications and the surveillance test scheduling system being used. As such, the proposed change is editorial in nature.

TS 4.3.6.1.1: The proposed change revises the reference from the non-existent TS Table 4.3.6.1.1-1 to provide a correct reference to TS Table 4.3.6.1-1, on page 3/4 3-88.

This error in the Technical Specifications first appears in the Amendment issued by the NRC on May 2, 1979. By letter dated April 27, 1979, CP&L supplemented several recent requests and requested issuance of an ATWS recirculation pump trip Specification based on Standard Technical Specifications. The ATWS recirculation pump trip specification was not included in earlier CP&L requests and was added at the request of the NRC.

Included in these earlier CP&L requests was an End of Cycle Recirculation Pump Trip (EOC-RPT) Specification for Unit 2. The Specifications issued by the NRC therefore had two specifications relating to recirculation pump trips. Because the EOC-RPT Specification was not included in the standards the NRC had to re-number the two recirculation pump trip Specifications for Unit 2. The ATWS Recirculation Pump Trip Specification was designated 3/4.3.6.1 and the EOC-RPT Specification was designated 3/4.3.6.2. For consistency, even though Unit 1 did not have two recirculation pump trip Specifications because the EOC-RPT had not been installed on that unit, the NRC issued the Unit 1 Specification for the ATWS Recirculation Pump Trip as 3/4.3.6.1.

The tables associated with these Specifications were also re-numbered but with a slightly different numbering scheme. While references to the tables cited in the limited condition for operation portion of these specifications were made consistent with the numbering scheme used for the tables, the reference to the table for surveillance requirements was not. This inconsistency has been corrected for Unit 2 but remains for Unit 1. The proposed change will correct this reference for Unit 1.

TS 3.4.2: The purpose of this proposed change is to correct a typographical error indicating an extraneous second footnote on page 3/4 4-4.

This Specification was last revised by Amendment 66 which was issued March 6, 1984. By a letter dated September 7, 1982 CP&L had requested a change to this Specification. The page submitted by CP&L included two footnotes. The second of these footnotes identified lift settings for the safety relief valves that would be effective until completion of the T-quencher modification. This second footnote was introduced with Amendment 29 (July 1, 1980) and modified with Amendment 30 (September 29, 1980). The amended page as issued by the NRC on March 6, 1984 omitted the second footnote but did not omit the indication of a second footnote in the limiting condition for operation.

Brunswick Unit 2

TS Table 2.2.1-1: The purpose of this proposed change is to correct a typographical error in item 2.b under allowable values by changing 115% to 115.5% on page 2-4

By letter dated October 12, 1989, the NRC issued Amendment 168 which included a revision to Specification Table 2.2.1-1 item 2.b. This revision specified an allowable value that was dependent on core flow but with a maximum value of 115.5% of rated thermal power. On February 6, 1990, the NRC issued Amendment 171 which revised items in Table 2.2.1-1 related to the main steam line radiation monitors. The changes to Table 2.2.1-1 effective with Amendment 168 were not incorporated into the changes to Table 2.2.1-1 effective with the issuance of Amendment 171. A correction to Amendment 171 was issued by the NRC on March 6, 1990. The March 6, 1990 correction however, introduced the typographical error specifying the maximum value of item 2.b to be 115% instead of 115.5% of rated thermal power. The proposed change will correct this typographical error.

Item 2 Bases Section 2.2.1: The proposed change removes references to the Rod Sequence Control System (RSCS) in Item 2 of Bases Section 2.2.1 on page B 2-4.

By letter dated March 14, 1990 CP&L submitted a change request that would permit removal of the RSCS and reduce the RWM cutoff setpoint from 20% of rated thermal power to 10% rated thermal power. That change overlooked the reference to RSCS in Item 2 of Bases Section 2.2.1. This proposed change corrects the previous oversight.

Item 10 Bases Section 2.2.1: The proposed change removes references to the Rod Sequence Control System (RSCS) and revises the bases description of the Select Rod Insertion consistent with removal of the RSCS in Bases Section 2.2.1, on pages B 2-7 and B 2-8.

By letter dated March 14, 1990 CP&L submitted a change request that would permit removal of the RSCS and reduce the RWM cutoff setpoint from 20% of rated thermal power to 10% rated thermal power. This change did not revise the bases description of the Select Rod Insert (SRI) contained in Bases Section 2.2.1 of the Unit 2 Technical Specifications (SRI is not installed in Unit 1).

TS 3.1.4.1: The proposed change corrects the spelling in action d.1 to BPWS, instead of BWPS. This spelling error is in TS 3.1.4.1 on page 3/4 1-14.

On March 14, 1990, CP&L requested a change to Technical Specifications for BSEP Units 1 and 2. This proposed change was to revise the Specifications to permit the removal of the Rod Sequence Control System (RSCS) and reduce the Rod Worth Minimizer (RWM) cutoff setpoint from 20% rated thermal power to 10% rated thermal power. On August 9, 1990 CP&L submitted the typed Technical Specification change request pages to the NRC staff which contained the typographical errors to Specification 3.1.4.1.

TS Table 4.3.1-1: The proposed change provides a grammatical correction by adding the word "is" in Note e of Technical Specification Table 4.3.1-1 for Unit 2, page 3/4 3-9.

This change will make the wording of the Unit 2 Technical Specification consistent with the existing wording of Unit 1.

TS Table 4.3.4-1: The proposed change removes references to the RSCS in item g of the Notes on page 3/4 3-52.

On March 14, 1990, CP&L requested a change to Technical Specifications for BSEP Units 1 and 2. These proposed changes were to revise the Specifications to permit the removal of the Rod Sequence Control System (RSCS) and reduce the Rod Worth Minimizer (RWM) cutoff setpoint from 20% rated thermal power to 10% rated thermal power. That submittal failed to identify references to the RSCS in Note g of Table 4.3.4-1.

TS Table 3.3.5.5-1: The proposed change revises the labeling of items included in Technical Specification Table 3.3.5.5-1, on page 3/4 3-64a, to permit identification consistent with other Technical Specifications and the surveillance test scheduling system.

The proposed change allows the items listed in Table 3.3.5.5-1 to be labeled in a manner consistent with other Technical Specification and the surveillance test scheduling system being used. As such, the proposed change is editorial in nature.

TS Table 4.3.5.5-1: The proposed change revises the labeling of items included in Technical Specification Table 4.3.5.5-1, on page 3/4 3-64c, to permit identification consistent with other Technical Specifications and the surveillance test scheduling system.

The proposed change allows the items listed in Table 3.3.5.5-1 to be labeled in a manner consistent with other Technical Specifications and the surveillance test scheduling system being used. As such, the proposed change is editorial in nature.

TS 3.3.6.2: The proposed change to Specification 3.3.6.2 eliminates the footnote, revises the specification whose actions are referenced in actions d and e and revises the applicability statement to make the limiting condition for operation applicable only when the MCPR limits obtained from the COLR and used in Specification 3.2.2.1 require the End of Cycle Recirculation Pump Trip.

Specification 3.3.6.2 which identifies operability requirements of EOC-RPT instrumentation was first issued by the NRC on May 2, 1979. This original Specification did not include actions d and e. With issuance of Amendment 51 on June 11, 1980, Specification 3.3.6.2 was revised to include actions d and e. Actions d and e included references to Specification 3.2.3 which was also introduced with Amendment 51.

The limiting condition of operation of Specification 3.2.3 specified the MCPR limits that were applicable and required the EOC-RPT system to be operable per Specification 3.3.6.2. In the event the EOC-RPT was not operable, action a of Specification 3.2.3 provided alternative fuel design and cycle exposure dependent MCPR limits. The MCPR limits with and without EOC-RPT operable were calculated by the fuel vendor and reported

to CP&L in cycle specific reload analysis documents. Action a also directly references Specification 3.3.6.2. Action b of Specification 3.2.3 was applicable if the MCPR was not within specifications with requirements to restore the MCPR within limits within 15 minutes or reduce THERMAL POWER to less than 25% of rated thermal power within the next 4 hours. The required actions of action b of Specification 3.2.3 are identical with the requirements of the action statement of current Specification 3.2.2.1.

The original planned mode of operation during cycle 4 for which Amendment 51 was requested was with the EOC-RPT operable. However, difficulties were experienced with making and keeping the EOC-RPT operable. The persistence of the EOC-RPT operability difficulties and the capability to operate without significant impact using alternative MCPR limits led eventually to the situation where operation without EOC-RPT was the de facto mode of operation as evidenced by the amendments issued for subsequent operating cycles.

The changes to Specification 3.2.3 made for cycle 5 and implemented with Amendment 71 (July 12, 1982) included creating Specifications 3.2.3.1 and 3.2.3.2. Specification 3.2.3.1 specified exactly the same action as Specification 3.2.3 for conditions when the MCPR was less than the applicable limit (action b) but revised action a to include MCPR limits that were applicable based on whether or not the scram time requirements for using ODYN Option B MCPR limits were met. The scram time requirements for applying ODYN Option B MCPR limits was specified in the new specification, Specification 3.2.3.2. Specification 3.2.3.1 included ODYN Option B and Option A MCPR limits that were fuel design and cycle exposure dependent. To maintain consistency with these changes, actions d and e of Specification 3.3.6.2 should have been changed at that time to reference Specification 3.2.3.1 instead of Specification 3.2.3. The proposed change to actions d and e corrects this omission.

With Amendment 101 (September 22, 1984) MCPR limits applicable for operation with the EOC-RPT operable were omitted from Specification 3.2.3.1. Amendment 101 also introduced a footnote to the applicability statement in specification 3.3.6.2 indicating the EOC-RPT would be manually bypassed during cycle 6. These changes precluded the use of MCPR limits with greater thermal limit margin that would otherwise be appropriate with the EOC-RPT operable.

Consistent with industry and NRC efforts to reduce the number of cycle dependent changes to the Technical Specifications, Amendment 123 (April 30, 1986) modified the footnote to refer to the "current" cycle instead of a specific cycle. Amendment 161 (July 13, 1989) implemented a non-cycle specific format for Specification 3.2.3.1 and finally with Amendment 168, Specification 3.2.3.1 was re-designated as 3.2.2.1.

Consistent with the need to know if the MCPR limits used in Specification 3.2.2.1 require EOC-RPT operability and to maintain a non-cycle specific format the proposed change revises the applicability statement of Specification 3.3.6.2 to specify that the limiting condition for operation is only applicable if the values of the MCPR limits used in Specification 3.2.2.1 require EOC-RPT. This revision also ensures that a cycle specific document will be used to determine applicability of the specification and justifies removal of the footnote.

Bases Section 3/4.1.4: The proposed change corrects the Reference number, from Reference 5 to Reference 6, in the bases for Specification 3/4.1.4 on page B 3/4 1-4.

On March 14, 1990, CP&L requested a change to Technical Specifications for BSEP Units 1 and 2. These proposed changes were to revise the Specifications to permit the removal of the Rod Sequence Control System (RSCS) and reduce the Rod Worth Minimizer (RWM) cutoff setpoint from 20% rated thermal power to 10% rated thermal power. On August 9, 1990 CP&L submitted the typed Technical Specification change request pages to the NRC staff with the typographical errors to Specification 3.1.4.1. Therefore Amendment 144 was issued with typographical errors.

ENCLOSURE 2

BRUNSWICK STEAM ELECTRIC PLANT, UNITS 1 AND 2
NRC DOCKET NOS. 50-325 & 50-324
OPERATING LICENSE NOS. DPR-71 & DPR-62
REQUEST FOR LICENSE AMENDMENT
ADMINISTRATIVE CHANGES

10 CFR 50.91(a) EVALUATION

Pursuant to 10 CFR 50.91(a)(ii) this evaluation responds to the staffs proposed determination, under the standards in 10 CFR 50.92(c) for determining whether a significant hazards consideration exists. A proposed amendment to an operating license for a facility involves no significant hazards consideration if operation of the facility in accordance with the proposed amendment would not: (1) involve a significant increase in the probability or consequences of an accident previously evaluated, (2) create the possibility of a new or different kind of accident from any accident previously evaluated, or (3) involve a significant reduction in a margin of safety. Carolina Power & Light Company has reviewed this proposed license amendment request and believes its adoption would not involve a significant hazards consideration. The bases for this determination are as follows:

Proposed Change:

The proposed change revises the Brunswick Technical Specifications by correcting several typographical errors, incorporating material implicitly contained in a footnote to an applicability statement, providing detailed labels for items listed in a table, correcting the citation of references, and removing references to the Rod Sequence Control System (RSCS) not included in a previous change request. These changes are administrative in nature:

Basis

The change does not involve a significant hazards consideration for the following reasons:

1. The proposed amendment does not involve a significant increase in the probability or consequences of an accident previously evaluated because the proposed change is administrative in nature. These changes do not alter the configuration or operation of the facility. The Limiting Safety Systems Settings and Safety Limits specified in the current Technical Specifications remain unchanged.

2. The proposed amendment does not create the possibility of a new or different kind of accident from any accident previously evaluated. The safety analysis of the facility remains complete and accurate. There are no physical changes to the facility and the plant conditions for which the design basis accidents have been evaluated are still valid. The operating procedure and emergency procedures are unaffected with the possible exception of resolving special notations that may have recognized the typographical errors that are being corrected.

3. The margins of safety are established through the Limiting Conditions of Operation, Limiting Safety Systems Settings and Safety Limits specified in the Technical Specifications. Since there are no changes to the physical design or operation of the facility, these margins will not be changed.

ENCLOSURE 3

BRUNSWICK STEAM ELECTRIC PLANT, UNITS 1 AND 2
NRC DOCKET NOS. 50-325 & 50-324
OPERATING LICENSE NOS. DPR-71 & DPR-62
REQUEST FOR LICENSE AMENDMENT
ADMINISTRATIVE CHANGES

ENVIRONMENTAL CONSIDERATIONS

10 CFR 51.22(c)(9) provides criterion for and identification of licensing and regulatory actions eligible for categorical exclusion from performance of an environmental assessment. A proposed amendment to an operating license for a facility requires no environmental assessment if operation of the facility in accordance with the proposed amendment would not: (1) involve a significant hazards consideration; (2) result in a significant change in the types or significant increase in the amounts of any effluents that may be released offsite; (3) result in an increase in individual or cumulative occupational radiation exposure. Carolina Power & Light Company has reviewed this request and believes the proposed amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment needs to be prepared in connection with the issuance of the amendment. The basis for this determination follows:

Proposed Change:

The proposed change will revise the Brunswick Technical Specifications by correcting several typographical errors, directly incorporating material implicitly contained in a footnote to an applicability statement, providing detailed labels for items listed in a table, correcting the citation of references, and removing references to the Rod Sequence Control System (RSCS) not included in a previous change request. These changes are administrative in nature.

Basis:

The change meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9) for the following reasons:

1. As demonstrated in Enclosure 2, the proposed amendment does not involve a significant hazards consideration.

2. The proposed amendment does not result in a significant change in the types or significant increase in the amounts of any effluents that may be released offsite. The proposed amendment corrects several typographical errors, incorporates material implicitly contained in a footnote to an applicability statement, provides detailed labels for items listed in a table, corrects the citation of references, and removes references to the Rod Sequence Control System (RSCS) not included in a previous change request. These changes are administrative in nature. Therefore, the amendment has no effect on the amounts of any effluents that may be released offsite.

3. The proposed amendment does not result in an increase in individual or cumulative occupational radiation exposure. The proposed amendment involves no plant or equipment modifications. The proposed amendment corrects several typographical errors, directly incorporates material implicitly contained in a footnote to an applicability statement, provides detailed labels for items listed in a table, corrects the citation of references, and removes references to the Rod Sequence Control System (RSCS) not included in a previous change request. These changes are administrative in nature. Therefore, the amendment has no effect on either individual or cumulative occupational radiation exposure.

ENCLOSURE 4

BRUNSWICK STEAM ELECTRIC PLANT, UNITS 1 AND 2
NRC DOCKET NOS. 50-325 & 50-324
OPERATING LICENSE NOS. DPR-71 & DPR-62
REQUEST FOR LICENSE AMENDMENT
ADMINISTRATIVE CHANGES

PAGE CHANGE INSTRUCTIONS

UNIT 1

<u>Removed Pages</u>	<u>Inserted Pages</u>
B 2-4	B 2-4
B 2-6	B 2-6
3/4 1-14	3/4 1-14
3/4 3-52	3/4 3-52
3/4 3-64a	3/4 3-64a
3/4 3-64c	3/4 3-64c
3/4 3-88	3/4 3-88
3/4 4-4	3/4 4-4

PAGE CHANGE INSTRUCTIONS (CONTINUED)

Unit 2

<u>Removed Pages</u>	<u>Insert Pages</u>
2-4	2-4
B 2-4	B 2-4
B 2-7	B 2-7
B 2-8	B 2-8
3/4 1-14	3/4 1-14
3/4 3-9	3/4 3-9
3/4 3-52	3/4 3-52
3/4 3-64a	3/4 3-64a
3/4 3-64c	3/4 3-64c
3/4 3-93	3/4 3-93
B 3/4 1-4	B 3/4 1-4