

UNITED STATES NUCLEAR REGULATORY COMMISSION

WASHINGTON, D.C. 20556-0001

February 22, 1994

Docket No. 52-004

Mr. Patrick W. Marriott, Managgar Licensing & Consulting Services GE Nuclear Energy 175 Curtner Avenue San Jose, California 95125

Dear Mr. Marriott:

SUBJECT: REQUEST FOR ADDITIONAL INFORMATION (RAI) REGARDING THE SIMPLIFIED BOILING WATER REACTOR (SBWR) DESIGN (Q260.1-Q260.8)

The staff has determined that it needs additional information to support its review activities related to the SBWR design certification. Some additional information on quality assurance described in Sections 17.1 and 17.2 of the SBWR standard safety analysis report (SSAR) is needed (Q260.1-Q260.8). Please provide a written response to the enclosed questions within 90 days of the date of this letter.

You have previously requested that portions of the information submitted in the August 1992, application for design certification of the SBWR plant, as supplemented in February 1993, be exempt from mandatory public disclosure. The staff has not completed its review of your request in accordance with the requirements of 10 CFR 2.790; therefore, that portion of the submitted information is being withheld from public disclosure pending the staff's final determination. The staff concludes that this RAI does not contain those portions of the information for which you are seeking exemption. However, the staff will withhold this letter from public disclosure for 30 calendar days from the date of this letter to allow GE the opportunity to verify the staff's conclusions. If, after that time, you do not request that all or portions of the information in the enclosure be withheld from public disclosure in accordance with 10 CFR 2.790, this letter will be placed in the NRC's Public Document Room.

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^{*}The numbers in parentheses designate the tracking numbers assigned to the questions.

This RAI affects nine or fewer respondents, and therefore, is not subject to review by the Office of Management and Budget under P.L. 96-511.

If you have any questions regarding this matter, please contact me at (301) 504-1178 or Mr. Son Ninh at (301) 925-1125.

Sincerely,

Original Signed By:

Melinda Malloy, Project Manager Standardization Project Directorate Associate Directorate for Advanced Reactors and License Renewal Office of Nuclear Reactor Regulation

Enclosure:

RAI on the SBWR Design

cc w/enclosure: See next page

Distribution (w/enclosure):

*Central File PDST R/F *PDR SNinh RBorchardt PShea

TMurley/FMiraglia,12G18 WRussell, 12G18

RGramm, 10A19 MFinkelstein, 15818 GZech, 10A19 TCarter, 11A1

GSuh (2), 12E4

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FAllenspach, 10A19

JMoore, 15818 ACRS (11)

*To be held for 30 days

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| OFC | LA: PDST | PM: PDST | PM: PDST | SC: POST |
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| DATE | 2/3/94 | 2/30/94 | 2/2/94 | 2/22/94 |

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DOCUMENT NAME: SBWR9412.MM

Mr. Patrick W. Marriott GE Nuclear Energy

cc: Mr. Laurence S. Gifford GE Nuclear Energy 12300 Twinbrook Parkway Suite 315 Rockville, Maryland 20852

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Mr. Victor G. Snell, Director Safety and Licensing AECL Technologies 9210 Corporate Boulevard Suite 410 Rockville, Maryland 20850

REQUEST FOR ADDITIONAL INFORMATION (RAI) ON THE SIMPLIFIED BOILING WATER REACTOR (SBWR) DESIGN

Quality Assurance (QA)

- 260.1 Describe what is meant by the term "SBWR Team Members," which is used in several places in Section 17.1 of the standard safety analysis report (SSAR). In addition, identify the Team Members.
- 260.2 SSAR Section 17.1.2 states that GE Nuclear Energy (GE-NE) and each of its Team Members have their own quality assurance program as described in Reference 17.1.4 ("SBWR Design and Certification Program Quality Assurance Plan," NEDG-31831). Provide a copy of NEDG-31831 and describe the method by which GE found the Team Member's QA programs to be acceptable.
- 260.3 Describe the provisions that control the design interfaces between GE-NE and the SEWR Team Members.
- 260.4 SSAR Section 19G.2.41, "Procedures for Feedback of Operating, Design and Construction Experience," regarding compliance with 10 CFR 50.34(f)(3)(i) states that this is a combined operating license (COL) license information requirement. Describe provisions for the feedback of important industry design experience into the SBWR design.
- 260.5 Describe what QA provisions apply to design-phase test programs, including the software QA controls.
- 260.6 Describe the QA provisions that apply to the appropriate items of Section 3.3.1.4 of ANSI/ANS-52.1-1983, "Nuclear Safety Criteria for the Design of Stationary Boiling Water Reactor Plants," including the safety parameter display system or its equivalent.
- 260.7 Provide a statement in SSAR Section 17.2 that the QA responsibilities for the operations phase is a COL responsibility.
- 260.8 Provide a statement that QA requirements for construction are a COL responsibility.