

APR 06 1981

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Docket Nos. 50-324/325

MEMORANDUM FOR: T. A. Ippolito, Chief
Operating Reactors Branch No. 2, DL

FROM: W. P. Gammill, Chief
Effluent Treatment Systems Branch, DSI

SUBJECT: PROGRESS REPORT ON AOG MODIFICATIONS AT
BRUNSWICK STEAM ELECTRIC PLANT

By letter, March 23, 1981, Carolina Power and Light Company provided a modification on the schedule for the redesign, purchase and installation of the recombiner/charcoal adsorber (chilled) system to replace the cryogenic offgas system at Brunswick, Unit Nos. 1 and 2. The schedule commits to having Unit No. 1 modification installed, tested and operable May 1, 1983, and Unit No. 2 fully operational by December 31, 1983, with the old cryogenic equipment removed. Present operation has been in accordance with the interim technical specifications addressing the requirements of 10 CFR Part 50, Appendix I. We find that the modified schedule is acceptable.

The applicant should be informed that we consider that this major change to a radwaste treatment system will require a 10 CFR Part 50.90 review. For that review, we will need information to show conformance with Appendix I and that the modification can meet the acceptance criteria of Standard Review Plans in NUREG-0800 (formally NUREG-75/087). The amendment to the FSAR should be scheduled such that our safety evaluation can be completed and proposed changes to the radiological effluent technical specifications submitted six months prior to use of the modified AOG system.

Original signed by:
William P. Gammill,

William P. Gammill, Chief
Effluent Treatment Systems Branch
Division of Systems Integration

- cc: W. Kreger
- J. Hannon
- J. Van Vliet
- F. Congel
- R. Bangart
- C. Willis
- J. Boegli

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JSBoegli	CWillis	WPGammill			
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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

APR 8 1981

MEMORANDUM FOR: Harold R. Denton, Director
Office of Nuclear Reactor Regulation

FROM: Carlyle Michelson, Director
Office for Analysis and Evaluation
of Operational Data

SUBJECT: H.B. ROBINSON RCS LEAK ON JANUARY 29, 1981

The Engineering Evaluation Report for the Robinson event is enclosed for your information and consideration. The primary reason for the evaluation was the loss of approximately 6,000 gallons of reactor coolant water from two separate leaks in the letdown train of the Chemical and Volume Control Letdown System (CVCS).

The evaluation of the event did not identify any safety concerns or any required immediate actions by the NRC. There are four areas, however, which are referred to NRR for your consideration. These include:

1. Whether a requirement should be placed upon operating plants to establish a procedure for identification and recovery from a spurious safety injection actuation (if such a procedure is not already in place).
2. Whether criteria for terminating SI should include provisions for isolating charging since charging flow could be considered high pressure safety injection for very small breaks.
3. Whether there is a need for a direct reactor trip on a spurious safety injection actuation at other Westinghouse plants which do not have a direct trip.
4. Whether operation of the isolation valves in the CVCS at Robinson is causing the system to be operated in a manner which is contrary to its design bases.

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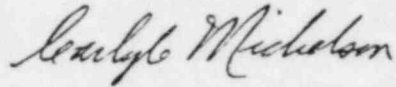
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Harold R. Denton

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Since these areas have been identified for information, a written response is not requested. No additional AEOD action is anticipated.

This event and the operator's response is a good example of an operating experience which should be disseminated to other licensees for information and training purposes. We have proposed this event to be included in Power Reactor Events.



Carlyle Michelson, Director
Office for Analysis and Evaluation
of Operational Data

Enclosure:
Engineering Evaluation Report
for H.B. Robinson

cc w/enclosure:
See Distribution