



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

PHILADELPHIA ELECTRIC COMPANY  
PUBLIC SERVICE ELECTRIC AND GAS COMPANY  
DELMARVA POWER AND LIGHT COMPANY  
ATLANTIC CITY ELECTRIC COMPANY

DOCKET NO. 50-277

PEACH BOTTOM ATOMIC POWER STATION, UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 75  
License No. DPR-44

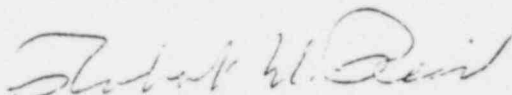
1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Philadelphia Electric Company, et al. (the licensee) dated August 8, 1980, as supplemented October 3, 1980, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and paragraph 2.C.(2) of Facility Operating License No. DPR-44 is hereby amended to read as follows:

Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 75, are hereby incorporated in the license. PECO shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

A handwritten signature in cursive script, appearing to read "Robert W. Reid".

Robert W. Reid, Chief  
Operating Reactors Branch #4  
Division of Licensing

Attachment:  
Changes to the Technical  
Specifications

Date of Issuance: November 19, 1980

ATTACHMENT TO LICENSE AMENDMENT NO. 75

FACILITY OPERATING LICENSE NO. DPR-44

DOCKET NO. 50-277

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages. The revised pages are identified by amendment number and contain vertical lines indicating the area of change.

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240n-240q (new pages)

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LIMITING CONDITIONS FOR OPERATION

SURVEILLANCE REQUIREMENTS

3.15 Seismic Monitoring  
Instrumentation

Applicability

Applies to the operational status of the seismic monitoring instrumentation.

Specifications

- A. The seismic monitoring instrumentation shown in Table 3.15 shall be operable.
- B. With one or more seismic monitoring instruments inoperable for more than 30 days, in lieu of any other report required by Specification 6.9.2, prepare and submit a Special Report to the Director of the appropriate Regional Office pursuant to Specification 6.9.3 within the next 10 working days outlining the cause of the malfunction and the plans for restoring the instrument(s) to operable status.
- C. The provisions of Specification 3.0.c are not applicable.

4.15 Seismic Monitoring  
Instrumentation

Applicability

Applies to the surveillance requirements of the seismic monitoring instrumentation.

Specifications

- A. Each of the required seismic monitoring instruments shall be demonstrated operable by the performance of the Instrument Check, Instrument Functional Test, and Instrument Calibration operations at the frequencies shown in Table 4.15.
- B. Each of the required seismic monitoring instruments actuated during a seismic event shall be restored to operable status within 24 hours and an Instrument Calibration performed within 5 days following the seismic event. Data shall be retrieved from actuated instruments and analyzed to determine the magnitude of the vibratory ground motion. In lieu of any other report required by Specification 6.9.2, a Special Report shall be prepared and submitted to the Director of the appropriate Regional Office pursuant to Specification 6.9.3 within 10 working days describing the magnitude, frequency spectrum and resultant effect upon facility features important to safety.

POOR ORIGINAL

TABLE 3.15\*\*

SEISMIC MONITORING INSTRUMENTATION

<u>Instruments and Sensor Locations#</u>	<u>Measurement Range</u>	<u>Minimum Instruments Operable</u>
1. Triaxial Time-History Accelerographs		
a. Containment Foundation (torus compartment)	0.1-10g	1
b. Refueling Floor	0.1-10g	1
c. RCIC Pump (Rm #7)	0.1-10g	1
d. "C" Diesel Generator	0.1-10g	1
2. Triaxial Peak Accelerographs		
a. Reactor Piping (Drywell)	0.01-2g	1
b. Refueling Floor	0.01-2g	1
c. "C" Diesel Generator	0.01-2g	1
3. Triaxial Response-Spectrum Recorders		
a. Cable Spreading Rm	0.1-10g	1*

\* With reactor control room annunciation

\*\* Effective upon completion of installation.

# Seismic instrumentation located in Unit 2

POOR ORIGINAL

TABLE 4.15\*\*

SEISMIC MONITORING INSTRUMENTATION SURVEILLANCE REQUIREMENTS

<u>Instruments and Sensor Locations#</u>	<u>Instrument*</u> <u>Check</u>	<u>Instrument*</u> <u>Functional</u> <u>Test</u>	<u>Instrument*</u> <u>Calibration</u>
1. Triaxial Time-History Accelerographs			
a. Containment Foundation (torus compartment)	M	SA	R
b. Refueling Floor	M	SA	R
c. PCIC Pump (Rm #7)	M	SA	R
d. "C" Diesel Generator	M	SA	R
2. Triaxial Peak Accelerographs			
a. Reactor Piping (Drywell)	NA	NA	R
b. Refueling Floor	NA	NA	R
c. "C" Diesel Generator	NA	NA	R
3. Triaxial Response-Spectrum Recorders			
a. Cable Spreading Rm	M	SA	R

\* Surveillance Frequencies

M: every month  
SA: every 6 months  
R: every 18 months

\*\* Effective on completion of installation.

# Seismic instrumentation located in Unit 2.



3.15/4.14 BASES

The operability of the seismic monitoring instrumentation ensures that sufficient capability is available to promptly determine the magnitude of a seismic event and evaluate the response of those features important to safety. This capability is required to permit comparison of the measured response to that used in the design basis for the plant.

The time-history recordings of the triaxial time-history accelerographs are done in the cable spreading room on a digital cassette accelerograph. In addition to being recorded, the containment foundation sensor is analyzed on line by a response spectrum analyzer. The spectrum of any sensor can be obtained by playing back its time-history cassette through the response spectrum analyzer.

## 6.9.2 Continued

- (3) Observed inadequacies in the implementation of administrative or procedural controls which threaten to cause reduction of degree of redundancy provided in reactor protection systems or engineered safety feature systems.
- (4) Abnormal degradation of systems other than those specified in item 2.a(3) above designed to contain radioactive material resulting from the fission process.

Note: Sealed sources or calibration sources are not included under this item. Leakage of valve packing or gaskets within the limits for identified leakage set forth in technical specifications need not be reported under this item.

6.9.3 Unique Reporting Requirements

Special reports shall be submitted to the Director of the appropriate Regional Office within the time period specified for each report. These reports shall be submitted covering the activities identified below pursuant to the requirements of the applicable reference specification:

- a. Loss of shutdown margin, Specification 3.3.A and 4.3.A within 14 days of the event.
- b. Reactor vessel inservice inspection, Specification 3.6.G and 4.6.G within 90 days of the completion of the reviews.
- c. Report seismic monitoring instrumentation inoperable for more than 30 days (Specification 3.15B) within the next 10 working days. Submit a seismic event analysis (Specification 4.15B) within 10 working days of the event.
- d. Primary containment leak rate testing approximately three months after the completion of the periodic integrated leak rate test (Type A) required by Specification 4.7.A.2.c.2. For each periodic test, leakage test results from Type A, B and C tests shall be reported. B and C tests are local leak rate tests required by Specification 4.7.A.2.f. The report shall contain an analysis and interpretation of the Type A test results and a summary analysis of periodic Type B and Type C tests that were performed since the last Type A test.



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PHILADELPHIA ELECTRIC COMPANY  
PUBLIC SERVICE ELECTRIC AND GAS COMPANY  
DELMARVA POWER AND LIGHT COMPANY  
ATLANTIC CITY ELECTRIC COMPANY

DOCKET NO. 50-278

PEACH BOTTOM ATOMIC POWER STATION, UNIT NO. 3

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 74  
License No. DPR-56

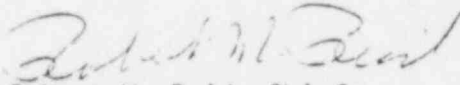
1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Philadelphia Electric Company, et al. (the licensee) dated August 8, 1980, as supplemented October 3, 1980, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and paragraph 2.C.(2) of Facility Operating License No. DPR-56 is hereby amended to read as follows:

Technical Specifications

The Technical Specifications contained in Appendix A and B, as revised through Amendment No. 74, are hereby incorporated in the license. PECO shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Robert W. Reid, Chief  
Operating Reactors Branch #4  
Division of Licensing

Attachment:  
Changes to the Technical  
Specifications

Date of Issuance: November 19, 1980

ATTACHMENT TO LICENSE AMENDMENT NO. 74

FACILITY OPERATING LICENSE NO. DPR-56

DOCKET NO. 50-278

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages. The revised pages are identified by amendment number and contain vertical lines indicating the area of change.

Remove Pages

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257

Insert Pages

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vii (new page)

240n-240q (new pages)

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A. Water Fire Protection System	A	240c
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LIMITING CONDITIONS FOR OPERATIONSURVEILLANCE REQUIREMENTS3.15 Seismic Monitoring  
InstrumentationApplicability

Applies to the operational status of the seismic monitoring instrumentation.

Specifications

- A. The seismic monitoring instrumentation shown in Table 3.15 shall be operable.
- B. With one or more seismic monitoring instruments inoperable for more than 30 days, in lieu of any other report required by Specification 6.9.2, prepare and submit a Special Report to the Director of the appropriate Regional Office pursuant to Specification 6.9.3 within the next 10 working days outlining the cause of the malfunction and the plans for restoring the instrument(s) to operable status.
- C. The provisions of Specification 3.0.c are not applicable.

4.15 Seismic Monitoring  
InstrumentationApplicability

Applies to the surveillance requirements of the seismic monitoring instrumentation.

Specifications

- A. Each of the required seismic monitoring instruments shall be demonstrated operable by the performance of the Instrument Check, Instrument Functional Test, and Instrument Calibration operations at the frequencies shown in Table 4.15.
- B. Each of the required seismic monitoring instruments actuated during a seismic event shall be restored to operable status within 24 hours and an Instrument Calibration performed within 5 days following the seismic event. Data shall be retrieved from actuated instruments and analyzed to determine the magnitude of the vibratory ground motion. In lieu of any other report required by Specification 6.9.2, a Special Report shall be prepared and submitted to the Director of the appropriate Regional Office pursuant to Specification 6.9.3 within 10 working days describing the magnitude, frequency spectrum and resultant effect upon facility features important to safety.



TABLE 3.15\*\*

SEISMIC MONITORING INSTRUMENTATION

<u>Instruments and Sensor Locations<sup>#</sup></u>	<u>Measurement Range</u>	<u>Minimum Instruments Operable</u>
1. Triaxial Time-History Accelerographs		
a. Containment Foundation (torus compartment)	0.1-10g	1
b. Refueling Floor	0.1-10g	1
c. RCIC Pump (Rm #7)	0.1-10g	1
d. "C" Diesel Generator	0.1-10g	1
2. Triaxial Peak Accelerographs		
a. Reactor Piping (Drywell)	0.01-2g	1
b. Refueling Floor	0.01-2g	1
c. "C" Diesel Generator	0.01-2g	1
3. Triaxial Response-Spectrum Recorders		
a. Cable Spreading Rm	0.1-10g	1*

\* With reactor control room annunciation

\*\* Effective upon completion of installation.

# Seismic instrumentation located in Unit 2

TABLE 4.15\*\*

SEISMIC MONITORING INSTRUMENTATION SURVEILLANCE REQUIREMENTS

<u>Instruments and Sensor Locations#</u>	<u>Instrument*</u> <u>Check</u>	<u>Instrument*</u> <u>Functional</u> <u>Test</u>	<u>Instrument*</u> <u>Calibration</u>
<b>1. Triaxial Time-History Accelerographs</b>			
a. Containment Foundation (torus compartment)	M	SA	R
b. Refueling Floor	M	SA	R
c. PCIC Pump (Rm #7)	M	SA	R
d. "C" Diesel Generator	M	SA	R
<b>2. Triaxial Peak Accelerographs</b>			
a. Reactor Piping (Drywell)	NA	NA	R
b. Refueling Floor	NA	NA	R
c. "C" Diesel Generator	NA	NA	R
<b>3. Triaxial Response-Spectrum Recorders</b>			
a. Cable Spreading Rm	M	SA	R

\* Surveillance Frequencies

M: every month  
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 R: every 18 months

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3.15/4.14 BASES

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## 6.9.2 Continued

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- b. Reactor vessel inservice inspection, Specification 3.6.G and 4.6.G within 90 days of the completion of the reviews.
- c. Report seismic monitoring instrumentation inoperable for more than 30 days (Specification 3.15B) within the next 10 working days. Submit a seismic event analysis (Specification 4.15B) within 10 working days of the event.
- d. Primary containment leak rate testing approximately three months after the completion of the periodic integrated leak rate test (Type A) required by Specification 4.7.A.2.c.2. For each periodic test, leakage test results from Type A, B and C tests shall be reported. B and C tests are local leak rate tests required by Specification 4.7.A.2.f. The report shall contain an analysis and interpretation of the Type A test results and a summary analysis of periodic Type B and Type C tests that were performed since the last Type A test.