

ENCLOSURE



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U.S. NUCLEAR REGULATORY COMMISSION  
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November 20, 1980

MP-1-1659

Mr. Boyce H. Grier  
Director, Region 1  
Office of Inspection and Enforcement  
U.S. Nuclear Regulatory Commission  
631 Park Avenue  
King of Prussia, Pennsylvania 19406

Reference: Provisional License DPR-21  
Docket No. 50-245  
Reportable Occurrence RO-80-19/1T

Dear Mr. Grier:

This letter forwards the Licensee Event Report for Reportable Occurrence RO-80-19/1T required to be submitted within 14 days pursuant to the requirements of the Millstone Unit 1 Technical Specifications, Section 6.9.1.8.i. An additional three copies of the report are enclosed.

Yours truly,

E. J. Mroczka  
Station Superintendent  
Millstone Nuclear Power Station

EJM/MJB:mw

Attachment: LER RO-80-19/1T

cc: Director, Office of Inspection and Enforcement, Washington, D.C. (30)  
Director, Office of Management Information and Program Control, Washington, D.C. (3)  
U. S. Nuclear Regulatory Commission, c/o Document Management Branch, Washington, D.C. 20555

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ATTACHMENT TO LER 80-19/1T  
NORTHEAST NUCLEAR ENERGY COMPANY  
MILLSTONE NUCLEAR POWER STATION-UNIT 1  
PROVISIONAL LICENSE NUMBER DPR-21  
DOCKET NUMBER 50-245

IDENTIFICATION OF OCCURRENCE

Performance of systems were discovered that require corrective measures to prevent operation in a manner less conservative than assumed in the Safety Analysis Report.

CONDITIONS PRIOR TO OCCURRENCE

Prior to the occurrence the plant was shut down for a planned refueling outage; the Isolation Condenser system was out of service for performance of Class II Inservice Inspection.

DESCRIPTION OF OCCURRENCE

On November 6, 1980 at 0925 hours during Class II Inservice Inspection of the Isolation Condenser System, liquid penetrant examination of the inside diameter of the steam supply piping nozzle-to-isolation condenser vessel weld revealed circumferential cracking on both sides of the weld in the heat affected zone.

APPARENT CAUSE OF OCCURRENCE

The cause of weld cracking is not known at the present time; an investigation is in progress. Results will be provided in an update report when the investigation is completed.

ANALYSIS OF OCCURRENCE

The Isolation Condenser System is provided for core decay heat removal following reactor isolation and scram, and is therefore not required to be operable at the present time. Degradation of the weld in question did not result in a condition that has not been previously analyzed.

CORRECTIVE ACTION

Evaluation of the weld in question is in progress. The affected area of pipe will be repaired, replaced or modified as necessary to restore it to its original design integrity. In addition, ultrasonic testing of all remaining welds in the Isolation Condenser System supply and return piping is being performed. Alternate means of examination, radiographic or liquid penetrant, will be used in addition to ultrasonic testing in areas where indications are noted, and evaluation of these areas will also be performed. Results of examinations and any corrective action will be provided in an update report following completion of the tasks.