NRC FORM 366 **U.S. NUCLEAR REGULATORY COMMISSION** (7.77) LICENSEE EVENT REPORT $\square(1)$ (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION) CONTROL BLOCK: 0 0 3 0 0 0 0 0 0 -4 0 0 GAE I CON!T 3 2 1 7 1 2 0 9 8 2 8 0 1 0 6 8 3 ER 68 69 EVENT DATE 74 75 REPORT DATE 8 REPORT 0 5 0 0 0 1 L(6)01 SOURCE DOCKET NUMBER EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10) On 12/9/82, the "F" & "H" Main Steam Safety Relief valves operated at 0 1076 & 1108 psig respectively instead of the 1090 psig +/- 1%, require- | 0 3 |ment of Tech. Specs. 2.2.A.l.b. The "G" valve was missing the pilot 04 [inle't tube. The same model valve is used on Unit 2. The health and safety of the public were not affected by this repetitive event as last reported on LER 50-321/1982-060; these events are generic for the Model 7567F valve. 80 CAUSE CAUSE COMP VALVE COMPONENT CODE SUBCODE F (15) E (12) X (13) V A L V E X (14 B (16) H B (11) OCCURRENCE REVISION SEQUENTIAL REPORT TYPE EVENT YEAR REPORT NO. CODE REPORT 191 01 L 01 0 9 31 SHUTDOWN ATTACHMENT NPRD-4 FORM SUB COMPONENT PRIME COMP. EFFECT ON PLANT ACTION FUTURE HOURS (22) SUPPI IFR MANUFACTURER Y T1012 Z (20) Z (21) 010101 IN N Z (19 (24) (25) (23) 18) CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27) Event attributed to setpoint drift for "F" and "H" valves. Missing pilot inlet tube attributed to tube support welds failure. Missing tube replaced on "G" valve, and "F" and "H" valves calibrated. Design Change was incorporated on Unit 1 valves to improve tube-to-valve body attachment. Unit 2 "E" valve modified previously. Others scheduled next refueling. 80 METHOD OF DISCOVERY (30)DISCOVERY DESCRIPTION (32) OTHER STATUS % POWER 0 0 0 29 D (31) Notification from Wyle Labs 80 ACTIVITY CONTENT LOCATION OF RELEASE (36) AMOUNT OF ACTIVITY (35 Z (33) Z (34) NA 80 PERSONNEL EXPOSURES DESCRIPTION (39) 0 0 0 (37) Z (38) NA 80 PERSONNEL INJURIES DESCRIPTION (41) NUMBER 0 0 (40) NA 80 LOSS OF OR DAMAGE TO FACILITY (43) DESCRIPTION 7 (42) NA 8301180333 830106 PUBLICITY NRC USE ONLY DESCRIPTION (45) PDR ADOCK 0500032 PDR N (44) NA 69 80 PHONE (912) 367-7851 NAME OF PREPARER S. B. Tipps

LER No.: 50-321/1982-099 Facility: Edwin I. Hatch Licensee: Georgia Power Company Docket #: 50-321

Narrative Report for LER 50-321/1982-099

On December 9, 1982, problems were discovered with three of the eleven Main Steam Safety Relief Valves; all eleven had been removed for testing per Tech. Specs. 4.6.H.l. Two of the valves setpoints were found to be out of the specified tolerances per Tech. Specs. 2.2.A.l.b., which allows a plus or minus 1% error. Thus, Tech. Specs. required setting is 1090 psig +/- 1% (1079 co 1101 psig). The "F" valv, was found to be set at 1076 psig. The "H" valve was found to be set at 1108 psig. The "G" valve was found to be missing the pilot inlet tube from the valve body. The missing tube did not affect the valve actuation setpoint. The same model safety relief valve is used on Unit 2. Plant operation was not affected by these events. This is a repetitive occurrence for the setpcint drift of the "H" valve as last reported on LER 50-321/1982-060. These events (i.e., setpoint drift and pilot inlet tube problems) are generic for this Target Rock model 7567F safety relief valve.

The events are attributed to setpoint drift for the "F" and "H" valves. The missing pilot inlet tube for the "G" valve is attributed to failure of the tube support welds. The tube-to-plate welds were found broken, with the support plates intact and the tube missing. A Design Change was incorporated on the Unit 1 valves to improve the tube-to-valve body attachment. A new support comprised of a single strap which fits over the tube is used. The strap is welded to the valve body wall on both sides and is tack welded to the tube as well. The Unit 2 "E" valve was previously modified in this manner, and the remainder of the Unit 2 valves are scheduled for modification during the next Refueling Outage.