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7)	Followup report LICENSEE EVENT REPORT
	CONTROL BLOCK:
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N'T 1	REPORT SOURCE L 6 0 5 0 0 2 5 9 0 0 7 1 1 8 0 3 1 0 0 3 8 0 9 EVENT DESCRIPTION AND PROBABLE CONSEQUENCES 10
2	During normal operation, the technical specification limit of 15.20 kw/rc. for
[3]	linear heat generation rate was exceeded in one node. The node was 13.41 Kw/ft.
4	A re-evaluation of the event indicated that the unit was put at steady-state. The
5	rod moves associated with the event and subsequent corrective action all occurred
6	within less than 15 minutes. Therefore, the limits of technical specifications
7	section 3.5.J were not applicable and the event should not have been reported.
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9	SYSTEM CAUSE CAUSE COMPONENT CODE COMPONENT CODE SUBCODE SUBCO
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Tennessee Valley Authority Browns Ferry Nuclear Plant Form BF-17 BF 15.2 1/10/79

LER SUPPLEMENTAL INFORMATION

BFRO-50- 259 / 8056 Technical Specification Involved 3.5.J

Reported Under Technical Specification 6.7.2.b(2)

Date of Occurrence 7/11/80 Time of Occurrence 2233 Unit 1

Identification and Description of Occurrence:

Following four control-rods being withdrawn 5 notches, the process computer showed one node out of limits for linear heat generation rate by .13 Kw/ft.

(Cancelled (see form 366)

Conditions Prior to Occurrence:

Unit 1 @ 863 MWe Unit 2 @ 976 MWe Unit 3 @ 0 MWe

Action specified in the Technical Specification Surveillance Requirements metdue to inoperable equipment. Describe:

N/A

Apparent Cause of Occurrence:

Problem caused by withdrawing the four control rods 5 notches instead of 3 notches.

Analysis of Occurrence:

There was no damage to plant equipment. There was no activity release, no personnel exposure or injury and no danger to the health or safety of the public.

Corrective Action:

Immediately, the four withdrawn control rods were inserted three notches, which brought the LHGR to within limits.

Failure Data:

*Revision:

*Retention: Period - Lifetime; Responsibility - Administrative Supervisor