

LIMITING CONDITIONS FOR OPERATION

SURVEILLANCE REQUIREMENTS

3.7 Primary Containment

3. Pressure Suppression Chamber - Reactor Building Vacuum Breakers

a. Except as specified in 3.7.A.3.b below, two pressure suppression chamber - reactor building vacuum breakers shall be operable at all times when primary containment integrity as required. The setpoint of the differential pressure instrumentation which actuates the pressure suppression chamber - reactor building breakers shall be 0.5 psig.

b. From and after the date that one of the pressure suppression chamber - reactor building vacuum breakers is made or found to be inoperable for any reason, reactor operation is permissible only during the succeeding seven days unless such vacuum breaker is sooner made operable, provided that the repair procedure does not violate primary containment integrity.

4. Drywell-Pressure Suppression Chamber Vacuum Breakers

a. When primary containment is required, all drywell-pressure suppression chamber vacuum breakers shall be operable except during testing and as stated in Specifications 3.7.A.4.b, c and d, below. Drywell-pressure suppression chamber vacuum breakers shall be considered operable if:

(1) The valve is demonstrated to open with the applied force of the installed test actuator as indicated by the position switches and remote position indicating lights.

(2) The valve shall return by gravity when released after being opened by remote or manual means, to within 3/32" of the fully closed position.

(3) Neither of the two position alarm systems which annunciate on Panel C-7 and Panel 905 when any vacuum breaker opening exceeds 3/32", are in alarm.

4.7 Primary Containment

3. Pressure Suppression Chamber - Reactor Building Vacuum Breakers

a. The pressure suppression chamber - reactor building vacuum breakers and associated instrumentation including set point shall be checked for proper operation every three months.

4. Drywell-Pressure Suppression Chamber Vacuum Breakers

a. Periodic Operability Tests

(1) Once each month each drywell-pressure suppression chamber vacuum breaker shall be exercised and the operability of the valve and installed position indicators and alarms verified.

(2) A drywell to suppression chamber differential pressure decay rate test shall be conducted at least every 3 months.

3.7 Primary Containment

- b. Any drywell-suppression chamber vacuum breaker may be non-fully closed as determined by the position switches provided that the drywell to suppression chamber differential decay rate is demonstrated to be not greater than 25% of the differential pressure decay rate for the maximum allowable bypass area of 0.2 ft².
- c. Reactor operation may continue provided that no more than 2 of the drywell-pressure suppression chamber vacuum breakers are determined to be inoperable provided that they are secured or known to be in the closed position.
- d. If a failure of one of the two installed position alarm systems occurs for one or more vacuum breakers, reactor operation may continue provided that a differential pressure decay rate test is initiated immediately and performed every 15 days thereafter until the failure is corrected. The test shall meet the requirements of Specification 3.7.A.4.b.

5. Oxygen Concentration

- a. After completion of the startup test program and demonstration of plant electrical output, the primary containment atmosphere shall be reduced to less than 5% oxygen by volume with nitrogen gas during reactor power operation with reactor coolant pressure above 100 psig, except as specified in 3.7.A.5.b.

4.7 Primary Containment

- b. During each refueling outage:
- (1) Each vacuum breaker shall be tested to determine that the disc opens freely to the touch and returns to the closed position by gravity with no indication of binding.
 - (2) Vacuum breaker position switches and installed alarm systems shall be calibrated and functionally tested.
 - (3) At least 25% of the vacuum breakers shall be visually inspected such that all vacuum breakers shall have been inspected following every fourth refueling outage. If deficiencies are found, all vacuum breakers shall be visually inspected and deficiencies corrected.
 - (4) A drywell to suppression chamber leak rate test shall demonstrate that the differential pressure decay rate does not exceed the rate which would occur through a 1 inch orifice without the addition of air or nitrogen.

5. Oxygen Concentration

The primary containment oxygen concentration shall be measured and recorded at least once weekly.

BASES:

3.7.A & 4.7.A Primary Containment (Cont'd)

margin of safety for the primary containment in the event of a small break in the primary system.

Each drywell suppression chamber vacuum breaker is equipped with three switches. One switch provides full open indication only. Another switch provides closed indication and an alarm on Panel C-7 should any vacuum breaker come off its closed seat by greater than 3/32". The third switch provides a separate and redundant alarm on Panel 905 should any vacuum breaker come off its closed seat by greater than 3/32". The two alarms above are those referred to in Section 3.7.A.4.a(3) and 3.7.A.4.d.

Attachment B

Proposed Technical Specification Change

Proposed Change

Reference is made to Pilgrim Nuclear Power Station, Unit #1 Technical Specification, Appendix A, Section 3.3.B, "Control Rods", Item 3.3.B.5.

Remove the following: Item 3.3.B.5.c

"1.06"

Add the following: Item 3.3.B.5.c

...will remain above "the Safety Limit MCPR" assuming a single...

Reason for Change

In Item 3.3.B.5.c, reference is made to an MCPR of 1.06. From the basis it is clear that this refers to the Safety Limit MCPR. When retrofit fuel was loaded into the core during Reload 4, the Safety Limit was changed to 1.07 in Specification 1.1.A and other references to MCPR of 1.06 were replaced by "Safety Limit MCPR". Item 3.3.B.5.c was apparently overlooked.

Safety Considerations

This change does not present an unreviewed safety question as defined in 10 CFR 50.59(c). It has been reviewed and approved by the Operations Review Committee and reviewed by the Nuclear Safety Review and Audit Committee.

Schedule of Change

This change will be put into effect upon receipt of approval by NRC.

Fee Determination

Pursuant to 10 CFR 170.22, Boston Edison Company proposes this change as Class II.

LIMITING CONDITION FOR OPERATION

SURVEILLANCE REQUIREMENT

3.3.B. Control Rods

- 4. Control rods shall not be withdrawn for startup or refueling unless at least two source range channels have an observed count rate equal to or greater than three counts per second.
- 5. During operation with limiting control rod patterns, as determined by the Reactor Engineer, either:
 - a. Both RBM channels shall be operable: or
 - b. Control rod withdrawal shall be blocked: or
 - c. The operating power level shall be limited so that the MCPR will remain above the Safety Limit MPCR assuming a single error that results in complete withdrawal of any single operable control rod.

C. Scram Insertion Times

- 1. The average scram insertion time, based on the deenergization of the scram pilot valve solenoids as time zero, of all operable control rods in the reactor power operation condition shall be no greater than:

<u>% Inserted From Fully Withdrawn</u>	<u>Average Scram Insertion Times (set)</u>
10	.55
30	1.275
50	2.00
90	3.50

4.3.B. Control Rods

- 4. Prior to control rod withdrawal for startup or during refueling, verify that at least two source range channels have an observed count rate of at least three counts per second.
- 5. When a limiting control rod pattern exists, an instrument functional test of the RBM shall be performed prior to withdrawal of the designated rod(s) and daily thereafter.

C. Scram Insertion Times

- 1. Following each refueling outage, each operable control rod shall be subjected to scram time tests from the fully withdrawn position. If testing is not accomplished with the nuclear system pressure above 950 psig, the measured scram insertion time shall be extrapolated to reactor pressures above 950 psig using previously determined correlations. Testing of all operable control rods shall be completed prior to exceeding 40% rated thermal power.

Attachment C

Proposed Technical Specification Change

Proposed Change

Reference is made to Pilgrim Nuclear Power Station, Unit #1 Technical Specification, Appendix A, Section 6.9, "Reporting Requirements", Item 6.9.A.2.

Remove the following: Item 6.9.A.2

"a monthly basis ... appropriate Regional Office,"

Add the following: Item 6.9.A.2

... be submitted on "a monthly basis to Director, Office of Inspection and Enforcement, U.S. Nuclear Regulatory Commission, Washington, D. C. 20555," to arrive no ...

Reason for Change

The reporting requirements of Item 6.9.A.2 presently reflects the guidance specified in USNRC Regulatory Guide 10.1, "Compilation of Reporting Requirements for Persons Subject to NRC Regulation", Revision 3, dated May 1977. This change is proposed to reflect reporting guidelines specified in USNRC Regulatory Guide 10.1, Revision 4, dated October 1981. Compliance with the latest revision to this Regulatory Guide will assure timely and correct distribution of this report in accordance with the provisions of Title 10, Chapter 1, Code of Federal Regulations.

Safety Considerations

This change does not present an unreviewed safety question as defined in 10 CFR 50.59(c). It has been reviewed and approved by the Operations Review Committee and reviewed by the Nuclear Safety Review and Audit Committee.

Schedule of Change

This change will be put into effect upon receipt of approval by NRC.

Fee Determination

Pursuant to 10 CFR 170.22, Note 2 Boston Edison proposes this change be exempt from any fees, as it will be issued for the convenience of the NRC.

2. Monthly Operating Report. Routine reports of operating statistics, shutdown experience and forced reductions in power shall be submitted on a monthly basis to Director, Office of Inspection and Enforcement, U.S. Nuclear Regulatory Commission, Washington, D.C. 20555, to arrive no later than the 15th of each month following the calendar month covered by the report.

The Monthly Operating Report shall include a narrative summary of operating experience that describes the operation of the facility, including safety-related maintenance, for the monthly report period.

3. Occupational Exposure Tabulation. A tabulation of the number of station, utility and other personnel (including contractors) receiving exposures greater than 100 mrem/yr and their associated man-rem exposure according to work and job functions, e.g. reactor operations and surveillance, inservice inspection, routine maintenance, special maintenance (including a description), waste processing, and refueling shall be submitted on an annual basis. This tabulation supplements the requirements of 20.407 of 10 CFR 20. The dose assignment to various duty functions may be estimates based on pocket dosimeter, TLD, or film badge measurements. Small exposures totalling less than 20% of the individual total dose need not be accounted for. In the aggregate, at least 80% of the total whole body dose received from external sources shall be assigned to specific major work functions.