Mailing Address
Alabama Power Company
600 North 18th Street
Post Office Box 2641
Birmingham, Alabama 35291
Telephone 205 783-6081

F. L. Clayton, Jr.
Senior Vice President
Flintridge Building

Docket No. 50-348

Alabama Power
the southern electric system

January 10, 1983

Director of Nuclear Reactor Regulation
U. S. Nuclear Regulatory Commission
Washington, D. C. 20555

Attention: Mr. S. A. Varga

Joseph M. Farley Nuclear Plant - Unit 1
Inservice Inspection Program for ASME
Code Class 1, 2 and 3 Components

Gentlemen:

In accordance with the requirements of 10CFR50.55a(g)(6)(i), Alabama Power Company hereby requests that relief be granted from the requirements of the 1974 Edition through the Summer 1975 Addenda of the ASME Code, Section XI, Table IWB-2600 Item Numbers B1.14, B2.9 and B3.8. A complete description of the affected components, existing examination requirements, bases of request for relief and proposed alternative examinations are contained in Attachment I.

This request for relief supplements the request for relief submitted for Unit 1 by our letter of April 17, 1981. In addition, Alabama Power Company inadvertently omitted the required fee from this request.

This request is designated as Class III in accordance with 10CFR170.22 requirements. Enclosed is a check for $4,000.00 to cover the total amount of fees required.

Yours truly,

F. L. Clayton, Jr.

STB:bs
Enclosures
cc: Mr. R. A. Thomas
Mr. G. F. Trowbridge
Mr. J. P. O'Reilly
Mr. E. A. Reeves
Mr. W. H. Bradford

A001
W/Check
$4,000

Note 17 to the Class 1 Table

COMPONENT: CODE CLASS: 1

Reactor Vessel Cladding, Pressurizer Cladding and Steam Generator Cladding (Item Numbers B1.14, B2.9, and B3.8, Examination Categories B-I-1 and B-I-2).

EXAMINATION REQUIREMENT:

Visual examination of the reactor vessel cladding shall include at least six patches (each 36 sq. in.) evenly distributed in accessible sections of the vessel shell. The examinations performed during each inspection interval shall cover 100% of the patch areas. Visual examination of the pressurizer and steam generator vessel cladding shall include at least one patch (36 sq. in.) near each manway in the primary side of the vessel. The examinations performed during each inspection interval shall cover 100% of the patch areas. The examination of the patch areas in the pressurizer and steam generator may be performed at or near the end of the inspection interval.

BASIS FOR RELIEF:

Pressurizer and steam generator cladding examinations which must be performed by manual inspections inside the vessels are impractical because of the high radiation levels in the areas required to be examined. The reactor vessel cladding visual examination is impractical to perform because of the critical outage time required to remove the access plugs or fuel and core barrel that enable the use of cameras to assist in the examination.

ALTERNATIVE EXAMINATION:

The reactor vessel longitudinal and circumferential shell welds are volumetrically examined per Table IWB-2600, Item B1.1 Category B-A and Item B1.2 Category B-B. The pressurizer longitudinal and circumferential welds are volumetrically examined per Table IWB-2600, Item B2.1 Category B-B. The tube sheet-to-head weld on the steam generators are volumetrically examined per Table IWB-2600, Item B3.1 Category B-B. These weld examinations include the cladding on the weld and the adjacent base metal for a distance of one wall thickness on each side of the weld.

CONCLUSION:

Volumetric examinations which are required for the longitudinal and circumferential welds in vessels provide sufficient evidence of component structural integrity. Additional support for not requiring the present inspections is that later NRC approved editions of Section XI no longer require cladding inspections but allow for volumetric examinations. Based on this guidance, no additional examinations will be performed for categories B-I-1 and B-I-2 for the cladded areas included in this request and, in the cladded areas described in the original April 17, 1981 relief request.