

BOSTON EDISON COMPANY  
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BOSTON, MASSACHUSETTS 02199

WILLIAM D. HARRINGTON  
SENIOR VICE PRESIDENT  
NUCLEAR

January 14, 1983  
BECO. Ltr. #83-19  
Proposed Change #83-1

Mr. Domenic B. Vassallo, Chief  
Operating Reactors Branch #2  
Division of Licensing  
Office of Nuclear Reactor Regulation  
U.S. Nuclear Regulatory Commission  
Washington, D. C. 20555

License No. DPR-35  
Docket No. 50-293

PNPS Proposed Technical Specification Changes

Dear Sir:

Pursuant to 10 CFR 50.90, Boston Edison Company hereby proposes the following modifications to Facility Operating License No. DPR-35, Appendix A. The desired changes are described in Attachments A-C to this letter. In accordance with 10 CFR 170.22, "Schedule of Fees for Facility License Amendments", a check for the amount of \$7,600 will be submitted in the near future.

It should be noted that the Technical Specification Change shown in Attachment A is the result of commitments made in response to an IE Inspection Report Notice of Violation. Specifically, the submittal of Attachment A to the NRC addresses INC 82-22-04.

Should you have any questions concerning this submittal, please do not hesitate to contact us.

Very truly yours,

*W.D. Harrington*

*A001*

Attachments: A,B,C

3 signed originals and 37 copies

Commonwealth of Massachusetts )  
County of Suffolk )

Then personally appeared before me William D. Harrington, who, being duly sworn, did state that he is Senior Vice President - Nuclear of Boston Edison Company, the applicant herein, and that he is duly authorized to execute and file the submittal contained herein in the name and on behalf of Boston Edison Company and that the statements in said submittal are true to the best of his knowledge and belief.

My Commission expires: 4-14-89

*Marion M. DeCamp*  
Notary Public

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## Attachment A

### Proposed Technical Specification Change

#### Proposed Change

Reference is made to Pilgrim Nuclear Power Station, Unit #1 Technical Specification, Appendix A, Section 3.7.A, "Primary Containment", Item 3.7.A.4.a(3).

Remove the following: 3.7.A.4.a(3)

"The position alarm...opening exceeds 3/32".

Add the following: 3.7.A.4.a(3)

"Neither of the two position alarm systems which annunciate on Panel C-7 and Panel 905 when any vacuum breaker opening exceeds 3/32", are in alarm."

Remove the following: Item 4.7.A.4.a(3)

"Until such time...shall be performed."

Remove the following: Bases 3.7.A and 4.7.A

"Each dry-well suppression...longer be applicable."

Add the following: Bases 3.7.A and 4.7.A

Each drywell suppression chamber vacuum breaker is equipped with three switches. One provides full open indication only. Another provides closed indication and an alarm on Panel C-7 should any vacuum breaker come off its closed seat by greater than 3/32". The last provides a separate and redundant alarm on Panel 905 should any vacuum breaker come off its closed seat by greater than 3/32". The two alarms above are those referred to in Section 3.7.A.4.a(3) and 3.7.A.4.d.

#### Reason for Change

A change to Item 3.7.A.4.a(3) will clarify the ambiguous language that now exists. Specifically the words "when installed" are deleted and the proposed change details the applicable Drywell to Torus Vacuum Breaker alarms.

Item 4.7.A.4.a(3) is no longer needed because the position indicators with alarm in the control room are installed on the drywell-pressure suppression chamber vacuum breakers.

#### Safety Considerations

This change does not present an unreviewed safety question as defined in 10 CFR 50.59(c). It has been reviewed and approved by the Operations Review Committee and reviewed by the Nuclear Safety Review and Audit Committee.

#### Schedule of Change

This change will be put into effect upon receipt of approval by NRC.

#### Fee Determination

Pursuant to 10 CFR 170.22 Boston Edison Company proposes this change as Class II.