



Northern States Power Company

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January 14, 1983

Director
Office of Nuclear Reactor Regulation
U S Nuclear Regulatory Commission
Washington, DC 20555

PRAIRIE ISLAND NUCLEAR GENERATING PLANT
Docket Nos. 50-282 License Nos. DPR-42
50-306 DPR-60

License Amendment Request dated January 14, 1983
Response to NRC Generic Letter 82-16 dated September 20, 1982

Attached are three originals and 37 conformed copies of a request for a change of Technical Specifications, Appendix A, of Operating Licenses DPR-42 and 60. Also attached is one copy of the License Amendment class determination and a check in the amount of \$1,600 for the amendment fee.

This request contains an addition to Section 6.1 of the Technical Specifications regarding the overtime allowed for individuals working at the plant. With this submittal, all items in NRC Generic Letter 82-16 have been addressed:

1. STA Training (I.A.1.1.3)
Your September 20, 1982 letter still identifies this as an open item pending a decision by the Commission.
2. Limit Overtime (I.A.1.3)
A license amendment request reflecting NSP position is included with this letter.
3. Short Term Auxiliary Feedwater System (II.E.1.1)
The letter from NRC to NSP dated March 27, 1982, "NRC Letter and Safety Evaluation Report NUREG-0737, Items II.E.1.1 and II.E.1.2, Evaluation of Prairie Island Auxiliary Feedwater Systems", contained the resolution of this item. No additional Technical Specification changes are required in addition to those requested in our September 14, 1982 submittal.
4. Safety Grade AFW System Initiation and Flow Indication (II.E.1.2)
Same as item 3
5. Dedicated Hydrogen Penetrations (II.E.4.1)
Prairie Island is installing an internal recombiner system in each unit. Each unit is also provided with a purge and vent combustible gas control system with dedicated penetrations. Testing of these penetrations is required by the Technical Specifications. Your letter dated June 1, 1982 confirmed your acceptance of the Prairie Island Nuclear Generating Plant design with respect to the dedicated hydrogen control penetrations.

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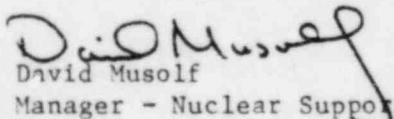
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6. Containment Pressure Setpoint (II.E.4.2.5)
The letter from NRC to NSP dated December 2, 1981, "TMI Task Action Plan, Item II.E.4.2(5) Containment Pressure Setpoint Prairie Island Units 1 & 2" resolved the issue. No Technical Specification changes are required.
7. Containment Purge Valve (II.E.4.2.6)
A license amendment request was submitted on October 29, 1982 which addressed this issue.
8. Radiation Signal on Purge Valves (II.E.4.2.7)
Prairie Island already has this requirement in their Technical Specifications in Table TS 3.5-4.
9. Upgrade B&W AFW System (II.K.2.8)
This item is not applicable to Prairie Island.
11. B&W Thermal-Mechanical Report (II.K.2.13)
This item is not applicable to Prairie Island.
12. Reporting SV and RV Failures and Challenges (II.K.3.3)
A license amendment request was submitted on September 14, 1982.
13. Anticipatory Trip on Turbine Trip (II.K.3.12)
Prairie Island already has this requirement in their Technical Specifications in TS 3.5-2 and Table TS 4.1-1.

Exhibit A describes the proposed change, reason for change, and the safety evaluation. Exhibit B contains the revised Appendix A pages. Please contact us if you have any questions concerning this License Amendment Request.


David Musolf
Manager - Nuclear Support Services

DMM/SAF/bd

cc: Regional Administrator, NRC
NRR Project Manager, NRC
NRC Resident Inspector
G Charnoff