NORTHEAST UTILITIES
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August 24, 1979

Docket No. 50-336

Director of Nuclear Reactor Regulation
Attn: Mr. R. Reid, Chief
Operating Reactors Branch #4
U. S. Nuclear Regulatory Commission
Washington, D. C. 20555

References: (1) W. G. Counsil letter to R. Reid dated August 22, 1979.
(2) W. G. Counsil letter to R. Reid dated August 24, 1979.

Gentlemen:

Millstone Nuclear Power Station, Unit No. 2
Feedwater System Piping

In References (1) and (2), Northeast Nuclear Energy Company (NNECO) provided information to the NRC Staff regarding the feedwater piping system and instrumentation which is being installed at Millstone Unit No. 2. At the verbal request of the NRC Staff, the following additional information is provided.

Existing Emergency Procedure 2509, Steam Line Rupture, is applicable to both steam line ruptures and feedwater line ruptures. Should a rupture occur in the vicinity of either steam generator nozzle, the current procedure identifies the appropriate actions for the plant operators. An MSI signal would automatically terminate feedwater flow and close the main steam isolation valves, thereby isolating the break. The affected steam generator would boil dry. Feedwater would be manually re-initiated as required to the intact loop for decay heat removal.

The criteria provided in Reference (2) are supplemented with the following:

(1) On-site acceptance criteria have been established as an initial screening tool for piping system acceptability, and

(2) Off-site criteria are established to evaluate total system performance with respect to design and fatigue crack growth conditions.

On-Site Acceptance Criteria

The strain gauge data will be evaluated to determine the magnitude and frequency of the dynamic stress due to mechanical or flow induced vibrations. The resulting stresses are to be within the design allowable per the ASME Section III, Division I, Figure I-9.1 (S/N) curves.

Off-Site Acceptance Criteria

The measured parameters of temperature, strain, and acceleration will be reduced to evaluate total piping system performance with respect to design. The criterion for acceptability of the piping system performance is that the stress intensities obtained from the measured data for each operating condition are less than the allowable design stress intensities specified in the ASME Code, Section III, Subsection NC-3650 and that the projected fatigue crack growth is bounded by the analysis previously submitted in Reference (1). A representative amount of this data will be reduced and evaluated within three (3) weeks of initial plant startup.

We trust you find the above information sufficient to resolve your concerns.

Very truly yours,

NORTHEAST NUCLEAR ENERGY COMPANY

W. G. Counsil

W. G. Counsil
Vice President