Route 3, 80x 1370 PLASSIVIA AR 72801 Tol 501-964-3100

March 15, 1994

1CAN039403

U. S. Nuclear Regulatory Commission Document Control Desk Mail Station PI-137 Washington, DC 20555

Subject:

Arkansas Nuclear One - Unit 1

Docket No. 50-313 License No. DPR-51 Monthly Operating Report

## Gentlemen:

The Arkansas Nuclear One - Unit 1 Monthly Operating Report (MOR) for February, 1994 is attached. This report is submitted in accordance with ANO-1 Technical Specification 6.12.2.3.

Very truly yours,

Dwight C. Mims Director, Licensing

DCM/jrh Attachment

JE24 /

U. S. NRC March 15, 1994 .1CAN039403 Page 2

cc: Mr. Leonard J. Calian
Regional Administrator
U. S. Nuclear Regulatory Commission
Region IV
611 Ryan Plaza Drive, Suite 400
Arlington, TX 76011-8064

NRC Senior Resident Inspector Arkansas Nuclear One - ANO-1 & 2 Number 1, Nuclear Plant Road Russellville, AR 72801

Mr. George Kalman NRR Project Manager, Region IV/ANO-1 U. S. Nuclear Regulatory Commission NRR Mail Stop 13-H-3 One White Flint North 11555 Rockville Pike Rockville, Maryland 20852

Mr. Thomas W. Alexion
NRR Project Manager, Region IV/ANO-2
U. S. Nuclear Regulatory Commission
NRR Mail Stop 13-H-3
One White Flint North
11555 Rockville Pike
Rockville, Maryland 20852

### OPERATING DATA REPORT

DOCKET NO:

A TEL

50-313

DATE

March 1, 1994

COMPLETED BY: TELEPHONE:

K. R. Hayes (501) 964-5535

## **OPERATING STATUS**

| 1.  | Unit Name: Arkansas Nuclear One - Unit 1  |  |  |  |  |
|-----|---|--|--|--|--|
| 2.  | Reporting Period: February 1-28, 1994   |  |  |  |  |
| 3.  | Licensed Thermal Power (MWt): 2,568   |  |  |  |  |
| 4.  | Nameplate Rating (Gross MWe): 902.74  |  |  |  |  |
| 5.  | Design Electrical Rating (Net MWe): 850   |  |  |  |  |
| 6   | Maximum Dependable Capacity (Gross MWe): 883  |  |  |  |  |
| 7.  | Maximum Dependable Capacity (Net MWe): 836  |  |  |  |  |
| 8.  | If Changes Occur in Capacity Ratings (Items Number 3 Through 7) Since Last Report, Give Reasons |  |  |  |  |
| 9   | Power Level To Which Restricted. If Any (Net MWe): None   |  |  |  |  |
| 10. | Reasons For Restrictions. If Any: None  |  |  |  |  |
|     |   |  |  |  |  |

|            |  | MONTH   | YR-TO-DATE | CUMULATIVE |
|------------|--|---------|------------|------------|
| 11.<br>12. | Hours in Reporting Period<br>Number of Hours Reactor was | 672.0   | 1416.0     | 168283.0   |
| 13.        | Critical Reactor Reserve Shutdown                        | 647.5   | 1391.5     | 121989.9   |
|            | Hours  | 0.0     | 0.0        | 5044.0     |
| 14.        | Hours Generator On-Line                                  | 641.7   | 1385.7     | 119729.3   |
| 15         | Unit Reserve Shutdown Hours                              | 0.0     | 0.0        | 817.5      |
| 16.        | Gross Thermal Energy Generated                           |         |            |            |
|            | (MWH)  | 1635270 | 3544960    | 275595523  |
| 17.        | Gross Electrical Energy                                  |         |            |            |
|            | Generated (MWH)  | 560260  | 1215935    | 92008970   |
| 18.        | Net Electrical Energy                                    |         |            |            |
|            | Generated (MWH)  | 536281  | 1165269    | 87492405   |
| 19         | Unit Service Factor                                      | 95.5    | 97.9       | 71.1       |
| 20.        | Unit Availability Factor                                 | 95.5    | 97.9       | 71.6       |
| 21         | Unit Capacity Factor                                     |         |            |            |
|            | (Using MDC Net)  | 95.5    | 98.4       | 62.2       |
| 22         | Unit Capacity Factor                                     |         |            |            |
|            | (Using DEC Net)  | 93.9    | 96.8       | 61.2       |
| 23.        | Unit Forced Outage Rate                                  | 4.5     | 2.1        | 11.1       |
| 24.        | Shutdowns Scheduled Over Next 6 Mont                     |         |            | ***        |

If Shut Down At End of Report Period Estimated Date of Startup:
 Units in Test Status (Prior to Commercial Operation):

INITIAL CRITICALITY INITIAL ELECTRICITY COMMERCIAL OPERATION Forecast

Achieved 08/06/74 08/17/74 12/19/74

## AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO: 50-313

UNIT: One

DATE: March 1, 1994

COMPLETED BY: K. R. Hayes

TELEPHONE: (501) 964-5535

## MONTH February, 1994

| DAY | AVERAGE DAILY POWER LEVEL |
|-----|---------------------------|
|     | (MWe-Net)                 |

| 1  | 363  |
|----|------|
| 2  | 35   |
| 3  | 827  |
| 4  | 844  |
| 5  | 843  |
| 6  | 846  |
| 7  | 846  |
| R  | 846  |
| 9  | 846  |
| 10 | 845  |
| 11 | 845  |
| 12 | 845  |
| 13 | 845  |
| 14 | 845  |
| 15 | 845  |
| 16 | 846  |
| 17 | 845  |
| 18 | 845  |
| 19 | 845  |
| 20 | 845  |
| 21 | 846  |
| 22 | 845  |
| 23 | 845  |
| 24 | 840  |
| 25 | 841  |
| 26 | 845  |
| 27 | 845  |
| 28 | 845  |
| 29 | #N/A |
| 30 | #N/A |
| 31 | #N/A |
|    |      |

AVGS: 798

# INSTRUCTION

On this format, list the average daily unit power level in MWe-Net for each day in reporting month. Complete to the nearest whole megawatt.

## UNIT SHUTDOWNS AND POWER REDUCTIONS REPORT FOR February, 1994

| DOCKET NO.   | 50-313        |  |  |  |
|--------------|---------------|--|--|--|
| UNIT NAME    | ANO Unit 1    |  |  |  |
| DATE         | March 1, 1994 |  |  |  |
| COMPLETED BY | K. R. Haves   |  |  |  |
| TELEPHONE    | 501-964-5535  |  |  |  |

| NO.   | DATE   | TYPE' | DURATION<br>(HOURS) | REASON <sup>2</sup> | METHOD OF<br>SHUTTING DOWN<br>REACTOR <sup>3</sup> | EVENT<br>REPORT # | SYSTEM<br>CODE* | COMPENENT<br>CODE <sup>5</sup> | CAUSE & CORRECTIVE ACTION TO  |
|-------|--------|-------|---------------------|---------------------|--|-------------------|-----------------|--------------------------------|---|
| 54-01 | 940201 | F     | 30.3                | A                   | 1  | 94-001            | JE              | LT                             | Unit taken off-line to repair a Steam<br>Generator level transmitter and an excore<br>nuclear instrumentation sensor. |

F: Forced S: Scheduled Reason:

2

A - Equipment Failure (Explain)

B - Maintenance of Test

C - Refueling

D - Regulatory Restriction

E - Operator Training & License Examination

F - Administration

G - Operational Error

H - Other (Explain)

3

Method:

1 - Manual

2 - Manual Scram.

3 - Automatic Scram.

4 - Continuation 5 - Load Reduction

9 - Other

Exisibit G - Instructions for Preparation of Data Entry Sheets for Licensee

Event Report (LER) File (NUREG-0161)

Exhibit I - Same Source

# NRC MONTHLY OPERATING REPORT

## OPERATING SUMMARY

#### FEBRUARY 1994

## UNIT ONE

Arkansas Nuclear One, Unit One, began the month operating at 100% power. On the First at 09:05 hours, the unit began a power decrease to hot shutdown to repair a steam generator level transmitter and an excore nuclear instrumentation sensor. The unit went off line at 12:43 hours on the first. The steam generator level transmitter was replaced and the excore instrument was repaired. The unit was put back on line at 19:00 hours on the second. Full unit load was reached at 02:20 hours on the third.

On the twenty fourth at 09:25 hours, the unit load was decreased to 98% power to allow isolation of a condenser waterbox for tube leakage repair. The unit was returned to 100% power at 11:50 hours on the same day. Unit 1 operated at full power for the remainder of the month.

DATE: February, 1994

# REFUELING INFORMATION

| 1. | Name of facility: Arkansas Nuclear One - Unit 1   |
|----|---|
| 2. | Scheduled date for next refueling shutdown. February 14, 1995   |
| 3. | Scheduled date for restart following refueling. April 7, 1995   |
| 4. | Will refueling or resumption of operation thereafter require a technical specification change or other license amendment? If answer is yes, what, in general, will there be? If answer is no, has the reload fuel design and core configuration been reviewed by your Plant Safety Review Committee to determine whether any unreviewed safety questions are associated with the core reload (Ref. 10 CFR Section 50.59)? |
|    | Unknown at this time.   |
| 5. | Scheduled date(s) for submitting proposed licensing action and supporting information.  |
|    | Unknown at this time  |
| 6. | Important licensing considerations associated with refueling, e.g., new or different fue design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures.   |
| 7. | The number of fuel assemblies (a) in the core and (b) in the spent fuel storage pool.   |
|    | a) 177 b) 685   |
| 8. | The present licensed spent fuel pool storage capacity and the size of any increase in licensed storage capacity that has been requested or is planned, in number of fuel assemblies.  |
|    | present 968 increase size by 0  |
| 9. | The projected date of the last refueling that can be discharged to the spent fuel poor assuming the present licensed capacity.  |
|    |   |

(Loss of full core off-load capability)

DATE: 1996

# IMAGE EVALUATION TEST TARGET (MT-3)







