OPERATING DATA REPORT

DOCKET NO. 50-267
DATE December 7, 1982
COMPLETED BY L. M. McBride
TELEPHONE (303) 785-2224

OPERATING STATUS

NOTES

1. Unit Name: Fort St. Vrain
2. Reporting Period: 821101 through 821130
3. Licensed Thermal Power (MWt): 842
4. Nameplate Rating (Gross MWe): 342
5. Design Electrical Rating (Net MWe): 330
6. Maximum Dependable Capacity (Gross MWe): 342
7. Maximum Dependable Capacity (Net MWe): 330
8. If Changes Occur in Capacity Ratings (Items Number 3 Through 7) Since Last Report, Give Reasons:
None
9. Power Level To Which Restricted, If Any (Net MWe): 231
10. Reasons for Restrictions, If Any: Restriction to 70% pending resolution of contractual matters.

		This Month	Year to Date	Cumulative
11.	Hours in Reporting Period	720	8016	29977
12.	Number of Hours Reactor Was Critical	538.8	4745.0	19323.4
13.	Reactor Reserve Shutdown Hours	0	0	0
14.	Hours Generator On-Line	0	3266.2	13174.5
15.	Unit Reserve Shutdown Hours	0	0	0
16.	Gross Thermal Energy Generated (MWH)	6541.9	1849177.5	6783122.3
17.	Gross Electrical Energy Generated (MWH)	0	635548	2326904
18.	Net Electrical Energy Generated (MWH)	-5330	573085	2127344
19.	Unit Service Factor	0	40.7	43.9
20.	Unit Availability Factor	0	40.7	43.9
21.	Unit Capacity Factor (Using MDC Net)	0	21.7	21.5
22.	Unit Capacity Factor (Using DER Net)	0	21.7	21.5
23.	Unit Forced Outage Rate	100.0	36.3	34.6

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): Plant recovery 821201 through 830115-1104.0 hours. Surveillance testing 830823 through 830422-744.0 hrs..
25. If Shut Down at End of Report Period, Estimated Date of Startup: Reactor is critical at low power.
26. Units In Test Status (Prior to Commercial Operation):

	Forecast	Achieved
INITIAL CRITICALITY	N/A	N/A
INITIAL ELECTRICITY	N/A	N/A
OPERATION	N/A	N/A

AVERAGE DAILY UNIT POWER LEVEL

Docket No. 50-267

Unit Fort St. Vrain

Date December 7, 1982

Completed By L. M. McBride

Telephone (303) 785-2224

Month November

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)	DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1	0.0	17	0.0
2	0.0	18	0.0
3	0.0	19	0.0
4	0.0	20	0.0
5	0.0	21	0.0
6	0.0	22	0.0
7	0.0	23	0.0
8	0.0	24	0.0
9	0.0	25	0.0
10	0.0	26	0.0
11	0.0	27	0.0
12	0.0	28	0.0
13	0.0	29	0.0
14	0.0	30	0.0
15	0.0	31	N/A
16	0.0		

*Generator on line but no net generation.

UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 50-267
UNIT NAME Fort St. Vrain
DATE December 7, 1982
COMPLETED BY L. M. McBride
TELEPHONE (303) 785-2224

REPORT MONTH November 1982

NO.	DATE	TYPE	DURATION	REASON	METHOD OF SHUTTING DOWN REACTOR	LER #	SYSTEM CODE	COMPONENT CODE	CAUSE AND CORRECTIVE ACTION TO PREVENT RECURRENCE
82-014	821101	F	720.0	H	3	N/A	IBH	INSTRU	Loop 1 Shutdown followed by reactor scram and turbine-generator trip on 820930. Outage continued due to primary coolant chemistry impurities.

REFUELING INFORMATION

1. Name of Facility.	Fort St. Vrain Unit No. 1
2. Scheduled date for next refueling shutdown.	September 1, 1983
3. Scheduled date for restart following refueling.	November 1, 1983
4. Will refueling or resumption of operation thereafter require a technical specification change or other license amendment?	Yes
If answer is yes, what, in general, will these be?	Use of type H-451 graphite.
If answer is no, has the reload fuel design and core configuration been reviewed by your Plant Safety Review Committee to determine whether any unreviewed safety questions are associated with the core reload (Reference 10CFR Section 50.59)?	----------
If no such review has taken place, when is it scheduled?	----------
5. Scheduled date(s) for submitting proposed licensing action and supporting information.	Not scheduled at this time; to be determined.
6. Important licensing considerations associated with refueling, e.g., new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures.	----------
7. The number of fuel assemblies (a) in the core and (b) in the spent fuel storage pool.	1482 HTGR fuel elements. 11 spent HTGR fuel elements
8. The present licensed spent fuel pool storage capacity and the size of any increase in licensed storage capacity that has been requested or is planned, in number of fuel assemblies.	Capacity is limited in size to about one-third of core (approximately 500 HTGR elements). No change is planned.

REFUELING INFORMATION (CONTINUED)

9. The projected dat of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity.	1992 under Agreements AT(04-3)-633 and DE-SC07-79ID01370 between Public Service Company of Colorado, General Atomic Company, and DOE.*

* The 1992 estimated date is based on the understanding that spent fuel discharged during the term of the Agreements will be stored by DOE at the Idaho Chemical Processing Plant. The storage capacity has evidently been sized to accomodate eight fuel segments. It is estimated that the eighth fuel segment will be discharged in 1992.