

AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-285
 UNIT Fort Calhoun Station
 DATE January 10, 1983
 COMPLETED BY T. P. Matthews
 TELEPHONE (402)536-4733

MONTH December, 1982

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)	DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1	<u>378.1</u>	17	<u>0.0</u>
2	<u>377.8</u>	18	<u>0.0</u>
3	<u>345.5</u>	19	<u>0.0</u>
4	<u>0.0</u>	20	<u>0.0</u>
5	<u>0.0</u>	21	<u>0.0</u>
6	<u>0.0</u>	22	<u>0.0</u>
7	<u>0.0</u>	23	<u>0.0</u>
8	<u>0.0</u>	24	<u>0.0</u>
9	<u>0.0</u>	25	<u>0.0</u>
10	<u>0.0</u>	26	<u>0.0</u>
11	<u>0.0</u>	27	<u>0.0</u>
12	<u>0.0</u>	28	<u>0.0</u>
13	<u>0.0</u>	29	<u>0.0</u>
14	<u>0.0</u>	30	<u>0.0</u>
15	<u>0.0</u>	31	<u>0.0</u>
16	<u>0.0</u>		

INSTRUCTIONS

On this format, list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt.

(9/77)

OPERATING DATA REPORT

DOCKET NO 50-285
 DATE January 10, 1983
 COMPLETED BY T. P. Matthews
 TELEPHONE (402) 526-4733

OPERATING STATUS

1. Unit Name: Fort Calhoun Station
2. Reporting Period: December, 1982
3. Licensed Thermal Power (MWt): 1500
4. Nameplate Rating (Gross MWe): 501
5. Design Electrical Rating (Net MWe): 478
6. Maximum Dependable Capacity (Gross MWe): 501
7. Maximum Dependable Capacity (Net MWe): 478
8. If Changes Occur in Capacity Ratings (Items Number 3 Through 7) Since Last Report, Give Reasons:
N/A

Notes

9. Power Level To Which Restricted, If Any (Net MWe): N/A
10. Reasons For Restrictions, If Any: None

	This Month	Yr.-to-Date	Cumulative
11. Hours In Reporting Period	744.0	8,760.0	81,241.0
12. Number Of Hours Reactor Was Critical	70.0	7,871.5	64,110.5
13. Reactor Reserve Shutdown Hours	0.0	0.0	1,309.5
14. Hours Generator On-Line	70.0	7,857.5	62,947.5
15. Unit Reserve Shutdown Hours	0.0	0.0	0.0
16. Gross Thermal Energy Generated (MWH)	81,785.6	10,914,718.3	77,616,548.4
17. Gross Electrical Energy Generated (MWH)	28,014.0	3,661,387.9	25,735,333.5
18. Net Electrical Energy Generated (MWH)	26,433.3	3,482,165.9	24,330,034.4
19. Unit Service Factor	9.4	89.7	77.5
20. Unit Availability Factor	9.4	89.7	77.5
21. Unit Capacity Factor (Using MDC Net)	7.4	83.2	65.0
22. Unit Capacity Factor (Using DER Net)	7.4	83.2	64.7
23. Unit Forced Outage Rate	49.3	3.6	3.9

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
1983 refueling outage commenced early on December 6, 1982.

25. If Shut Down At End Of Report Period, Estimated Date of Startup: March 20, 1983
 26. Units In Test Status (Prior to Commercial Operation): N/A
- | | Forecast | Achieved |
|----------------------|----------|----------|
| INITIAL CRITICALITY | _____ | _____ |
| INITIAL ELECTRICITY | _____ | _____ |
| COMMERCIAL OPERATION | _____ | _____ |

UNIT SHUTDOWNS AND POWER REDUCTIONS

REPORT MONTH December, 1982

DOCKET NO. 50-285
 UNIT NAME Fort Calhoun Station
 DATE January 10, 1983
 COMPLETED BY T. P. Matthews
 TELEPHONE (402)526-4733

No.	Date	Type ¹	Duration (Hours)	Reason ²	Method of Shutting Down Reactor ³	Licensee Event Report #	System Code ⁴	Component Code ⁵	Cause & Corrective Action to Prevent Recurrence
82-05	821203	F	68	A	3	N/A	HA	TURBIN	The reactor tripped on December 3, 1982 because of high turbine vibration. After investigation, it was determined that the first stage rotor of the H.P. turbine had broken blades and wiped bearings. Subsequently, it was decided as of December 6, 1982 to commence the 1983 refueling outage approximately one month early.
82-06	821206	S	606	C	4	N/A	XX	XXXXXX	

¹
 F: Forced
 S: Scheduled

²
 Reason:
 A-Equipment Failure (Explain)
 B-Maintenance of Test
 C-Refueling
 D-Regulatory Restriction
 E-Operator Training & License Examination
 F-Administrative
 G-Operational Error (Explain)
 H-Other (Explain)

³
 Method:
 1-Manual
 2-Manual Scram.
 3-Automatic Scram.
 4-Other (Explain)

⁴
 Exhibit G - Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

⁵
 Exhibit I - Same Source

Refueling Information
Fort Calhoun - Unit No. 1

Report for the month ending December 1982.

- | | |
|--|-------------------|
| 1. Scheduled date for next refueling shutdown. | <u>March 1984</u> |
| 2. Scheduled date for restart following refueling. | <u>May 1984</u> |
| 3. Will refueling or resumption of operation thereafter require a technical specification change or other license amendment? | <u>Yes</u> |
| a. If answer is yes, what, in general, will these be? | |

A Technical Specification Change

- | | |
|---|---|
| b. If answer is no, has the reload fuel design and core configuration been reviewed by your Plant Safety Review Committee to determine whether any unreviewed safety questions are associated with the core reload. | |
| c. If no such review has taken place, when is it scheduled? | |
| 4. Scheduled date(s) for submitting proposed licensing action and support information. | <u>Methodology - Dec. 1983</u>
<u>Tech. Specs. - Feb. 1984</u> |
| 5. Important licensing considerations associated with refueling, e.g., new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures. | |
| 6. The number of fuel assemblies: | |
| a) in the core | <u>133</u> assemblies |
| b) in the spent fuel pool | <u>237</u> " |
| c) spent fuel pool storage capacity | <u>483</u> " |
| d) planned spent fuel pool storage capacity | <u>728</u> " |
| 7. The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity. | <u>1985</u> |

Prepared by JJK Gayer Date January 4, 1983

OMAHA PUBLIC POWER DISTRICT
Fort Calhoun Station Unit No. 1

December, 1982
Monthly Operations Report

I. OPERATIONS SUMMARY

Fort Calhoun Station entered the month of December at 78% power. On December 3, 1982, the plant tripped off line due to high vibration of the turbine bearings. Upon investigation, the turbine was found to have lost six buckets off the first stage high pressure rotor and wiped several of the bearing facings. It was then resolved to move up the scheduled date of the maintenance/refueling outage.

Presently, the control rods have been uncoupled, the reactor coolant system is at the middle of the hot leg and steam generator eddy current testing has been completed.

In preparation for the full off load of the reactor core to the spent fuel pool, cutting up of used in-core flux detectors for proper storage has been completed and the shuffle of spent fuel already in the pool has commenced for allotment of room.

Annual licensed operator requalification examinations were completed by December 2, 1982.

No safety valve or PORV challenges occurred.

A. PERFORMANCE CHARACTERISTICS

<u>LER Number</u>	<u>Deficiency</u>
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NONE

B. CHANGES IN OPERATING METHODS

NONE

C. RESULTS OF SURVEILLANCE TESTS AND INSPECTIONS

Surveillance tests as required by the Technical Specifications Section 3.0 and Appendix B, were performed in accordance with the annual surveillance test schedule. The following is a summary of the surveillance tests which resulted in Operation Incidents and are not reported elsewhere in the report:

<u>Operation Incidents</u>	<u>Deficiency</u>
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OI 1615	ST-ESF-3, F.2
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During the performance of ST-ESF-3, F.2, pressure switches/sensors were found out of calibration, A/PC-742-2, B/PC-742-2, C/PC-742-2, D/742-2, A/PC-742-1, C/PC-742-1, D/PC-742-1

C. RESULTS OF SURVEILLANCE TESTS AND INSPECTIONS (continued)

<u>Operation Incidents</u>	<u>Deficiency</u>
OI 1612 ST-MSSV-1 F.1	During the performance of ST-MSSV-1, F.1, valves MS-275, 278, 280 and 282 failed to actuate within tolerance of setpoints.
OI 1616 ST-RM-4 F.1	During performance of ST-RM-4, F.1, sampler OAA was found to be inoperable due to a blown fuse.
OI 1617 ST-FW-2 F.2	During performance of ST-FW-2, F.2, flow indicators FI-1109/FI-1100 located on AI-66A/B were found to be operating out of setpoint tolerance.

D. CHANGES, TESTS AND EXPERIMENTS CARRIED OUT WITHOUT COMMISSION APPROVAL

<u>Procedure</u>	<u>Description</u>
SP-PORV-2	PORV Setpoints. Completed per procedure. The performance of this procedure did not constitute an unreviewed safety question as defined in 10CFR 50.59 because it was performed using normal plant operating procedures ensuring the operability of redundant equipment during setpoint changes.
10/23/82	PORV Setpoints. Completed per procedure. The performance of this procedure did not constitute an unreviewed safety question as defined in 10CFR 50.59 because it was performed using normal plant operating procedures ensuring the operability of redundant equipment during setpoint changes.
10/27/82	PORV Setpoints. Completed per procedure. The performance of this procedure did not constitute an unreviewed safety question as defined in 10CFR 50.59 because it was performed using normal plant operating procedures ensuring the operability of redundant equipment during setpoint changes.

D. CHANGES, TEST AND EXPERIMENTS CARRIED OUT WITHOUT COMMISSION APPROVAL

<u>Procedure</u>	<u>Description</u>
SP-ECT-1	Eddy Current Testing of Heat Exchanger Tubes. Completed per procedure. This procedure did not constitute an unreviewed safety question as defined by 10CFR 50.59 since it only involved eddy current testing of feedwater heaters.
SP-IC-11	Disposal of Irradiated Incore Detector. Completed per procedure. This procedure did not constitute an unreviewed safety question as defined by 10CFR 50.59 since it only involved the cutting and loading of incore detectors. Appropriate radiation protection procedure were followed.

E. RESULTS OF LEAK RATE TESTS

Local leak rate tests conducted during refueling outages were performed during December. A summary report will be submitted as required by the Technical Specifications.

F. CHANGES IN PLANT OPERATING STAFF

NONE

G. TRAINING

Training in December was directed at General Employee initial and refresher badge certification. Classes were included to handle the influx of personnel due to the forced outage in early December. Topics include security, tagging, procedural compliance, radiation protection, crane operator certification, hydrotesting and fire barriers.

H. CHANGES, TEST AND EXPERIMENTS REQUIRING NUCLEAR REGULATORY COMMISSION AUTHORIZATION PURSUANT TO 10CFR 50.59.

NONE

II. MAINTENANCE (Significant Safety Related)

M.O. #	DATE	DESCRIPTION	CORRECTIVE ACTION
17298	11/16/82	RM-056 will not cal check properly.	Replaced high alarm switch.
17408	12/3/82	AC-3B component cooling water pump - inboard seal leaks.	Completed per MP-AC-3-1.
17500	12/9/82	Raw water strainer will not run electrically.	Cleaned and lubed breaker.

W. G. Gates

W. G. Gates
Manager
Fort Calhoun Station