NRC FORM 366
(7-77)

U. S. NUCLEAR REGULATORY COMMISSION

LICENSEE EVENT REPORT

CONTROL BLOCK: (1) (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

01 | V A S P S 2 (2) | 0 0 - 0 0 0 0 0 - 0 0 (3) | 4 1 1 1 1 (4) | (5)
LICENSEE CODE | LICENSE NUMBER | LICENSE TYPE | CAT

CON'T 01 | REPORT SOURCE L (6) | DOCKET NUMBER 0 5 0 0 0 2 8 1 (7) | EVENT DATE 1 2 1 1 8 2 (8) | REPORT DATE 0 1 0 7 8 3 (9)

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

02 With the unit at cold shutdown, PT 39B-2 revealed that snubbers 2-RC-HSS-162 &
03 163 and 2-RH-HSS-5 had low levels in their remote reservoirs. Also snubber
04 2-SHP-HSS-4A was discoverd to have no fluid in it's reservoir. This event is
05 contrary to T.S.3.20 and reportable per T.S.6.6.2.b.(2). Piping thermal movement
06 was not hampered and no seismic events occurred during the inspection interval,
07 therefore, the health and safety of the public were not affected.
08

09

SYSTEM CODE	CAUSE CODE	CAUSE SUBCODE	COMPONENT CODE	COMP SUBCODE	VALVE SUBCODE
C B (11)	E (12)	B (13)	S U P O R T (14)	D (15)	Z (16)

(17) LER/RO REPORT NUMBER

EVENT YEAR		SEQUENTIAL REPORT NO.		OCCURRENCE CODE	REPORT TYPE		REVISION NO.
8 2	—	0 7 4	/	0 3	L	—	0

ACTION TAKEN	FUTURE ACTION	EFFECT ON PLANT	SHUTDOWN METHOD	HOURS (22)	ATTACHMENT SUBMITTED	NPRD-4 FORM SUB.	PRIME COMP. SUPPLIER	COMPONENT MANUFACTURER
X (18)	Z (19)	Z (20)	Z (21)	0 0 0 0	Y (23)	N (24)	A (25)	B 2 0 9 (26)

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

10 Snubbers 2-RC-HSS-162 & 163 had a leak on their reservoir sight glass and 2-RH-HSS-
11 5 had a leak on a pipe fitting to the valve block. The sight glass was blanked
12 off and the pipe fitting retightened. Snubber 2-SHP-HSS-4A had a leak through
13 its tubing connection and was replaced.
14

	FACILITY STATUS	% POWER	OTHER STATUS (30)	METHOD OF DISCOVERY	DISCOVERY DESCRIPTION (32)
15	G (28)	0 0 0 (29)	N/A	B (31)	Periodic Test

	ACTIVITY RELEASED	CONTENT OF RELEASE	AMOUNT OF ACTIVITY (35)	LOCATION OF RELEASE (36)
16	Z (33)	Z (34)	N/A	N/A

PERSONNEL EXPOSURES

	NUMBER	TYPE	DESCRIPTION (39)
17	0 0 0 (37)	Z (38)	N/A

PERSONNEL INJURIES

	NUMBER	DESCRIPTION (41)
18	0 0 0 (40)	N/A

8301170511 830107
PDR ADOCK 05000281
S PDR

LOSS OF OR DAMAGE TO FACILITY (43)

	TYPE	DESCRIPTION
19	Z (42)	N/A

PUBLICITY

	ISSUED	DESCRIPTION (45)	NRC USE ONLY
20	N (44)	N/A	

NAME OF PREPARER J. L. Wilson PHONE (804) 357-3184

ATTACHMENT 1
SURRY POWER STATION, UNIT NO. 2
DOCKET NO: 50-281
REPORT NO: 82-074/03T-0
EVENT DATE: 12-11-82

TITLE OF THE EVENT: INOPERABLE SNUBBERS

1. Description of the Event

With the unit at cold shutdown, during the performance of regularly scheduled surveillance, PT-39B-2, (Visual Inspection - Inaccessible Snubbers as Found), two reactor coolant snubbers (2-RC-HSS-162 & 163) ,with common remote reservoirs and one RHR Snubber (2-RH-HSS-5) were discovered to have low fluid levels in their reservoirs. Also, a main steam snubber (2-SHP-HSS-4A) was discovered to have no fluid in its reservoir. This event is contrary to Technical Specification 3.20 and reportable per Technical Specification 6.6.2.b.(2).

2. Probable Consequences and Status of Redundant Equipment

Snubbers prevent unrestrained pipe motion that can occur during an earthquake or severe hydraulic transient while allowing for thermal pipe movement. The possible consequences of an inoperable snubber is an increased chance of pipe damage resulting from dynamic loads.

Remote reservoir low level does not prevent thermal pipe movement. There were no seismic events during the inspection interval, therefore, the health and safety of the public were not affected.

3. Cause

Snubbers 2-RC-HSS-162 & 163 had a leak around the sight glass of their common remote reservoir and 2-RH-HSS-5 had a leak from the fluid piping connection to its valve block. Snubber 2-SHP-HSS-4A had a leak from the tubing between its reservoir and valve block.

4. Immediate Corrective Action

The RC snubber's reservoir sight glass was removed, blanked off and sealed and the RHR snubber's leaking pipe fitting was retaped and tightened. Since air was suspected to have entered the main steam snubber's cylinder, it was removed and replaced with a snubber which had been tested satisfactorily.

5. Subsequent Corrective Action

None required.

6. Action Taken to Prevent Recurrence

The next performance of PT-39B-2 has been placed on an accelerated schedule.

7. Generic Implications

None.