N	R	C	8	0	R	м	36	ő	
1.4									

U. S. NUCLEAR REGULATORY COMMISSION

3-771	LICENSEE EVENT REPORT
	CONTROL BLOCK.
	V A S P S 2 C 0 - 0 0 0 0 - 0 0 0 0 0 0 0 0 0 0 0 0
CONT 0	REPORT L S 0 5 0 0 0 2 8 1 0 1 0 7 8 2 8 0 1 0 1 0 7 8 3 9 SOURCE 50 51 DOCKET NUMBER 58 69 EVENT DATE 74 15 REPORT DATE 80
	EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (19) During unit shutdown on 12/7/82, and during unit startup on 12/24/82, FI-474
0 2	drifted low and was declared inoperable. Inoperability of this instrument is
013	
014	contrary to Tech. Spec. 3.7 and is reportable in accordance with T.S.6.6.2.b(2).
0 5	Since redundant instrumentation remained operable and the failed instrument's
0 6	bi-stables were placed in trip, the health and safety of the public would not
0 7	have been affected.
516	80
7 8	SYSTEM CAUSE CAUSE COMPONENT CODE SUBCODE SUBCODE
0 9	9 10 11 12 13 13 19 20 REVISION
	IT LER/RO EVENT YEAR REPORT NO. CODE TYPE NO. 17 AEBORT 8 2 0 7 3 0 3 0 31 32
	ACTION BUTURE EFFECT SHUTDOWN HOURS 22 ATTACHMENT NERO PRIME COMPONENT MANUFACTURER MANUFACTURER MANUFACTURER
	B (B Z (3) Z (2) C
10	The cause was a partial loss of fluid in the sensing line to the high side of the
	transmitter, which was caused by a packing leak on a root valve. The valve was
1 2	repaired, the sensing line filled, and the instrument was returned to service.
1 2	
14	80
7 8	PACILITY NOWER OTHER STATUS 30 METHOD OF DISCOVERY DESCRIPTION 32
7 8	D 3 10 5 3 3 1 N/A A 45 46 Operator Observation 30
1 6	ACTIVITY CONTENT RELEASED OF RELEASE AMOUNT OF ACTIVITY 35 LOCATION OF RELEASE 36 N/A 80
, ,	NUMBER TYPE DESCRIPTION 39
1 7 1 a	0 0 0 0 0 0 0 0 0 0 0 0 0 0 0 0 0 0 0
1 4	
• 3	SSS OF OR DAMAGE TO FACILITY (2)
1 9	A/A go
1210	SSUED DESCRIPTION (45)
1 12	58 59 BO 5
	NAME DE PERSONAL J. L. Wilson CME (804) 357-3184

ATTACHMENT 1 SURRY POWER STATION, UNIT NO. 2 DOCKET NO: 50-281 REPORT NO: 82-073/03L-0 EVENT DATE: 12-07-82

TITLE OF THE EVENT: 2-FI-474 MALFUNCTION

1. Description of the Event

On 12/7/82, with the unit at 53% Reactor Power during a unit shutdown prior to a maintenance outage, operations personnel observed that one channel of the "A" Steam Generator Steam Flow indication (2-FI-474) had drifted lower than the redundant channel and was declared inoperable. This channel (2-FI-474) drifted low and was declared inoperable again during the unit startup on 12/24/82. Plant operation was continued as permitted by Tech. Spec. 3.7.B, Table 3.7-2. This event is reportable in accordance with Tech. Spec. 6.6.2.b(2).

2. Probable Consequences and Status of Redundant Equipment

The steam line flow instrumentation provides signals to actuate the Safety Injection System upon sensing the effects of a steam line break. SIS actuation following a steam line break is designed to occur upon sensing high steam line flow in coincidence with low reactor coolant average temperature or low steam line pressure. Since the bi-stable for the failed instrument (2-FI-474) was placed in the trip mode and the redundant channel remained operable, the health and safety of the public would not have been affected.

3. Cause

The cause was a partial loss of fluid in the sensing line to the high side of the transmitters, which was caused by a packing leak on a root valve to the sensing line. This resulted in a lower differential pressure across the transmitter, thereby causing a reduced flow indication.

4. Immediate Corrective Action

The immediate corrective action for these events was to place the bistables in the trip mode, as required by Tech. Spec. Table 3.7-2.

5. Subsequent Corrective Action

Following the shutdown of the unit, the transmitter and indicator calibrations were checked. No significant problems or differences from previous calibrations were found. The packing on the leaking root valve was tightened. During the unit startup following the maintenance outage, the flow error on FI-474 was still indicated. The condensate pot on the flow transmitter was inspected and filled. Filling the condensate pot returned the flow signal to an acceptable range as checked against the redundant channel. The instrumentation was declared operable and returned to service on 12/26/82.

6. Action Taken to Prevent Recurrence

No further actions were required.

**

Page 2 Report No. 82-073/03L-0

7. Generic Implications

None.