

LICENSEE EVENT REPORT

CONTROL BLOCK: _____ (1) (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

0	1	A	L	B	R	F	1	2	0	0	-	0	0	0	0	0	0	0	3	4	1	1	1	1	4	5
7	8	9	14							15	25							26	30					37	38	58

LICENSEE CODE LICENSE NUMBER LICENSE TYPE CAT 58

0	1	L	6	0	5	0	0	0	2	5	9	7	1	2	0	5	7	8	3	1	2	1	8	7	8	9
7	8	60	61	68							69	74					75	80								

REPORT SOURCE DOCKET NUMBER EVENT DATE REPORT DATE

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

0 2 | During a refueling outage, while performing local leak rate testing, six MSIV's, FCV's
 0 3 | 1-14, 1-15, 1-51, 1-52, 1-37, and 1-38, exceeded the leakage limits of T.S.4.7.A.2.1.
 0 4 | FCV's 1-14, 1-15, 1-51, and 1-52 leakage was greater than 2,196 SCFH. FCV's 1-37 and 1-38
 0 5 | leakage was greater than 1,986 SCFH. Redundant systems were not applicable since the
 0 6 | reactor was in the refueling mode. There was no hazard to the public health or safety.
 0 7 | Previous occurrence: BFRO-50-259/7723.

0	9	C	D	11	E	12	B	13	V	A	L	V	E	X	14	F	15	G	16
7	8	9	10	11	12	13	14	15	16	17	18	19	20	21	22	23	24	25	26

SYSTEM CODE CAUSE CODE CAUSE SUBCODE COMPONENT CODE COMP. SUBCODE VALVE SUBCODE

17	21	22	23	24	26	27	28	29	30	31	32
7	8	—	0	3	4	—	0	1	T	—	0

LER/RO REPORT NUMBER EVENT YEAR SEQUENTIAL REPORT NO. OCCURRENCE CODE REPORT TYPE REVISION NO.

Z	B	19	Z	20	Z	21	0	0	0	0	Y	23	N	24	N	25	A	5	8	5	26
33	34	35	36	37	38	39	40	41	42	43	44	45	46	47	48	49	50	51	52	53	54

ACTION TAKEN FUTURE ACTION EFFECT ON PLANT SHUTDOWN METHOD HOURS ATTACHMENT SUBMITTED NPRO-4 FORM SUB. PRIME COMP. SUPPLIER COMPONENT MANUFACTURER

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

1 0 | The defective valves were 26-inch Atwood-Morill globe valves, pneumatic actuation,
 1 1 | operating pressure rating of 1,250 psig at 575°F. The cause of the occurrence was
 1 2 | degradation of valve seating surfaces through normal operation.

1	5	H	28	0	0	0	29	NA	30	B	31	Surveillance Test	32
7	8	9	10	11	12	13	14	15	16	17	18	19	20

FACILITY STATUS % POWER OTHER STATUS METHOD OF DISCOVERY DISCOVERY DESCRIPTION

1	6	Z	33	Z	34	NA	35	NA	36
7	8	9	10	11	12	13	14	15	16

ACTIVITY CONTENT RELEASED OF RELEASE AMOUNT OF ACTIVITY LOCATION OF RELEASE

1	7	0	0	0	37	Z	38	NA	39
7	8	9	10	11	12	13	14	15	16

PERSONNEL EXPOSURES NUMBER TYPE DESCRIPTION

1	8	0	0	0	40	NA	41
7	8	9	10	11	12	13	14

PERSONNEL INJURIES NUMBER DESCRIPTION

1	9	Z	42	NA	43
7	8	9	10	11	12

LOSS OF OR DAMAGE TO FACILITY TYPE DESCRIPTION

2	0	N	44	NA	45
7	8	9	10	11	12

PUBLICITY ISSUED DESCRIPTION

78122702689

LER SUPPLEMENTAL INFORMATION

BFRO-50- 259 / 7834 Technical Specification Involved 4.7.A.2.i

Reported Under Technical Specification 6.7.2.a (3)

Date of Occurrence 12/5/78 Time of Occurrence unknown Unit 1

Identification and Description of Occurrence:

Six MSIV's, FCV's 1-14, 1-15, 1-51, 1-52, 1-37, and 1-38, exceeded the leak rate allowed by T.S. 4.7.A.2.i during local leak rate testing. The following are the leak rates: FCV's 1-14, 1-15, 1-51, and 1-52 indicated a leak rate greater than 2,196 SCFH. FCV's 1-37 and 1-38 indicated a leak rate greater than 1,986 SCFH.

Conditions Prior to Occurrence:

Unit shutdown for refueling. The reactor head was off.

Action specified in the Technical Specification Surveillance Requirements met due to inoperable equipment. Describe.

Repair and retest to be completed.

Apparent Cause of Occurrence:

Degradation of valve seating surface due to normal wear.

Analysis of Occurrence:

None

Corrective Action:

Valves will be repaired and satisfactorily retested prior to return to service of the unit.

Failure Data:

BFRO-50-259/7723

Tennessee Valley Authority - Browns Ferry Nuclear Plant