



UNITED STATES
NUCLEAR REGULATORY COMMISSION

WASHINGTON, D. C. 20555-0001

March 8, 1994

Docket No. 50-333

Mr. William A. Josiger, Acting
Executive Vice President-Nuclear
Generation
Power Authority of the State of
New York
123 Main Street
White Plains, New York 10601

Dear Mr. Josiger:

SUBJECT: RELIEF REQUEST FROM THE AMERICAN SOCIETY OF MECHANICAL ENGINEERS (ASME) CODE REQUIREMENT TO EXAMINE ALL PORTIONS OF ONE LONGITUDINAL WELD AND ONE CIRCUMFERENTIAL WELD IN THE REACTOR VESSEL BELTLINE REGION DURING THE CURRENT 10-YEAR INSPECTION INTERVAL - JAMES A. FITZPATRICK NUCLEAR POWER PLANT (TAC NO. M87158)

By letter dated July 30, 1993, the Power Authority of the State of New York (PASNY) requested relief from the ASME Section XI Code requirement to examine all portions of one longitudinal weld and one circumferential weld in the reactor vessel beltline region during the current 10-year inspection interval at the James A. FitzPatrick Nuclear Power Plant. The relief was requested in accordance with 10 CFR 50.55a(g)(5)(iii).

ASME Code, Section XI, Table IWB-2500-1, Examination Category B-A, Items B1.11 (circumferential shell welds) and B1.12 (longitudinal shell welds) require a 100% volumetric examination of one circumferential and one longitudinal beltline region weld, as defined by Figures IWB-2500-1 and IWB-2500-2, respectively. PASNY requested relief from the Code requirement to examine 100% of the length of one beltline region circumferential weld and 100% of the length of one beltline region longitudinal weld. The relief would apply only to the current inspection interval. PASNY's basis for requesting relief was that the design of the biological shield wall and permanent insulation system does not allow access from the outside of the reactor vessel to 100% of one longitudinal weld and 100% of one circumferential weld. Furthermore, the alternative examinations proposed would be representative of beltline region conditions and meet the intent of the Code.

As noted in the enclosed safety evaluation (SE), the NRC staff has reviewed the submittal and concludes that the proposed alternative for inservice inspection examination of the reactor pressure vessel beltline welds will provide an acceptable level of quality and safety. Therefore, pursuant to

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Mr. William A. Josiger

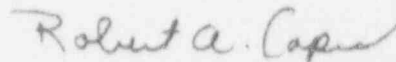
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10 CFR 50.55a(a)(3)(i), PASNY's proposed alternative is authorized on a one-time basis only, provided that the new regulations, as stated in the enclosed SE, are met during the first inspection period of the third 10-year interval.

This completes our action related to TAC No. M87158.

Sincerely,



Robert A. Capra, Director
Project Directorate I-1
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Enclosures:

1. Safety Evaluation
2. Technical Evaluation Summary

Mr. William A. Josiger
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James A. FitzPatrick Nuclear
Power Plant

cc:

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