

REPORT DATE: December 14, 1978

REPORTABLE OCCURRENCE 78-35

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FORT ST. VRAIN NUCLEAR GENERATING STATION
PUBLIC SERVICE COMPANY OF COLORADO
P. O. BOX 361
PLATTEVILLE, COLORADO 80651

REPORT NO. 50-267/78-35/03-L-0

Final

IDENTIFICATION OF
OCCURRENCE:

After performance of a routine surveillance test, four of six Linear Channel-High Power Rod Withdrawal Prohibit (RWP) trip points were left at a setting greater than 30%. Table 4.4-4 of LCO 4.4.1 specifies an RWP trip point setting of less than or equal to 30%. Therefore, this condition is reportable per Fort St. Vrain Technical Specification AC 7.5.2(b)1.

EVENT
DESCRIPTION:

On July 20, 1978, a routine surveillance test of the Linear Power Range Channel Calibration was performed. This surveillance test specified a high power RWP trip point voltage of 2.0 volts with a voltage tolerance of ± 0.05 volts. The 2.0 volt trip point corresponds to a reactor power of 30%. Therefore, when the ± 0.05 volt tolerance was utilized, a trip point in excess of 30% reactor power was possible.

The "as left" conditions for the surveillance test were as follows:

<u>Channel</u>	<u>"As Left" Volts</u>	<u>Equivalent Power</u>
3	2.01	30.2%
4	2.02	30.3%
5	2.02	30.3%
6	2.02	30.3%
7	1.99	29.8%
8	2.00	30.0%

The settings of Channels 3, 4, 5, and 6 were less conservative than those established by the Technical Specifications, but did not prevent the fulfillment of their functional requirements.

The discrepancy between the surveillance test and the Technical Specification setpoints was discovered during a routine audit performed by plant Quality Assurance personnel.

CAUSE
DESCRIPTION:

The procedure was defective because the acceptance voltage range stated in the surveillance test allowed four RWP trip points to be left at a setting greater than 30%. The voltage tolerance of ± 0.05 volts was specified because it is within the accuracy capability of the test instrumentation.

CORRECTIVE
ACTION:

The surveillance test setpoints have been revised to a voltage range of 1.93 ± 0.06 volts, which corresponds to a reactor power of $29 \pm 1\%$. The nuclear instrumentation has been readjusted using this revised criteria.

No further corrective action is anticipated or required.

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