

# Technical Guidance for Siting Criteria Development

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## FOREWORD

On July 29, 1980 an advance notice of rulemaking was published for the siting of nuclear power reactors. One of the principle elements contained in the advance notice was the development of a comprehensive analysis of all technical issues relevant to siting. Sandia National Laboratories was contracted by the Nuclear Regulatory Commission to perform the analysis and document the technical guidance to support the formulation of new regulations. This report completes the effort to provide the technical guidance.

The work has been primarily focused toward the development of generic siting criteria, uncoupled from specific plant design. To achieve this end, the NRC staff developed a representative set of severe accident release source terms which covers the full spectrum of postulated severe accident releases for typical light water reactors. NUREG-0773, "The Development of Severe Reactor Accident Source Terms: 1975-1981," provides the detailed description of the considerations that went into the development of the spectrum of source terms (SSTs) in general terms; a more specific discussion of the concept of a representative or generic spectrum of source terms is given in pages 6 through 21 of NUREG-0771, "Regulatory Impact of Nuclear Reactor Accident Source Term Assumptions." From the results of Probabilistic Risk Assessments available at the time of the preparation of this report, the NRC staff would assign typical probability values to the source terms for a range of light water reactor designs as follows:

Probability of SST1 release     $1 \times 10^{-5}$ /reactor year

Probability of SST2 release     $2 \times 10^{-5}$ /reactor year

Probability of SST3 release     $1 \times 10^{-4}$ /reactor year

Table 2.3.1-3 presents the comparative impact of these releases in terms of public health effects. These ratios indicate the relative importance of the source terms given equal probability of occurrence. Their absolute and relative probabilities of occurrence affect their significance for the selection of siting criteria.

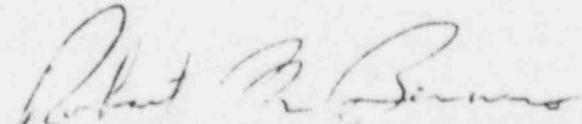
There are very large uncertainties associated with these numbers. The absolute values and the ratios of these probabilities for a given facility are design specific. To accurately portray the risk, very specific accident sequence probabilities and source terms are needed. Thus, the results presented in this report do not represent nuclear power risk.

The siting source terms were used to calculate accident consequences at 91 U. S. reactor sites using site specific meteorology and population data and assuming an 1120 MWe reactor. These calculations treat siting factors such as weather conditions and emergency response probabilistically but postulate the siting source term release. The results are thus conditional consequence values.

Currently there is significant controversy about the realism of accident source terms, that is, the accuracy with which they describe potential releases of radioactivity for a given sequence of events in a core melt accident. The work done to date on siting uses the source terms developed for the Reactor Safety Study, held unchanged by newer projections as explained in NUREG-0772, "Technical Bases for Estimating Fission Product Behavior During LWR Accidents." The staff expects newer information to be available by mid 1983 to modify these source terms. In the meanwhile, sensitivity analyses are given to explore how the calculated consequence values would change with various source term reductions.

Contained in this report are sensitivity studies for the major parameters important to siting decision making. Only through consideration of material such as this can reasoned decisions be made concerning recommendations for improved siting regulations.

This report represents some of the work being done to support the expanding use of probabilistic risk assessment in the regulatory process. The NRC must be careful with the results of such analyses, considering the very large uncertainties in the results. The studies shown in this report must be used in a manner that is consistent with the stated objectives. The results are to provide technical perspective on siting-related issues. Results presented in this report are not significantly different than results of consequence studies that have been available in the open literature for decades. Given the source term assumptions, large consequences are calculated. However, the risks (probabilities times consequences) posed by such accidents are very small. Therefore, the absolute numbers should only be quoted with the associated probabilities and with the stated assumptions recognizing the uncertainties in the analyses.



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### Abstract

Technical guidance to support the formulation and comparison of possible siting criteria for nuclear power plants has been developed for the Nuclear Regulatory Commission by Sandia National Laboratories. Information has been developed in four areas: (1) consequences of hypothetical severe nuclear power plant accidents, (2) characteristics of population distributions about current reactor sites, (3) site availability within the continental United States, and (4) socioeconomic impacts of reactor siting.

The impact on consequences of source term magnitude, meteorology, population distribution and emergency response have been analyzed. Population distributions about current sites were analyzed to identify statistical characteristics, time trends, and regional differences. A site availability data bank was constructed for the continental United States. The data bank contains information about population densities, seismicity, topography, water availability, and land use restrictions. Finally, the socioeconomic impacts of rural industrialization projects, energy boomtowns, and nuclear power plants were examined to determine their nature, magnitude, and dependence on site demography and remoteness.

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## 1. Introduction and Summary

### 1.1 Introduction

At the request of the Nuclear Regulatory Commission, Sandia National Laboratories has performed a study to develop technical guidance to support the formulation of new regulations for siting nuclear power reactors [1]. Guidance was requested regarding (1) criteria for population density and distribution surrounding future sites, and (2) standoff distances of plants from offsite hazards. Studies were performed in each of these two areas of concern.

The study of offsite hazards had two areas of concern: (1) determination of which classes of offsite hazards are amenable to regulation by fixed standoff distances, and (2) review of available methods for the determination of appropriate standoff distances. The hazards considered included aircraft, hazardous chemicals, dams, faults, adjacent nuclear power plants, tsunamis, meteorite impact, etc. The study concluded that none of the hazards are suitable to treatment by fixed standoff distances and that sufficient methods exist for evaluating the risk for most types of hazards. Because they have been published elsewhere [2], the results of the study of offsite hazards are not included in this report.

The studies of site characteristics, which are presented in this report, involved analyses in four areas, each of which could play a role in evaluating the impact of a siting policy. The four areas were: (1) consequences of possible plant accidents, (2) population distribution characteristics for existing sites, (3) availability of sites, and (4) socioeconomic impacts.

Accident consequence analyses were performed to help define the risks associated with existing sites and with alternative siting criteria. Consequence analyses also help to evaluate the dependence of risk on factors such as meteorology, population distribution, and emergency response which can be mandated or constrained by regulations. Population distributions at existing sites were examined to provide perspective on demographic characteristics as well as to determine whether there have been trends with time or regional differences in site selection. The site availability

analysis examined the impact of various population distribution criteria on the amount of land restricted from siting. Impacts of environmental and legal constraints were also examined. In addition, studies were performed to evaluate the extent of socioeconomic impacts and the degree to which they are dependent on site demographic characteristics. These four areas of analysis provide information that could be used to assess and compare alternative siting criteria.

The information developed by this study is presented in four chapters and six appendices. Chapter 2 presents the results of the consequence analyses that were performed to identify factors that have a significant impact upon risk. The factors examined include source term magnitude (Section 2.3), meteorology (Section 2.4.1), population (Section 2.4.2), emergency response (Section 2.5), consequence distances (Section 2.6), reactor size (Section 2.7.1), plume heat content (Section 2.7.2), dry deposition velocity (Section 2.7.3), characteristics of population distributions (Section 2.7.4), and criteria for the interdiction of contaminated land (Section 2.7.5). CRAC2 [3,4], the computer model used to perform these consequence analyses, is described briefly in Section 2.2.1 and more fully in Appendix E. Model input data are described in Section 2.2.2. Site specific input data are presented in Appendix A and core radionuclide inventory data in Appendix B. Data and model uncertainties are discussed in Section 2.2.4. Finally, a series of site specific calculations were made using a standard set of source terms uncorrected for the characteristics of the reactor at the site. The results of these calculations are presented in Appendix C.

Chapter 3 and Appendix 9 present an examination of the population distributions surrounding existing sites to provide perspective on demographic characteristics and to determine (1) whether there is evidence of a trend over time to less-dense siting and (2) whether site characteristics differ significantly in different regions of the country. The site availability analyses developed a capability for measuring the impact of population criteria on the availability of reactor sites. Also considered in these analyses were the seismicity, topographic character, availability of surface and ground water at potential sites, and the restriction of power plant siting because of the presence of

national parks or wilderness areas. This study, which was performed by Dames and Moore [5] under contract to Sandia, is presented in full in Chapter 4 and Appendix F. Finally, a study was performed to examine the socioeconomic impacts of reactor siting and the dependence of the magnitude of these impacts on site demography. The study examined impacts caused by large construction projects, energy boomtowns, and the construction of nuclear power plants. Also examined was the impact of site remoteness on transmission costs. The study, performed by Battelle-HARC under contract to Sandia, is summarized in Chapter 5 and presented in full in a separate report [6].

## 1.2 Summary

This report contains the results of numerous calculations and analyses performed at Sandia National Laboratories, Dames and Moore, and Batelle-HARC. The principal results or conclusions reached are:

- o Estimates of the number of early fatalities are very sensitive to source term magnitude. Mean early fatalities (average result for many weather sequences) are decreased dramatically (about two orders-of-magnitude) by a one order-of-magnitude decrease in source term SST1 (large core melt, loss of most safety systems). Because the core melt accident source terms SST1-3 used in this study neglect or underestimate several depletion mechanisms, which may operate efficiently within the primary loop or the containment, consequence magnitudes calculated using these source terms may be significantly overestimated.
- o The weather conditions at the time of a large release will have a substantial impact on the health effects caused by that release. In marked contrast to this, mean health effects (average result for many weather sequences) are relatively insensitive to meteorology. Over the range of meteorological conditions found within the continental United States (1 year meteorological records from 29 National Weather Service stations), mean early fatality values for a densely populated site show a range (highest value/lowest value) of only a factor of 2, and mean latent cancer fatalities a factor of 1.2.

- o Peak early fatalities (maximum value calculated for any weather sequence) are generally caused by rainout of the radioactive plume onto a population center. For an SST1 release, the peak result is about 10-times less probable in a dry locale than in a wet one.
- o The distances to which consequences might occur depend principally upon source term magnitude and meteorology. Frequency distributions of these distances, calculated using large numbers of weather sequences, yielded expected (mean), 99 percentile, and maximum calculated distances (expressed in miles) for early fatalities, early injuries, and land interdiction as follows:

<u>Source Term</u>	<u>Consequence</u>	<u>Mean</u>	<u>99%</u>	<u>Maximum Calculated</u>
SST1	Early Fatalities	<5	≤15	<25
	Early Injuries	~10	~30	≤50
	Land Interdiction	~20	>50	>50
SST2	Early Fatalities	~0.5	<2	<2
	Early Injuries	<2	<5	~5
	Land Interdiction	<2	~7	~10

The maximum calculated distances are associated with improbable events, (e.g., rain-out of the plume onto a population center). For the SST1 release reduced by a factor of 10, early fatalities are confined to ~5 miles, early injuries to ~20 miles, and interdiction of land to ~25 miles.

- o Calculated consequences are very sensitive to site population distribution. For each of the 91 population distributions examined, early fatality, early injury, and latent cancer fatality CCDFs were calculated assuming an SST1 release from an 1120 MWe reactor. The resulting sets of CCDFs had the following ranges:

Early Fatalities. ~3 orders-of-magnitude in the peak and mean numbers of early fatalities and in the probability of having at least one early fatality.

Early Injuries. ~3 orders-of-magnitude in the means, ~2 in the peaks, and ~1 in the probability of having at least one early injury.

Latent Cancer Fatalities. ~1 order-of magnitude in the peaks and the means and in the probability of having at least one latent cancer fatality.

Generally, mean results are determined by the average density of the entire exposed population, while peak results (especially for early fatalities) are determined by the distance to and size of exposed population centers.

- o Early fatalities and early injuries can be significantly reduced by emergency response actions. Both sheltering (followed by relocation) and evacuation can be effective provided the response is expeditious. Access to basements or masonry buildings significantly enhances the effectiveness of sheltering. Expeditious response requires timely notification of the public. If the evacuation is expeditious (timely initiation), evacuation speeds of 10 mph are effective. Evacuation before containment breach within 2 miles, after release within 10 miles, and sheltering from 10 to 25 miles appears to be a particularly effective response strategy.
- o Population densities (people/sq mi) about the 91 sites have the following maximum, 90th percentile and median values within the indicated distance intervals:

Distance (mi)	0-5	0-10	0-20
<u>Full Circle</u>			
Maximum	790	660	710
90th percentile	190	230	380
Median	40	70	90
<u>Most Populated 22.5° Sector</u>			
Maximum	4200	3800	4500
90th percentile	950	1000	1800
Median	330	270	480

- o At the 91 sites examined, the distance to the nearest exclusion zone boundary ranges from 0.1 to 1.3 miles and averages about 0.5 miles.
- o There appears to be a slight trend with time towards selection of reactor sites in less densely populated locations.
- o A site availability data base has been constructed on a 5 x 5 km grid cell for the continental United States. For each grid cell the data base contains information on population density, seismicity, topographic character, surface and ground water availability, and land use restrictions (wetlands, national parks, etc.)
- o Analysis of boomtown literature, studies of large non-nuclear energy projects, and economic data from existing nuclear power plant sites suggests that only siting in very remote regions has the potential for significant socioeconomic impacts, that these impacts may be both beneficial or detrimental and that the detrimental impacts can be mitigated by advance planning.
- o Outside of the Rocky Mountains, few potential reactor sites are located at a large distance from the national power grid. Consequently, site remoteness and transmission line costs are not strongly correlated.

This study examined a number of factors which could impact the development of siting criteria. The analyses, which are reported in the following chapters, can be used to determine many of the impacts of alternative criteria, and provide guidance in evaluating tradeoffs among criteria. In addition, the data and analyses contained in the study should be useful to the wider community of users interested in evaluating the consequences of reactor accidents.

## References for Chapter 1

1. Advance notice of this rulemaking appeared in the Federal Register, July 29, 1980, FR DOC 80-22643.
2. N. C. Finley, Nuclear Power Plant Siting: Consideration of Offsite Hazards, NUREG/CR-2380, Sandia National Laboratories (to be published).
3. L. T. Ritchie, J. D. Johnson, and R. M. Blond, Calculations of Reactor Accident Consequences, Version 2: User's Guide, NUREG/CR-2326, SAND81-1994, Sandia National Laboratories, Albuquerque, NM, (to be published).
4. L. T. Ritchie, et al., CRAC2, Calculation of Reactor Accident Consequences, Version 2, Model Description, SAND82-0342, NUREG/CR-2552, Sandia National Laboratories, Albuquerque, NM (to be published).
5. J. H. Robinson and K. I. Hansen, Impact of Demographic Siting Criteria and Environmental Suitability on Land Availability, Dames and Moore, Los Angeles, California, 1981.
6. C. Cluett, S. Malhotra, and D. Manninen, Socio-economic Impacts of Remote Nuclear Power Plant Siting, NUREG/CR-2537, SAND81-7230, Battelle Human Affairs Research Centers, Seattle, Washington, (to be published).

## 2. Consequences of Potential Reactor Accidents

### 2.1 Introduction

During this study, a large number of calculations were performed to provide a basis for understanding the dependence of reactor accident consequences on site characteristics. Some characteristics were examined because of the possibility of their inclusion in reactor siting criteria (e.g., population distribution, reactor power level). A number of additional parameters were investigated to determine the sensitivity of predicted consequences to variation or uncertainty in data used as input.

All consequence calculations for this study were performed using CRAC2, an improved version of CRAC,<sup>a</sup> the Reactor Safety Study [1] consequence model. Section 2.2.1 provides a brief overview of the CRAC2 model, while Section 2.2.2 describes the data used as input to the consequence calculations. Section 2.2.3 is a qualitative discussion of the sources and impacts of uncertainties associated with the consequence model. Section 2.2.4 defines the "base case" calculation which was used as a reference case for examination of the impact of variations in parameters and assumptions.

Section 2.3 briefly describes the five accident source terms used in the calculations. These source terms, denoted SST1-5, were developed by NRC and range from a full core-melt with uncontrolled release to a gap release with minimal leakage. Section 2.3.1 presents results of consequence calculations for each of the five source terms, and Section 2.3.2 examines the potential impact on consequences of reductions in the magnitude of the most severe accident (SST1).

Section 2.4 examines the impact of meteorology and population on consequence estimates. Meteorological data from 29 National Weather Service stations and wind rose and population data from each of the 91 currently approved reactor sites in the United States are examined. Section 2.5 presents the impact on consequences of various emergency response assumptions; both evacuation and sheltering scenarios are evaluated. Section 2.6 discusses the distances to which various consequences occur and the sensitivity of these distances to input

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a. CRAC stands for Calculation of Reactor Accident Consequences.

data and assumptions. Section 2.7 examines the sensitivity of consequences to variations in reactor size, energy-release rate, dry deposition velocity, population distribution, and land-interdiction criteria. Finally, Section 2.8 presents a summary of the insights gained from these calculations.

## 2.2 Background

### 2.2.1 Overview of Consequence Model

The accident consequence calculations described in this chapter were performed using CRAC2 [2,3], an improved version of the Reactor Safety Study (WASH-1400) consequence model, CRAC [1,4]. Modifications made in the upgrade from CRAC to CRAC2 are briefly described in Appendix E.<sup>a</sup> The model describes the progression of the cloud of radioactive material released from the containment structure during and following a reactor accident, and predicts its interaction with and influence on the environment and man. A schematic outline of the computational steps taken in the model is presented in Figure 2.2.1-1.

Analyses of potential plant system failures and accident phenomenology provide an estimate of accident probabilities and release characteristics (magnitudes, timing, etc.) that are used as input to the consequence model.<sup>b</sup> Given these estimates, a standard Gaussian dispersion model is used to calculate ground-level concentrations of airborne radioactive material downwind of the reactor site. Weather data for a 1-year period are input to the dispersion model in the form of hourly recordings of wind speed, thermal stability, and accumulated precipitation. The wind direction is assumed to be invariant during and following the release. Radionuclide concentrations within the cloud are depleted by deposition (both wet and dry) and radioactive decay, and integrated air and ground contamination are calculated for downwind distances.

- a. Results calculated using the two models are similar, as shown in the recent International Comparison Study of Reactor Accident Consequence Models [5,6].
- b. Specific release characteristics assumed in this study are described in Section 2.3.

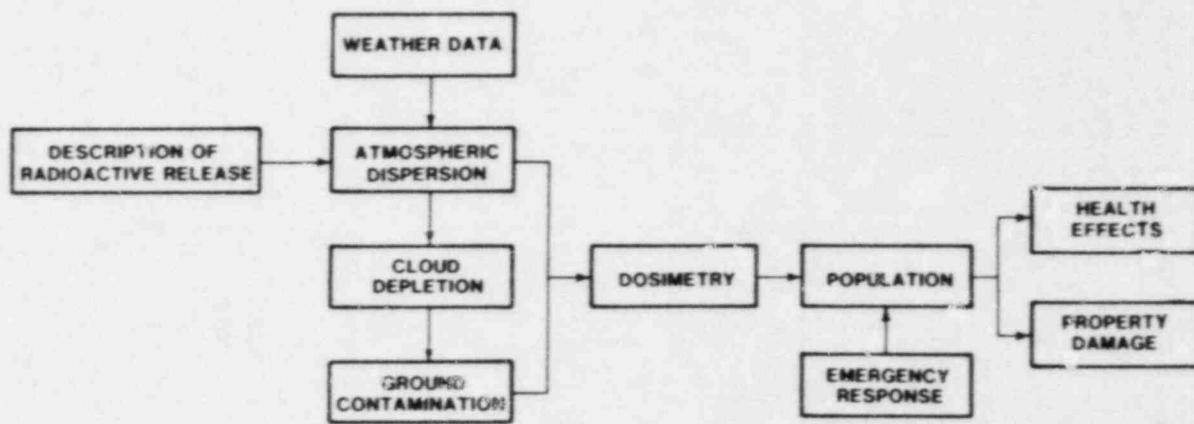


Figure 2.2.1-1. Schematic Outline of Consequence Model, CRAC2.

Hourly weather recordings are used to account for weather variations during the progression of the accident. Beginning at a selected hour within the year's data, the dispersion model uses the subsequent meteorological conditions to predict the dispersion, downwind transport, and deposition of the released cloud of radioactive material. Hourly recordings are sequentially incorporated until all of the released radioactive material (excluding the noble gases) has been deposited. By using an appropriate sample of weather sequences from the year's data, a frequency distribution of estimated consequences can be produced.

The consequence model uses the calculated airborne and ground radionuclide concentrations to estimate the public's exposure to external radiation from (1) airborne radionuclides in the cloud and (2) radionuclides deposited from the cloud onto the ground, and internal radiation from (1) radionuclides inhaled directly from the passing cloud, (2) inhaled resuspended radionuclides, and (3) the ingestion of contaminated food and milk. Radiation exposure from sources external to the body is calculated for time periods over which individuals are exposed to those sources, while the exposure from sources internal to the body is calculated over the remaining life of the exposed individual.

The consequence model allows the input of either site-specific or hypothetical population data as a function of distance and direction from the reactor site. A simple evacuation model is incorporated, which is based on a statistical analysis of evacuation data assembled by the U.S. Environmental Protection Agency [7-9] (see Appendix E). The model incorporates a delay time before public movement, followed by evacuation radially away from the reactor. A range of evacuation delay times, speeds, and distances have been assumed in this study, as is described in later sections.

Based on the calculated radiation exposure to downwind individuals, the consequence model estimates the number of public health effects that would result from the accidental release. Early injuries and fatalities, latent cancer fatalities, and thyroid and genetic effects may be computed. Early fatalities are defined to be those fatalities that occur within 1 year of the exposure period. They are estimated on the basis of exposure to the bone marrow, lung and gastrointestinal tract. Bone marrow damage is the dominant contributor to early

fatalities. In both the Reactor Safety Study and this study, early fatalities are calculated assuming an LD<sub>50/60</sub><sup>a</sup> of 510 rads to the bone marrow. Supportive medical treatment of the exposed individual is also assumed. Early injuries are defined as non-fatal, non-carcinogenic illnesses, that appear within 1 year of the exposure and require medical attention or hospital treatment. The late somatic effects considered include latent cancer fatalities plus benign and malignant thyroid nodules.

The consequence model also includes an economic model to estimate the potential extent of property damage associated with the release of radioactive material. The total offsite dollar cost of the accident is estimated as the sum of (1) the evacuation cost, (2) the value of condemned crops and milk, (3) the cost of decontaminating land and structures, (4) the cost of interdicting land and structures, and (5) relocation costs (moving costs and temporary loss of income).

#### 2.2.2 Input Data for Consequence Model

CRAC2 requires a large set of input data, including accident release characteristics and source terms, various site-related data (e.g., meteorology, population), reactor core radionuclide inventories, and emergency response scenarios. The accident release characteristics and source terms assumed in this study are described in Section 2.3.

The site-related data, gathered for use in this study, are presented in Appendix A. The data gathered includes:

1. General site and reactor data (e.g., reactor size, vendor, start-up date, site location) for each of the 91 U.S. sites at which a reactor is operating or a construction permit has been obtained.
2. Regional shielding factors for sheltered populations.
3. Site population data derived from the 1970 census.

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a. The dose that would be lethal to 50 percent of the population within 60 days.

4. Meteorological data consisting of hourly recordings of weather conditions from 29 National Weather Service stations plus mixing heights from Holzworth [10].
5. Annual site wind roses obtained from either Environmental Impact Reports or Safety Analysis Reports.
6. Site economic data, updated from those used in WASH-1400 to reflect inflation and changing economic conditions.

A core radionuclide inventory for a 3412 MWe (1120 MWe) reactor was calculated for this study using the SANDIA-ORIGEN [11] computer code. This calculation assumed an end-of-cycle fuel burnup of 33,000 MWd/MTU (about 25 percent greater than was assumed in WASH-1400) which is representative of the current generation of larger reactors. Differences in reactor size were accommodated by linearly scaling the inventory with rated thermal power level. A description of the inventory calculations and a discussion of the impact of inventories on predicted consequences are presented in Appendix B. The sensitivity of consequences to reactor size is examined in Section 2.7.1.

The emergency response submodel incorporated in CRAC2 is described in Section 2.5 and Appendix E. The model allows specification of up to six emergency response scenarios plus a weighted sum of these scenarios termed "Summary Evacuation." Unless otherwise specified, calculations were performed using the scenarios presented in Table 2.2.2-1. The scenarios range from a prompt evacuation to sheltering to no emergency response. The response distance of 10 miles was selected to coincide with the Emergency Planning Zone (EPZ) recommended by the NRC [12]. The delay times and speeds assumed were based on a statistical analysis of evacuation data gathered by the EPA (see Appendix E). The "Summary Evacuation" was defined as a 30 percent, 40 percent, 30 percent weighting<sup>a</sup> of scenarios 1, 2, and 3, and

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a. Thirty percent of the time, all people within 10 miles evacuate with a 1 hour delay and 10 mph speed; 40 percent of the time, all people within 10 miles evacuate with a 3-hour delay and 10 mph speed; and 30 percent of the time all people within 10 miles evacuate with a 5-hour delay and 10 mph speed.

represents a "best estimate" for consequence predictions. Most of the results presented in the following sections assumed this "Summary Evacuation." The sensitivity of predicted consequences to emergency response assumptions is examined in Section 2.5. Differences in emergency response due to site-specific characteristics were not addressed.

Table 2.2.2-1. Emergency Response Scenarios

Scenario Number	Type of Response	Response Distance	Delay Time Before Response	Response Speed
1	Evacuation	10 miles	1-hour	10 mph
2	Evacuation	10 miles	3-hours	10 mph
3	Evacuation	10 miles	5-hours	10 mph
4	Evacuation	10 miles	5-hours	1 mph
5	Sheltering, Relocation	10 miles	none, 6-hours	---
6	No Emergency Response	--	--	---

### 2.2.3 Uncertainties

Uncertainties in offsite consequence predictions stem principally from uncertainties in two areas: modeling and input data. Modeling uncertainty arises from (1) an incomplete understanding of the phenomena involved in the transport of released radionuclides to man and the consequent health impacts, and (2) simplifications of phenomena made in the modeling process to reduce costs or model complexity. Input data uncertainty arises from problems associated with the quality and availability of

data, selection or determination of appropriate values for model input (including radioactive source terms), and statistical variations in data. To date, a comprehensive assessment of these uncertainties in consequence predictions has not been performed. However, a number of partial uncertainty estimates have been derived using sensitivity analysis techniques [1,13,14].

Improvements in a number of model areas could substantially reduce current uncertainties. The most important of these include source terms (see Section 2.3), plume depletion processes (see Section 2.7.3), the effect of wind trajectories on population exposures, and the effectiveness of emergency response (see Section 2.5). Each of these areas is briefly described below.

Radioactive source terms for atmospheric releases are subject to a number of important uncertainties, including uncertainties about release magnitude and timing, and about aerosol size distributions. It has been suggested [15,16] that removal processes within the primary coolant system and containment could reduce the amount of material released to the atmosphere to levels significantly below those currently estimated. Possible removal processes include plate-out of hot vapors on cooler surfaces, agglomeration and deposition of aerosols, and dissolution in water. Better specification of the timing of a release is important for two reasons: (1) a longer warning period increases the chance of an effective emergency response and (2) a long, slow release spreads the radioactive material over a larger area, thereby decreasing individual doses and (usually) health effects. The particle-size distribution of the released material, and thus the efficiency of dry deposition processes during downwind transport, is determined principally by aerosol agglomeration rates. Resolution of these source-term uncertainties by ongoing or future research activities may require a reevaluation of some of the conclusions reached by this study. For example, some of the conclusions about emergency planning and response presented in Section 2.5 could be significantly altered.

A plume of radioactive material may be depleted during transport by dry deposition and/or washout processes. The dry-deposition removal rate is strongly dependent on the size distribution of particulate matter in the plume. Therefore, the current lack of information about this size distribution prevents reliable modeling

of dry deposition. Since washout of material by rainfall is a very efficient removal mechanism, it is important to account for the frequency, intensity, and spatial variability of rainfall. Moreover, because high-consequence events are usually associated with rainfall over population centers, failure to adequately model rainfall can lead to large inaccuracies in predicted peak consequences.

Wind trajectories determine the specific population exposed by downwind transport of the plume of radioactive material. With the exception of the computer code CRACIT [17,18], current consequence models neglect wind trajectories. Although results obtained with CRACIT indicate that treatment of wind trajectories may affect risk less than intuition suggests [6], a thorough examination of this subject (perhaps using a Gaussian puff model), particularly for sites with complex terrain, seems essential [19].

The sensitivity of predicted consequences to different emergency response scenarios is examined in Section 2.5. If consequence models are to be applied to evaluate the risk at specific sites, consideration should be given to those characteristics of the site and of local organizations that could influence the effectiveness of offsite emergency response. For example, local and utility emergency response plans, available mechanisms for warning the public, and characteristics of the surrounding road network should be examined. Road networks could be particularly important if population densities are sufficient to result in "traffic jams" or "bottleneck" conditions, or if terrain features are likely to cause evacuation routes and the plume trajectory to overlap.

Another area of uncertainty is the estimation of the late somatic effects, of which the incidence of cancer is the most important. The recent BEIR III report [20] discusses these uncertainties, which are largest for low doses (and dose rates) of low-LET radiation. In addition, Loewe and Mendelsohn [21] have recently conducted a reassessment of the dosimetry data for the populations exposed by the detonations at Hiroshima and Nagasaki. These new findings have led to major changes in the estimates of the neutron and gamma-ray doses received by survivors. Efforts are currently underway at the Los Alamos National Laboratory to redefine the source terms from the two detonations

and at Oak Ridge National Laboratory to recalculate dose estimates. When completed, these reassessments may result in some changes in estimates for late somatic effects.

#### 2.2.4 Base Case Calculation

The results of a large number of calculations are presented in Sections 2.3 through 2.7 of this report. These calculations examine the impact on predicted consequences of a wide variety of parameters and assumptions. To simplify the examination of the impact of variations in input parameters and assumptions, a "base case" calculation was defined. Assumed in the base case were:

- a standard 1120 MWe PWR
- an SST1 release (defined in Section 2.3)
- New York City meteorology
- the Indian Point wind rose and population
- Summary Evacuation

The values of all other input parameters were those typically used in CRAC2. The sensitivity of predicted consequences to the base case assumptions and to other input parameter values is discussed in later sections.

### 2.3 Reactor Accident Source Terms

This section describes the reactor accident source terms used to perform the consequence calculations. Consequences that might result from these source terms are compared and the most important source terms are identified. In addition, source term uncertainties are addressed. Results that show the impacts of these uncertainties on reactor accident consequences are presented and discussed.

#### 2.3.1 Accident Release Characteristics and Source Terms

The Nuclear Regulatory Commission recently sponsored an evaluation of the technical bases for reactor accident source term assumptions and the potential impact of possible source term changes on the regulatory process [16,22]. These studies found that the Design Basis Accidents (DBAs), which have been the basis for regulatory policies governing nuclear power plant siting and design, do not constitute a realistic representation

of the full spectrum of possible accident source terms for any reactor design. Therefore, they do not provide an adequate estimate of reactor risk at specific sites. Consequently, after review of current source term information, the NRC defined a spectrum of accidents [22], which more adequately spans the range of possible accident source terms and better reflects current understanding of fission product behavior during reactor accidents.

The spectrum of accidents that was defined ranges from accidents within the design basis envelope to core melt accidents which may release large quantities of radioactive material to the environment. Five accident groups were designated as being representative of the spectrum of potential accident conditions. Each group represents a different degree of core degradation and of failure of containment safety features. Brief descriptions of the characteristics of the accident types included in each group are presented in Table 2.3.1-1.

For the purpose of decision-making in such areas as siting and emergency response, NRC defined a set of five Siting Source Terms (denoted SST1-5) to represent the five accident groups. By adjusting the probabilities associated with each of the five source terms, the set can be made to approximately represent any current LWR design.<sup>a</sup> Table 2.3.1-2 summarizes the five NRC-defined source terms used in this study.

The consequences that could potentially result from each of the five source terms were determined by performing a series of CRAC2 calculations. Table 2.3.1-3 compares the relative magnitudes (normalized to 100 for source term SST1) of the mean values<sup>b</sup> of selected consequences, given the occurrence of each of the five source terms and assuming an 1120 MWe PWR, Indian Point population distribution and wind rose, New York City meteorology, and Summary Evacuation (see Sections 2.2.2 and 2.5 and Appendix E). These results indicate that source terms SST2 through SST5 would not be expected to produce substantial numbers of offsite consequences.

- a. Detailed Probabilistic Risk Assessments (PRAs) have not been performed for all reactors. Based on currently available PRAs, NRC has suggested that representative probabilities for the SSTs are:  $P_1$  for SST1 =  $1 \times 10^{-5}$ ,  $P_2$  for SST2 =  $2 \times 10^{-5}$ , and  $P_3$  for SST3 =  $1 \times 10^{-4}$ . There are very large variations (factors of 10 to 100) in the accident probabilities associated with a specific design.
- b. Using approximately 100 sampled weather sequences, the CRAC2 code calculates frequency distributions for consequences that might result from a radioactive release. The means of these distributions are the mean values referred to in the text.

compared to the SST1 source term. The mean consequences calculated for the SST1 release exceed those from the SST2 release by 1 to 4 orders of magnitude and exceed those from releases SST3, SST4, and SST5 by 4 to 7 orders of magnitude. Early fatalities, early injuries, and land interdiction do not result from releases SST3, SST4, and SST5 because these accidents do not release enough radioactivity to produce doses that exceed the dose thresholds for these consequences.

Table 2.3.1-1. Brief Descriptions Characterizing the Accident Groups Within the NRC "Accident Spectrum" [22]

Group 1	Severe core damage. Essentially involves loss of all installed safety features. Severe direct breach of containment.
Group 2	Severe core damage. Containment fails to isolate. Fission product release mitigating systems (e.g., sprays, suppression pool, fan coolers) operate to reduce release.
Group 3	Severe core damage. Containment fails by basemat melt-through. All other release mitigation systems function as designed.
Group 4	Modest core damage. Containment systems operate in a degraded mode.
Group 5	Limited core damage. No failures of engineered safety features beyond those postulated by the various design basis accidents. The most severe accident in this group assumes that the containment functions as designed following a substantial core melt.

Table 2.3.1-2. NRC Source Terms for Siting Analysis

<u>Release Characteristics<sup>a</sup></u>	<u>Source Term</u>				
	<u>SST1</u>	<u>SST2</u>	<u>SST3</u>	<u>SST4</u>	<u>SST5</u>
Accident Type	Core Melt	Core Melt	Core Melt	Gap Release	Gap Release
Containment Failure Mode	Overpressure	H <sub>2</sub> Explosion or Loss of Isolation	-	-	-
Containment Leakage	Large	Large	1%/day	1%/day	0.1%/day
Time of Release (hr)	1.5	3	1	0.5	0.5
Release Duration (hr)	2	2	4	1	1
Warning Time (hr)	0.5	1	0.5	-	-
Release Height (meters)	10	10	10	10	10
Release Energy	0	0	0	0	0
<u>Inventory Release Fractions</u>					
Xe-Kr Group	1.0	0.9	6 x 10 <sup>-3</sup>	3 x 10 <sup>-6</sup>	3 x 10 <sup>-7</sup>
I Group	0.45	3 x 10 <sup>-3</sup>	2 x 10 <sup>-4</sup>	1 x 10 <sup>-7</sup>	1 x 10 <sup>-8</sup>
Cs-Rb Group	0.67	9 x 10 <sup>-3</sup>	1 x 10 <sup>-5</sup>	6 x 10 <sup>-7</sup>	6 x 10 <sup>-8</sup>
Te-Sb Group	0.64	3 x 10 <sup>-2</sup>	2 x 10 <sup>-5</sup>	1 x 10 <sup>-9</sup>	1 x 10 <sup>-10</sup>
Ba-Sr Group	0.07	1 x 10 <sup>-3</sup>	1 x 10 <sup>-6</sup>	1 x 10 <sup>11</sup>	1 x 10 <sup>-12</sup>
Ru Group	0.05	2 x 10 <sup>-3</sup>	2 x 10 <sup>-6</sup>	0	0
La Group	9 x 10 <sup>-3</sup>	3 x 10 <sup>-4</sup>	1 x 10 <sup>-6</sup>	0	0

a. As defined in the Reactor Safety Study [1].

Table 2.3.1-3. Comparison of Conditional Mean Consequences Predicted for Five Source Terms<sup>a,b</sup>

Source Term	Mean Early Fatalities	Mean Early Injuries	Mean Latent Cancer Fatalities	Mean Thyroid Nodules	Mean Interdicted Land Area
SST1	100 <sup>b</sup>	100	100	100	100
SST2	$1 \times 10^{-2}$	0.5	7	3	1
SST3	0	0	$2 \times 10^{-2}$	$5 \times 10^{-2}$	0
SST4	0	0	$4 \times 10^{-4}$	$8 \times 10^{-5}$	0
SST5	0	0	$4 \times 10^{-5}$	$8 \times 10^{-6}$	0

a. Assumptions: 1120 MWe PWR, population distribution and wind rose for Indian Point, New York City meteorology, "Summary Evacuation" of persons within 10 miles.

b. All consequences are normalized to 100 for source term SST1.

Figures 2.3.1-1 and 2.3.1-2 present mean bone marrow dose and mean thyroid dose to exposed individuals as a function of distance for each of the five source terms.<sup>a</sup> The doses were calculated assuming no emergency response, an 1120 MWe PWR, and New York City meteorology. The mean doses at any distance vary by nearly 8 orders of magnitude over the spectrum of five releases. For any pair of releases, relative doses are roughly proportional to the ratios of curies of released radioactivity excluding noble gases (Xe-Kr group). These figures also show that individual bone marrow and thyroid doses would generally not be expected to exceed a few tens of millirem for the SST4 release and a few millirem for the SST5 release.

Figure 2.3.1-3 displays the variation with distance of the mean individual risks (averaged over 360 degrees<sup>b</sup>) of early fatality and early injury for source terms SST1 and SST2, and of latent cancer fatality (from early exposure only<sup>c</sup>) for all five source terms. These curves were calculated assuming an 1120 MWe PWR, New York City meteorology, a uniform wind rose, and no emergency response. Because early fatalities and injuries have dose thresholds, their risks of occurrence decrease rapidly with distance for large source terms (e.g., SST1 and SST2) and are zero offsite ( $\geq 0.25$  mi) for small source terms (e.g., SST3, SST4, and SST5). Since no offsite risk of early fatality or injury was predicted for source terms SST3, SST4, or SST5, in Figures 2.3.1-3a and 2.3.1-3b no curves were plotted for these source terms. In contrast to this, because no dose threshold is assumed for latent cancer fatalities, the risk of latent cancer fatality decreases more slowly with distance and is non-zero for all five source terms. Therefore, in Figure 2.3.1-3c a

- a. The doses are the means of the frequency distributions of estimated individual dose calculated using an appropriate sample of weather sequences from a single year of meteorological data.
- b. Individual risks shown are the product of two probabilities: (1) the probability of exposure to the plume given that the release occurs, and (2) the probability that the individual dies following the exposure.
- c. Early exposure includes exposure to the radioactive plume, all exposures resulting from inhalation of radioactive materials from the plume, and short-term exposure to radioactivity deposited on the ground from the plume.

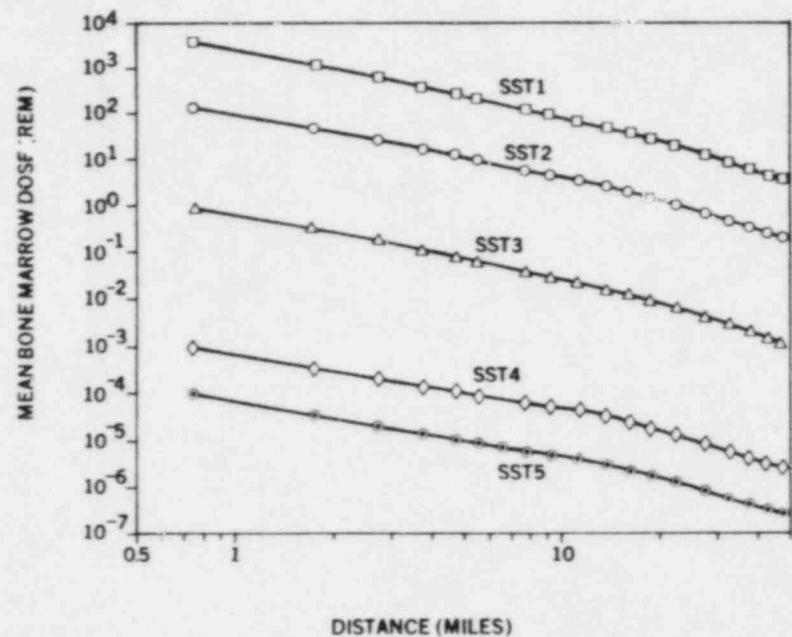


Figure 2.3.1-1. Comparison of Predicted Mean Bone Marrow Dose to Exposed Individuals vs Distance for the Five Source Terms.

Assumptions: 1120 MWe reactor, New York City meteorology, no emergency response, one day exposure to radionuclides deposited on the ground.

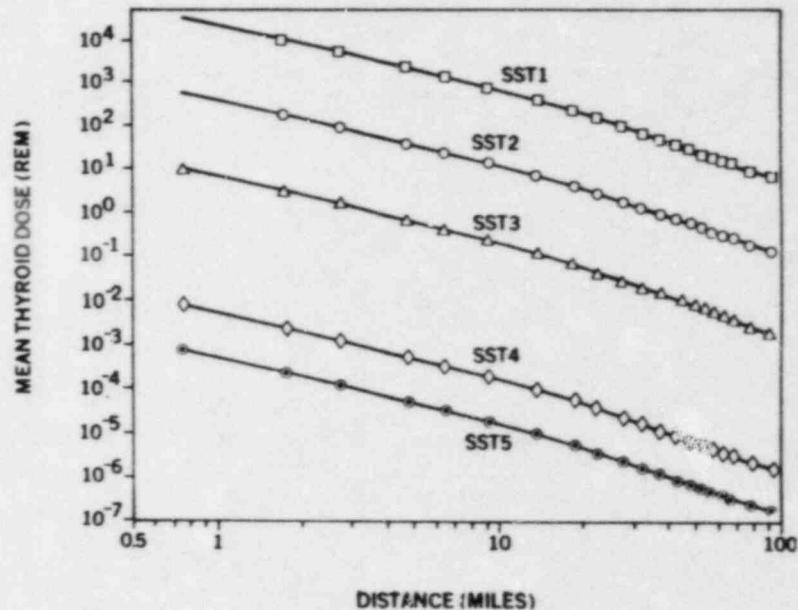


Figure 2.3.1-2. Comparison of Predicted Mean Thyroid Dose to Exposed Individuals vs Distance for the Five Source Terms.

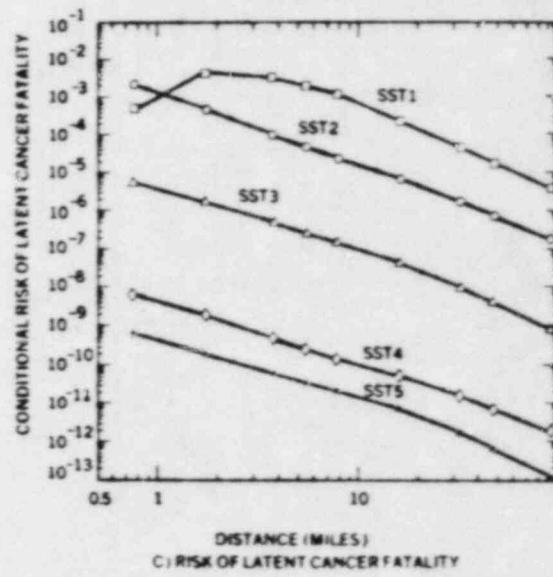
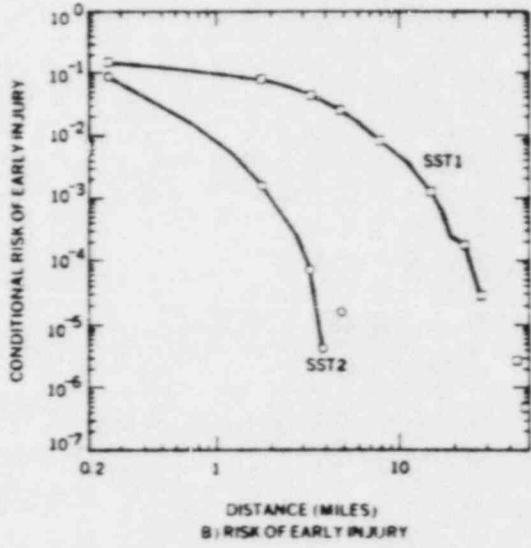
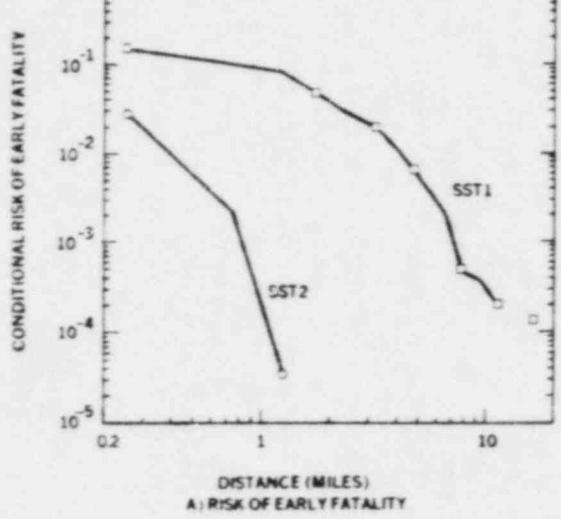


Figure 2.3.1-3. Risk to an Individual of a) Early Fatality, b) Early Injury, and c) Latent Cancer Fatality (from early exposure only) vs Distance Conditional on Each of the Five Siting Source Terms. Assumptions: 1120 MWe PWR, New York City meteorology, no emergency response, and a uniform wind rose.

risk curve is plotted for each source term. The latent cancer risk curve for the SST1 release crosses the risk curve for the SST2 release at short distances. The falloff in the latent cancer fatality risk at short distances ( $\lesssim 2$  mi) for SST1 is caused by the very high risk of early fatality at these distances. Because of the high early fatality risk, the latent cancer fatality risk is essentially conditional on surviving the high early radiation doses produced close to the reactor by SST1. Finally, comparison of Figure 2.3.1-3c with Figures 2.3.1-1 and 2.3.1-2 shows that the relative differences between the five latent cancer fatality risk curves are similar to those between the five dose vs distance curves for bone marrow or thyroid doses.

Together, the results presented in Table 2.3.1-3 and Figures 2.3.1-1 through 2.3.1-3 show that the SST1 accident would likely dominate overall reactor risk to the public.<sup>a</sup> Furthermore, consequences resulting from the SST4 and SST5 accidents were shown to be much smaller than those resulting from the core melt accidents (source terms SST1, SST2, and SST3). Therefore, because these non-melt releases probably have little influence on off-site reactor risk, the SST4 and SST5 releases will not be considered further. In addition, because offsite risk is dominated by the most severe core-melt accidents, the remainder of this chapter will concentrate principally on the SST1 release, although results for the SST2 and SST3 releases will be presented when appropriate.

### 2.3.2 Uncertainty in Source Term Magnitudes

At present there is a great deal of controversy over the magnitude and nature of source terms for severe reactor accidents. A recent study [15] suggested that source terms for atmospheric releases could be substantially smaller than those assumed in WASH-1400 (or also in this report). The study cited evidence that removal processes, which have generally been neglected but which should operate within the primary coolant system and containment, would decrease the amount of material released following an accident to amounts substantially below those usually assumed. Such removal processes include plate-out of hot vapors, agglomeration and deposition of aerosols, and dissolution of soluble materials in water.

a. This conclusion depends on the relative probabilities of releases.

The effectiveness of these removal processes would be strongly dependent on the conditions inside the coolant system and containment and on the chemical and physical form of the fission products. For example, Campbell et al. [23] suggest that under accident conditions in LWRs, fission product iodine would be in the form of a soluble metallic iodide (probably CsI) rather than volatile, molecular iodine, as is currently assumed. Also, Morewitz [24], after review of past reactor accidents and destructive tests, concluded that in all cases where water was present, no fission product tellurium had been released. Morewitz proposed two explanations for this observation: Either tellurium remains in solution in the form of soluble  $\text{CsTe}_2$ , or tellurium particles are efficiently scavenged by rapid droplet growth caused by condensation of water vapor. Morewitz further noted that even in the absence of water droplet formation, the generation of large quantities of aerosol from structural materials (steel, concrete, etc.) would produce rates of aerosol agglomeration rapid enough to ensure that a large fraction of the radioactive particles would quickly settleout inside the containment.

These suggestions have received substantial support in a recent NRC report [16]. The significance of these proposals is that the solubility of volatile fission products and potential aerosol removal mechanisms could limit the quantity of released radionuclides to levels one to two orders of magnitude below those currently assumed.

To evaluate the impact on predicted consequences of significant reductions in the amount of released material, a series of calculations was performed with arbitrary reductions in the quantities of released fission products. The impact of potential reductions due to the solubility of fission products in water was evaluated by arbitrarily reducing the release fractions of iodine, cesium, and tellurium<sup>a</sup> to 50, 10 and 0 percent of the standard SST1 level, singly,

- 
- a. The tellurium release fraction includes both tellurium and antimony and the cesium release fraction includes both cesium and rubidium (see Table 2.3.1-2). Cesium and tellurium, however dominate the predicted consequences for each release group.

in pairs (Cs and I only), and all simultaneously (50 percent reduction only). To evaluate the impact on predicted consequences of potential reductions in source terms due to efficient aerosol removal processes, calculations were performed with the release fraction of all isotopes except noble gases arbitrarily reduced to 50, 10, 5, and 1 percent of the SST1 release.

The results of the calculations are summarized in Tables 2.3.2-1 and 2.3.2-2. Assumed in these calculations were the Indian Point site, New York City meteorology, an 1120 MWe reactor, and Summary Evacuation. The results in Table 2.3.2-1 indicate that a factor of 10 reduction in the release fraction of either iodine or tellurium results only in about a factor of 2 reduction in early effects. Because of the dose-threshold for early effects, this does not imply that iodine or tellurium "account" for half of the early effects.

Table 2.3.2-1 does, however, present a measure of the relative doses resulting from exposure to individual elements. Iodine isotopes account for about 35 percent of the expected acute bone marrow dose and for about 80 percent of the thyroid dose. Bone-marrow dose has been shown to be the dominant cause of early fatalities. Tellurium isotopes account for about 35 percent of the acute bone marrow dose and about 20 percent of the thyroid dose. Because of the long half-lives of Cs<sup>134</sup> (2 years) and Cs<sup>137</sup> (30 years), cesium is the dominant element for long-term exposure. However, a factor of 10 reduction in the release fraction of cesium reduces the mean number of latent cancer fatalities by only 25 percent.

The small reduction in the number of latent cancer fatalities is a result of the assumption in CPAC2 that land will be interdicted to reduce long-term exposure. Thus, reducing the release fraction of cesium reduces the amount of interdicted land but does not significantly alter the total population exposure. The amount of interdicted land is very sensitive to the release fraction of cesium. A factor of ten reduction in the cesium release fraction results in an 85% reduction in the interdicted land area. The sensitivity of latent cancer fatalities to the criterion used for the interdiction of land is discussed in Section 2.7.5.

Table 2.3.2-2 presents the impact on consequences of reductions in the SST1 release fractions of all

Table 2.3.2-1. Sensitivity of Mean Consequences to Reductions in SST1 Release Fractions of Iodine, Cesium, and Tellurium<sup>a,b</sup>

Accident Release	Early Fatalities	Early Injuries	Latent Cancer Fatalities	Acute Dose <sup>c</sup>		Area of Land Interdiction
				Bone Marrow	Thyroid	
SST1 (Standard)	100 <sup>b</sup>	100	100	100	100	100
50% I	75	75	98	85	60	100
10% I	60	55	95	70	30	100
0% I	50	55	95	65	20	100
50% Cs	95	95	90	95	100	55
10% Cs	90	95	75	90	100	15
0% Cs	85	90	60	90	100	1
50% Te	75	65	95	85	90	100
10% Te	50	45	90	70	80	100
0% Te	45	40	90	65	80	100
50% I,Cs	70	70	90	80	60	55
10% I,Cs	45	55	70	60	30	15
0% I,Cs	40	50	55	55	20	1
50% I,Cs,Te	40	45	85	60	50	55

a. Assumptions: 1120 MWe reactor, Indian Point site, New York City meteorology, Summary Evacuation.

b. All consequences normalized to 100 for source term SST1.

c. Relative doses are approximately independent of distance.

Table 2.3.2-2. Sensitivity of Mean Consequences to Reductions in SST<sub>1</sub>  
Release Fractions of All Elements Except Noble Gases<sup>a,b</sup>

Accident Release	Early Fatalities	Early Injuries	Latent Cancer Fatalities	Acute Doses <sup>c</sup>		Interdicted Land Area
SST <sub>1</sub> (Standard)	100 <sup>b</sup>	100	100	Bone Marrow	Thyroid	100
50% SST <sub>1</sub> <sup>d</sup>	30	35	74	53	50	55
10% SST <sub>1</sub> <sup>d</sup>	1	4	32	16	10	10
5% SST <sub>1</sub> <sup>d</sup>	0.2	2	19	11	5	5
1% SST <sub>1</sub> <sup>d</sup>	0.03	1	5	8	1	1

- a. Assumptions: 1120 MWe reactor, Indian Point Site, New York City meteorology, Summary Evacuation.
- b. All consequences normalized to 100 for source term SST<sub>1</sub>.
- c. Relative doses are approximately independent of distance.
- d. Release fractions reduced for all isotopes except noble gases.

elements except the noble gases. The results indicate that an order-of-magnitude decrease in the release fractions causes the mean number of early fatalities to decrease by about 2 orders-of-magnitude and other consequences to decrease by about 1 order-of-magnitude. The 99th percentile<sup>a</sup> of the calculated distribution of early fatalities for the standard SST1 release was 8,300. When the SST1 release fractions for elements other than noble gases were reduced to 10 and 1 percent of the standard values, the 99th percentile values for early fatalities fell to 100 and 0, respectively.

Only the impact on consequences of potential reductions in the magnitude of source terms has been examined in this section. Two other areas of large uncertainty, the energy release rate accompanying a radioactive release and the physical characteristics of the released material (as reflected in the dry deposition velocity) are discussed in Sections 2.7.2 and 2.7.3, respectively. Other areas of uncertainty, such as release timing (including variable and long duration releases) and release height, have not been addressed in this study.

In summary, if resolution of present uncertainties concerning Source term magnitudes determines that the amount of material released to the atmosphere is significantly less than that currently assumed, there could be large decreases in the predicted consequences of large core melt accidents (e.g., SST1 and SST2). Therefore, the reader should bear in mind that the consequences presented in this report may be significantly overestimated and, thus, some conclusions drawn may not remain valid.

#### 2.4 Site Meteorology and Population

In very general terms, the predicted consequences of an accidental release of radioactive material are dependent on four factors: 1) the assumed source term, 2) the meteorological conditions during and following the release, 3) the number of people exposed to the released material, and 4) the effectiveness of population protective measures. In the previous section, the sensitivity of consequences to the source term was discussed. In this section, the impact on consequences of the mete-

a. Those consequences that would be equalled or exceeded by 1 out of every 100 releases.

investigated. The impact of emergency protective measures on consequences is discussed in Section 2.5.

#### 2.4.1 Sensitivity to Meteorological Record

Predictions of the potential consequences of reactor accidents normally assume that an accident may occur at any time, day or night, under any possible weather conditions. So that all possible weather conditions are adequately represented in the calculations, CRAC2 samples weather sequences from an actual record of meteorological conditions. The meteorological record required by CRAC2 consists of the site wind rose and 8760 hourly observations (1 year) of wind speed, atmospheric stability, and accumulated precipitation. As described in Section 2.2.1 and Appendix E, approximately 100 weather sequences are sampled from the meteorological record and used in the calculations to generate frequency distributions for various consequences. Current regulatory policy requires a licensee to monitor meteorological conditions for at least 1 year as part of the site approval process [25]. Data from reactor sites, however, are often of poor quality. Some site meteorological files do not include observations of precipitation and there are often "gaps" in the recordings. For this study, meteorological records from 29 National Weather Service (NWS) stations were used with the site wind rose. The 29 records represent the broad range of climatic conditions found in the United States, ranging from arid climates, such as Phoenix, AZ, to wet climates, such as Apalachicola, FL. NWS data have several potential advantages over reactor site data in that they are generally of higher quality, are readily available, contain more detailed observations, and are of durations of up to 30 years. A description of the 29 meteorological records may be found in Section A.3 of Appendix A.

A sensitivity analysis was performed to examine the impact that the meteorological record used in the calculations has on predicted consequences. Each of the 29 records was used as input for calculations at the Indian Point and Diablo Canyon sites (i.e., the population distributions and wind rose for each site were used with each of the 29 NWS records). Indian Point was selected because it has one of the highest population densities surrounding the site, while Diablo Canyon has one of the lowest.

The calculations assumed Summary Evacuation (see Section 2.5), an 1120 MWe plant, and an SST1 release.

Any observed variation in the predicted consequences at either of the two sites must be due either to differences in the 29 meteorological records or to inadequacies in the procedure used to sample weather sequences.

The weather sequence sampling procedure currently used with CRAC2 has several deficiencies. Because only one year of data is sampled, very low probability sequences (e.g., intense rain at a specific distance) may not be adequately represented. Sequences that contain rain events are currently properly weighted as to frequency of occurrence only when the rain event occurs within 30 miles of the site. This is probably adequate for early fatalities, which typically do not occur beyond 25 miles. However, consequences such as early injuries and interdiction of land, that have dose thresholds and which occur to distances substantially greater than 30 miles, are probably not properly represented by a sampling procedure that does not characterize weather sequences beyond 30 miles. Finally, because rainfall sequences are not weighted for rainfall intensity, ground contamination also may not be adequately characterized by the current sampling procedure.

Figure 2.4.1-1 presents the 29 early fatality CCDFs<sup>a</sup> for the Indian Point site obtained using the 29 meteorological records. Probabilities are conditional on the occurrence of an SST1 accident. The means of the 29 conditional distributions vary by less than a factor of 2. At the 90th percentile of the distributions, the consequences range from about 2000 to 4000 early fatalities. At the 99th percentile, the range is about 7000 to 14,000. The higher-consequence events with conditional probabilities less than  $10^{-2}$  typically result from sequences with an onset of precipitation over a populated area. The frequency of precipitation (fraction of hours with recorded precipitation) in the 29 records varies by about a factor of 10, ranging from 1 percent for the Phoenix record to 10 percent at Caribou, ME (see Table A.3-3). Therefore, the probabilities of the high-consequence events also vary by about a factor of 10. The peaks (maximum calculated number of early fatalities) of the 29 early fatality CCDFs also vary by about a factor of ten ( $10^4$  to  $10^5$  fatalities). This

a. Complementary Cumulative Distribution Functions are log-log plots of the probability that a consequence of a given magnitude will be equalled or exceeded.

Figure 2.4.1-1

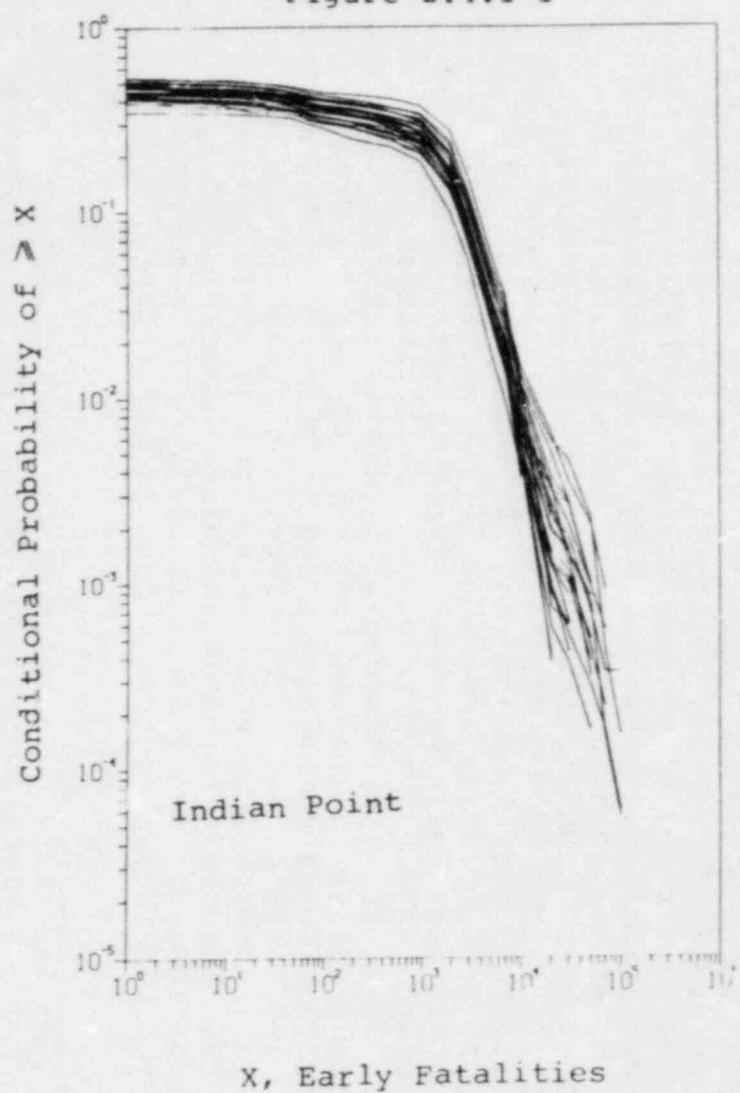
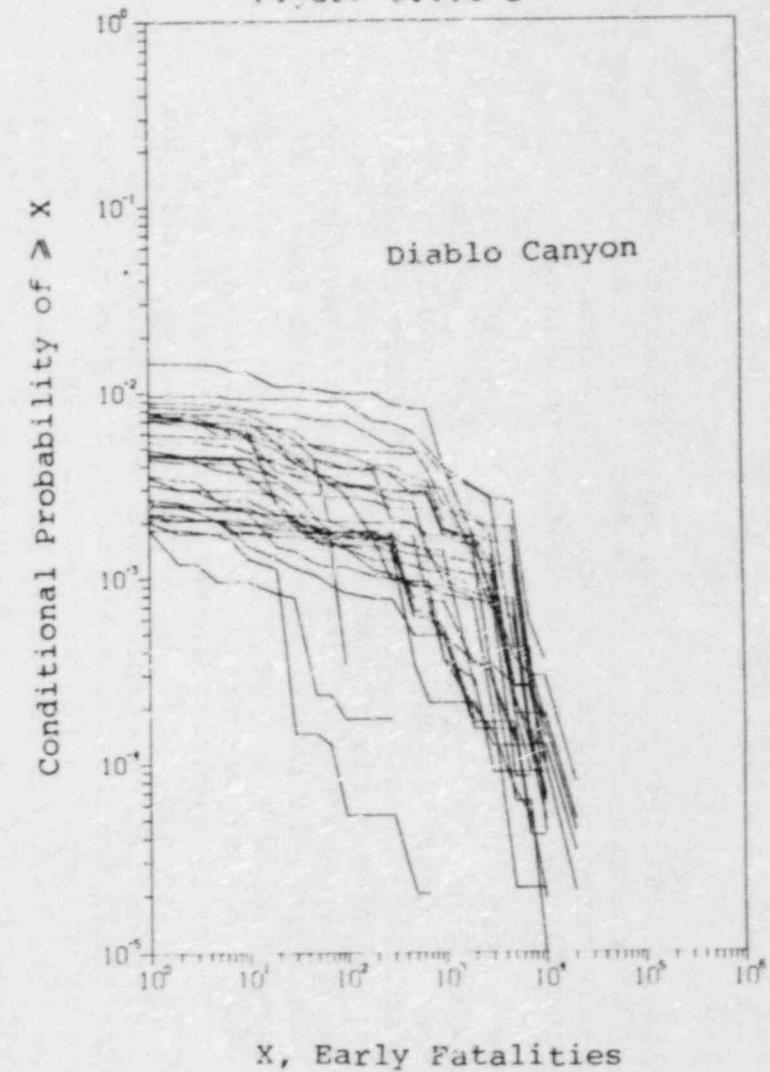


Figure 2.4.1-2



Early Fatality Complementary Cumulative Distribution Functions (CCDFs) Generated With Meteorological Data From 29 National Weather Service Stations. Probabilities are conditional on an SST1 accident occurring. The means of the distributions have the following ranges: Indian Point 710-1300, Diablo Canyon 0.1-18.  
Assumptions: Summary Evacuation, 1120 MWe reactor.

range probably is caused by inadequacies in the weather sequence sampling procedure used in the calculations.

In marked contrast to the Indian Point result, the 29 early fatality CCDFs for the Diablo Canyon site (Figure 2.4.1-2) are not closely clustered. Because of the very low population density surrounding the Diablo Canyon site, early fatalities occur above the 99th percentile of the distributions<sup>a</sup> for only one of the 29 meteorological records. Examination of the sequences which produced any early fatalities showed that almost all were sequences containing precipitation. The spread of the distributions (as much as 2 orders of magnitude in both probabilities and consequences) is caused by variations in the frequency of precipitation among the 29 records and inadequacies in the weather sequence sampling procedure.

Results similar to those presented in Figure 2.4.1-2 were found by Sprung [26] for calculations with buoyant plumes where, again, the occurrence of precipitation is required to produce significant numbers of early fatalities (Note that all releases in the present study are assumed to be non-buoyant. The effect of plume buoyancy on predicted consequences is discussed in Section 2.7.2.)

Figures 2.4.1-1 and 2.4.1-2 indicate that out to the 99th percentile of the conditional distributions, the meteorological record used in the calculations does not have a significant impact on the predicted distributions of early fatalities (CCDF mean values differ by less than a factor of 2). Figures 2.4.1-3 and 2.4.1-4 show the 29 early-injury CCDFs for the two sites. Except for three of the meteorological records, there is again very little variation among consequences with conditional probabilities greater than  $10^{-2}$ . The outlying curves are for the Apalachicola, Seattle, and El Paso meteorological records at the Indian Point site and the Apalachicola and Seattle records at Diablo Canyon. Apalachicola and Seattle are two of the "wetter" meteorological records; inexplicably, El Paso is one of the driest. The source of these anomalies is not certain, but is probably due to inadequacies of the weather sequence sampling procedure (i.e., rain events beyond 30 miles are not appropriately weighted).

a. Those consequences that would be equalled or exceeded by 1 out of every 100 releases.

Figure 2.4.1-3

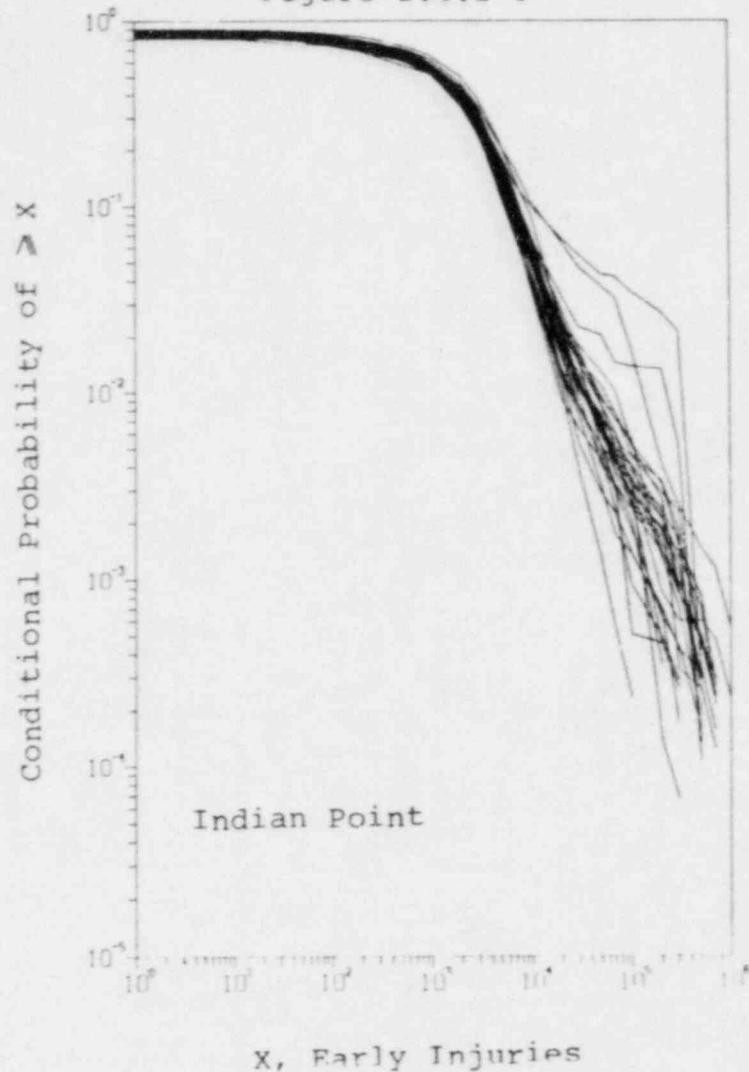
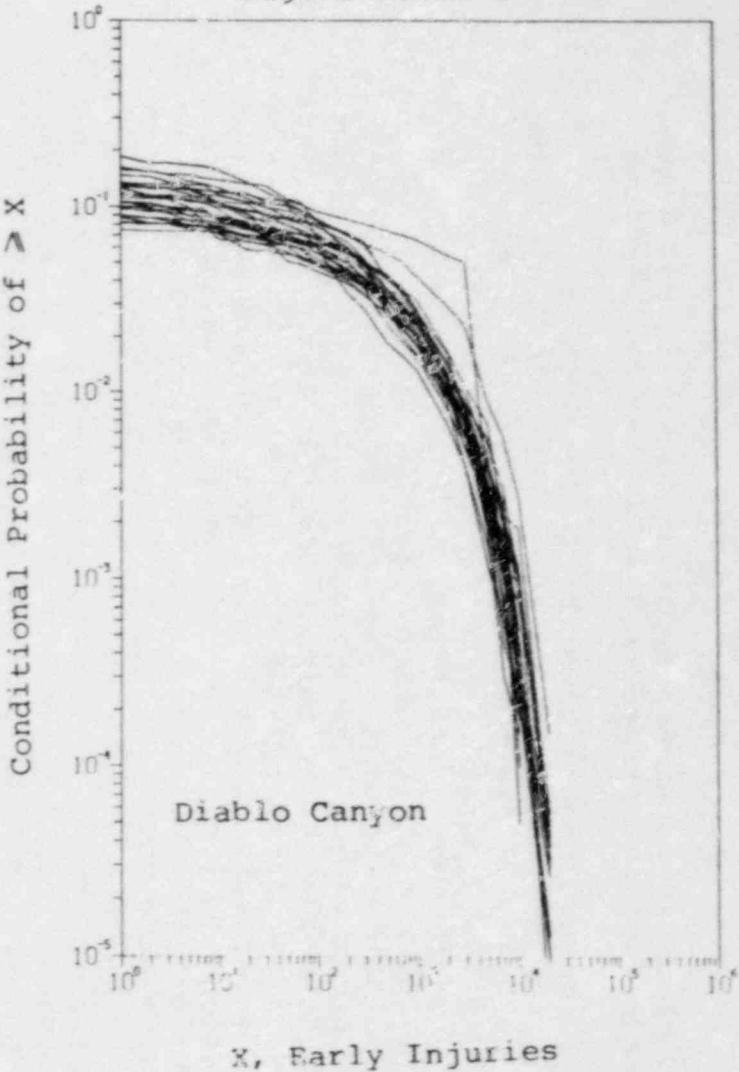


Figure 2.4.1-4



Early Injury Complementary Cumulative Distribution Functions (CCDFs) Generated With Meteorological Data From 29 National Weather Service Stations. Probabilities are conditional on an SST1 accident occurring. The means of the distributions have the following ranges: Indian Point 2400-14,000 (2,400 - 5,000 without the 3 high outlying CCDFs), Diablo Canyon 64-240. Assumptions: Summary Evacuation, 1120 MWe reactor.

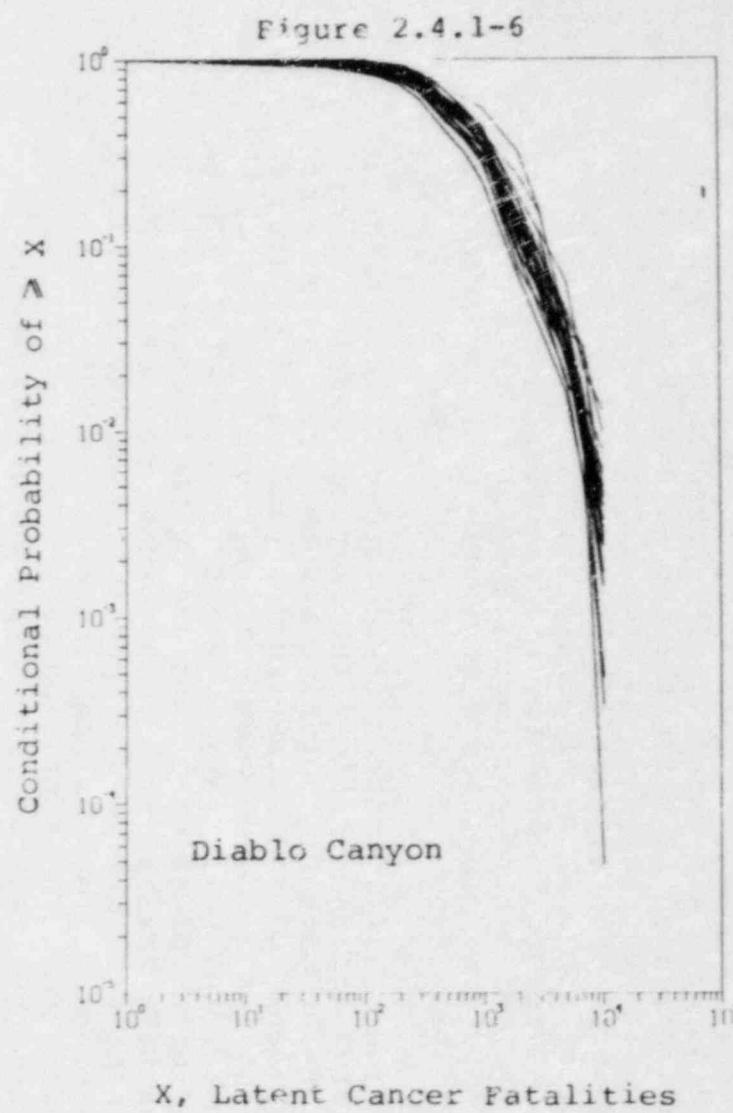
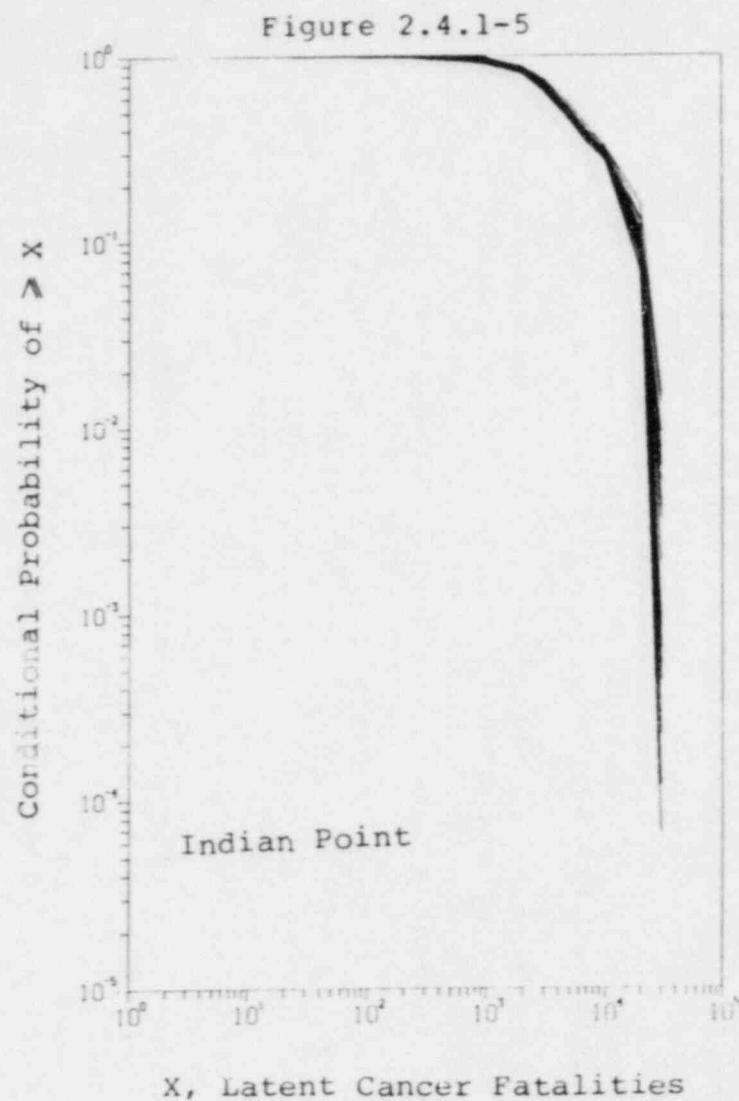
Figures 2.4.1-5 and 2.4.1-6 present the 29 latent cancer fatality CCDFs for the two sites. Both figures show variations only in the probabilities of the high-consequence events, most likely a reflection of the different probability of precipitation in each meteorological record. These two figures clearly indicate that the meteorological record does not have a significant impact on predicted distributions of latent cancer fatalities.

Figure 2.4.1-7 shows the interdicted-land area CCDFs for the 29 records. Interdicted land is a measure of the potential offsite economic consequences of an accident and is calculated independent of population distribution and wind rose. At the 90th percentile, the predicted areas vary by about a factor of 3. There is a 2-order of magnitude spread in the probabilities of the CCDF maxima (high-consequence sequences). The different probabilities of precipitation among the 29 meteorological records can account for about 1 order of magnitude. The remaining factor of 10 most likely is caused by inadequacies in the weather-sequence categorization procedure (see Appendix E).

This section has examined the sensitivity of consequence magnitudes to meteorological record. The sensitivity to meteorological record of the distances to which consequences occur is discussed in Section 2.6.

The following conclusions can be drawn from this sensitivity analysis:

- o Given a specific release, the one-year meteorological record used in the calculations does not have a significant impact on predicted consequences out to the 99th percentile of the distributions. Therefore, when suitable meteorological data is not available from the site, the use of substitute meteorological data, such as that available from a nearby National Weather Service station, is probably adequate for performing consequence calculations with CRAC2.
- o Major differences in predicted consequences among the 29 meteorological records occur at probabilities less than  $10^{-2}$  and probably arise from variations in the frequency of precipitation and inadequacies in the procedure used to sample weather sequences.



Latent Cancer Fatality Complementary Cumulative Distribution Functions (CCDFs) Generated With Meteorological Data From 29 National Weather Service Stations. Probabilities are conditional on an SSTD accident occurring. The means of the distributions have the following ranges: Indian Point 7600-9300, Diablo Canyon 750-1600. Assumptions: Summary Evacuation, 1120 MWe reactor.

- Further refinement is needed in the CRAC2 treatment of meteorological data. Possible improvements include the use in the weather sequence sampling procedure of more than 1 year of weather data and the consideration of precipitation intensity. In addition, sequences with an onset of precipitation may need to be categorized to distances beyond the present 30 miles, perhaps to 100 miles.

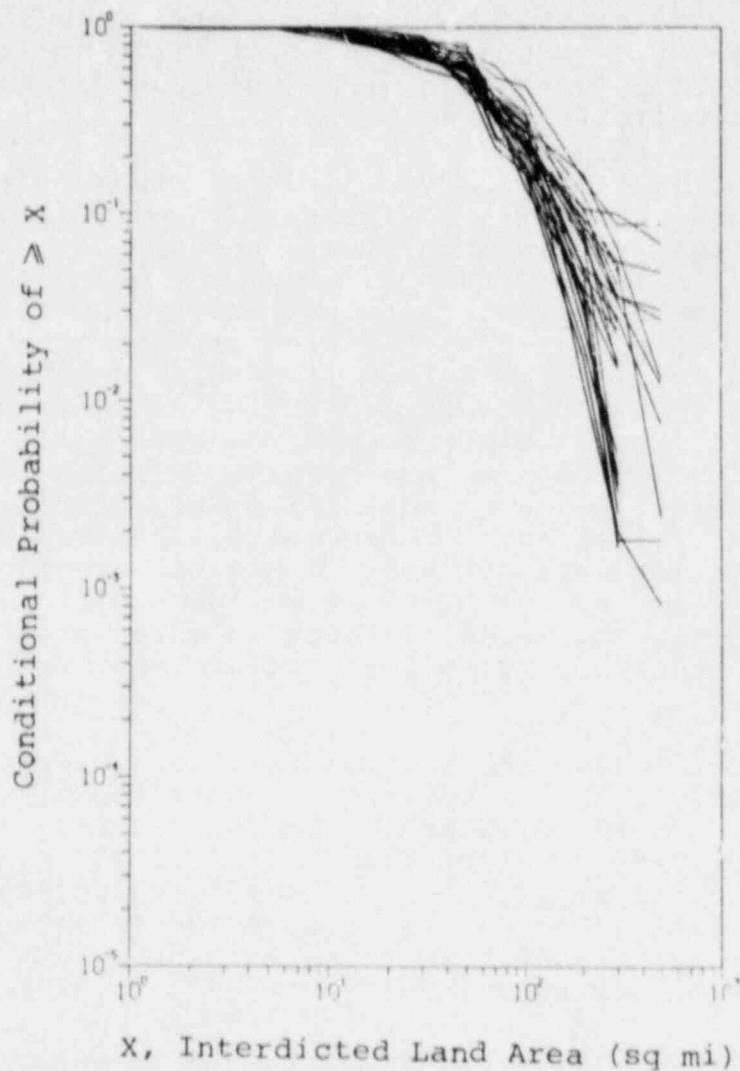


Figure 2.4.1-7. Interdicted Land Area Complementary Cumulative Distribution Functions (CCDFs) Generated with Meteorological Data from 29 National Weather Service Stations. Probabilities are conditional on an SST1 accident occurring. The means of the distributions range from 72 to 140 square miles. Assumption: 1120 MWe reactor.

#### 2.4.2 Sensitivity to Site Population Distribution

To examine the role of population distribution in determining reactor accident consequences, a sensitivity study was performed using the actual population distribution and 1-year average wind rose from each of the 91 U.S. reactor sites having either an operating license or a construction permit. Calculations performed using actual site population distributions also provide a better understanding of past siting policy and a reference against which the consequences of proposed siting policies can be compared.

For each of the 91 sites, a representative meteorological record was selected from the 29 National Weather Service records used in this study (see Appendix A). As discussed in the previous section, the meteorological record used in the calculations has only a marginal impact on the predicted distribution of consequences. Thus, the uncertainty resulting from using a substitute record (rather than one obtained at the site) is probably not significant. Since the purpose of this study was to examine the impact on consequences of specific site characteristics, a standard 1120 MWe reactor was assumed at all 91 sites. Consequently, the results of these calculations are not assessments of existing reactor-site combinations, and it would be misleading to use them as such. Finally, each calculation also assumed the occurrence of an SST1 release and of Summary Evacuation.

Figures 2.4.2-1a through 2.4.1-1c show early fatality, early injury, and latent cancer fatality CCDFs for all of the 91 sites. The figures have been truncated at conditional probabilities of  $10^{-3}$  (one in a thousand releases). This was done because consequence probabilities and magnitudes for improbable events (those with conditional probabilities less than  $10^{-3}$ ) are very uncertain. A large part of this uncertainty is due to the assumption of an evacuation only within 10 miles. Because of this assumption, all persons beyond 10 miles were assumed to be exposed to deposited radionuclides for 1 day, regardless of dose rate<sup>a</sup>. Any emergency actions taken beyond 10 miles

a. Under some meteorological conditions, the 1-day bone marrow dose at 10 miles can exceed 1000 rem.

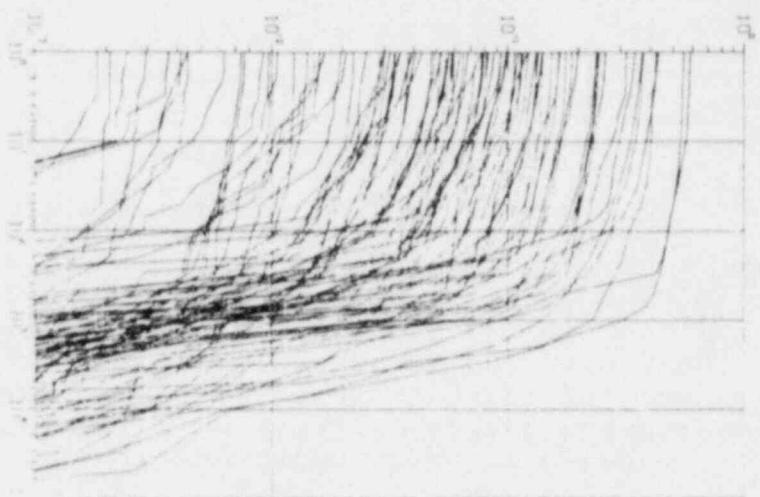
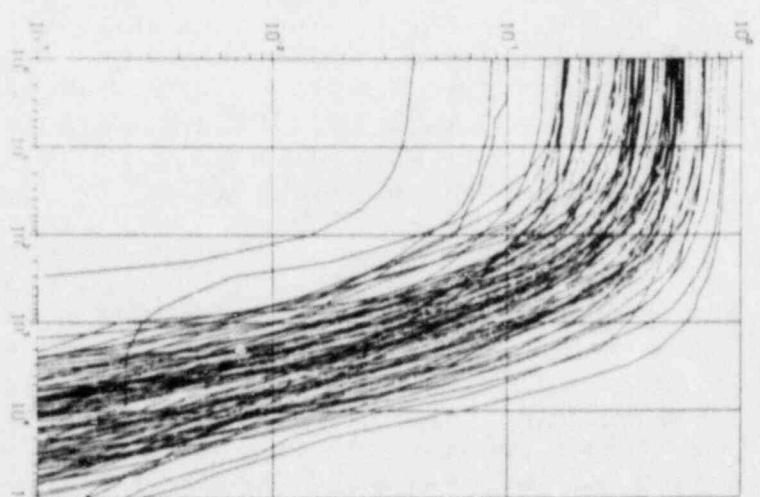
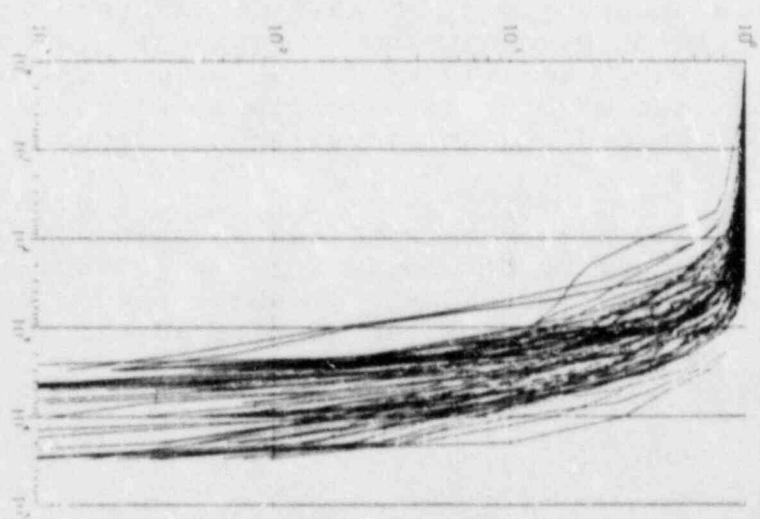
Conditional Probability of  $\geq X$ Conditional Probability of  $\geq X$ Conditional Probability of  $\geq X$ 

Figure 2.4.2-1. (a) Early Fatality, (b) Early Injury, and (c) Latent Cancer Fatality CCDFs Conditional on an SSTI Release at all 91 Current U.S. Reactor Sites. Assumptions: 1120 MWe reactor, Summary Evacuation, representative meteorology. Range of means: early fatalities 0.4 to 970, early injuries 4 to 3600, and latent cancer fatalities 230 to 8100.

(e.g., sheltering or prompt relocation) would significantly mitigate the consequences of low-probability, high consequence events [27]. The effect on consequences of different emergency response scenarios is discussed in Section 2.5.

The 91 early fatality CCDFs range (on the probability axis) over almost 3 orders of magnitude in the conditional probability of any early fatalities [i.e.,  $P(\geq 1)$ ] and over nearly 4 orders of magnitude in consequences at a conditional probability of  $10^{-3}$  (consequence axis). The conditional means of the 91 CCDFs range from 0.4 to 970 fatalities. Figure 2.4.2-2 presents a histogram of the conditional means of the early fatality CCDFs versus number of sites. Only four sites have means above 250 fatalities; over half are less than 50. Table C-1 in Appendix C lists the conditional mean number of early fatalities, early injuries, and latent cancer fatalities for each of the 91 sites. The 99th percentile<sup>a</sup> of the conditional distributions of early fatalities range from zero to 8000. Figure 2.4.2-3 presents a histogram of the 99th percentile of the distributions versus number of sites.

The 91 early injury CCDFs (Figure 2.4.2-1b) range over approximately 1 order of magnitude in the conditional probability of having any injuries [ $P(\geq 1)$ ] and over 2 orders in consequence magnitude at a conditional probability of  $10^{-3}$ . The conditional mean numbers of early injuries range from 4 to 3600. The latent cancer fatality CCDFs (Figure 2.4.2-1c) show less than 1 order of magnitude spread on both axes. The conditional means of the latent cancer fatality CCDFs range from 230 to 8100.

In Section 2.4.1, it was shown that the meteorological record does not significantly affect the calculated distributions of consequences. Therefore, the wide variability in calculated distributions displayed in Figures 2.4.2-1a through c (early fatalities, early injuries, latent cancer fatalities) can be due only to differences in the 91 population distributions since all other factors were either held constant or have no significant effect on predicted consequences.

a. Those consequences that would be equalled or exceeded by 1 out of every 100 releases.

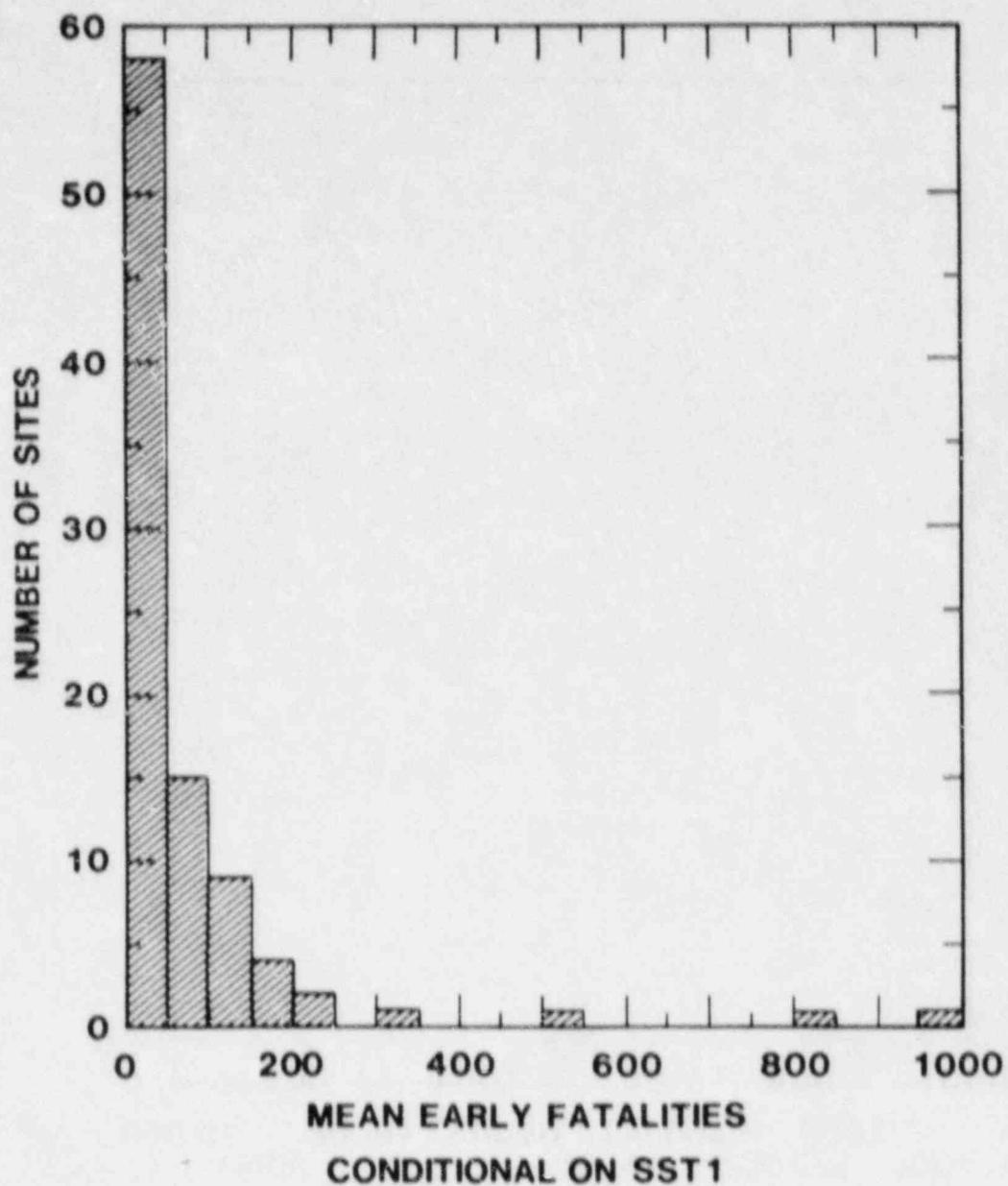


Figure 2.4.2-2. Histogram of Mean Early Fatalities for 91 Sites, Conditional on an SST1 release. Assumptions: 1120 MWe reactor, a representative meteorological record, and Summary Evacuation.

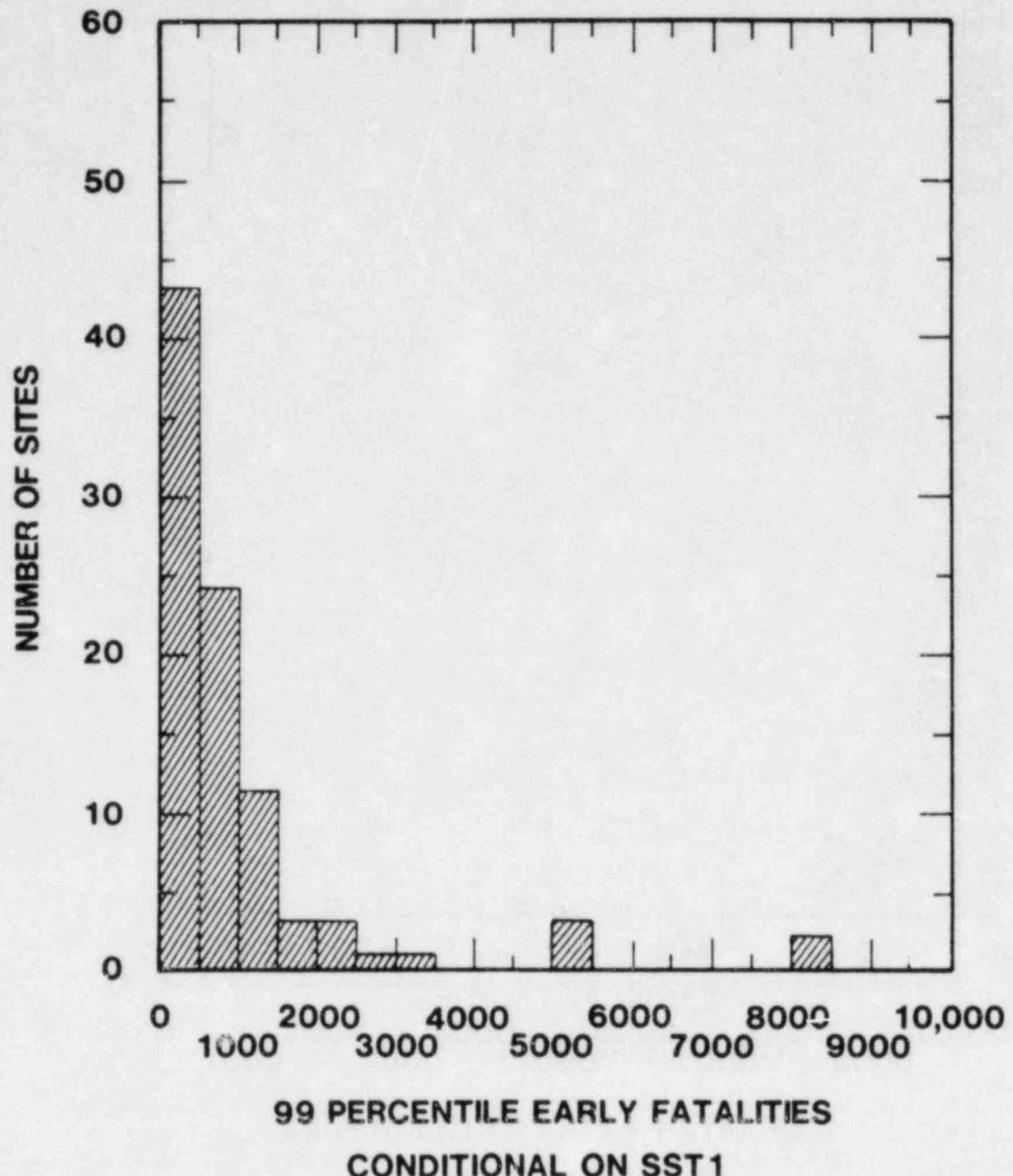


Figure 2.4.2-3. Histogram of the 99th Percentile of the Distribution of Early Fatalities for 91 Sites, Conditional on an SST1 Release.  
Assumptions: 1120 MWe reactor, a representative meteorological record, and Summary Evacuation.

The different degrees of variability of the three consequences are primarily due to the different distances to which each consequence occurs. Within 20 miles of the reactor there is tremendous variability in the 91 population distributions. Within this distance, the population densities range from 1 to 710 people per square mile (see Section 3). Therefore, the distributions of early fatalities, which are confined to areas within a few tens of miles of the site (most occur within a few miles, see Section 2.6), show the greatest variability. Early injuries can occur to many tens of miles, but most occur within about 30 miles. Within 50 miles of the 91 sites, average population densities range from 10 to 2100 people per square mile. Since this range (factor of 210) is less than that observed to 20 miles (factor of 710), the variability in the 91 early injury CCDFs is less than that obtained for early fatalities. Finally, when averaged over very large areas, the variability in the 91 population distributions is greatly reduced. The population densities within 200 miles of the 91 sites vary between 14 and 335 people per square mile (factor of 24). Thus, the distributions of latent cancer fatalities, which can occur over very large areas, show the least variability.

Some specific characteristics of population distributions which might impact the variability of consequences are discussed in Section 2.7.4. Finally, for each of the 91 sites examined in this report, early fatality, early injury, and latent cancer fatality CCDFs conditional on an SST1 release are presented in Appendix C. When examining these CCDFs, it is important to remember that they are not truly site specific. Although each CCDF was calculated using the site's wind rose, the population distribution about the site, and an appropriate substitute meteorological record, the SST1 release assumed in each calculation was not modified to reflect the specific design of the site's reactor. Instead, a standard 1120 MWe PWR was assumed in each calculation.

## 2.5 Sensitivity to Emergency Response

Should an accident at a nuclear power plant lead to a significant release of radioactivity, public radiation exposures could be mitigated by evacuation, sheltering, relocation, or medical prophylaxis<sup>a</sup>. Summary Evacuation within 10 miles was assumed in most of the calculations presented in other sections of this report. In this section the sensitivity of early fatalities and early injuries to emergency response is examined by a series of parametric calculations. All of these calculations assume an SST1 release from an 1120 MWe reactor, Indian Point population and wind rose, and New York City meteorology.

The emergency response submodel in CRAC2 was briefly described in Section 2.2.2 and is more fully described in this section and in Appendix E. The model allows for the mitigation of radiation exposures by evacuation or by sheltering followed by relocation. Evacuation is characterized by the delay time between accident warning and the initiation of evacuation, by the distance within which people evacuate, and by the evacuation speed [8]. Sheltering is characterized by the distance within which all people take shelter, the shielding factors afforded by the structures in which they take shelter [29-31], and the delay time between cloud passage and the relocation of sheltered population. The parameters that describe these emergency response scenarios are first defined and then the results of the parametric calculations are presented.

- 
- a. Evacuation is the expeditious movement of people to avoid exposure to the passing cloud of radioactive material. Sheltering is the expeditious movement of people indoors, if possible, into basements or masonry buildings which afford enhanced shielding from radiation. Relocation is the movement of exposed persons out of contaminated areas after the passage of the radioactive cloud. Medical Prophylaxis is the administration of agents which decrease or block internal exposures (e.g., KI prophylaxis decreases thyroid exposures [28]).

The following eight parameters essentially determine the impact of the CRAC2 emergency response model on consequence predictions:

Warning Time: Time from accident notification by plant personnel to release of radioactivity due to containment failure (e.g., 0.5 hr for SST1).

Delay Time: Time from accident notification to the initiation of emergency response (0 hr for sheltering; 1-5 hr for evacuation).

Evacuation Radius: The radius within which all occupants of a 90° sector (centered on the plume centerline) evacuate (10 mi in the base case calculation).

Evacuation Speed: The effective speed at which evacuees move radially away from the reactor (10 mph in the base case calculation).

Evacuation Distance: The radial distance to which the evacuees move (5 mi beyond the evacuation radius; therefore, 15 mi for the base case calculation) before they are removed from the calculation because they are assumed to have enough information to avoid additional exposure.

Sheltering Radius: The radius within which all non-evacuating occupants of a 90° sector (centered on the plume) take shelter. If the sheltering radius is less than or equal to the evacuation radius, only evacuation takes place. If the sheltering radius is larger than the evacuation radius, then all persons between the evacuation radius and the sheltering radius take shelter. Beyond the sheltering radius, normal activity is assumed to continue (i.e., some people are outdoors).

Shielding Factor [29]: The fraction of the dose to an unsheltered individual received by an individual sheltered in a building or in a vehicle (i.e., during evacuation). Shielding factors for buildings depend on the housing stock (percent brick, availability of basements) and, therefore, vary by geographic region. Different shielding factors are used to decrease unshielded exposures to the radioactive plume and to contaminated ground (see Appendix A).

Relocation Time: The period which elapses after passage of the radioactive plume before non-evacuating individuals are moved from contaminated areas (24 hr in the base case calculation)

Relationships between several of these eight emergency response model parameters are schematically depicted in Figure 2.5-1.

The CRAC2 emergency response submodel allows for the specification of up to six different emergency response scenarios and will calculate a weighted average of the results for any designated set of scenarios. CRAC2 calculations presented in other sections of this report generally assume "Summary Evacuation," which is the weighted summation of three different evacuation scenarios as follows:

<u>Scenario Number</u>	<u>Weight<sup>a</sup></u>	<u>Type of Response</u>	<u>Response Distance</u>	<u>Response Speed</u>	<u>Delay Before Response</u>
1	30%	evacuation	10 miles	10 mph	1 hour
2	40%	evacuation	10 miles	10 mph	3 hours
3	30%	evacuation	10 miles	10 mph	5 hours

- a. The 30%/40%/30% weighting provides a best fit to EPA evacuation data [7] (See Appendix E).

The sensitivity of the CRAC2 evacuation model to evacuation speed has been previously investigated by Aldrich, et al. [9], who found that, for evacuation within 10 miles after a 3 hour delay, early fatalities were minimally affected by effective evacuation speed provided that the evacuation speed was at least 10 mph. The impact of delay time on early health effects is illustrated in Figure 2.5-2, which presents early fatality CCDFs for 10 mph evacuations within 10 miles after delays of 1, 3, and 5 hours, respectively (scenarios 1, 2, and 3). Also plotted is the CCDF for Summary Evacuation, which is the 30:40:30 weighted summation of the CCDFs for scenarios 1, 2, and 3. Figure 2.5-2 shows (1) that early fatalities are substantially decreased by short delay times ( $\lesssim 1$  hr); and (2) that Summary Evacuation yields results nearly identical to those obtained for scenario 2 (3 hr delay).

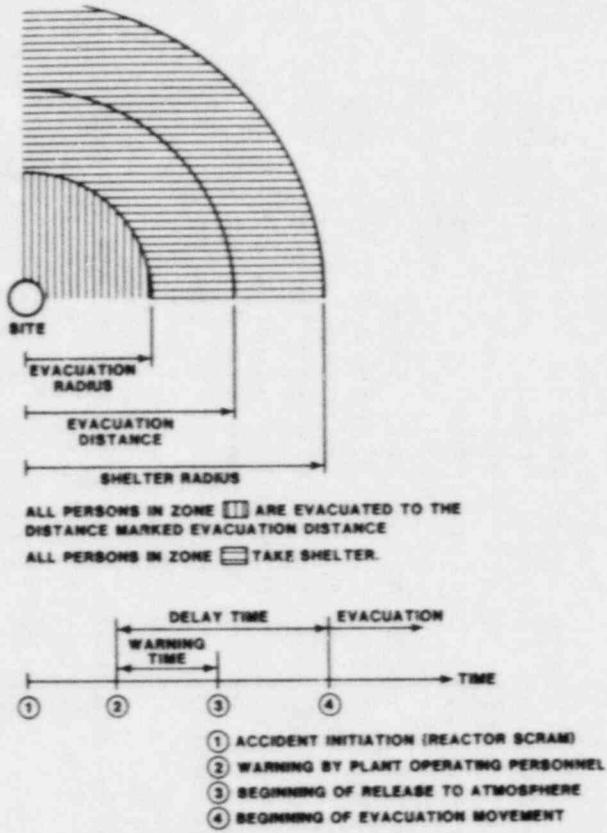


Figure 2.5-1.

Relationships Between  
Evacuation Model Parameters.

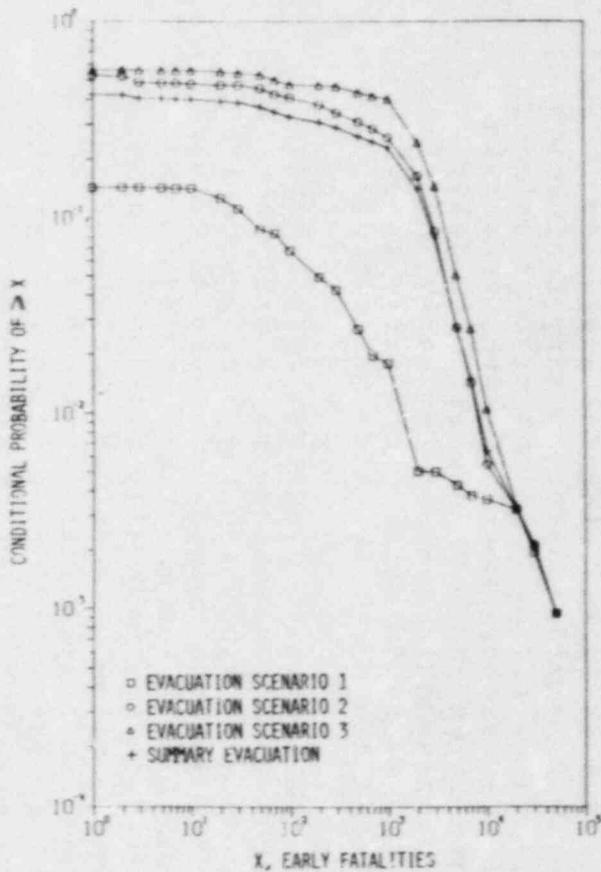


Figure 2.5-2.

Early Fatality Complementary Cummulative Distribution Functions for 10 mph Evacuations within 10 Miles after Delays of 1, 3, and 5 Hours (Scenarios 1, 2, and 3, respectively) and for Summary Evacuation. Assumptions: 1120 MWe reactor, Indian Point population and wind rose, New York City meteorology.

Table 2.5-1 presents mean and 99th percentile<sup>a</sup> values of early fatalities and early injuries for emergency response scenarios 1, 2, and 3 and for Summary Evacuation. The table shows (1) that, for evacuations of population within 10 miles of the reactor, mean and 99th percentile values of early fatalities are more sensitive to delay time than are the corresponding values for early injuries; and (2) that for both early fatalities and early injuries, 99th percentile values are about 10 times mean values.

The different sensitivities displayed result largely from the fact that each consequence has a different characteristic distance within which the consequence is calculated to occur (distance dependencies are discussed in detail in Section 2.6). For most weather sequences, fatal doses of radiation are generally confined to distances of less than 10 miles. Therefore, for almost all of the weather sequences sampled, the entire population potentially subject to fatal radiation doses is evacuating. Consequently, mean and 99 percentile values for early fatalities are highly sensitive (factors of  $8 \approx 1400/180$  and  $7 \approx 10,000/1400$ ) to delay time. In contrast to this, doses of radiation sufficient to cause early injuries frequently occur to distances significantly greater than 10 miles. Therefore, because a significant fraction of the population potentially subject to doses sufficient to cause injuries (i.e., the population beyond 10 miles) is not evacuating, mean and 99th percentile values of early injuries are less sensitive (factors of 1.7 and 1.1) to delay time than are the corresponding values for early fatalities. Finally, for evacuations of population within 10 miles, peak values (worst case calculated for any weather sequence, conditional probabilities of  $\leq 10^{-3}$ ) of early fatalities and early injuries are essentially insensitive to evacuation delay time e.g., in Figure 2.5-2 the four early fatality CCDFs have identical tails). This is because early fatality and injury worst case results (CCDF tails) are caused by rainout of radioactivity from the plume onto population centers (cities) located more than 10 miles from the reactor. Since these cities were not evacuated in this set of calculations, these calculations yield peak values of early fatalities and early injuries that are not affected by evacuation delay time.

Table 2.5-2 presents the effect of the distance within which population is evacuated upon early fatalities

a. Consequence magnitude that would be equalled or exceeded following 1 out of every 100 releases.

Table 2.5-1. Effect of Delay Time on Early Fatalities and Early Injuries for Evacuation to 10 Miles. Results are Conditional on an SST1 Release.

Delay Time (hr)	Early Fatalities		Early Injuries	
	Mean	99th Percentile	Mean	99th Percentile
1	180	1,400	2500	30,000
3	920	8,000	4000	32,000
5	1400	10,000	4300	34,000
Summary	830	8,300	3600	33,000

Assumptions: 1120 MWe reactor, SST1 release, Indian Point population and wind rose, New York City meteorology.

Table 2.5-2. Effect of Evacuation Distance on Early Fatalities and Early Injuries for Summary Evacuation. Results are Conditional on an SST1 Release.

Evacuation Distance (mi)	Early Fatalities		Early Injuries	
	Mean	99th Percentile	Mean	99th Percentile
0 <sup>a</sup>	3600	18,000	6300	41,000
5	1100	11,000	5500	40,000
10	830	8,300	3600	33,000
25	700	7,200	1800	9,400

a. No evacuation

Assumptions: 1120 MWe reactor, SST1 release, New York City meteorology, Indian Point population and wind rose.

and early injuries for Summary Evacuation. The table shows that mean and 99th percentile values of early fatalities and injuries are all quite sensitive to the distance within which population is evacuated. Because worst case results (conditional probabilities of  $\leq 10^{-3}$ ) for early fatalities are generally caused by rainout of the radioactive plume onto a city located further than 10 but less than 25 miles from the reactor, evacuation within 25 miles lowers the worst case number of early fatalities from 57,000 (for evacuation within 10 mi) to 15,000 (for evacuation within 25 mi).

The next three tables examine the sensitivity of early health effects to sheltering parameters. Table 2.5-3 displays the effect of the distance within which population takes shelter in preferred locations (building interiors, basements if available) on early fatalities and early injuries. Examination of the table shows that the effect of response distance for sheltering is similar to that for evacuation. Mean and 99th percentile values of early fatalities and injuries are all quite sensitive to sheltering distance. As before, 99th percentile values are about 10 times the mean result and a 25 mile response distance significantly decreases (by about a factor of 5) the worst case result (conditional probability of  $\leq 10^{-3}$ ) below the result obtained with a 10 mile response distance.

Table 2.5-4 illustrates the impact of the availability of basements upon the degree of shielding (and thereby the reductions in consequences) afforded by sheltering. The table shows that mean and 99th percentile values of early fatalities are substantially decreased, if Northeast regional shielding factors (building characteristics: 87% basements, 47% brick) are used rather than Pacific Coast regional shielding factors (building characteristics: 23% basements, 27% brick) [29]. Because sheltering was assumed to take place only to 10 miles, mean and 99th percentile values of early injuries show a lessened sensitivity. These results are consistent with results previously obtained by Aldrich et al. [27].

Table 2.5-3. Effect of Sheltering Distance on Early Fatalities and Early Injuries for Preferential Sheltering Followed by Relocation. Results are Conditional on an SST1 Release.

Sheltering Distance (mi)	Early Fatalities		Early Injuries	
	Mean	99th Percentile	Mean	99th Percentile
5	830	9,300	5600	40,000
10	560	5,500	3700	32,000
15	490	4,900	2700	25,000
25	420	4,500	1800	11,000

Assumptions: 1120 MWe reactor, SST1 release, Indian Point population and wind rose, New York City meteorology, no evacuation, Northeast regional shielding factors, relocation after 6 hr.

Table 2.5-4. Effect of Early Fatalities and Early Injuries for Sheltering to 10 Miles Followed by Relocation. Results are Conditional on an SST1 Release.

Number of Basements	Early Fatalities		Early Injuries	
	Mean	99th Percentile	Mean	99th Percentile
Few <sup>a</sup>	1200	9,300	4100	34,000
Many <sup>b</sup>	560	5,500	3700	32,000

a. 23% basements (Pacific Coast regional shielding factors used, see Appendix A).

b. 87% basements (Northeast regional shielding factors used, see Appendix A).

Assumptions: 1120 MWe reactor, SST1 release, Indian Point population and wind rose, New York City meteorology, no evacuation, relocation after 6 hr.

After plume passage, relocation of sheltered populations decreases exposure to contaminated ground. The effect upon early fatalities and early injuries of decreasing relocation time from 24 to 6 hours is presented in Table 2.5-5. As before, because sheltering was assumed to take place only to 10 miles, mean and 99th percentile early injury values show little sensitivity, while mean and 99th percentile values for early fatalities decrease by a factor of two.

Table 2.5-5. Effect of Relocation Time on Early Fatalities and Early Injuries for Sheltering to 10 Miles. Results are Conditional on an SST1 Release.

Relocation Time (hr)	Early Fatalities		Early Injuries	
	Mean	99th Percentile	Mean	99th Percentile
6	560	5,500	3700	32,000
12	750	7,500	3800	33,000
24	1200	9,300	4100	34,000

Assumptions: 1120 MWe reactor, SST1 release, Indian Point population and wind rose, New York City meteorology, no evacuation, Northeast regional shielding factors.

Table 2.5-6 gathers together in a single table the results of all the calculations which examined evacuation or sheltering separately. The table presents the variation with response distance of early health consequences for five evacuation scenarios and three sheltering scenarios. Examination of Table 2.5-6 shows that for any response distance, expeditious evacuation (1 hr delay, 10 mph) and sheltering with expeditious relocation (after 6 hr) yield the smallest predictions of early health consequences. The table also confirms the strong dependence of mean early health consequences on response time and the less strong dependence on response distance.

Table 2.5-6. Dependence of Early Fatalities and Early Injuries on Response Distance for Eight Emergency Response Scenarios. Results are Conditional on an SSTI Release

Emergency Response		Response Distance (mi)									
Type	Characteristics	0 <sup>a</sup>	5	10	15	25	0 <sup>a</sup>	5	10	15	25
<u>Mean Early Fatalities</u>											
Evacuation	5 hr delay, 1 mph	3,600	2,100	1,900	1,800	1,800	6,300	6,200	5,300	5,100	4,700
	5 hr delay, 10 mph	3,600	1,600	1,400	1,300	1,250	6,300	6,000	4,300	3,300	2,500
	3 hr delay, 10 mph	3,600	1,200	920	860	790	6,300	5,800	4,000	3,000	2,200
	Summary Evacuation	3,600	1,100	830	780	700	6,300	5,500	3,600	2,700	1,800
	1 hr delay, 10 mph	3,600	440	180	110	40	6,300	4,600	2,500	1,500	700
Sheltering <sup>b</sup>	24 hr relocation	3,600	c	1,200	c	c	6,300	c	4,100	c	c
	12 hr relocation	3,600	c	750	c	c	6,300	c	3,800	c	c
	6 hr relocation	3,600	830	560	490	420	6,300	5,600	3,700	2,700	1,800
<u>99th Percentile Early Fatalities<sup>d</sup></u>											
Evacuation	5 hr delay, 1 mph	18,000	16,000	14,000	12,000	11,000	41,000	41,000	40,000	41,000	28,000
	5 hr delay, 10 mph	18,000	14,000	10,000	9,400	8,800	41,000	40,000	34,000	26,000	10,000
	3 hr delay, 10 mph	18,000	11,000	8,000	7,300	7,000	41,000	40,000	32,000	26,000	10,000
	Summary Evacuation	18,000	11,000	8,300	7,600	7,200	41,000	40,000	33,000	26,000	9,400
	1 hr delay, 10 mph	18,000	7,000	1,400	1,200	1,000	41,000	39,000	30,000	24,000	5,200
Sheltering <sup>b</sup>	24 hr relocation	18,000	c	9,300	c	c	41,000	c	34,000	c	c
	12 hr relocation	18,000	c	7,500	c	c	41,000	c	33,000	c	c
	6 hr relocation	18,000	9,300	5,500	4,900	4,500	41,000	40,000	32,000	25,000	11,000

Assumptions: 1120 MWe reactor, SSTI release, Indian Point population and wind rose, New York City Meteorology.

- a. No emergency response.      b. Northeast Regional Shielding Factors.      c. Not calculated.      d. Consequence magnitude equalled or exceeded following 1 out of every 100 releases.

Figures 2.5-3 and 2.5-4 present the variation with distance of the risk to an individual of early health effects (death or injury) for seven emergency response scenarios. The figures show that, as distance decreases, the different scenarios predict increasingly similar individual risks (the seven risk curves converge). The curves converge at short distances because many weather sequences result in radiation doses large enough to have fatalities or injuries for each of the seven emergency response scenarios. For example, expeditious evacuation (1 hr delay) is not always adequate because for many weather sequences the radioactive plume reaches people before they begin to evacuate. And sheltering with expeditious relocation is inadequate because for many weather sequences fatal or injury causing doses are still received by sheltered persons even with expeditious relocation. Accordingly, because at short distances each of the seven scenarios fails to provide sufficient protection for a substantial number of weather sequences, at these distances little sensitivity to differences in emergency response is observed. In agreement with Table 2.5-6, both figures show that individual risk of early health consequences decreases most rapidly with distance for expeditious evacuation (1 hr delay, 10 mph) or sheltering with expeditious relocation (after 6 hr).

The emergency response submodel in CRAC2 is able to apply one emergency response scenario to an inner region and a second scenario to an outer region. Using this option, the impact of emergency response scenarios, which call for both evacuation and sheltering, and the effect of response beyond 10 miles were briefly examined. Table 2.5-7 presents some evacuation data from Table 2.5-2 and contrasts that data with results obtained for emergency response scenarios which call for evacuation of population within 10 miles and sheltering of population from 10 to 25 miles. The table shows that for Summary Evacuation, increasing the response distance from 10 to 25 miles decreases mean and 99th percentile early injury values by factors of 2 and 3.5, respectively, while mean and 99th percentile early fatality values are somewhat lowered (mean, 19%; 99th, 15%). The table also shows (1) that Summary Evacuation to 10 miles in combination with sheltering (relocation after 24 hr) from 10 to 25 miles is as effective as Summary Evacuation to 25 miles; and (2) that in comparison to Summary Evacuation, expeditious evacuation (1 hr delay, 10 mph)

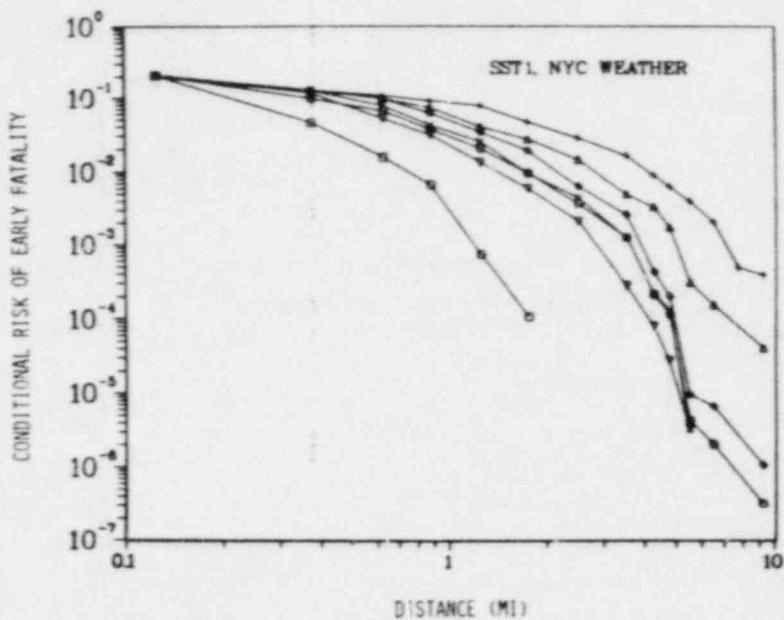


Figure 2.5-3. Conditional Risk of Early Fatality

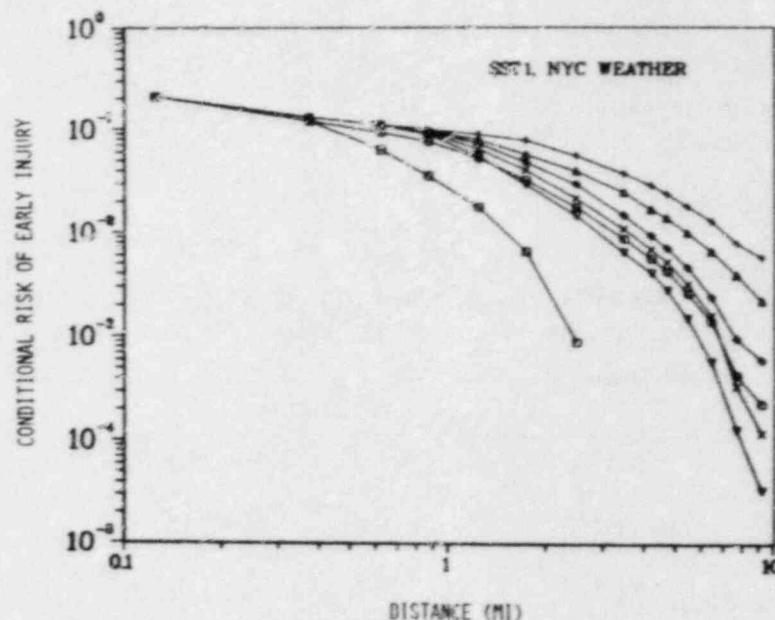


Figure 2.5-4. Conditional Risk of Early Injury

#### Legend

- + - No evacuation
- △ - 5 hr delay, 1 mph, within 10 mi
- ◊ - 5 hr delay, 10 mph, within 10 mi
- × - 3 hr delay, 10 mph, within 10 mi
- - Summary Evacuation, within 10 mi
- ▽ - Sheltering within 10 mi, 6 hr relocation
- - 1 hr delay, 10 mph, within 10 mi

Assumptions: 1120 MWe reactor, uniform wind rose, New York City meteorology, results conditional on an SST1 release.

Table 2.5-7. Impact of Emergency Response Beyond 10 Miles on Early Fatalities and Early Injuries. Results are Conditional on an SST1 Release.

Evacuation Distance (mi)	Evacuation Delay	Sheltering Distance (mi)	Early Fatalities		Early Injuries	
			Mean	99th Percentile	Mean	99th Percentile
0 - 10	Summary	None	830	8,300	3600	33,000
0 - 25	Summary	None	700	7,200	1800	9,400
0 - 10	Summary	10 - 25	690	5,400	1900	8,400
0 - 10	1 hr	10 - 25	40	750	750	5,800

Assumptions: 1120 MWe reactor, SST1 release, Indian Point population and wind rose, New York City Meteorology, Northeast regional shielding factors, relocation of sheltered individuals after 24 hr.

to 10 miles combined with the sheltering (relocation after 24 hr) from 10 to 25 miles substantially reduces mean and 99th percentile values for early fatalities (factors of 17 and 7, respectively) and significantly reduces mean and 99th percentile values for early injuries (factors of 2.5 and 1.5, respectively). Further, peak early fatalities (conditional probabilities  $\leq 10^{-3}$ ) are reduced by a factor of almost 10 (peak 15,000 to 1,600). Because of the substantial impact of emergency response beyond 10 miles upon peak early fatalities, it should be noted that most results presented in other sections of this report assume no immediate emergency response beyond 10 miles and consequently may significantly overestimate early fatality peaks.

Finally, Figure 2.5-5 indicates the sensitivity of early fatalities to the range of emergency response scenarios examined. In Figure 2.5-5 the CCDF for Summary Evacuation is the "base case" (see Section 2.2.4) result. The two bounding early fatality CCDFs for no emergency response and for expeditious evacuation to 25 miles show that the emergency response scenario selected has a substantial impact on consequence magnitude.

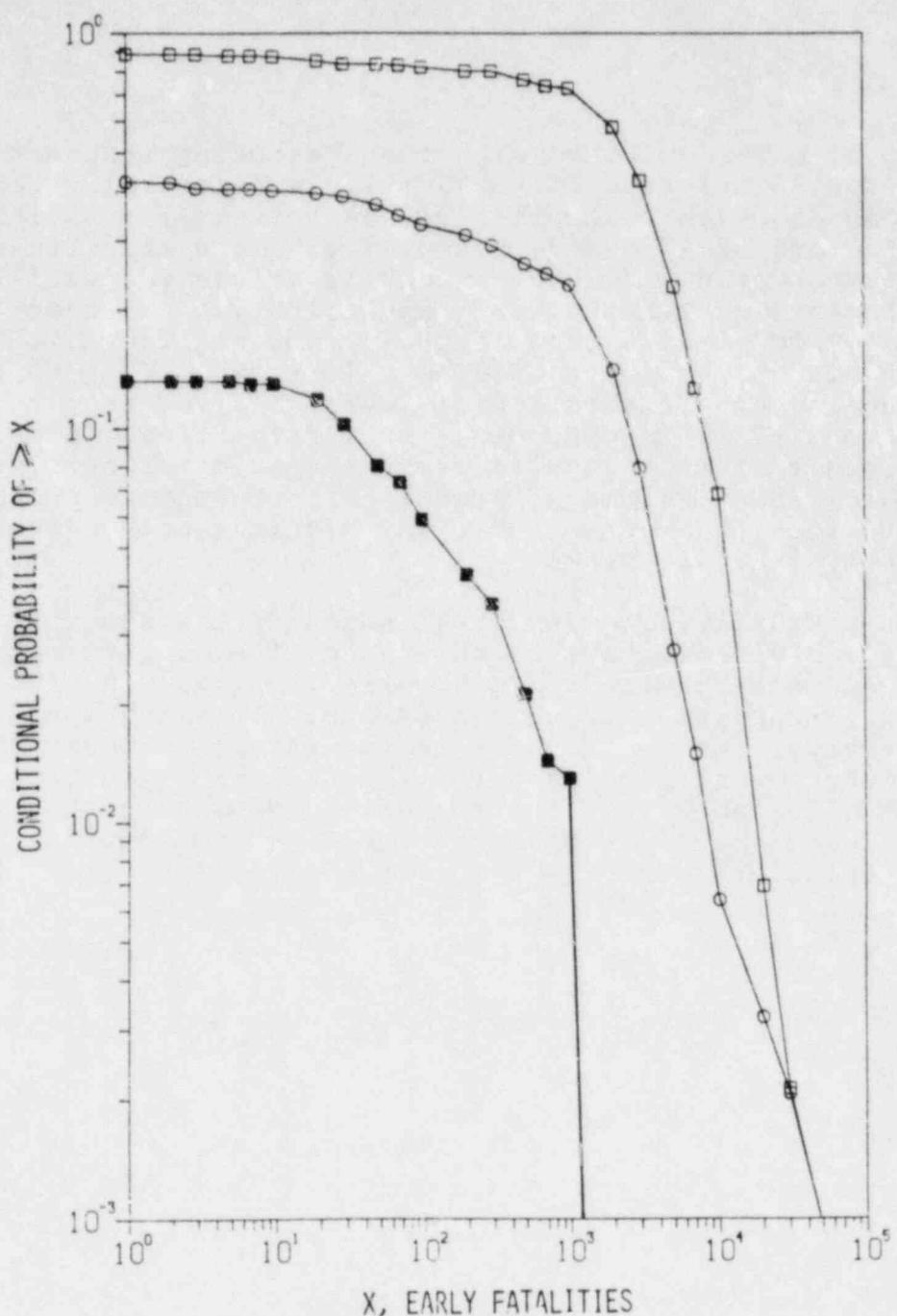


Figure 2.5-5. Impact of Range of Emergency Response Scenarios upon Early Fatalities. Results Conditional upon an SST1 Release

- - No emergency response
- - Summary Evacuation, within 10 mi
- - 1 hr delay, 10 mph, within 25 mi

Assumptions: 1120 MWe reactor, SST1 release, Indian Point population and wind rose, New York City meteorology.

## 2.6 Distance Dependencies of Reactor Accident Consequences

This section considers distances within which selected consequences might occur, as well as distances within which Protective Action Guides (PAGs) for radiation exposure [32] might be exceeded following a severe reactor accident. The sensitivities of these distances to meteorological conditions at the time of the accident, to differences between meteorological records, to accident severity, and to emergency response are examined. Because of the current controversy concerning the magnitudes of source terms for severe accidents (see Section 2.3.2), the impact of source term reductions on distance estimates is also considered.

The consequences that could result from a severe reactor accident include short-term effects such as early fatalities and injuries and long-term effects such as delayed cancer deaths and interdiction of land. Because early consequences would occur only after large, acute doses of radiation, these effects would be limited to areas close to the reactor (a few tens of miles). Population restrictions within these areas could therefore significantly impact the number of early consequences. As a result, estimates of distances to which fatal or injury-causing doses of radiation could be received are of interest for the development of reactor siting criteria. Following a severe reactor accident, contamination could be sufficiently high to require interdiction of property (buildings and land) to substantial distances (several tens of miles). Because interdiction of large areas could be a significant, and possibly dominant, contributor to the offsite costs of a reactor accident, distances to which land might be interdicted could also be an important consideration for the development of siting criteria. Since latent cancers can be induced by small doses of radiation, they can occur at large distances from the reactor. As a result, latent cancers would generally be less affected by population restrictions close to a reactor than would early fatalities or early injuries.

For each sampled meteorological sequence, the CRAC2 code calculates the maximum distances at which selected consequences might occur. These distances will depend on the magnitude and characteristics of

the source term as well as plume dispersion and depletion processes. By using the weather sequence sampling technique discussed in Section 2.2.1, the CRAC2 code can generate CCDFs of "maximum" consequence distances for any given source term. These curves illustrate the impact that radionuclide dispersion, which is determined by the weather conditions at the time of the accident, has on distances to which consequences occur.

Figures 2.6-1, 2.6-2, and 2.6-3 show SST1 and SST2 early fatality distance, early injury distance, and interdiction distance<sup>a</sup> CCDFs for the 29 meteorological records discussed in Section 2.4. The figures show that for an SST1 release early fatality distances range from 1 to 20 miles, early injury distances from 1 to 80 miles, and interdiction distances from 1 to 100 miles. Thus, for a single event, consequence distances are strongly influenced by the weather at the time of the release. However, the figures also show that for a specific release (e.g., SST1), CCDFs calculated using different meteorological records are quite similar. For example, the 90th percentile values of the 29 early fatality CCDFs calculated assuming an SST1 release range only from 6 to 9 miles.

These results also show that for the SST1 release, early fatalities would be limited to about 20 miles, injuries to about 50 miles, and land interdiction to about 100 miles. For the SST2 release, early fatalities would generally be limited to about 2 miles, injuries to about 8 miles, and land interdiction to about 10 miles. For each set of CCDFs, the variation in the peaks, and probabilities of the peaks, is principally due to a combination of (1) the order of magnitude variation in rain frequencies for the 29 meteorological records and (2) errors inherent in the weather sequence sampling procedure (see Section 2.4).

- a. Fatality and injury distances are defined to be distances within which individuals are at risk of being an early fatality or injury given the assumed release (SST1 or SST2). The interdiction distance is defined to be the distance within which land would be interdicted following the assumed release. The SST3 release is not large enough to cause early fatalities, early injuries, or interdiction of land offsite.

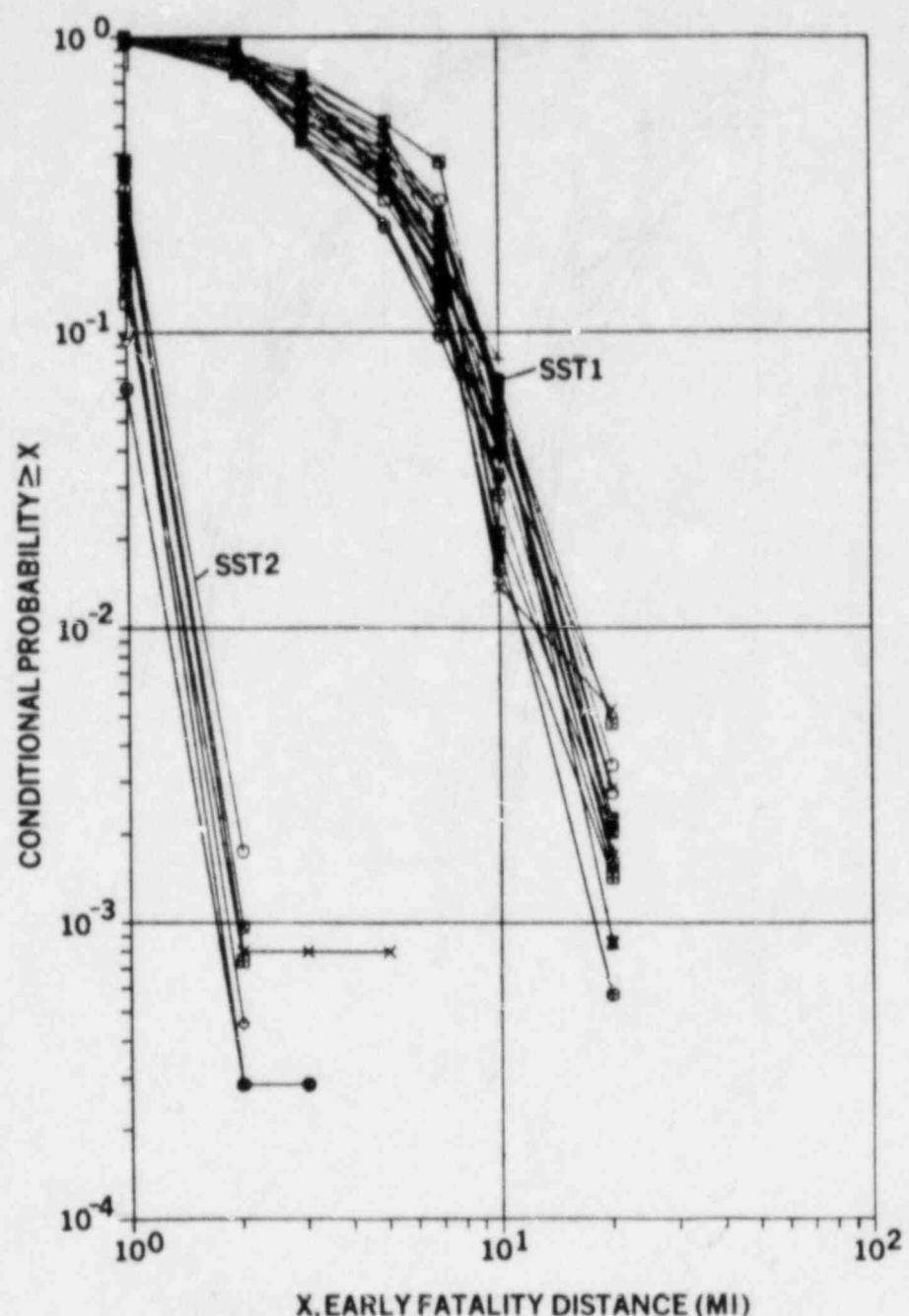


Figure 2.6-1. Conditional CCDFs of Early Fatality Distance for 29 Meteorological Records Calculated Assuming SST1 and SST2 Releases from an 1120 MWe PWR and No Emergency Response.

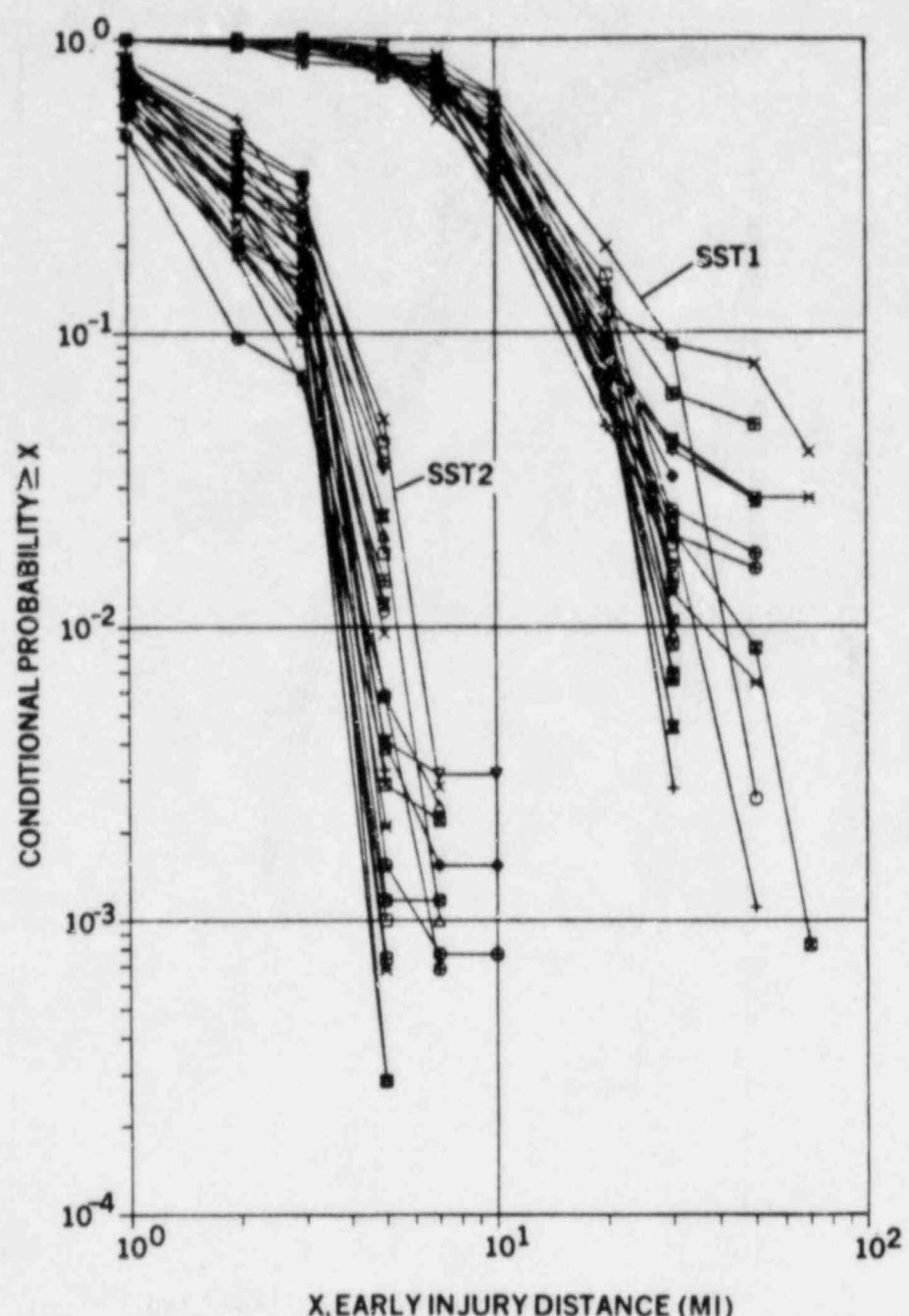


Figure 2.6-2. Conditional CCDFs of Early Injury Distance for 29 Meteorological Records Calculated Assuming SST1 and SST2 Releases from an 1120 MWe PWR and No Emergency Response.

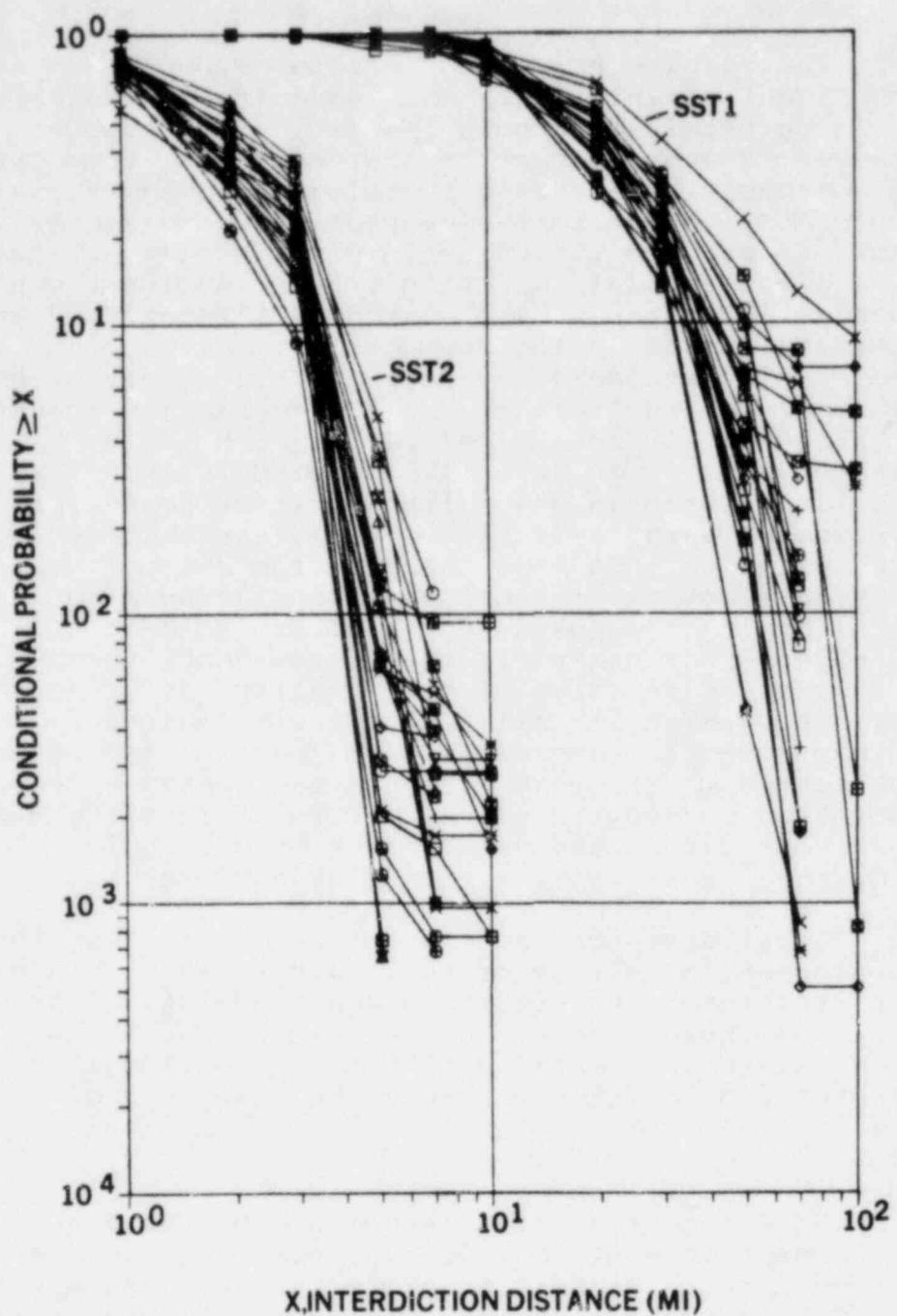


Figure 2.6-3. Conditional CCDFs of Interdiction Distance for 29 Meteorological Records Calculated Assuming SST1 and SST2 Releases from an 1120 MWe PWR.

The results presented thus far show the distances to which fatal or injury-causing doses of radiation could be received assuming no emergency response. However, given a severe reactor accident, some type of emergency response would be expected and therefore, acute doses close to the reactor could be reduced. As shown in Section 2.5, emergency protective actions can have a substantial impact on reactor accident consequences. Figure 2.6-4 compares SST1 fatality distance CCDFs calculated using New York City meteorology and four different emergency response scenarios: no emergency response, sheltering, and two evacuation scenarios (1 hr delay, 10 mph, within 25 mi; 5 hr delay, 10 mph, within 25 mi). In general, these CCDFs show that early fatality distances are quite sensitive to emergency response. Thus, effective implementation of emergency protective actions in areas near the reactor could result in substantial reductions in distances to which fatal or injury-causing doses of radiation could be received. For example, with no emergency response the 90th percentile value of the fatality radius for an SST1 release is  $\gtrsim$  8 miles, while with sheltering the 90th percentile distance is 4 miles and with expeditious evacuation (1 hr delay, 10 mph) the distance is further decreased to about 2 miles. CCDFs of fatality distance that were calculated using other meteorological records show the same sensitivity to emergency response.

Other distances that might be of interest for the development of siting criteria are those within which the EPA Protective Action Guides (PAGs) [32] for whole body and thyroid dose might be exceeded. A PAG is defined as the projected dose<sup>a</sup> to an individual in the general public which warrants the initiation of emergency

- a. The "projected dose" is defined by the EPA as the dose that would be received within a few days following the release if no protective actions are taken. PAGs range from 1 to 5 rem for whole body exposure and from 5 to 25 rem for projected dose to the thyroid. The lower value of these ranges should be used if there are no major local constraints limiting the ability to provide protection at that level. However, when determining the need for protective action, in no case should the higher value be exceeded.

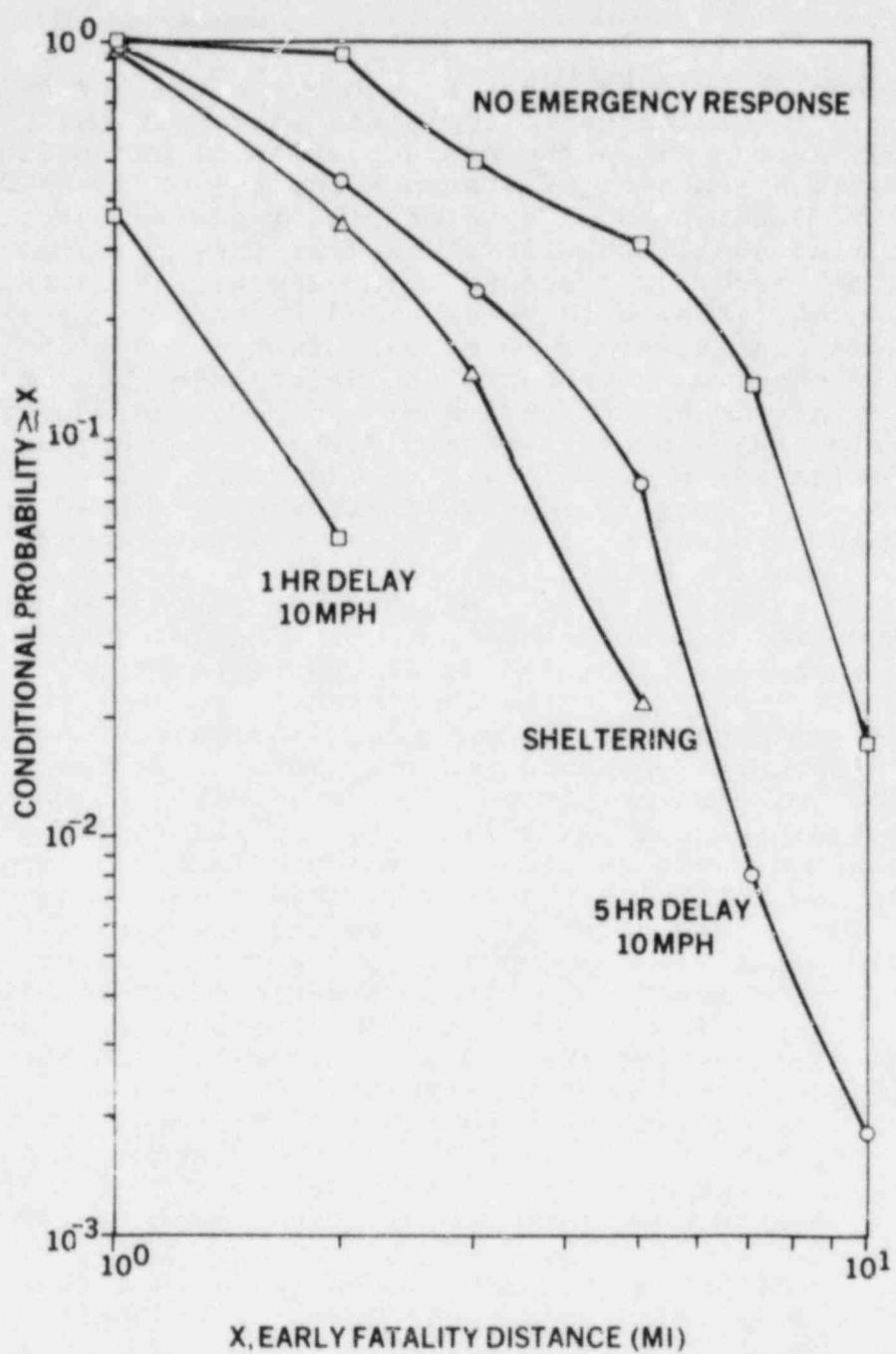
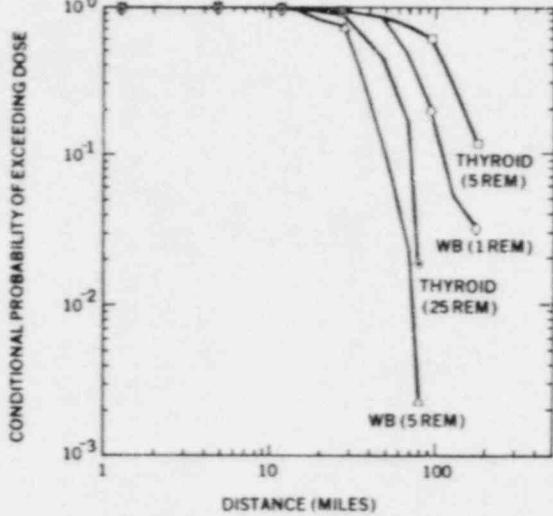


Figure 2.6-4. Sensitivity of SST1 Early Fatality Distances to Emergency Response.  
Assumptions: New York City meteorology, 1120 MWe PWR, and 25 Mile Response Radius.

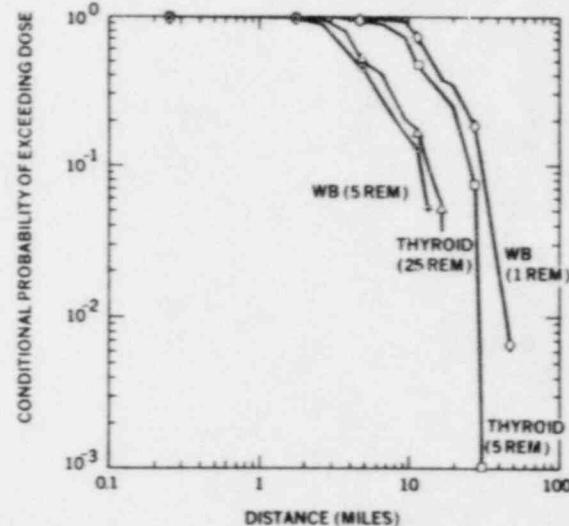
protective actions and, as such, is a trigger value to aid in decisions to implement these actions. Figure 2.6-5 shows the probabilities of exceeding the PAGs as a function of distance for the SST1, SST2, and SST3 releases. The probabilities were calculated assuming an 1120 MWe PWR, New York City meteorology, and no emergency response. In general, these results show that PAGs could be exceeded to very large distances (in excess of 50 miles) given an SST1 accident while they would probably not be exceeded beyond about 30 miles for an SST2 release. In addition, doses would nearly always exceed PAGs to distances of approximately 30 miles for the SST1 release and 2 miles for the SST2 release. Doses from an SST3 release are shown not to exceed PAGs beyond about 3 miles of the reactor.

The results discussed thus far in this section are summarized in Table 2.6-1. In the table consequence distances are presented for three releases (SST1, SST2, and SST3) and for three conditional probability levels: mean, 99th percentile, and peak (maximum calculated). The distances presented in the table summarize the large number of distance CCDFs calculated using the 29 meteorological records. The fatality and injury distances presented could be reduced by any effective emergency response action. In general, Table 2.6-1 suggests that: (1) for severe core melt accidents, early fatalities would generally not occur beyond about 15 miles, and in the worst case, would be confined to about 25 miles, while early injuries would probably be confined to downwind distances of about 50 miles; (2) for smaller core melt accidents (on the order of SST2 in severity), early fatalities would be confined to about 2 miles, and injuries and land interdiction to about 7 miles; and (3) for accidents on the order of SST3 in severity, PAGs would probably not be exceeded beyond a few miles.

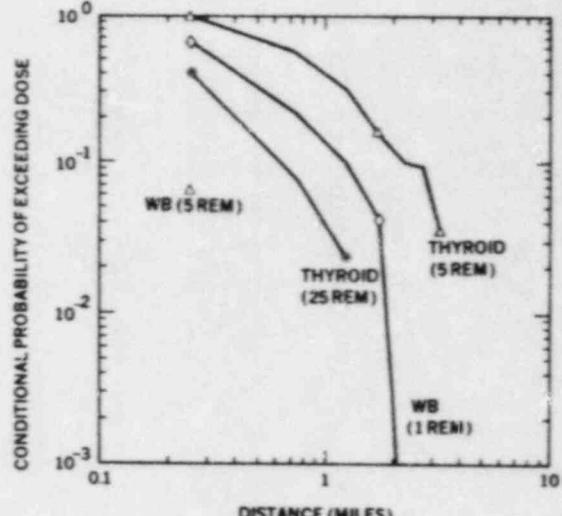
As discussed earlier, latent somatic effects could result from relatively small doses of radiation. Therefore, given a reactor accident, these consequences could occur at large downwind distances from the reactor. Figure 2.6-6 shows the cumulative fraction of latent cancer fatalities versus distance for the SST1, SST2, and SST3 releases. These curves were calculated assuming an 1120 MWe PWR, New York City meteorology, and a one mile per hour evacuation to ten miles after a five hour delay. In general, the results show that significant fractions of latent health effects could occur at large distances from the reactor. For the uniform



a) SST1



b) SST2



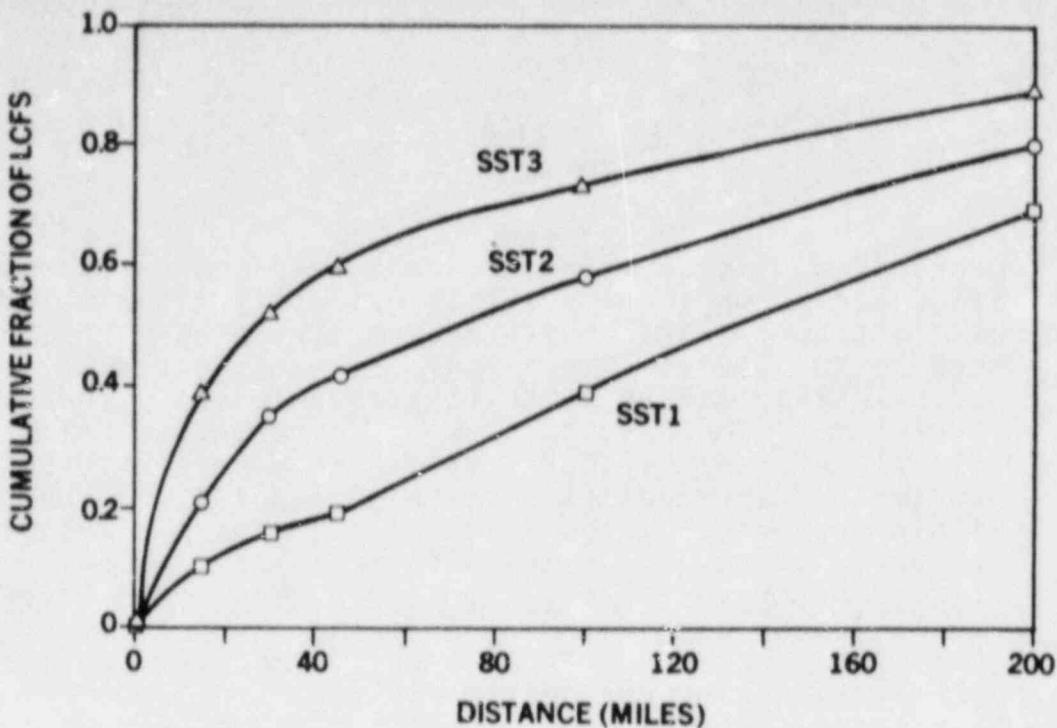
c) SST3

Figure 2.6-5. Conditional Probability of Exceeding PAGs Versus Distance for SST1, SST2, and SST3 Source Terms. Assumptions: 1120 MWe PWR, New York City meteorology, and no emergency response.

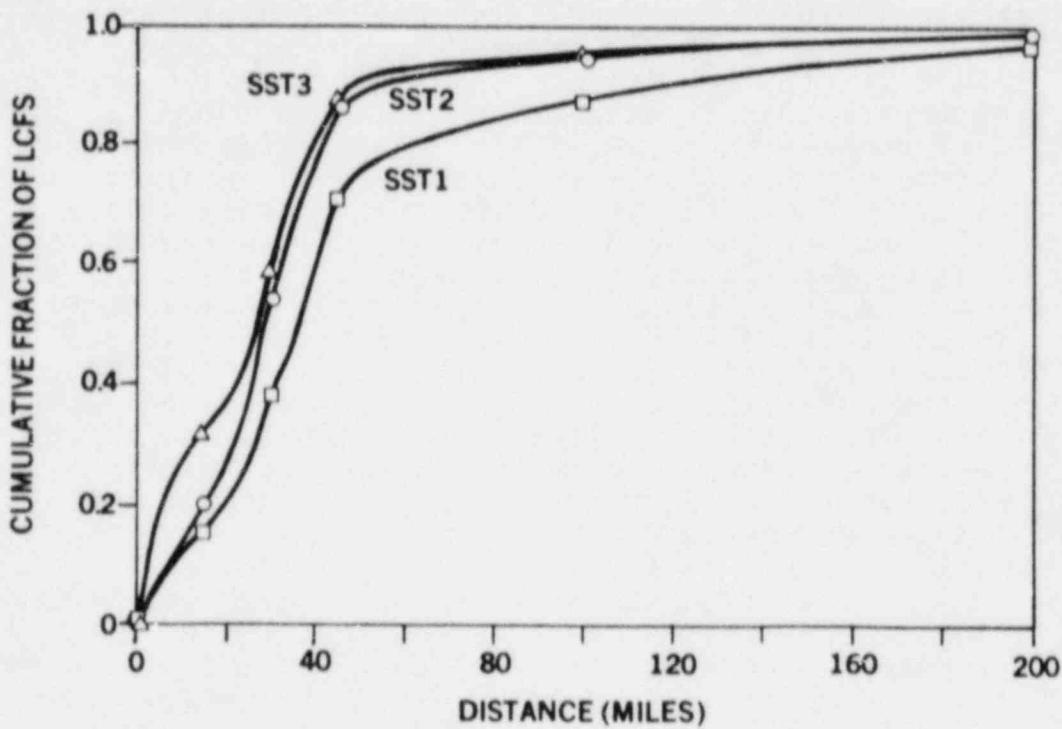
Table 2.6-1. Summary of Consequence Distances<sup>a</sup> (miles)

<u>Source Term</u>	<u>Consequence</u>	<u>Conditional Probability Level<sup>b</sup></u>		
		<u>Mean</u>	<u>99%</u>	<u>Calc Max</u>
SST1	Early Fatalities	<5	≤15	<25
	Early Injuries	~10	~30	≥50
	Land Interdiction	~20	>50	>50
	PACs <sup>c</sup>	≥50	>50	>50
SST2	Early Fatalities	~0.5	<2	≤2
	Early Injuries	<2	<5	~5
	Land Interdiction	<2	~7	~10
	PAGs <sup>c</sup>	≤20	~20	<50
SST3	PAGs <sup>c</sup>	≤0.5	<2	<3

- a. These distances are for a 1120 MWe PWR which is comparable in size to many of the most recently sited nuclear reactors.
- b. Mean distances are the average of the probability distributions of distance; 99% distances refer to those beyond which a consequence or dose is calculated to occur in 1 in 100 accidents; and the calculated maxima represent the largest distances calculated.
- c. A PAG is defined as the "projected" dose to an individual in the general public which warrants the initiation of emergency protective actions. PACs range from 1 to 5 rem for whole body exposure and from 5 to 25 rem for projected dose to the thyroid.



a) Uniform Population Distribution



b) Indian Point Population Distribution

Figure 2.6-6. Cumulative Fraction of Latent Cancer Fatalities as a Function of Distance from the Reactor a) for a Uniform Population Distribution and b) for the Indian Point Population Distribution.

Assumptions: 1120 MWe PWR, New York City meteorology, and a slow evacuation (5 hr delay, 1 mph, 10 mi response distance).

population distribution, the calculated cancer fatalities are shown to be somewhat uniformly distributed with distance. This uniform distribution results because the decrease in cancer risk with distance is approximately offset by the increase in the exposed population. The results shown for the Indian Point site illustrate the impact of a highly non-uniform population distribution. The high population densities within approximately 50 miles of the Indian Point site (relative to lower densities further away) cause a significantly larger fraction of the predicted cancer fatalities to occur within 50 miles of the reactor. Thus, the high non-uniformity of the exposed population distribution also causes the distribution of cancer fatalities to be non-uniform with distance.

Section 2.3.2 discussed recent reviews of accident phenomenology which indicate that the magnitudes of current source terms for severe reactor accidents may be significantly too large. To investigate the impact of source term reductions on distances to which consequences might occur, a series of calculations was performed for the SSTI release reduced by arbitrary factors of 2, 10, 20, and 100. Important assumptions for the calculations included New York City meteorology, an 1120 MWe PWR, and no emergency response. Table 2.6-2 summarizes the results and in general shows that reductions in severe accident source terms substantially reduce consequence distances. An order of magnitude reduction in the SSTI release reduced the peak fatal distance from about 20 miles to 5 miles while a two-order of magnitude reduction reduced the peak distance to 1 mile. Similar reductions are shown for early injury and land interdiction distances.

This section has examined the impact of meteorological conditions, accident severity, and emergency response on consequence distances. Four factors, that also could influence consequence distances, are discussed in other sections of this report. They are reactor size (i.e., size of radionuclide inventory, see Section 2.7.1), plume heat content (determines plume rise, see Section 2.7.2), dry deposition velocity (see Section 2.7.3) and interdiction criteria (see Section 2.7.5).

Table 2.6-2. Sensitivity of Fatal, Injury, and Interdiction Distances to Release Magnitude<sup>a</sup>

---

<u>Source Term</u>	<u>Fatal Distance (mi)</u>			<u>Injury Distance (mi)</u>			<u>Interdiction Distance (mi)</u>		
	<u>Mean</u>	<u>99%<sup>b</sup></u>	<u>Peak<sup>b</sup></u>	<u>Mean</u>	<u>99%<sup>b</sup></u>	<u>Peak<sup>b</sup></u>	<u>Mean</u>	<u>99%<sup>b</sup></u>	<u>Peak<sup>b</sup></u>
SST1	3.9	12	18	11	35	50	19	55	85
1/2 SST1 <sup>c</sup>	2.5	10	18	7.0	20	25	14	45	50
1/10 SST1 <sup>c</sup>	0.9	2.2	5.0	2.8	10	18	5.5	18	25
1/20 SST1 <sup>c</sup>	0.5	2.0	2.0	1.9	7.0	10	3.6	12	18
1/100 SST1 <sup>c</sup>	0	1.0	1.0	0.9	4.0	5.0	1.1	10	10

---

a. Assumptions: New York City meteorology, 1120 MWe PWR, and no emergency response.

b. The 99 percent distances refer are the distances beyond which a consequence is calculated to occur in only 1 in 100 accidents. The peak result is that obtained for the most unfavorable weather sequence sampled.

c. Release fractions reduced for all isotopes except noble gases.

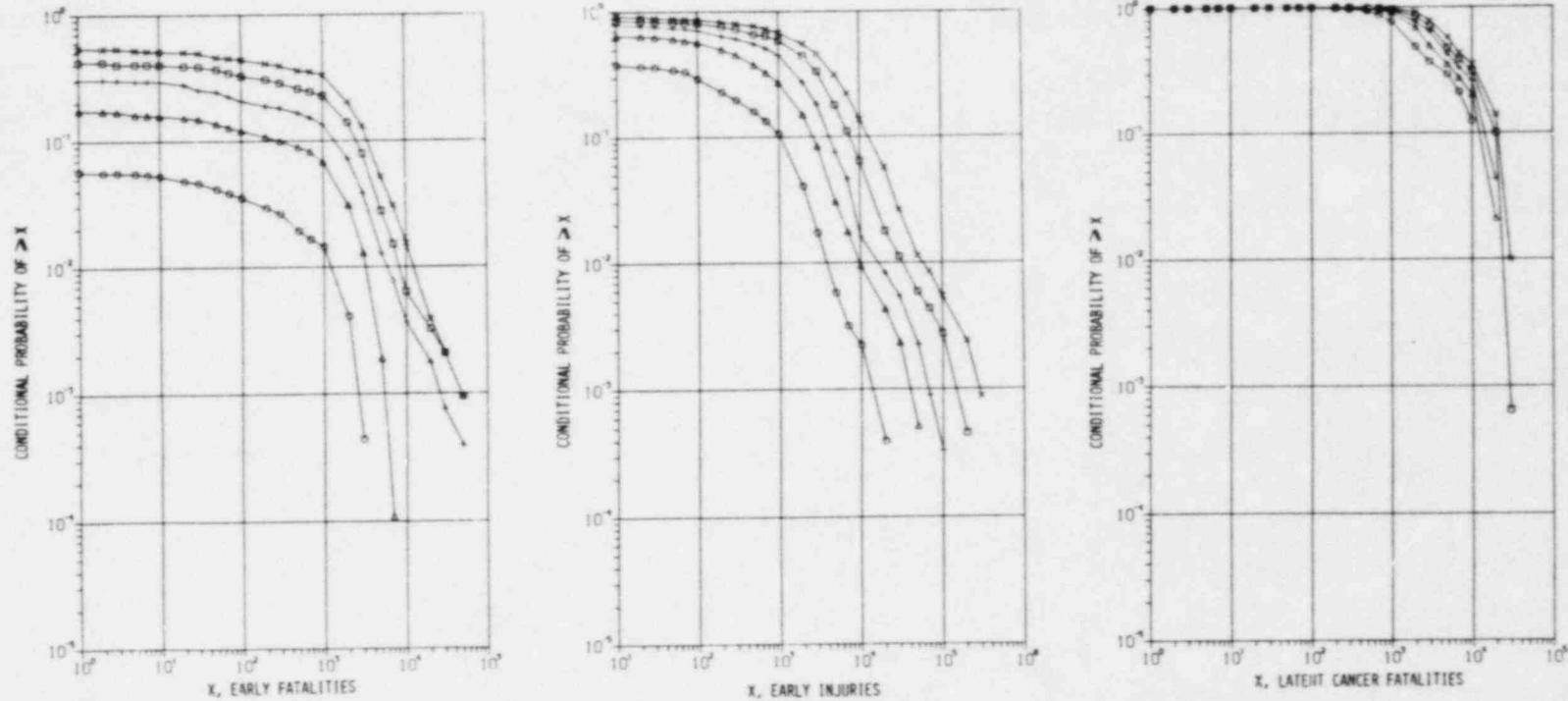
## 2.7 Other Sensitivity Calculations

### 2.7.1 Reactor Size

All of the calculations presented in previous sections of this report assume an 1120 MWe reactor. This reactor size was selected because many reactors currently operating and most under construction are about this size. Because consequences depend strongly on the amount of radioactivity released (see Section 2.3, Accident Source Terms), which in turn is dependent on reactor size, the sensitivity of consequences to reactor size was examined. Calculations were performed for nine reactor sizes ranging from 11.2 to 1500 MWe. All calculations assumed a 1120 MWe core radionuclide inventory scaled according to reactor size, an SST1 release, New York City meteorology, and the Indian Point population distribution and wind rose. The linear scaling procedure used is described in Appendix B, Core Radionuclide Inventories, which also discusses inventory changes due to annual operating cycle and differences between PWR and BWR inventories.

Figures 2.7.1-1 and 2.7.1-2 present conditional CCDFs of early fatalities, early injuries, latent cancer fatalities, interdiction distance, and interdicted land area for five of the nine reactor sizes examined, assuming Summary Evacuation. Table 2.7.1-1 presents the mean and 99th percentile values of these distributions. The effects of emergency response and reactor size on mean early fatalities are presented in Table 2.7.1-2. Finally, Figure 2.7.1-3 presents plots of the mean values presented in each table versus reactor size.

Several conclusions can be drawn from these results. First, Figure 2.7.1-3 shows that mean values of all five consequences increase roughly linearly with reactor size. The rates of increase are largest for early fatalities and smallest for interdiction distance. Table 2.7.1-1 shows that mean values increase more rapidly than 99th percentile values. The mean early fatality results presented in Table 2.7.1-2 clearly display the significant impact of emergency response, seen previously (see Section 2.5). For an 1120 MWe reactor, No Evacuation yields a mean result of almost 3600 early fatalities, while Best Evacuation (1 hr delay, 10 mph, 10 mi response region) decreases this number to less than 300. Figure 2.7.1-3a shows that for an emergency response of a given effectiveness, there is a reactor size (x-axis



**Figure 2.7.1-1.** Effect of Reactor Size upon a) Early Fatalities, b) Early Injuries, and c) Latent Cancer Fatalities, Conditional on an SSTI Release.

Legend

- - 1500 MWe reactor
- ◻ - 1120 MWe reactor
- + - 750 MWe reactor
- △ - 500 MWe reactor
- - 250 MWe reactor

Assumptions: 1120 MWe core radionuclide inventory scaled to reactor size, SSTI release, New York City Meteorology, Indian Point wind rose and population, Summary Evacuation.

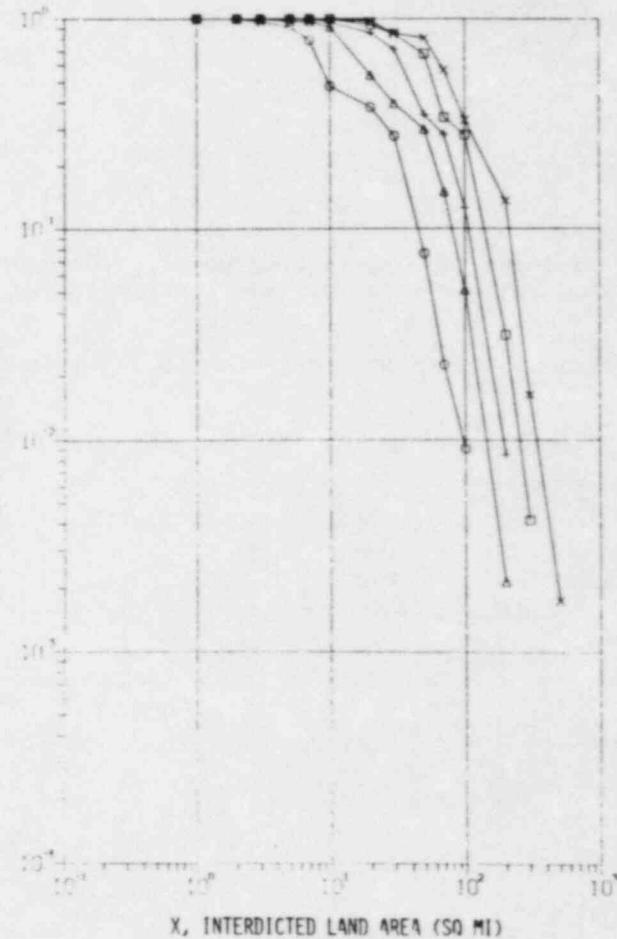
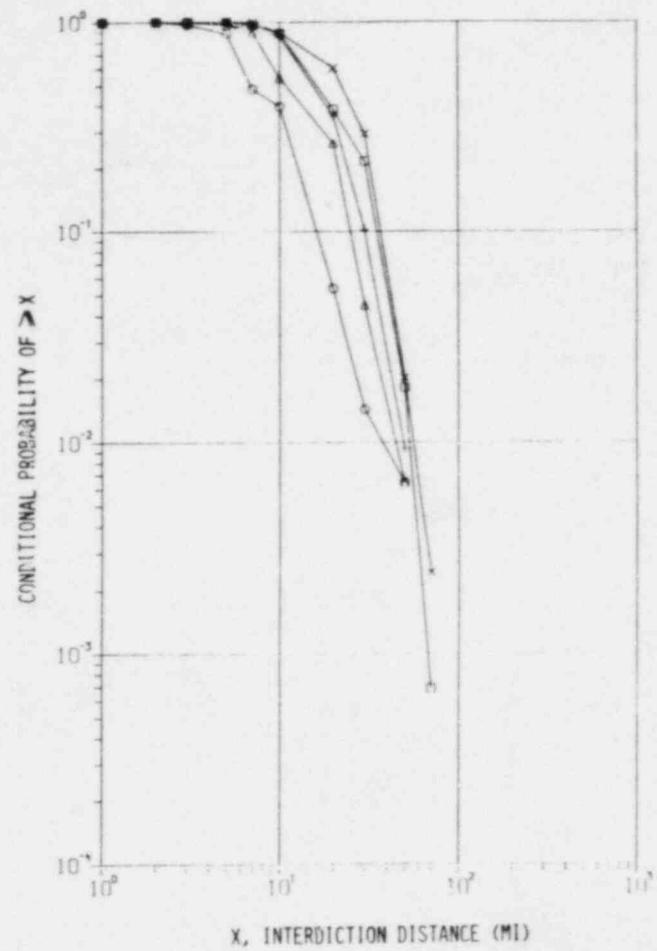


Figure 2.7.1-2. Effect of Reactor size upon a) Interdiction Distance (mi) and b) Interdicted Land Area (sq mi), Conditional on an SST1 Release.

#### Legend

- $\times$  - 1500 MWe reactor
- $\square$  - 1120 MWe reactor
- $+$  - 750 MWe reactor
- $\triangle$  - 500 MWe reactor
- $\circ$  - 250 MWe reactor

Assumptions: 1120 MWe core radionuclide inventory scaled to reactor size, SST1 release, New York City meteorology, Indian Point wind rose and population, Summary Evacuation.

Table 2.7.1-1. Dependence of Consequences Upon Reactor Size, Conditional  
on an SST1 Release<sup>a</sup>

Reactor Size (MWe)	Early Fatalities		Early Injuries		Latent Cancer Fatalities		Interdiction Distance (mi)		Interdicted Land Area (sq mi)	
	Mean	99th	Mean	99th	Mean	99th	Mean	99th	Mean	99th
250	34	1,200	323	3,800	3970	10,000	9.7	38	20.8	97
500	172	3,200	1020	9,700	5560	20,000	13.1	45	37.2	120
750	455	5,900	1880	16,000	6710	20,000	16.0	49	53.7	190
1120	831	8,200	3640	33,000	8110	24,000	19.3	54	75.8	250
1500	1250	12,000	6340	57,000	9600	30,000	22.8	56	106	340

a. Assumptions: 1120 MWe core radionuclide inventory scaled according to reactor size, SST1 release, New York City meteorology, Indian Point population and wind rose, Summary Evacuation.

Table 2.7.1-2 Dependence of Mean Early Fatalities Upon Reactor Size and Evacuation Scenario, Conditional on an SST1 Release<sup>a</sup>

Reactor Size (MWe)	Evacuation Scenario		
	Best Evacuation <sup>b</sup>	Summary Evacuation	No Evacuation
11.2 <sup>c</sup>	0	0.3	1
56 <sup>c</sup>	0	2	34
112 <sup>c</sup>	0	9	147
250	0.01	34	551
500	6	172	1490
560 <sup>c</sup>	17	224	1700
750	102	455	2380
1120	176	831	3580
1500	287	1250	4880

- a. 1120 MWe core radionuclide inventory scaled according to reactor size, SST1 release, New York City meteorology, Indian Point population and wind rose.
- b. 1 hour delay, 10 mph, 10 mi response region (see Section 2.5).
- c. Noble gas release fractions not scaled; this has no significant impact on early fatalities (see Section 2.3, Accident Source Terms).

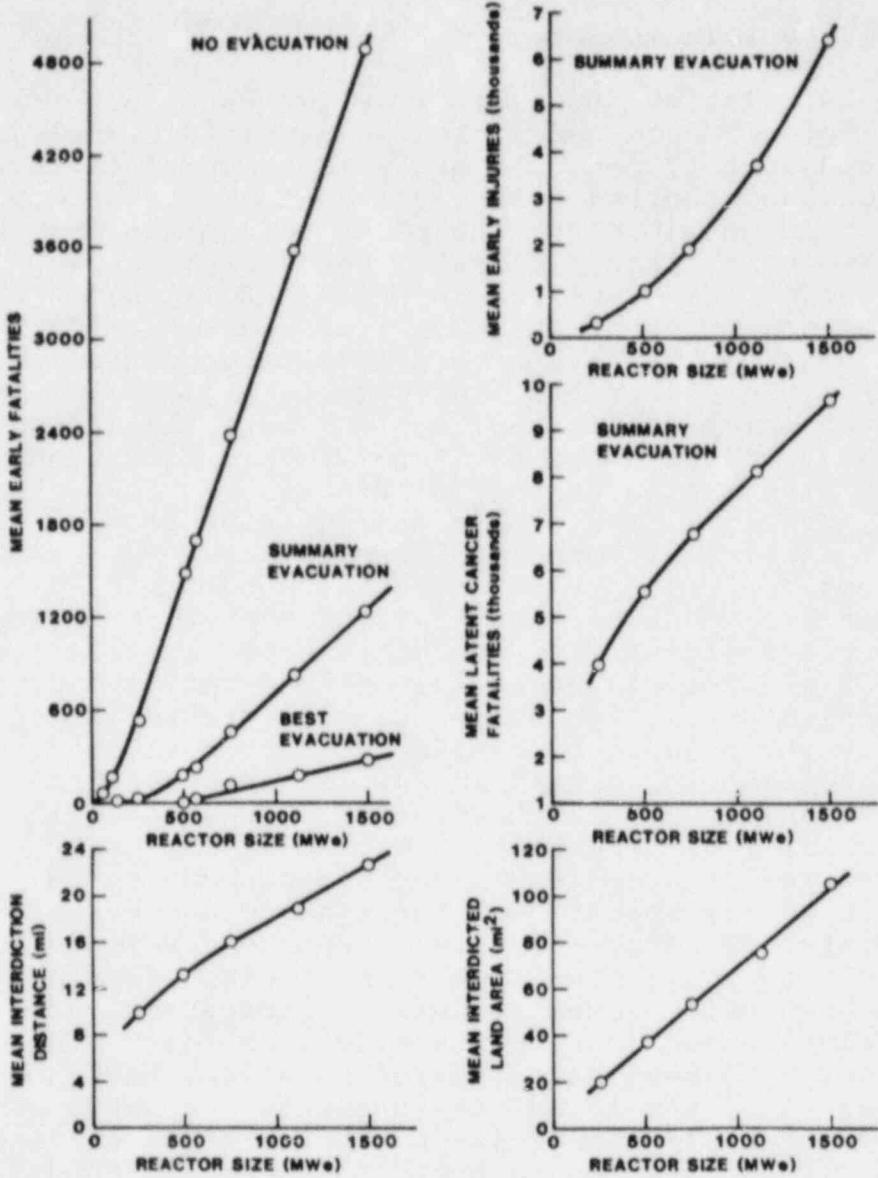


Figure 2.7.1-3. Plots of Mean Values of a) Early Fatalities, b) Early injuries, c) Latent Cancer Fatalities, d) Interdiction Distance (mi), and e) Interdicted Land Area (sq mi) vs Reactor Size, Conditional on an SSTI Release.

Assumptions: 1120 Mwe core radionuclide inventory scaled to reactor size, SSTI release, New York City meteorology, Indian Point population and wind rose.

intercept) for which on the average (mean result) few early fatalities would be expected. For Best Evacuation that size is ~500 MWe; for Summary Evacuation, ~100 MWe; and for no evacuation, ~10 MWe.

### 2.7.2 Energy Release Rate

The calculations considered so far have been for ground-level releases containing no sensible heat, i.e., nonbuoyant plumes. In an accident where there is a large uncontrolled release directly to the atmosphere, it is possible for the plume to contain a sizable amount of sensible heat. For example, the release categories described in WASH-1400 [1] had energy release rates of up to several hundred million BTUs per hour.<sup>a</sup> The rate of energy release determines the final plume height and, therefore, the downwind distance at which the plume first contacts the ground (touchdown). Since under the same weather conditions a buoyant plume would be more dilute at touchdown than a nonbuoyant plume, a significant reduction in the number of early health effects is possible. However, since plume depletion by dry deposition occurs only after touchdown, buoyant plumes might therefore produce ground concentrations high enough to produce early effects at greater distances than nonbuoyant plumes. Furthermore, for highly buoyant plumes, precipitation-washout is the primary mechanism by which radioactive material reaches the ground in sufficient concentrations to cause early health effects. Thus, for a buoyant release the probability of having any early fatalities and injuries is strongly dependent on the occurrence of precipitation. The final plume height is calculated in CRAC2 using the formulae developed by Briggs [33] for emissions from smokestacks. Considerable differences could exist between smokestack plumes and plumes released in a reactor accident [34]. These differences have been investigated by Russo, Wayland, and Ritchie [35] who found that predicted consequences were only marginally sensitive to the moisture content of the plume and atmosphere but, under certain conditions, consequences could be quite sensitive to radioactive heating and initial plume momentum.

For the present study, the sensitivity of predicted consequences to energy release rate was investigated

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a. In WASH-1400, an energy release rate of  $170 \times 10^6$  BTU/hr was assumed for a PWR-2 accident.

by performing calculations for an SST1 release with three arbitrary energy release rates: 17, 170, and 430 million BTU/hour. New York City meteorology and a uniform population density of 50 people per square mile beyond 1 mile were assumed. Table 2.7.2-1 compares selected results for these energy release rates with a cold (no sensible heat) SST1 release (the base case, see Section 2.2.4).

Table 2.7.2-1. Sensitivity of Estimated Consequences to Energy Release Rate<sup>a</sup>

<u>Release</u>	SST1	SST1	SST1	SST1
<u>Energy Release Rate (BTU/hr)</u>	0	$17 \times 10^6$	$170 \times 10^6$	$430 \times 10^6$
<hr/>				
Mean Early Fatalities				
Summary Evacuation	22	12	9	10
No Evacuation	140	140	47	47
<hr/>				
Mean Early Injuries				
Summary Evacuation	140	180	110	85
No Evacuation	350	390	270	150
<hr/>				
Mean Latent Cancer Fatalities	730	790	830	860
<hr/>				
Maximum Calculated Fatal Distance (mi)	17.5	17.5	25	25
<hr/>				
Maximum Calculated Injury Distance (mi)	50	50	50	60
<hr/>				
Maximum Calculated Land Interdiction Distance (mi)	85	85	85	85
<hr/>				

a. Assumptions: New York City meteorology, uniform population of 50 people per square mile beyond 1 mile.

The results for the low-energy release ( $17 \times 10^6$  BTU/hr) differ only slightly from those for the cold release, because this release rate is not large enough to cause substantial differences in the plume touchdown point. The two high-energy release rates result in consequences markedly different from the cold release. Because the occurrence of precipitation is necessary to cause significant numbers of early health effects for hot releases, the mean number of early effects is lower for the high-energy releases.

At very large distances, the amount of initial plume-rise does not significantly affect the transport and deposition of radioactive material. Consequently, latent cancer fatalities, which occur to great distances (see Section 2.6), are not significantly affected by plume buoyancy. The maximum observed fatal distance is 8 miles farther for the high-energy releases, although the maximum calculated injury distance is only slightly increased and interdicted land distance is unaffected. Neither land interdiction nor injury distances are very sensitive to energy release rate because these consequences also occur to distances where initial plume rise is generally not important.

Figure 2.7.2-1 plots the conditional individual risk of early fatality versus distance for the four energy release rates, assuming a uniform wind rose. Within 10 miles, the hot releases have lower risks than the cold releases. However, for low probability events (i.e., precipitation), the hot releases could result in fatalities out to 25 miles. The non-monotonicity in the risk at about 8 miles for the two hot releases ( $170 \times 10^6$  and  $430 \times 10^6$  BTU/hr) is believed to be an artifact of the weather-sequence sampling procedure used (see Section 2.4.1).

In summary, for an SST1 release the estimated numbers of early fatalities and injuries and the distance to which early fatalities occur are both quite sensitive to the energy release rate. However, consequences which can occur to great distances, such as latent cancer fatalities, are not sensitive to energy release rate. The maximum distances, to which early injuries may occur or land may be interdicted, are also not sensitive to energy release rate. A cautionary note: these conclusions may not hold for source terms significantly smaller than SST1.

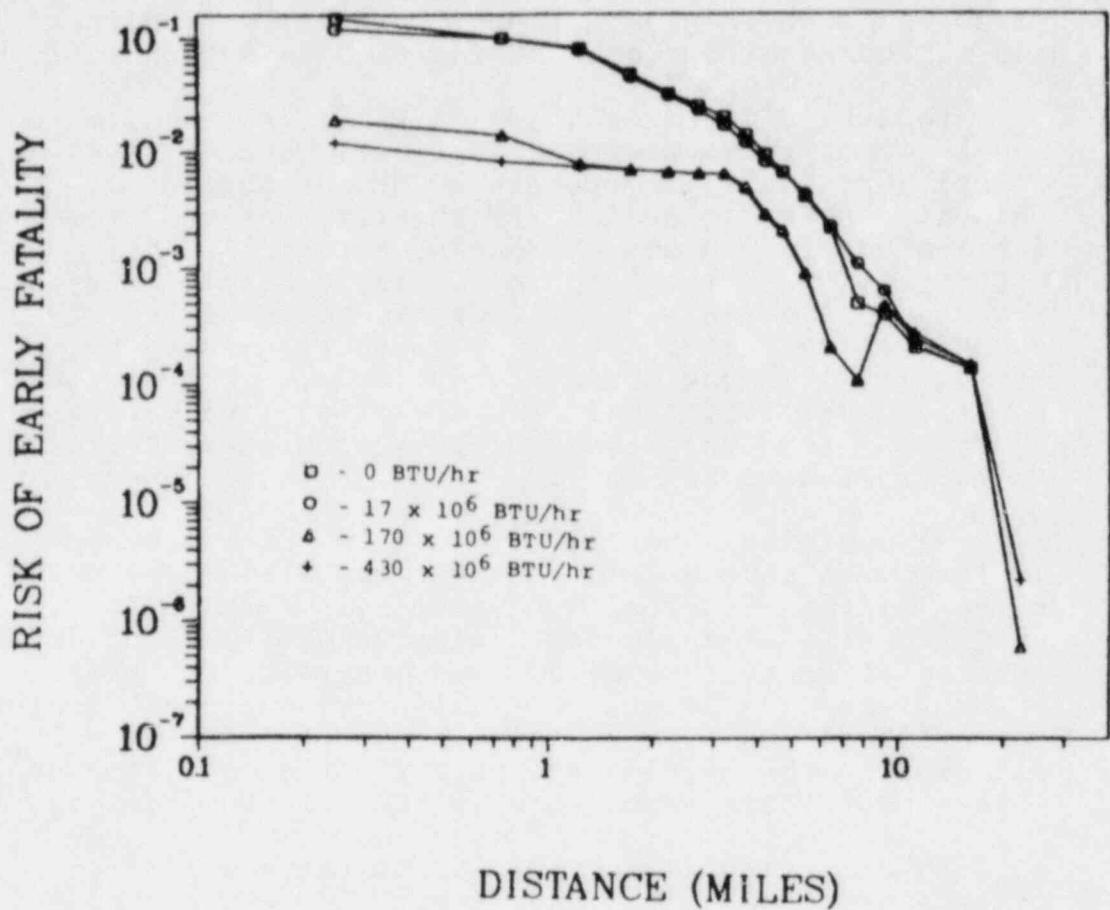


Figure 2.7.2-1. Individual Risk of Early Fatality Versus Distance for 4 Energy Release Rates, Conditional on an SST1 Release.  
Assumptions: SST1 release, New York City meteorology, uniform wind rose, no emergency response.

### 2.7.3 Dry Deposition Velocity

The deposition of radioactive material on the ground is the first step in many of the pathways by which radioactive material can reach people. Dry deposition of airborne material onto a surface is a complex process which includes a number of different phenomena such as gravitational settling, turbulent and molecular diffusion, and inertial impaction [36].

Hosker [37] and Kaul [38] have reviewed current models of dry removal processes. All current dry-deposition models incorporate a "dry-deposition velocity" which is defined as the ratio of the time-integrated air concentration of a material to the concentration of the material on the ground. A large number of parameters can affect the value of the deposition velocity. About 80 have been listed by Sehmel [39]. Among these are surface roughness, relative humidity, chemical composition, and particle diameter. Dry deposition velocity is highly sensitive to particle diameter [39].

Radioactive material released to the atmosphere is likely to have a range of particle diameters, each with a different deposition velocity. Despite this, in CRAC2 only a single deposition velocity may be input for each element considered, and generally the same value (1 cm/sec) is used for all elements except noble gases (the deposition velocity of noble gases is zero). All CRAC2 calculations presented in other sections of this report treat deposition velocity in this manner.

As discussed in Section 2.2.3, there are large uncertainties about the characteristics of the radioactive aerosol released from containment. Because predicted ground concentrations can be very sensitive to deposition velocity, a sensitivity analysis was performed to assess the impact of dry deposition velocity on predicted consequences. The analysis was somewhat simplistic in that only a single deposition velocity was used. Thus, no attempt was made to account for a range of particle sizes by use of a distribution of deposition velocities. Also neglected were effects of chemical composition and the possibility that different elements may be associated with particles of different sizes. Gravitational settling of particles, which can be treated by "tilted plume" models [40] was also ignored (gravitational settling would be the dominant

contributor to dry removal for particle diameters greater than about 5 microns).

Calculations were performed for an SST1 release with five deposition velocities: 0.1, 0.3, 1.0, 3.0, and 10.0 cm/sec.<sup>a</sup> These values are believed to span the range of possible deposition velocities. Only non-buoyant releases were considered. For buoyant releases, early consequences are dominated by the occurrence of precipitation; therefore, the variation of consequences with dry deposition velocity could be substantially smaller for buoyant releases (see Section 2.7.2). Other assumptions included Summary Evacuation, an 1120 MWe reactor, the Indian Point population distribution and wind rose, and New York City meteorology. Different population distributions and emergency response assumptions could impact the observed variation of early consequences with deposition velocity (see Sections 2.4 and 2.5).

Figure 2.7.3-1 presents the early fatality CCDFs for the set of deposition velocities examined. Except for the low-probability, high-consequence events, there are only very minor differences. Mean numbers of early fatalities vary by less than a factor of 1.5. Deposition velocities of 0.1, 0.3, and 1.0 cm/sec yield the highest consequence events (over 50,000 fatalities) from weather sequences with precipitation beginning between 10 and 20 miles from the reactor. With either a 3 or 10 cm/sec deposition velocity, the particulate matter in the plume is sufficiently depleted before this distance range is reached and, thus, rain does not produce a ground concentration in this interval high enough to cause significant numbers of early fatalities.

Figure 2.7.3-2 shows the conditional individual risk of early fatality versus distance within 10 miles of the reactor. Larger values of deposition velocity result in slightly greater individual risk within 2 miles of the reactor but a much reduced risk farther out. Table 2.7.3-1 lists the means, 90th and 99th percentiles, and maxima of the CCDFs of early fatality distance, early injury distance, and interdicted land

a. In all calculations a single deposition velocity was used for all elements except noble gases. The deposition velocity of the noble gases was assumed to be zero.

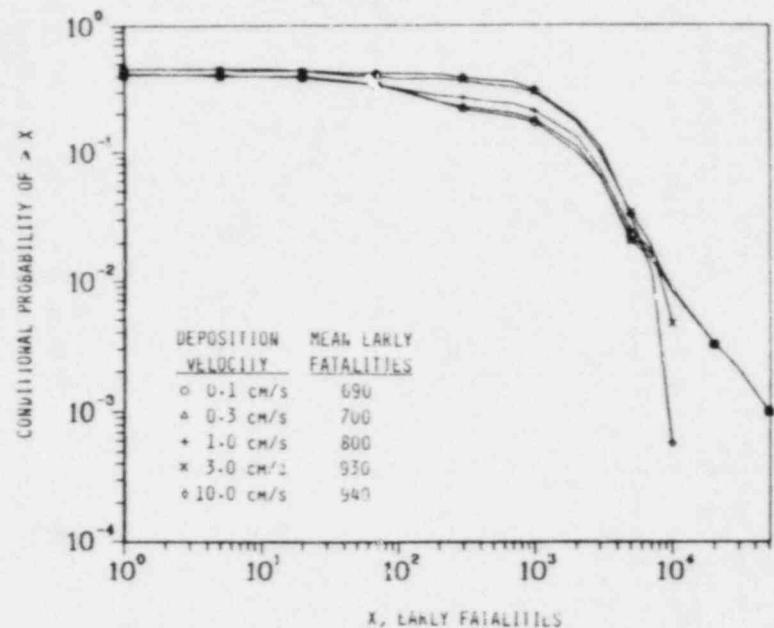


Figure 2.7.3-1. Early Fatality CCDFs for Five Different Deposition Velocities (for particulate matter only), Conditional on an SST1 Release.

Assumptions: 1120 MWe reactor, SST1 release, Indian Point wind rose and population, New York City meteorology, Summary Evacuation.

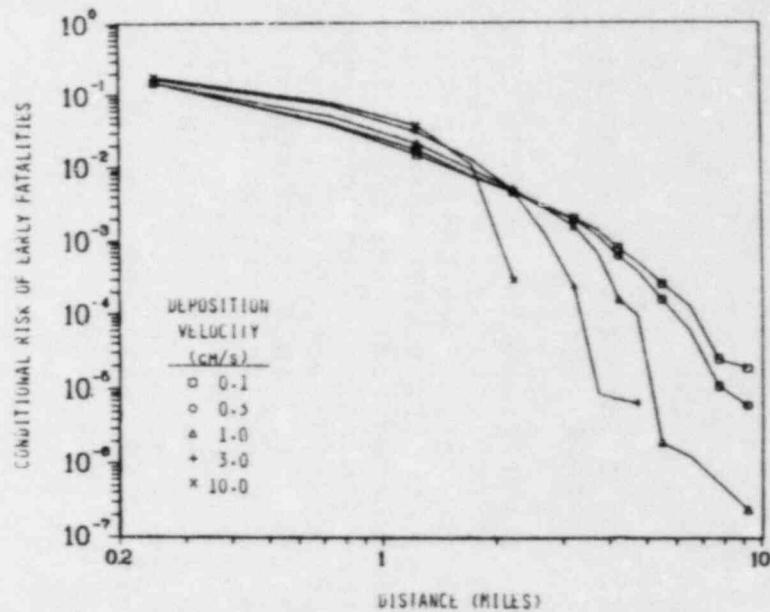


Figure 2.7.3-2. Individual Risk of Early Fatality vs Distance for 5 Deposition Velocities, Conditional on an SST1 Release

Table 2.7.3-1 Sensitivity of the Distances (miles) to which Consequences Occur for Various Deposition Velocities.

Dry- Deposition Velocity (cm/sec)	Early Fatality Distance				Early Injury Distance				Land Interdiction Distance			
	Mean	90%	99%	Maximum Calcu- lated	Mean	90%	99%	Maximum Calcu- lated	Mean	90%	99%	Maximum Calcu- lated
0.1	2.1	4	15	25	7.2	15	55	65	11	30	60	100
0.3	1.9	4	15	25	7.1	20	40	50	16	40	65	85
1.0	1.7	4	12	18	8.3	25	35	50	19	40	60	85
3.0	1.6	3	4	18	6.6	12	23	25	20	25	40	45
10	1.4	3	3	3	3.5	6	15	18	13	22	23	25

Assumptions: 1120 MWe reactor, SST1 release, New York City meteorology, Summary Evacuation within 10 miles.

distance (see Section 2.6). The mean distances for each consequence are only marginally sensitive to deposition velocity. However, the tail of the distributions (99th percentile and maximum calculated) are very sensitive to deposition velocity. As the deposition velocity increases, there is a large reduction in the 99th percentile and maximum calculated distances. Again, the tails of each distribution result from sequences with precipitation beginning some distance from the reactor. Deposition velocities above about 3 cm/sec deplete the plume closer to the reactor, and thus the distance to which precipitation can produce significant ground concentrations is much reduced.

Despite the narrow scope of this sensitivity analysis (only the deposition velocity has been studied rather than trying to account for the more realistic condition of a distribution of deposition velocities), the following conclusions can be drawn:

- o For a single deposition velocity applicable to all particulate matter, the maximum distance to which land is interdicted and early fatalities and injuries occur is very sensitive to deposition velocity. These maximum distances occur for low-probability, worst-case weather conditions.
- o For the population distribution and emergency response scenario assumed (Summary Evacuation), the mean number of early fatalities is only moderately sensitive to deposition velocity and thus may be largely insensitive to the particle-size distribution of the released material.

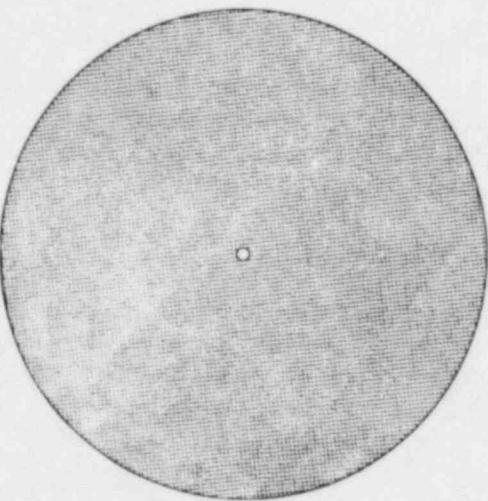
#### 2.7.4 Population Distribution

Results presented in Section 2.4, Site Meteorology and Population, showed that early fatalities and early injuries are strongly sensitive to the characteristics of the surrounding population distribution. Three sets of calculations were performed to better define the sensitivity of early fatalities and injuries to the following features of population distributions: (1) radial and angular variations in population density, (2) the size and distance of population centers, and (3) exclusion zone size.

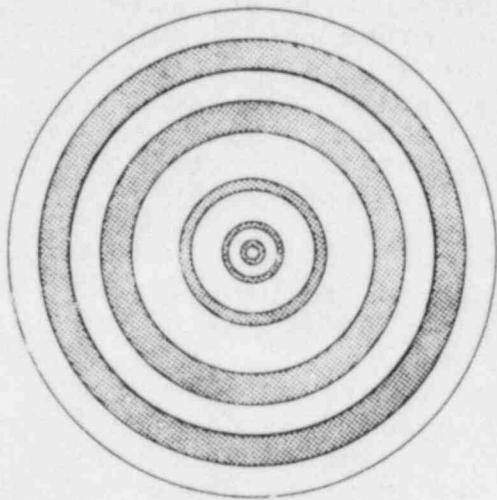
Radial and Angular Variations. Radial and angular variations in population density were examined by constructing a hypothetical reference population distribution and then calculating consequences for that distribution and eight transformations of that distribution. Beyond 20 miles all of the distributions were identical. Each had uniform populations of 750 people per square mile from 20 to 30 miles, 2500 from 30 to 50 miles, 500 from 50 to 100 miles, and 300 from 100 to 500 miles. None of the distributions had any people within 0.5 miles of the reactor (0.5 mile Exclusion Zone). All nine distributions met the following criterion: within 5, 10, 15, 20, and 30 miles of the reactor, the average population density was either zero (the distribution is empty to that radial distance) or 750 people per square mile (if there are any people within a given radial distance, then on average within that distance there are 750 people per sq mi). In addition, all nine distributions had 939,000 people within 20 miles of the reactor, but each had a different distribution of those people, as is schematically depicted in Figure 2.7.4-1.

Figure 2.7.4-1 indicates that the reference distribution (Distribution 1) was uniform from 0.5 to 20 miles. It had 530 people per square mile from 0.5 to 2 miles and 750 people per square mile from 2 to 20 miles. Distribution 2 was constructed from the reference distribution by moving the population within 20 miles forward into 5 high density rings. Distribution 3 moved the population within 20 miles entirely into a single 22.5° sector. Distributions 4 through 8 moved all of the population within 2, 5, 10, 15, or 20 miles, respectively, into a single 22.5° sector toward the back of the vacated region. Distribution 9 was constructed by scaling the actual population distribution around a New England reactor site, so that the resulting distribution had 530 people per square mile from 0.5 to 2 miles and 750 people per square mile in each of four distance intervals: 2-5, 5-10, 10-15, and 15-20 miles.

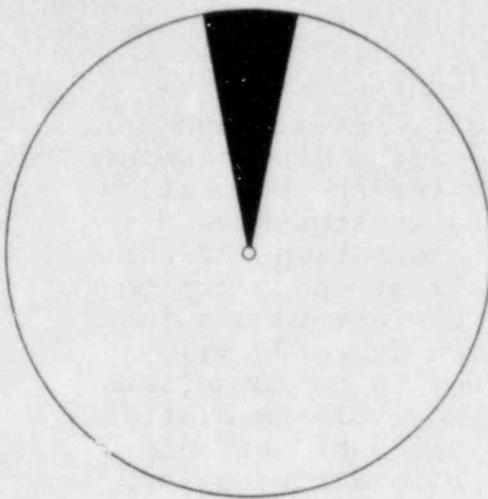
The transformations used to generate Distributions 4 through 8 in effect created population centers by vacating 15 of the 16 sectors of the reference distribution out to 2, 5, 10, 15, or 20 miles, respectively. The population centers thereby created had the following sizes and distances from the reactor:



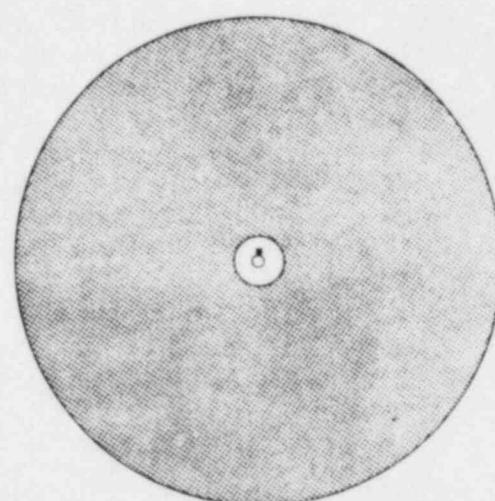
Distribution 1



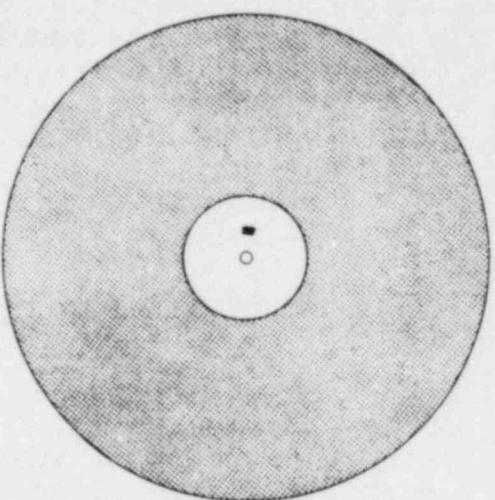
Distribution 2



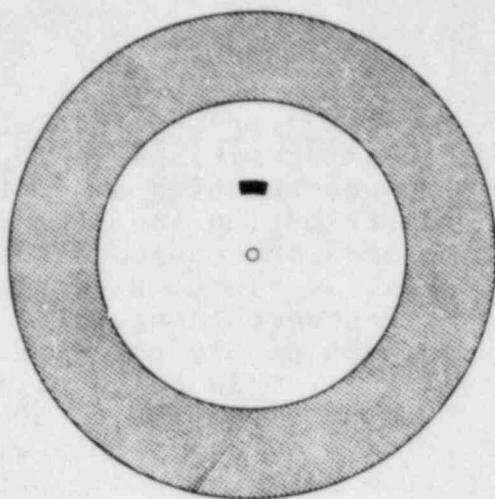
Distribution 3



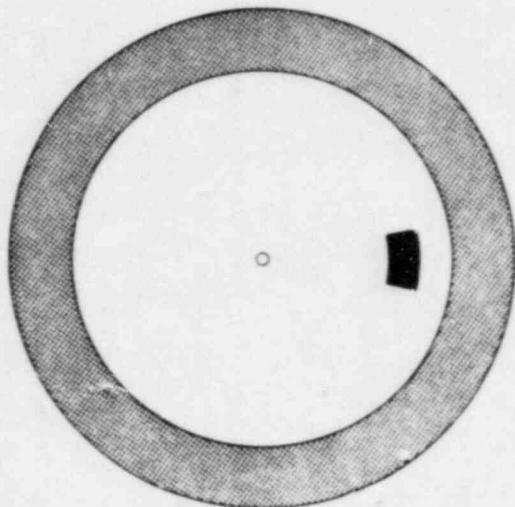
Distribution 4



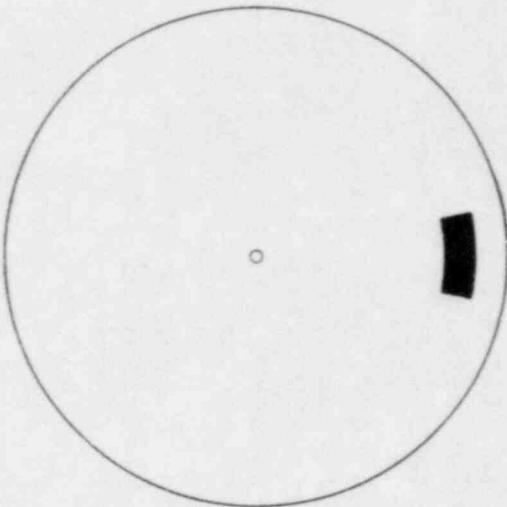
Distribution 5



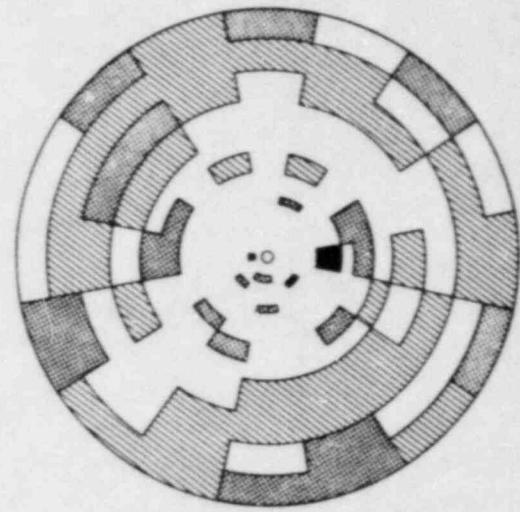
Distribution 6



Distribution 7



Distribution 8



Distribution 9

Figure 2.7.4-1. Schematic Representations of the Nine Hypothetical Population Distributions Used to Examine the Impact on Consequences of Radial and Angular Variations in Population Density.

- 1) Distribution 1 (Reference Distribution): uniform to 20 mi.
- 2) Distribution 2: 4 high density population rings.
- 3) Distribution 3: all population in 1 sector.
- 4) Distribution 4: city at 1.0 mi, uniform beyond 2 mi.
- 5) Distribution 5: city at 3.0 mi, uniform beyond 5 mi.
- 6) Distribution 6: city at 6.8 mi, uniform beyond 10 mi.
- 7) Distribution 7: city at 12.5 mi, uniform beyond 15 mi.
- 8) Distribution 8: city at 16.3 mi, uniform beyond 20 mi.
- 9) Distribution 9: real distribution scaled to match the densities of Distribution 1.

Distribution	City Size	City Distance (mi)
4	6,300	1
5	55,800	3
6	232,000	6.75
7	527,000	12.5
8	940,000	16.25

For each of the nine population distributions, early fatality and early injury CCDFs were calculated assuming an SST1 release from an 1120 MWe reactor, Summary Evacuation, New York City meteorology, and a uniform wind rose. The early fatality CCDFs are presented in Figures 2.7.4-2 through 2.7.4-5. For each early fatality and early injury CCDF, mean (expected) and 99th percentile (consequence magnitude equalled or exceeded following 1 out of every 100 releases) values and the probability of having at least one early fatality or injury are presented in Table 2.7.4-1.

Figure 2.7.4-2 compares the second population distribution to the Reference Distribution. Moving population forward into five high-density rings (densities of 2700, 7000, 5100, 1700, 1600, respectively) increases the number of early fatalities calculated at each probability level (the reference CCDF is shifted toward higher consequences).

Figure 2.7.4-3 compares the third population distribution to the Reference Distribution. Moving all of the population into 1 sector (vacating 15 sectors out to 20 miles) reduces the likelihood of having any early fatalities (the CCDF shifts downward) but increases the number observed, whenever fatalities do occur (the CCDF shifts to the right).

The CCDF shifts downward because, with 15 sectors vacant to 20 miles, many plumes do not intersect any population before plume concentrations fall below fatality dose thresholds. Therefore, the probability of having at least 1 early fatality is substantially decreased. If plumes were always exactly 1 sector wide, then the probability of having at least 1 early fatality would decrease by a factor of exactly 16. Because plume meander frequently causes plumes to be much wider

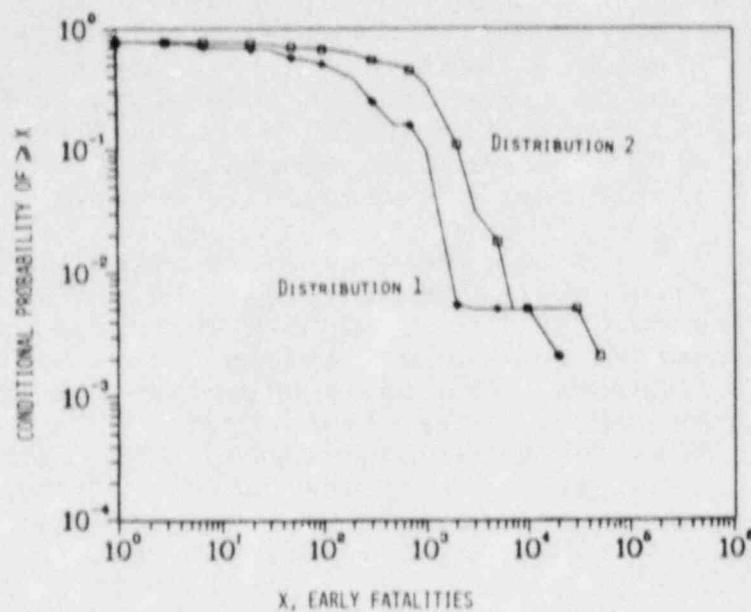


Figure 2.7.4-2. Comparison of the Early Fatality CCDF for Population Distribution 2 (4 high density rings) to that of the Reference Distribution.<sup>a</sup>

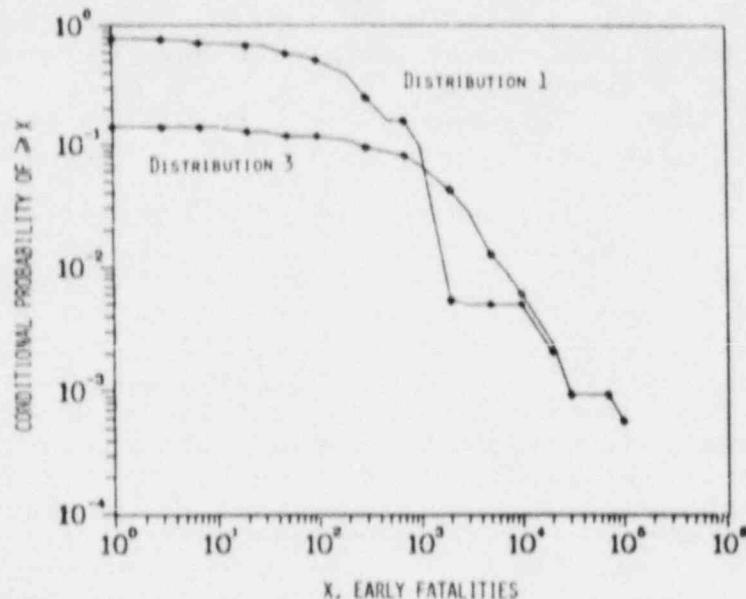


Figure 2.7.4-3. Comparison of the Early Fatality CCDF for Population Distribution 3 (all population in 1 sector) to that of the Reference Distribution.<sup>a</sup>

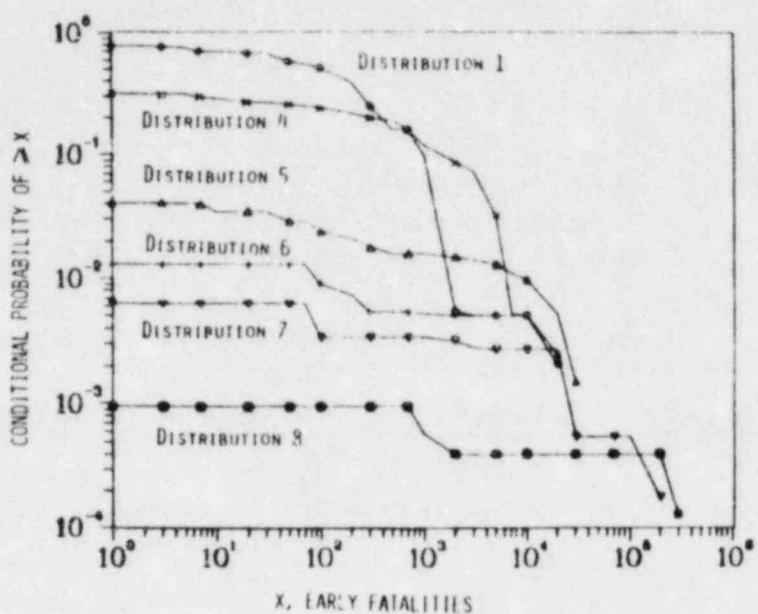
- a. Assumptions: 1120 MWe reactor, SST1 release, New York City meteorology, uniform wind rose, Summary Evacuation.

than 1 sector, the probability of observing at least 1 early fatality actually decreases by only a factor of  $\sim 6$ . Conversely, because all of the people out to 20 miles are now in 1 sector, when the plume goes out that sector, consequence magnitudes increase by about the same factor. Therefore, the mean (expected) result (400 early fatalities) is unchanged (see Table 2.7.4-1).

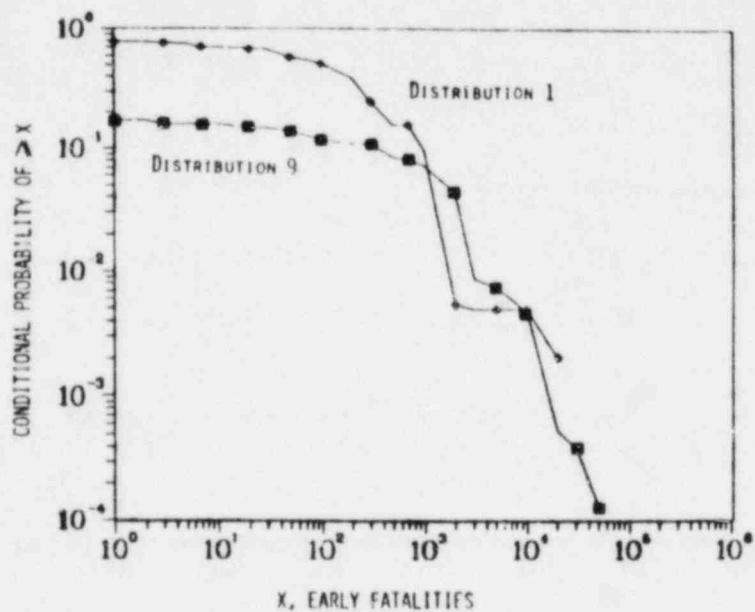
Figure 2.7.4-4 compares the early fatality CCDFs calculated using population distributions 4 through 8 to the Reference Distribution CCDF. The presence of population centers and vacant land in Distributions 4 through 8 produces two effects which are related. First, because increasingly larger areas of land surrounding the reactor are being vacated, the probability of observing any early fatalities decreases from 0.8 for the Reference Distribution to 0.001 for Distribution 8. Second, because the population centers are increasing in size (from 6000 people in Distribution 4 to 1,000,000 in Distribution 8), the maximum number of early fatalities (conditional probabilities of  $\lesssim 10^{-3}$ , caused by adverse weather) also increases from  $2.5 \times 10^4$  early fatalities for the Reference Distribution (which contains no population center) to  $4.0 \times 10^5$  for Distribution 8 (which contains a population center of almost 1 million people). Finally, the mean number of early fatalities for these distributions ranges from a low of 110 for Distributions 6 and 8 to a high of 560 for Distribution 4, while 99th percentile values range from 0 for Distributions 7 and 8 to 8500 for Distribution 5.

Figure 2.7.4-5 compares the CCDF calculated using the Reference Distribution to that calculated using Distribution 9. Figure 2.7.4-5 shows that incorporation into the Reference Distribution of radial and angular irregularities characteristic of a "real" population distribution alters the early fatality CCDF of the Reference Distribution in a predictable way. Because Distribution 9 is not uniform, the probability of having any early fatalities falls to 0.2 from the Reference Distribution value of 0.8, mean early fatalities decrease to 260 from 400, but the 99th percentile result increases from 1200 to 2800. Because Distribution 9 contains population centers (17,700 at 2.75 miles; 62,800 at 5.5 miles; 150,000 at 19 miles), the largest calculated number of early fatalities increased to  $6.5 \times 10^4$  from the Reference Distribution value of  $2.5 \times 10^4$ .

Examination of Table 2.7.4-1 and Figures 2.7.4-2 through 2.7.4-4 shows that the chance of having any early fatalities or early injuries, and the numbers that



**Figure 2.7.4-4.** Comparison of the Early Fatality CCDFs for Distributions 4 thru 8 (distributions that contain cities) to that of the Reference Distribution.<sup>a</sup>



**Figure 2.7.4-5.** Comparison of the Early Fatality CCDF of Distribution 9 (scaled real population distribution) to that of the Reference Distribution.<sup>a</sup>

a. Assumptions: 1120 MWe reactor, SST1 release, New York City meteorology, uniform wind rose, Summary Evacuation.

Table 2.7.4-1. Early Fatalities and Early Injuries for Population Distributions 1 Through 9, Conditional on an SST1 Release

Distri- bution	Early Fatalities			Early Injuries		
	P( $\geq 1$ )	Mean	99th Percentile	P( $\geq 1$ )	Mean	99th Percentile
1	0.79	400	1200	0.99	$2.2 \times 10^3$	19,000
2	0.79	1000	2700	0.99	$3.9 \times 10^3$	30,000
3	0.14	400	5600	0.17	$2.2 \times 10^3$	67,000
4	0.32	560	5800	0.82	$2.3 \times 10^3$	17,000
5	0.04	250	8500	0.48	$2.2 \times 10^3$	26,000
6	0.01	110	90	0.38	$1.5 \times 10^3$	27,000
7	0.006	160	0	0.20	$1.9 \times 10^3$	59,000
8	0.001	110	0	0.05	$1.2 \times 10^3$	34,000
9	0.17	260	2800	0.62	$1.8 \times 10^3$	24,000

P( $\geq 1$ ) = probability of having at least 1 early fatality or early injury (CCDF probability-axis intercept).

Mean = expected number of early fatalities or early injuries.

99th Percentile = consequence magnitude equalled or exceeded following 1 out of every 100 releases.

Assumptions: 1120 MWe reactor, SST1 release, New York City meteorology, uniform wind rose, Summary Evacuation.

might occur, are both highly variable. Therefore, because each of the nine distributions met the same radial population density criterion (populated radial intervals have population densities of 750 people per sq mi), it appears that any siting population criterion that restricts only the number of people within various radial distances may allow population distributions with significantly different risk characteristics. For this reason, consideration should perhaps be given to additional criteria which limit the number of people in any single sector or annular region.

Size and Distance of Population Centers. The effect of the size and distance of population centers upon consequences was further examined by imposing population centers of three sizes ( $10^4$ ,  $10^5$ , and  $10^6$  people) upon a 50 people per square mile background population density at the distances given in Table 2.7.4-2, thereby generating 13 population distributions, the background distribution and 12 distributions with population centers. Early fatality CCDFs were calculated for each of the 13 distributions assuming an SST1 release from a 1120 MWe reactor, New York City meteorology, a uniform wind rose, a 1-mile population exclusion zone, and evacuation to 10 miles at 10 mph with a distribution of delay times (Summary Evacuation, see Section 2.5). Mean, 90th, 99th, and maximum early fatality values for each CCDF are presented in Table 2.7.4-2.

Four conclusions may be drawn from the results presented in Table 2.7.4-2. First, irrespective of size, population centers beyond 25 miles do not contribute to early fatalities, i.e., these population centers have early fatality CCDFs identical to the background CCDF. Early fatalities are confined to 25 miles because, even for unfavorable meteorological conditions, plume concentrations fall below all early fatality thresholds before that distance.<sup>a</sup>

Second, population centers between 10 and 20 miles cause peak early fatality values<sup>b</sup> to increase substantially and mean values to increase by up to factors

- a. The maximum distance to which early fatalities occur for an SST1 release was shown in Section 2.6 to range from 13 to 25 miles, depending on meteorology, and is 18 miles for New York City meteorology.
- b. Improbable events with conditional probabilities of  $\leq 10^{-3}$  caused by adverse weather, e.g., rainout of the radioactive plume onto a population center.

Table 2.7.4-2. Effects of Size and Distance of Population Centers on Early Fatalities, Conditional on an SST1 Release

Center Population	Center Distance (mi)	Early Fatalities			
		Mean	90 Per- centile	99 Per- centile	Maximum Calculated <sup>a</sup>
Background <sup>b</sup>	--	23	67	150	1,700
$10^6$	175.0	23	67	150	1,700
	92.5	23	67	150	1,700
	52.5	23	67	150	1,700
	32.5	23	67	150	1,700
	52.5	23	67	150	1,700
$10^5$	27.5	23	67	150	1,700
	16.25	37	67	150	51,000
	11.25	44	67	160	49,000
$10^4$	16.25	26	67	150	11,000
	11.5	27	67	150	10,000
	5.5	24	68	160	1,700
	2.25	120	190	2,300	5,100

a. Maximum value calculated for any weather sequence. An improbable event (conditional probability  $\leq 10^{-3}$ ) typically caused by adverse weather (rainout of the radioactive plume onto a city).

b. Background population density = 50 people per sq mi.

Assumptions: 1120 Mwe reactor, SST1 release, New York City meteorology, uniform wind rose, Summary Evacuation.

of 2, but do not affect 90th or 99th percentile values (only mean and peak values differ from those of the background CCDF). Examination of individual calculations shows that population centers between 10 and 20 miles experience early fatalities principally when rain falls on the radioactive plume after it arrives over the population center. Because this is an improbable event, it affects only the CCDF peak and not its 90th, or 99th percentile values.<sup>a</sup>

Third, if effectively evacuated, population centers between 5 and 10 miles probably can avoid early fatalities (the CCDF for the population center at 5.5 miles is almost identical to the background CCDF). The population center at 5.5 miles experiences few early fatalities because the characteristics of Summary Evacuation (delay times, evacuation speed, see Section 2.5) assure that most persons in the population center avoid large exposures to radioactivity by evacuation for most weather sequences sampled.

Fourth, population centers very close to a reactor ( $\leq 5$  miles) are more likely to experience early fatalities even with evacuation (the CCDF of the population center at 2.25 miles differs from the background CCDF at all levels of probability). Early fatalities are likely to occur because only a timely warning followed by a very prompt evacuation could assure that all people in population centers within 5 miles of a reactor will escape plume exposures (see Section 2.5).

Exclusion Zone Size. All existing reactors are surrounded by an exclusion zone, which has no permanent inhabitants and is controlled exclusively by the utility operating the reactor. At current reactor sites exclusion zones are irregularly shaped with minimum exclusion distances which range from 0.1 to 1.3 miles (average 0.6 miles, see Appendix D). Larger exclusion zones would be expected to reduce the incidence of early health effects (those health effects induced by relatively large doses to individuals). The influence of exclusion zone size on early fatalities and injuries was examined for each

a. The effects of rain are discussed more fully in Sections 2.4 and 2.6; the effects of assuming emergency response beyond 10 miles are considered in Section 2.5.

of four emergency response scenarios (Scenarios 1, 5, 6, and 7 as defined in Section 2.2.2). Scenario 1 is an expeditious evacuation (1 hr delay, 10 mph), Scenario 5 is No Emergency Response, Scenario 6 is Poor Evacuation (5 hr delay, 1 mph), and Scenario 7 is Summary Evacuation. All calculations assumed no immediate emergency response beyond 10 miles, a uniform population distribution (100 persons per square mile), an SST1 release from an 1120 MWe reactor, and New York City meteorology.

Table 2.7.4-3 presents for each emergency response scenario the mean number of early fatalities calculated to occur within each of 20 distance intervals to 17.5 miles (for New York City meteorology, early fatalities are confined to 17.5 miles). Without any emergency response, the expected total number of early fatalities is 338, given an SST1 release at a reactor having a surrounding population density of 100 persons per square mile and no exclusion zone. However, if the reactor had a 1-mile exclusion zone, 58 fatalities would be avoided. Alternatively, an effective emergency response within 10 miles (e.g., Best Evacuation) would reduce the mean number of fatalities observed from 338 to 23 without any exclusion zone, and to 14 fatalities (those occurring beyond 10 miles) with a 1-mile exclusion zone.

The combined effects of exclusion zone size and emergency response effectiveness are further illustrated by the data in Table 2.7.4-4, which is drawn from Table 2.7.4-3. Table 2.7.4-4 presents for various combinations of emergency response effectiveness and exclusion zone size the number of early fatalities occurring within and beyond 10 miles and their sum. Table 2.7.4-4 shows that for large core-melt accidents mean early fatalities are reduced 16-fold (from 320 to <20) by an 0.5-mile exclusion zone and a very effective evacuation (Best Evacuation), by a 3-mile exclusion zone and a reasonably effective evacuation (Summary Evacuation), or by a 5-mile exclusion zone and an ineffective evacuation (Poor Evacuation). Alternatively, an 0.5-mile exclusion zone and a very effective evacuation within 2 miles (achieved possibly by early warning [41]) and a reasonably effective evacuation from 2 to 10 miles reduced mean early fatalities 12-fold (320 to 26).

Table 2.7.4-5 shows how the probability of having at least 1 early fatality or early injury varies with

Table 2.7.4-3. Mean Early Fatalities by Distance Intervals for Four Emergency Response Scenarios, All Evacuations<sup>a</sup>

<u>Distance Interval</u>	<u>Emergency Response<sup>b</sup></u>			
	None	Poor	Summary	Best
0.0				
0.25	6.3	6.3	5.6	3.9
0.5	11.5	11.4	8.6	2.4
- - - - -				
0.75	17.6	16.6	9.9	1.6
1.0	22.2	16.3	8.2	0.6
1.5	51.4	26.1	12.6	0.2
2.0	42.3	25.7	7.7	0.1
2.5	38.9	21.0	4.5	0.0
3.0	29.5	10.0	2.3	0
3.5	26.6	6.5	1.5	0
4.0	19.6	5.1	0.7	0
4.5	14.7	3.9	0.2	0
5.0	11.3	2.1	0.1	0
6.0	15.2	0.6	0.0	0
7.0	7.8	0.2	0.0	0
8.5	3.1	0	0	0
10.0	6.4	0.6	0.0	0
- - - - -				
12.5	6.9	6.9	6.9	6.9
15.0	0	0	0	0
17.5	7.1	7.1	7.1	7.1
<u>Total</u>	338	166	76	23

- a. Assumptions: SST1 release, 1120 MWe reactor, New York City meteorology, uniform wind rose, 100 people per square mile.
- b. No emergency response beyond 10 miles; relocation after 1 day (i.e., 1-day exposure to radioactivity deposited on the ground).

Table 2.7.4-4. Dependence of Mean Early Fatalities  
on Emergency Response Effectiveness  
and Exclusion Zone Size<sup>a</sup>

<u>Emergency Response</u>	<u>Exclusion Zone (mi)</u>	<u>Mean Early Fatalities</u>			<u>Total</u>
		>10 mi	≤10 mi		
Best Evacuation <sup>b</sup>	0.5	14	2.5		16.5
Summary Evacuation <sup>b</sup>	3.0	14	2.5		16.5
	2.0	14	9.3		23.3
	1.0	14	29.6		43.6
	0.5	14	47.7		61.7
Poor Evacuation <sup>b</sup>	5.0	14	1.4		15.4
	3.0	14	19.0		33.0
	2.0	14	50.0		64.0
	1.0	14	101.8		115.8
	0.5	14	134.7		148.7
No Evacuation	5.0	14	32.5		46.5
	3.0	14	104.7		118.7
	2.0	14	173.1		187.1
	1.0	14	266.8		280.8
	0.5	14	306.6		320.6
Best ≤2 mi Summary >2 mi	{ 0.5	14	11.8		25.8

a. Assumptions: SSTI release, 1120 MWe reactor, New York City meteorology, 100 people per square mile.

b. No emergency response beyond 10 miles; relocation after 1 day (i.e., 1-day exposure to radioactivity deposited on the ground).

Table 2.7.4-5. Probability of Having at Least  
1 Early Fatality or Injury<sup>a</sup> by  
Exclusion Zone Distance<sup>b</sup>

---

Emergency Response	None	Poor	Summary	Best	None	Poor	Summary	Best
Distance (mi)	Early Fatalities				Early Injuries			
0	1.00	1.00	0.96	0.88	1.00	1.00	1.00	1.00
0.25	1.00	1.00	0.81	0.38	1.00	1.00	1.00	1.00
0.5	1.00	0.97	0.76	0.26	1.00	1.00	0.92	0.72
0.75	0.97	0.85	0.55	0.21	1.00	1.00	0.85	0.50
1.0	0.97	0.60	0.37	0.10	1.00	1.00	0.82	0.41
2.0	0.59	0.40	0.19	0.01	0.98	0.97	0.76	0.36
5.0	0.20	0.10	0.02	0.01	0.78	0.57	0.39	0.36

---

a. CCDF intercept on probability axis (y-axis).

b. Assumptions: SST1 release, 1120 MWe reactor, New York City meteorology,  
100 people per square mile.

exclusion zone size. The table shows that the probability of having at least 1 early fatality following a large core-melt accident (SST1 release) can be reduced to 0.2 by the following combinations of an Emergency Response and an Exclusion Zone distance:

<u>Emergency Response</u>	None	Poor	Summary	Best
<u>Exclusion Zone (mi)</u>	5	4	2	0.75

Taken together Tables 2.7.4-3 through 2.7.4-5 suggest that a large Exclusion Zone without an emergency response is not nearly as effective as a substantially smaller Exclusion Zone and a timely emergency response.

Finally, because atmospheric releases of radioactivity of the size of SST1 are improbable (possibly extremely improbable, see Section 2.3.2, Source Term Uncertainties), it is important to note that for smaller releases (e.g., SST1 reduced an order of magnitude or SST2) the mean and peak distances to which early fatalities and injuries are likely to occur is much reduced, even with no emergency response (see Section 2.6, Distance Dependencies). Thus, for SST1 reduced 10-fold, on the average (mean result) fatalities would be confined to 1 mile and injuries to 3 miles, while for SST2 these distances are 0.5 miles and 2 miles, respectively. Thus, for releases substantially smaller than SST1, because early health effects are usually confined to only a few miles, typical Exclusion Zones ( $\sim 1$  mi) can have a substantial impact even without an emergency response.

#### 2.7.5 Interdiction Dose Criterion

Following a nuclear power plant accident, continued usage of land contaminated by radioactive material deposited from the plume would result in increased population exposures, and thus would increase latent health effects. Chronic exposure to contaminated land can be avoided by interdicting the usage of the land until removal processes (decontamination, radioactive decay, weathering, runoff) have decreased exposures to acceptable levels. The dose criterion (allowed ground-shine dose to an individual accumulated in 30 years) for interdiction of land is called the "interdiction dose." As interdiction dose increases, latent health effects increase (because more people are continuing to use contaminated land) and interdicted land area

and interdiction costs decrease (because less land is interdicted).

All of the calculations presented in other sections of this report used an interdiction dose of 25 rem due to a 30-year exposure to contaminated land. This section examines the sensitivity of latent cancer fatalities and of interdiction distance (distance to which land is interdicted), area, and costs to interdiction dose. Calculations were performed for four different 30-year interdiction doses (5, 10, 25, and 50 rem) and also for no interdiction. All of these calculations used an 1120 MWe reactor, the SST1 source term, the Indian Point population distribution and wind rose, and New York City meteorology.

Figures 2.7.5-1a through 2.7.5-1c present CCDFs for latent cancer fatalities and the interdiction distance and area. Table 2.7.5-1 presents mean and 90 percentile (conditional probability of  $10^{-1}$ ) values of latent cancer fatalities and of interdiction distance, area, and costs as a function of interdiction dose. In Figures 2.7.5-2a through 2.7.5-2c the mean values in Table 2.7.5-1 (except the cost data) are plotted versus interdiction dose. Examination of the CRAC2 code showed that the near linear dependence of mean latent cancer fatalities upon interdiction dose displayed in Figure 2.7.5-2a was to be expected.<sup>a</sup> Figure 2.7.5-2a shows that, if all contaminated ground were interdicted (interdiction dose of zero), then 3200 latent cancer fatalities would still result due to the pre-interdiction dose (cloudshine dose; inhalation dose, which includes the chronic dose from

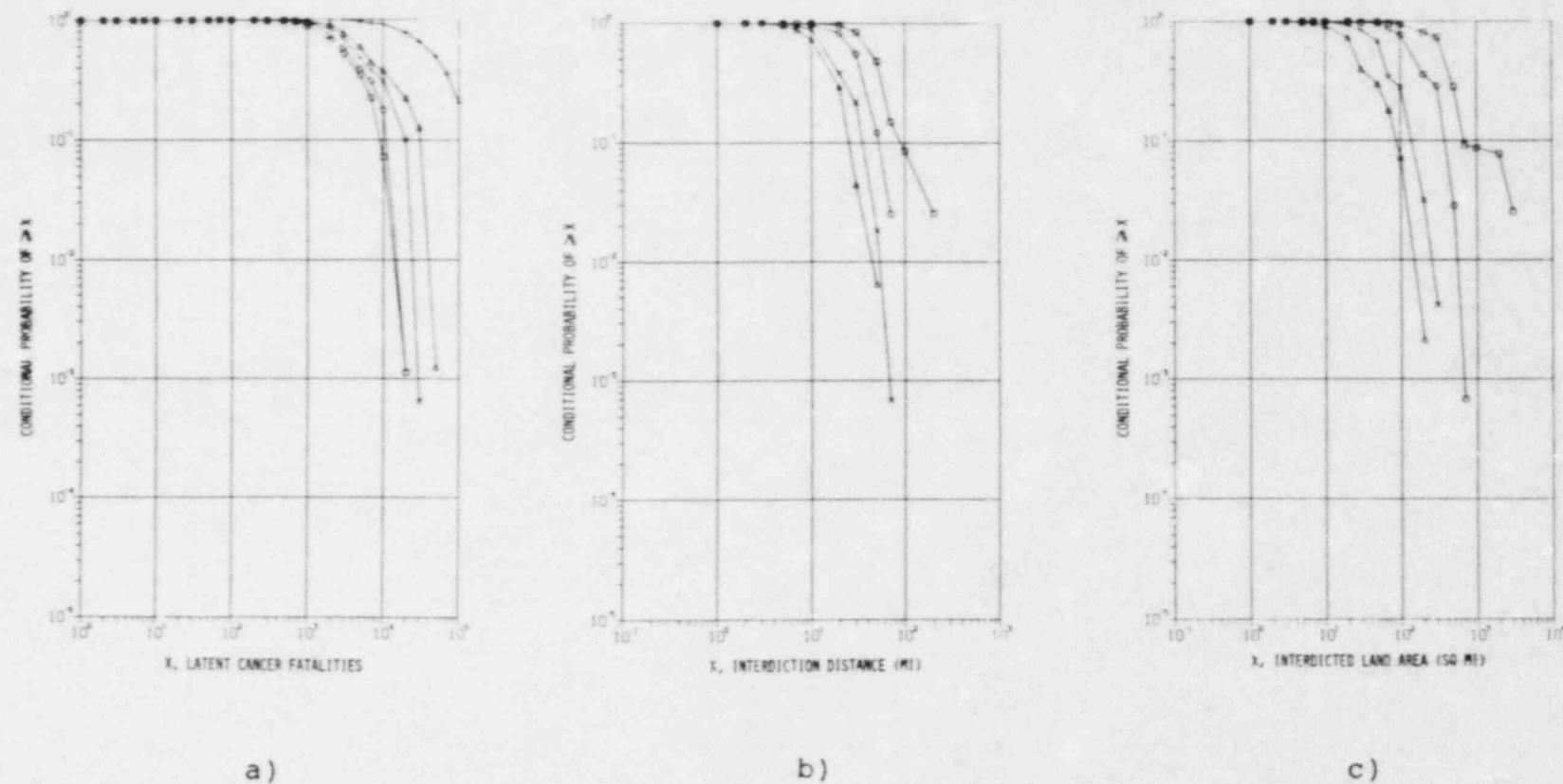
500

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$$a. \text{ Latent cancer fatalities} \sim \text{population dose} \sim \rho \int D(x) x dx,$$

where  $\rho$  = population density (approximately constant over large areas),  $D(x)$  = dose at distance  $x$ ,  $x_0$  = interdiction distance, and 500 mi = maximum distance for latent cancers (variable but large). From the transport and deposition algorithms used in CRAC2,  $D(x) \sim x^{-2}$ . So latent cancer fatalities

$$\sim \rho \ln x \left|_{x_0}^{500} \right. \text{ which is approximately linear in } x_0 \text{ for } x_0 \leq 50 \text{ mi.}$$



**Figure 2.7.5-1: Impact of 30-Year Interdiction Dose upon a) Latent Cancer Fatalities, b) Interdiction Distance (mi), and c) Interdicted Land Area (sq mi)**

#### Legend

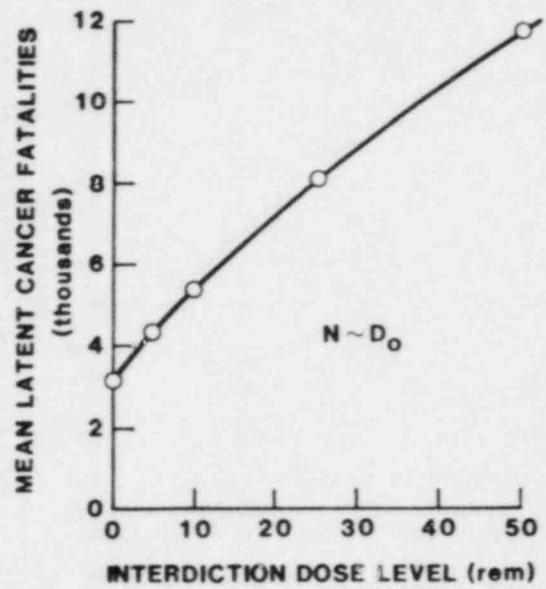
- + - no interdiction
- Δ - 50 rem interdiction dose
- X - 25 rem interdiction dose
- O - 10 rem interdiction dose
- - 5 rem interdiction dose

Assumptions: 1120 MWe reactor, SST1 release, Indian Point population and wind rose, New York City meteorology.

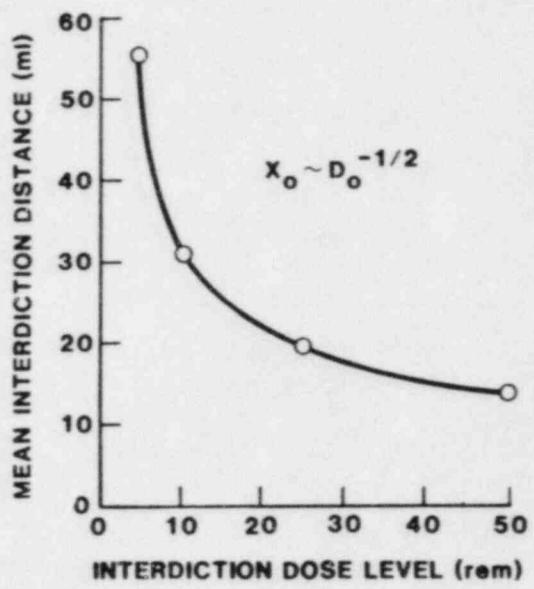
Table 2.7.5-1. Mean and 90th Percentile Values of Several Consequences by Interdiction Dose Level<sup>a</sup>

Interdiction Dose (rem)	Latent Cancer Fatalities		Interdiction Distance (mi)		Interdicted Land Area (sq. mi)		Interdiction Costs (billions)
	Mean	90 Per- centile	Mean	90 Per- centile	Mean	90 Per- centile	
5	4,300	9,100	56	90	580	640	36
10	5,400	11,000	32	52	200	380	17
25	8,100	20,000	19	35	76	140	5
50	12,000	31,000	14	25	41	86	2
None	68,000	130,000	0	0	0	0	0

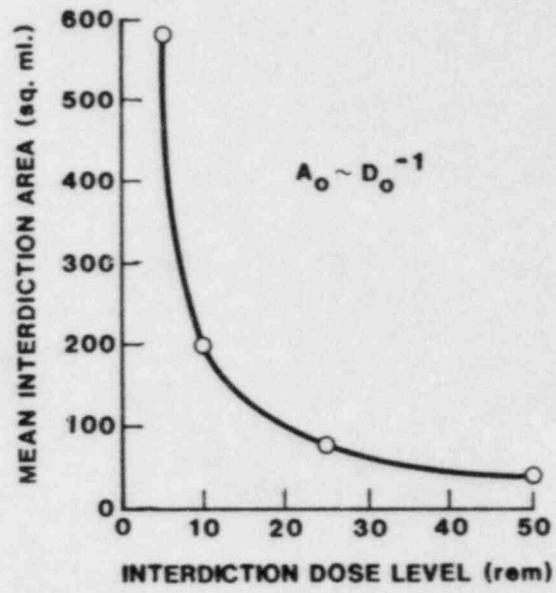
a. SST1 release, 1120 MWe reactor, Indian Point population and wind rose, New York City meteorology.



a)



b)



c)

Figure 2.7.5-2: Plots of a) Mean Latent Cancer Fatalities, b) Mean Interdiction Distance (mi), and c) Mean Interdicted Land Area vs Interdiction Dose Level (rem).

Assumptions: 1120 MWe reactor, SSTI release, Indian Point population and wind rose, New York City meteorology.

radioactivity deposited in the respiratory system; and pre-interdiction groundshine dose, which is assumed to be 1 day in duration). Figure 2.7.5-2b shows that interdiction distance is inversely proportional to the square root of the interdiction dose ( $x_o \sim D_o^{-1/2}$ ) and Figure 2.7.5-2c shows that interdiction area is inversely proportional to interdiction dose ( $A_o \sim D_o^{-1}$ ), which is not surprising since interdiction area should be roughly proportional to the square of interdiction distance ( $A_o \sim x_o^{-2}$ ).

Table 2.7.5-1 and Figures 2.7.5-1a through 2.7.5-1c show that latent cancer fatalities, and interdiction distance, area, and costs are all quite sensitive to interdiction dose. If all contaminated land were interdicted, the mean number of latent cancer fatalities would be reduced by about a factor of 20 from the number that would occur, if no land were interdicted (at the 90 percentile level the reduction factor is 15). Similarly, a 10-fold increase (5 to 50 rem) in interdiction dose produces about a 10-fold decrease in mean interdiction area and nearly a 20-fold decrease in mean interdiction costs.

Data in Table 2.7.5-1 can be used to illustrate the inverse relationship between latent fatalities and interdiction costs. For example, changing the interdiction dose criterion from no interdiction (all doses are tolerated) to an interdiction dose of 50 rem decreases mean latent fatalities by 57,000 and produces interdiction costs of  $\$1.9 \times 10^9$  or  $\sim \$3 \times 10^4$  per life saved. Further decrease from 50 rem to 25 rem saves an additional 4000 lives at a cost of  $\sim \$7 \times 10^5$  per life, while the decrease from 25 rem to 10 rem saves 3000 lives at a cost of  $\sim \$5 \times 10^6$  per life. Therefore, because of the inverse relationship between latent cancer fatalities and interdiction area, the high cost of interdicting land may make the interdiction of large areas (selection of a low interdiction dose) unacceptable.

## 2.8 Summary

This chapter has presented results from a large number of CRAC2 calculations, which characterize the sensitivity of accident consequences to input data and model parameters. Sensitivities were determined by comparison to a Base Case Calculation which assumed an SST1 release from a standard 1120 MWe reactor, meteorology typical of New York City, the Indian Point wind rose and population distribution, and Summary Evacuation. The principal conclusions derived from the results of these calculations are as follows:

- Estimates of the number of early fatalities are very sensitive to source term magnitude. Mean early fatalities (average result for many weather sequences) are decreased dramatically (about two orders-of-magnitude) by a one order-of-magnitude decrease in source term SST1 (large core melt, loss of most safety systems). Because the core melt accident source terms SST1-3 used in this study neglect or underestimate several depletion mechanisms, which may operate efficiently within the primary loop or the containment, consequence magnitudes calculated using these source terms may be significantly overestimated.
- The weather conditions at the time of a large release will have a substantial impact on the health effects caused by that release. In marked contrast to this, mean health effects (average result for many weather sequences) are relatively insensitive to meteorology. Over the range of meteorological conditions found within the continental United States (1 year meteorological records from 29 National Weather Service stations), mean early fatality values for a densely populated site show a range (highest value/lowest value) of only a factor of 2, and mean latent cancer fatality values a factor of 1.2.
- Peak early fatalities (maximum value calculated for any weather sequence) are generally caused by rainout of the radioactive plume onto a population center. For an SST1 release, the peak result is about 10-times less probable in a dry locale than in a wet one.

- The distances to which consequences might occur depend principally upon source term magnitude and meteorology. Frequency distributions of these distances, calculated using large numbers of weather sequences, yielded expected (mean), 99 percentile, and maximum calculated distances (expressed in miles) for early fatalities and early injuries as follows:

<u>Source Term</u>	<u>Consequence</u>	<u>Mean</u>	<u>99%</u>	<u>Maximum Calculated</u>
SST1	Early Fatalities	<5	≤15	<25
	Early Injuries	~10	~30	≤50
	Land Interdiction	~20	>50	>50
SST2	Early Fatalities	~0.5	<2	<2
	Early Injuries	<2	<5	~5
	Land Interdiction	<2	~7	~10

The maximum calculated distances are associated with very improbable events, (e.g., rain-out of the plume onto a population center). For the SST1 release reduced by a factor of 10, early fatalities are confined to ~5 miles, early injuries to ~20 miles, and interdiction of land to ~25 miles.

- Calculated consequences are very sensitive to site population distribution. For each of the 91 population distributions examined, early fatality, early injury, and latent cancer fatality CCDFs were calculated assuming an SST1 release from an 1120 MWe reactor. The resulting sets of CCDFs had the following ranges:

Early Fatalities. ~3 orders-of-magnitude in the peak and mean numbers of early fatalities and in the probability of having at least one early fatality.

Early Injuries. ~3 orders-of-magnitude in the means, ~2 in the peaks, and ~1 in the probability of having at least one early injury.

Latent Cancer Fatalities. ~1 order-of-magnitude in the peaks and the means and in the probability of having at least one latent cancer fatality.

Generally, mean results are determined by the average density of the entire exposed population, while peak results (especially for early fatalities) are determined by the distance to and size of exposed population centers.

- o Early fatalities and early injuries can be significantly reduced by emergency response actions. Both sheltering (followed by relocation) and evacuation can be effective, provided the response is expeditious. Access to basements or masonry buildings significantly enhances the effectiveness of sheltering. Expeditious response requires timely notification of the public. If the evacuation is expeditious (timely initiation), evacuation speeds of 10 mph are effective. Evacuation before containment breach within 2 miles, after release within 10 miles, and sheltering from 10 to 25 miles appears to be a particularly effective response strategy.
- o Because accident source terms increase with reactor size, smaller reactors pose lesser risks to the public than are posed by larger reactors.
- o Buoyant plumes (high heat content) can be lofted over close-in populations, thereby decreasing the risk of early health effects at short distances ( $\leq 10$  mi) but increasing that risk at longer distances ( $\sim 20$  mi). Because only rainout of lofted plumes is able to produce fatal exposures, mean early fatality values for buoyant plumes are substantially decreased by comparison to non-buoyant plumes (early fatalities result from fewer weather sequences).
- o Dry deposition velocity has a substantial impact on the distance to which land is interdicted and early health effects occur. However, the number of early health effects calculated are only moderately sensitive to dry deposition velocity.

- o Exclusion zones (unless very large) are unlikely to significantly reduce early health effects for very large core melt accidents such as SST1. However, for smaller accidents (e.g. 1/10 SST1, SST2) early health effects could be significantly mitigated by exclusion zones of 1 to 2 miles.
- o Decreasing the level of contamination at which land is interdicted decreases latent cancer fatalities and increases the amount of land interdicted. As interdiction dose is increased, interdiction costs (value of interdicted land and buildings) increase more rapidly than does the number of latent cancer fatalities avoided.

## References for Chapter 2

1. Reactor Safety Study, Appendix VI: Calculation of Reactor Accident Consequences, U.S. Nuclear Regulatory Commission, WASH-1400 (NUREG-75/014), 1975.
2. L. T. Ritchie, J. D. Johnson, and R. M. Blond, Calculations of Reactor Accident Consequences, Version 2 (CRAC2): Computer Code User's Guide, NUREG/CR-2326, SAND81-1994, Sandia National Laboratories, Albuquerque, NM (to be published).
3. L. T. Ritchie, et al., CRAC2, Calculation of Reactor Accident Consequences, Version 2, Model Description, SAND82-0342, NUREG/CR-2552, Sandia National Laboratories, Albuquerque, NM (to be published).
4. I. B. Wall et al., Overview of the Reactor Safety Study Consequence Model, U.S. Nuclear Regulatory Commission, NUREG-0340, 1977.
5. R. M. Blond, D. C. Aldrich and E. H. Johnson, "International Standard Problem for Consequence Modeling," International ANS/ENS Topical Meeting on Probabilistic Risk Assessment, Port Chester, New York, September 20-24, 1981.
6. D. C. Aldrich et al., "International Standard Problem for Consequence Modeling: Results," International ANS/ENS Topical Meeting on Probabilistic Risk Assessment, Port Chester, New York, September 1981.
7. J. M. Hans, Jr., and T. C. Sell, Evacuation Risks - An Evaluation, U.S. Environmental Protection Agency, EPA-520/6-74-002, 1974.
8. D. C. Aldrich, R. M. Blond, and R. B. Jones, A Model of Public Evacuation for Atmospheric Radiological Releases, SAND78-0092, Sandia Laboratories, Albuquerque, NM, 1978.
9. D. C. Aldrich, L. T. Ritchie, and J. L. Sprung, Effect of Revised Evacuation Model on Reactor Safety Study Accident Consequences, SAND79-0095, Sandia National Laboratories, 1979.

10. G. C. Holzworth, Mixing Heights, Wind Speeds, and Potential for Urban Air Pollution Throughout the Contiguous United States, Publ. No. AP-101, U.S. Environmental Protection Agency, Office of Air Programs, Research Triangle Park, NC, 1972.
11. D. E. Bennett, Radionuclide Core Inventories for Standard PWR and BWR Fuel Management Plans, Sandia National Laboratories, SAND82-1111. NUREG/CR-2724 (to be published).
12. Planning Basis for the Development of State and Local Government Radiological Emergency Response Plans in Support of Light Water Nuclear Power Plants, NUREG-0396, U. S. Nuclear Regulatory Commission, 1978.
13. Deutsche Risikostudie Kernkraftwerke, Fachband 8: Unfallfolgenrechnung und Risikoergebnisse, TUV Rheinland, Koln, Federal Republic of Germany, 1981.
14. PRA Procedures Guide, Review Draft, U.S. Nuclear Regulatory Commission, NUREG/CR-2300, September 1981.
15. M. Levenson and F. Rahn, "Realistic Estimates of the Consequences of Nuclear Accidents," Nucl. Technol. 38, 99 (1981).
16. Technical Basis for Estimating Fission Product Behavior During LWR Accidents, NUREG-0772, U.S. Nuclear Regulatory Commission, June 1981.
17. K. Woodard and T. E. Potter, "Modification of the Reactor Safety Study Consequence Computer Program (CRAC) to Incorporate Plume Trajectories," Trans. Amer. Nucl. Soc., 33, 193 (1979).
18. Zion Probabilistic Safety Study, Commonwealth Edison Co., Chicago, IL, September 1981.
19. D. J. Alpert, P. H. Gudiksen, and K. Woodard, "Modeling Atmospheric Dispersion for Reactor Accident Consequence Evaluation," International ANS/ENS Topical Meeting on Probabilistic Risk Assessment, Port Chester, NY, September 20-24, 1981.

20. BEIR (Committee on the Biological Effects of Ionizing Radiation), The Effects on Populations of Exposure to Low Levels of Ionizing Radiation, National Academy of Sciences, Washington, DC, 1980.
21. W. E. Loewe and E. Mendelsohn, "Revised Dose Estimates at Hiroshima and Nagasaki", UCRL-85446, Lawrence Livermore National Laboratory, 1980.
22. Regulatory Impact of Nuclear Reactor Accident Source Term Assumptions, NUREG-0771, U.S. Nuclear Regulatory Commission, June 1981.
23. R. O. Campbell, A. P. Malinauskas, and W. R. Stratton, "The Chemical Behavior of Fission Product Iodine in Light Water Reactor Accidents," Nucl. Technol., 38, 111, (1981).
24. H. A. Morewitz, "Fission Product and Aerosol Behavior Following Degraded Core Accidents," Nucl. Technol. 38, 120, (1981).
25. U.S. Nuclear Regulatory Commission, "Onsite Meteorological Programs," Regulatory Guide 1.23, 1972.
26. J. L. Sprung, An Investigation of the Adequacy of the Composite Population Distributions Used in the Reactor Safety Study, SAND78-0556, Sandia National Laboratories, Albuquerque, NM, 1978.
27. D. C. Aldrich, P. E. McGrath and N. C. Rasmussen, Examination of Offsite Radiological Emergency Protective Measures for Nuclear Reactor Accidents Involving Core Melt, SAND78-0454, NUREG/CR-1131, Sandia National Laboratories, Albuquerque, New Mexico, October (1979).
28. D. C. Aldrich, and R. M. Blond, Examination of the Use of Potassium Iodide (KI) as an Emergency Protective Measure for Nuclear Reactor Accidents, SAND80-0981, NUREG/CR-1433, Sandia National Laboratories, Albuquerque, NM, March 1980.
29. D. C. Aldrich, D. M. Ericson, Jr., and J. D. Johnson, Public Protection Strategies for Potential Nuclear Reactor Accidents: Sheltering Concepts With Existing Public and Private Structures, SAND77-1725, Sandia Laboratories, Albuquerque, NM, February 1978.

30. D. C. Aldrich, and D. M. Ericson, Jr., Public Protection in the Event of a Nuclear Reactor Accident: Multicompartment Ventilation Model for Shelters, SAND77-1555, Sandia Laboratories, Albuquerque, NM, February 1978.
31. A. F. Cohen, and B. L. Cohen, Infiltration of Particulate Matter Into Buildings, NUREG/CR-1151, SAND79-2079, Sandia Laboratories, March 1980.
32. Manual of Protective Action Guides and Protective Actions for Nuclear Incidents, EPA-520/1-75-001, U. S. Environmental Protection Agency, September 1975.
33. G. A. Briggs, "Plume Rise Predictions" in Lectures on Air Pollution and Environmental Impact Analysis, D. A. Haugen, ed., American Meteorological Society, Boston, MA, 1975, pp. 59-105.
34. A. J. Russo, Reactor Accident Plume Rise Calculations, SAND76-0340, Sandia National Laboratories, Albuquerque, NM, 1976.
35. A. J. Russo, J. R. Wayland, L. T. Ritchie, Influence of Plume Rise on the Consequences of Radioactive Material Releases, SAND76-0534, Sandia National Laboratories, Albuquerque, NM, 1977.
36. T. W. Horst, "A Surface Depletion Model for Deposition From a Gaussian Plume," Atmos. Environ. 11, 41 (1977).
37. R. P. Hosker, Jr., "Practical Application of Air Pollutant Deposition Models -- Current Status, Data Requirements, and Research Needs," in Proc. Intern. Conf. on Air Pollutants and Their Effects on the Terrestrial Ecosystem, Banff, Alberta, Canada, May 16-17, 1980, S. V. Krupa and A. H. Legge, ed., John Wiley and Sons, NY, 1980.
38. D. C. Kaul, "The Effect of Plume Depletion Model Variations on Risk Assessment Uncertainties," International ANS/ENS Topical Meeting on Probabilistic Risk Assessment, Port Chester, NY, September 1981.
39. G. A. Sehmel, "Particle and Gas Dry Deposition - A Review," Atmos. Environ. 14, 983 (1980).

40. T. J. Overcamp, "A General Gaussian Diffusion-Deposition Model for Elevated Point Sources," *J. Appl. Meteor.* 15, 1167 (1976).
41. R. P. Burke, In-Plant Considerations for Optimal Offsite Response to Reactor Accidents, Master's Thesis, MIT Department of Nuclear Engineering, Cambridge, MA, November 1981.

### 3. Population Statistics for Current Reactor Sites

#### 3.1 Introduction

This chapter examines a variety of characteristics of the population distributions about the 91 reactor sites first discussed in Section 2.4 and described in detail in Appendices A and C. Each of these sites has either an operating license or a construction permit. The site characteristics examined include distance to the boundary of the reactor site exclusion zone, site population factors, the distribution of population densities within different radial annuli, and distances, maximum population densities within 22.5 and 45 sectors, and time-dependent trends in site population densities. As a group these analyses delineate the demographic characteristics of current reactor sites and provide a perspective of past siting decisions.

The population distributions examined in this chapter were derived from 1970 census data. A computer program was used (see Appendix A) to construct from U. S. Census Enumeration District (CED) data, the population distribution (16 sectors, 34 radial intervals) surrounding each of the 91 reactor sites. The procedure used may produce a distribution with significant errors close to the site. Errors may result because the computer program assumes that the entire population of each CED is located entirely at the "centroid" of the CED, when it may actually be dispersed over areas which are substantially larger than the area of the spatial interval in which the centroid is located. Because a CED typically contains about 1000 persons, the magnitude of this error decreases as population density increases. Given the spacing of the circular polar grid, the error is most likely negligible beyond 20 miles even for sparsely populated regions ( $\leq 40$  people per sq mi). Beyond 7 miles, errors are unlikely to be substantial for population densities greater than 500 people per square mile.

Throughout this chapter results are frequently presented for each of the five NRC administrative regions. Figure 3-1 displays the boundaries of these regions and the locations of the 91 reactor sites examined. In Section 3.2 scatter plots of site exclusion zone distances and site population factors are presented by region. Section 3.3 presents population density CCDFs and displays percentile values drawn from the CCDFs for each region. Scatter plots of these data are also

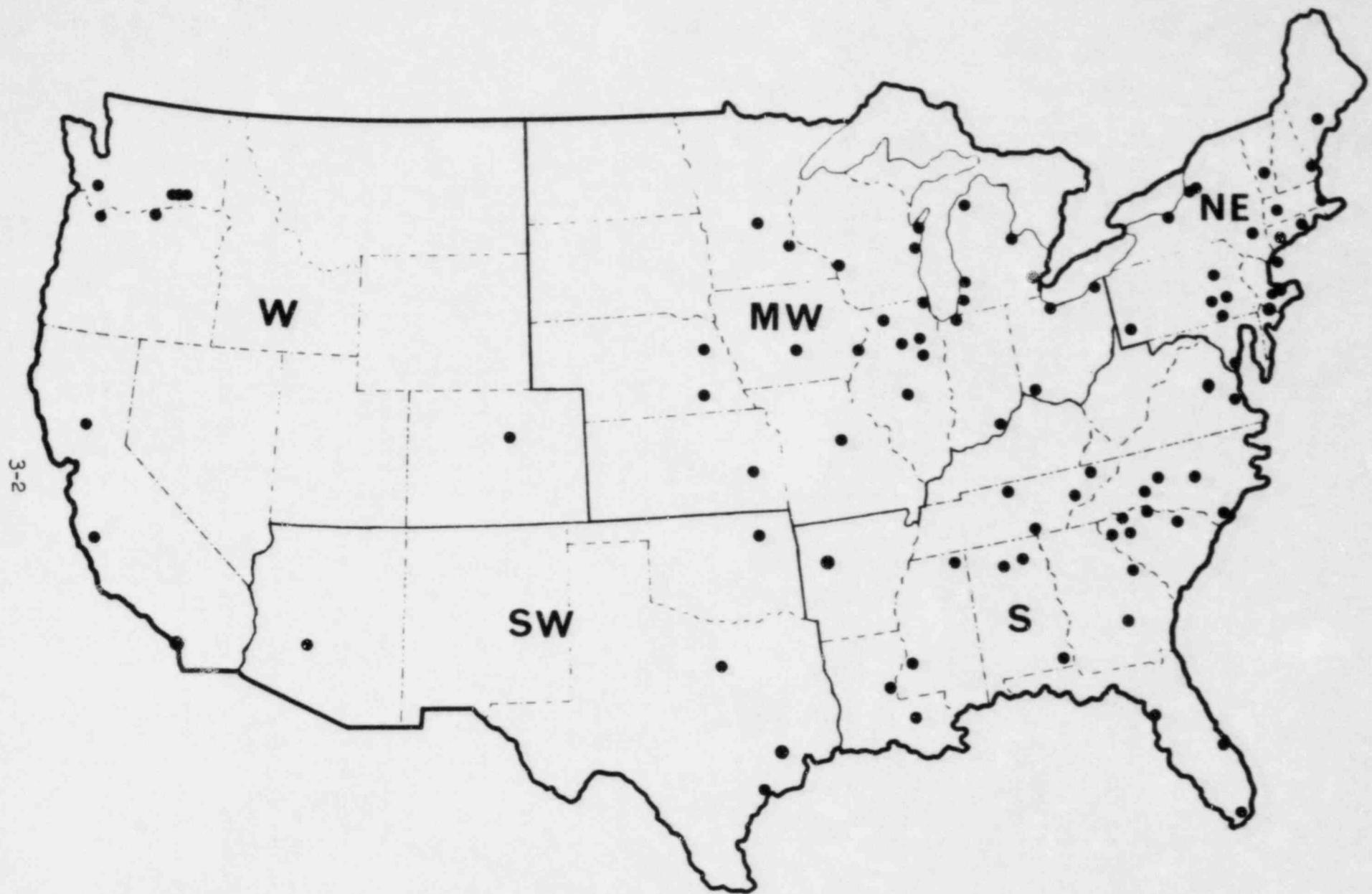


Figure 3-1. The Five NRC Administrative Regions and the Location of the 91 Reactor Sites.

presented. Time trends of site population characteristics are analyzed by region in Section 3.4. Finally, population characteristics for individual sites and additional regional results are presented in Appendix D, and additional population data are available in NUREG-0348 [1].

### 3.2 Exclusion Zones and Site Population Factors

Distance to the exclusion zone boundary, distance to nearby cities, and site population factors have all been used by the NRC to describe population distributions about reactor sites. Consequence sensitivity to exclusion zone size and to distance to nearby cities was examined in Section 2.7.4. This section examines regional variation (1) of the minimum distance to the exclusion zone boundary and (2) of site population factors, with and without wind rose weighting.

All reactors are surrounded by an exclusion zone, which has no permanent inhabitants and is controlled exclusively by the utility operating the reactor. Exclusion zones are usually irregularly shaped. For the 91 sites examined in this study, minimum distances to the exclusion zone boundary range from 0.1 to 1.3 miles with 0.5 miles being about average. The value for each of the 91 sites is presented in Appendix D. Figure 3-2 displays these values as scatter plots, one for each NRC administrative region. Median values for each scatter plot are indicated on the figure. The median values increase in the order NE, MW, W, S, SW.

Site population factors were developed by the NRC [2] to provide a way to compare populations around different sites. The factors are intended to be dimensionless measures of the total risk to the population within a specified radial distance. Since correlations between population distribution and wind direction may significantly influence risk at some sites, a wind rose weighted formulation of the site population factor was also developed.

The Site Population Factor (SPF) and Wind Rose weighted Site Population Factor (WRSPF) are defined as follows:

4-3  
Exclusion Distance, Miles

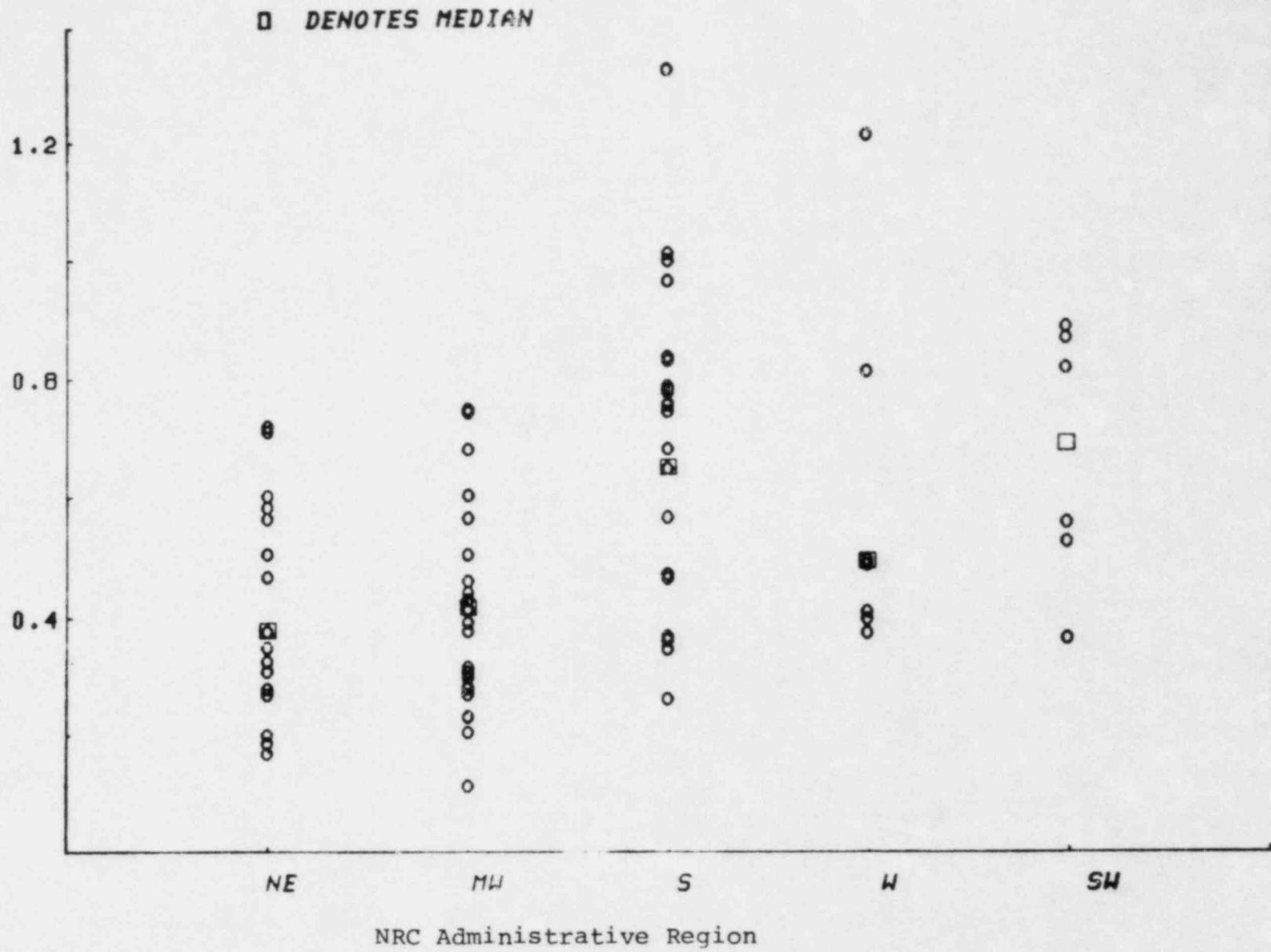


Figure 3-2. Exclusion Distances for 91 Reactor Sites by Geographic Area.

$$SPF_n = \frac{\sum_{i=1}^m p_i / r_i^{1.5}}{\sum_{i=1}^m \bar{p}_i / r_i^{1.5}}$$

$$WRSPF_n = \frac{16 \sum_{i=1}^m \left[ \sum_{j=1}^{16} w_j p_{i,j} \right] / r_i^{1.5}}{\sum_{i=1}^m \bar{p}_i / r_i^{1.5}}$$

where  $r_i$  is the outer radius of annulus i of m concentric annuli ( $r_0 = 0$ ,  $r_m = n$ ).

$n$  is the outer radius of the outermost annulus, annulus m.

$\bar{p}_i$  is the population of annulus i assuming a uniform population density of  $1000^2$  people per sq mi, i.e.,  $\bar{p}_i = 10^3 \pi (r_i - r_{i-1})$

$p_i$  is the actual population of annulus i.

$p_{i,j}$  is the actual population of the ith radial interval of wind rose sector j.

$w_j$  is the fraction of time that the wind blows into sector j.

Finally, the power 1.5 to which the radius  $r_i$  is raised was selected because it approximates the functional relationship between risk and distance; and  $WRSPF_n = SPF_n$  whenever  $w_j = 1/16$  for all j, i.e., whenever the wind rose is uniform.

Site population factors (both  $SPF_n$  and  $WRSPF_n$  for  $n = 5, 10, 20$ , and 30 miles) are presented in Appendix D for each of the 91 sites. Table 3-1 presents average values for these factors for each of the five NRC administrative regions. Examination of Table 3-1 shows that, for each distance and for both factors, the regional average values are highest for the Northeast region and lowest for the Southwest region, and decrease in the order NE, MW, S, W, SW.

Table 3-1

SPF and WRSPP Values for the Five  
NRC Administrative Regions<sup>a</sup>

	<u>NE</u>	<u>MW</u>	<u>S</u>	<u>W</u>	<u>SW</u>
SPF <sub>5</sub>	0.16±0.22	0.09±0.15	0.03±0.04	0.01±0.02	0.01±0.01
SPF <sub>10</sub>	0.17±0.19	0.10±0.14	0.05±0.03	0.03±0.03	0.01±0.01
SPF <sub>20</sub>	0.20±0.18	0.12±0.12	0.08±0.06	0.04±0.03	0.03±0.02
SPF <sub>30</sub>	0.25±0.24	0.14±0.13	0.09±0.06	0.05±0.04	0.04±0.04
WRSPPF <sub>5</sub>	0.17±0.29	0.10±0.18	0.04±0.04	0.02±0.02	0.01±0.01
WRSPPF <sub>10</sub>	0.18±0.22	0.11±0.16	0.05±0.03	0.04±0.06	0.02±0.01
WRSPPF <sub>20</sub>	0.22±0.20	0.13±0.14	0.08±0.07	0.05±0.04	0.03±0.02
WRSPPF <sub>30</sub>	0.26±0.26	0.15±0.14	0.09±0.07	0.06±0.06	0.04±0.03

<sup>a</sup>Standard Deviations are indicated as bounds

### 3.3 Site Population Statistics

The 91 population distributions examined in this chapter are all constructed on a 16 sector, circular polar grid. For any specified portion (a circle, an annulus, a sector) of that grid, 91 values of population density are available, one for each of the 91 population distributions. By cumulation of the 91 values for a given portion of the grid, a population density CCDF may be constructed.\* Six different sets of population density CCDFs have been constructed for the following areas of the population distribution grid:

Set 1: eight annuli (0-2, 2-5, 5-10, 10-20, 20-30, 30-50, 50-100, and 100-200 mi).

Set 2: eight radial distances (0-2, 0-5, 0-10, 0-20, 0-30, 0-50, 0-100, and 0-200 mi).

\*Population density CCDFs are Log-Log plots of the fraction of sites vs population density. Any point on the distribution gives the fraction of sites (y-axis value), which have a population density within the specified portion of the grid (annulus, circle, sector), that is greater than or equal to the specified population density (x-axis value).

Set 3: the most populated  $22.5^\circ$  sector in each of six annuli (0-2, 2-5, 5-10, 10-20, 20-30, and 30-50 mi) on the 16 sector grid.

Set 4: the most populated  $22.5^\circ$  sector in each of six radial distances (0-2, 0-5, 0-10, 0-20, 0-30 mi, and 0-50 mi) on the 16 sector grid.

Set 5: the most populated  $45^\circ$  sector (two adjacent  $22.5^\circ$  sectors) in each of six annuli (0-2, 2-5, 5-10, 10-20, 20-30, and 30-50 mi) on the 16 sector grid.

Set 6: the most populated  $45^\circ$  sector (two adjacent  $22.5^\circ$  sectors) in each of six radial distances (0-2, 0-5, 0-10, 0-20, 0-30, and 0-50 mi) on the 16 sector grid.

Each set of CCDFs contains CCDFs for each of the five NRC administrative regions (NE, MW, S, W, SW) and for all regions combined (All). CCDFs were also calculated for  $45^\circ$  sectors because atmospheric dispersion can produce plumes with an angular dispersion greater than  $22.5^\circ$ .

Because of the large number of CCDFs calculated (total of 240) most of the CCDFs are presented in Appendix D. Also presented in Appendix D are the site specific data from which the CCDFs were constructed. In this section, Figure 3-3 presents CCDFs of population density at the 91 sites for six radial annuli (0-2, 2-5, 5-10, 10-20, 20-30, and 30-50 mi) and Figure 3-4 presents CCDFs for six radial distances (0-2, 0-5, 0-10, 0-20, 0-30, and 0-50 mi). CCDFs of population density, in the most populated  $22.5^\circ$  and  $45^\circ$  sectors at each of the 91 sites, are presented for the same two sets of six annuli and six radial distances in Figures 3-5 through 3-8. Tables 3-2 and 3-3 list maximum, 90th percentile, median, and minimum population densities for each of the five NRC administrative regions and for all regions combined for eight annuli and eight radial distances. Table 3-4 presents population densities for 4 radial distances of the most populated  $22.5^\circ$  sector for each of the five administrative regions and for all regions combined. Finally, Figures 3-9 through 3-11 present scatter plots

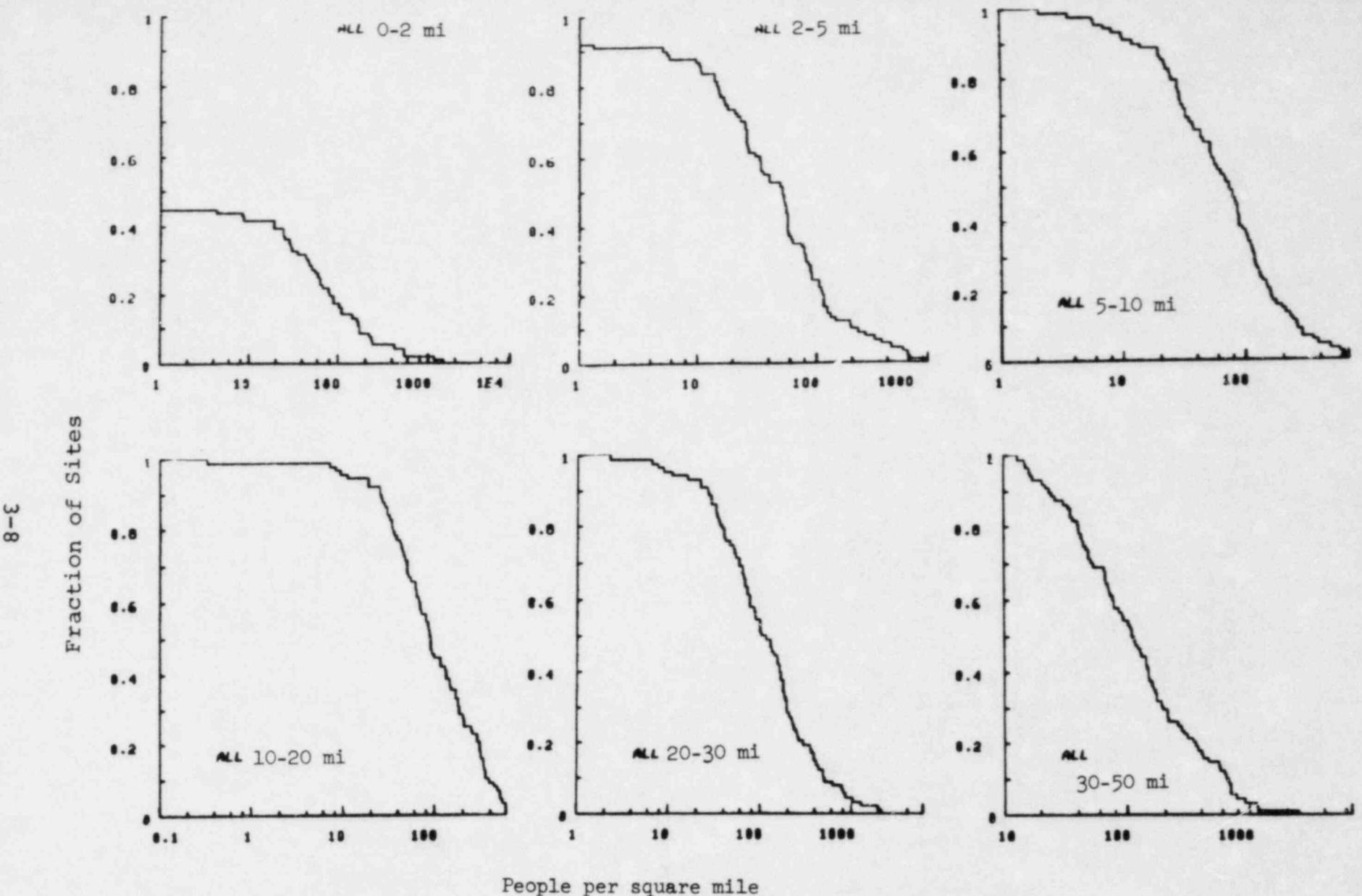


Figure 3-3. CCDFs of Population Density ( $\text{People}/\text{Mile}^2$ ) at 91 Sites for Six Radial Annuli.

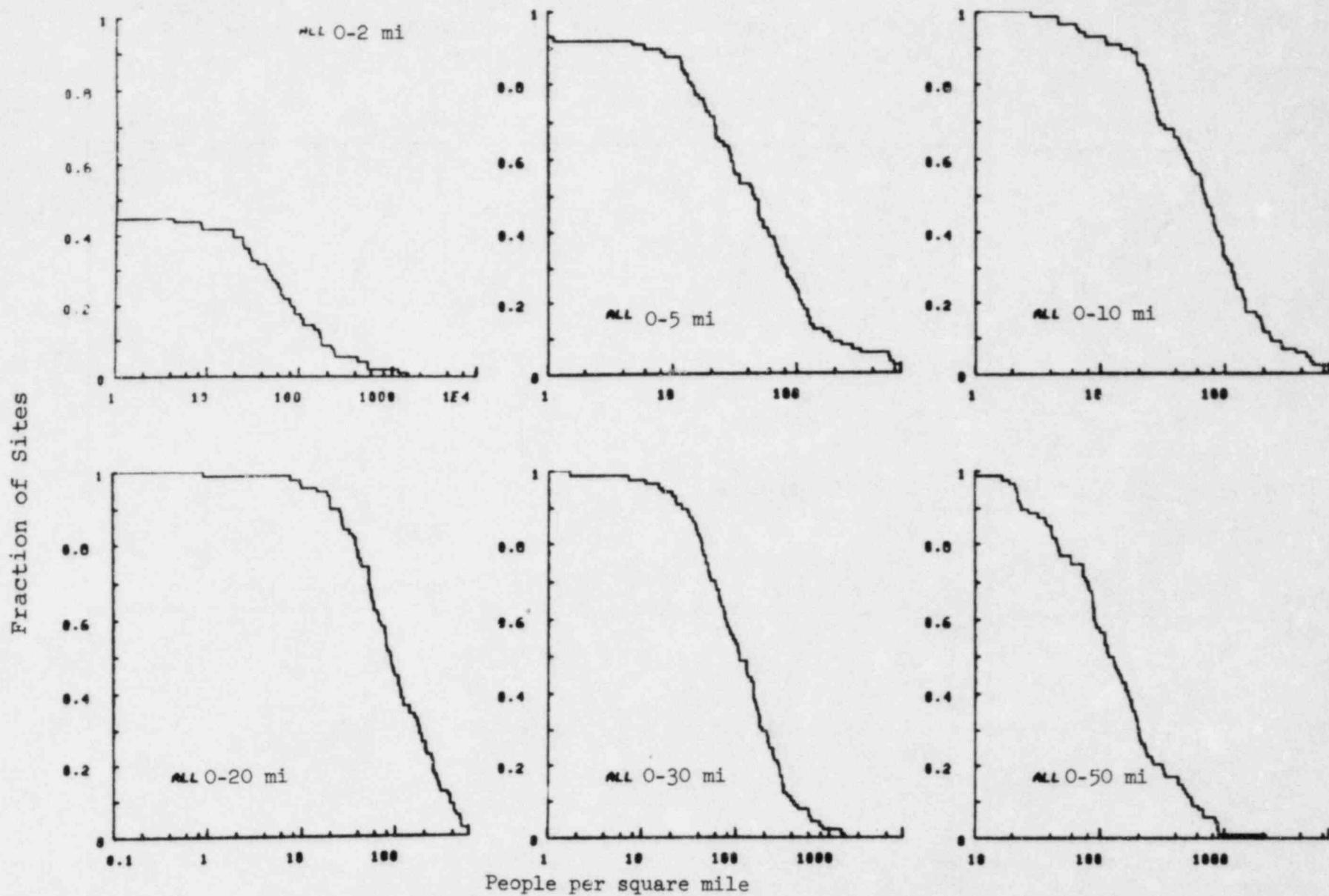


Figure 3-4. CCDFs of Population Density ( $\text{People}/\text{Mile}^2$ ) at 91 Sites for Six Radial Distances

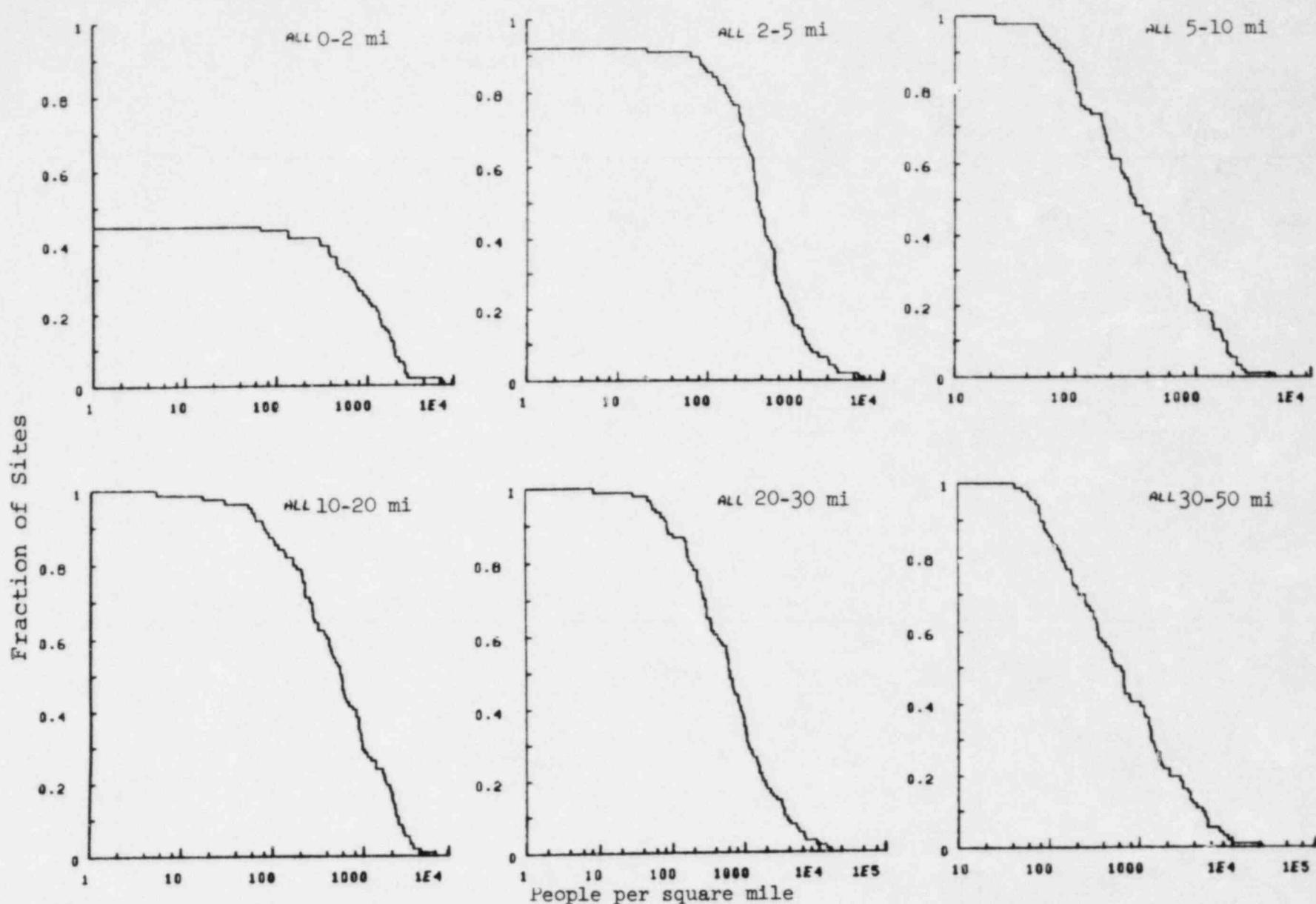


Figure 3-5. CCDFs of Population Density ( $\text{People}/\text{Mile}^2$ ) in the Most Populated 22.5 Degree Sector at 91 Sites for Six Radial Annuli.

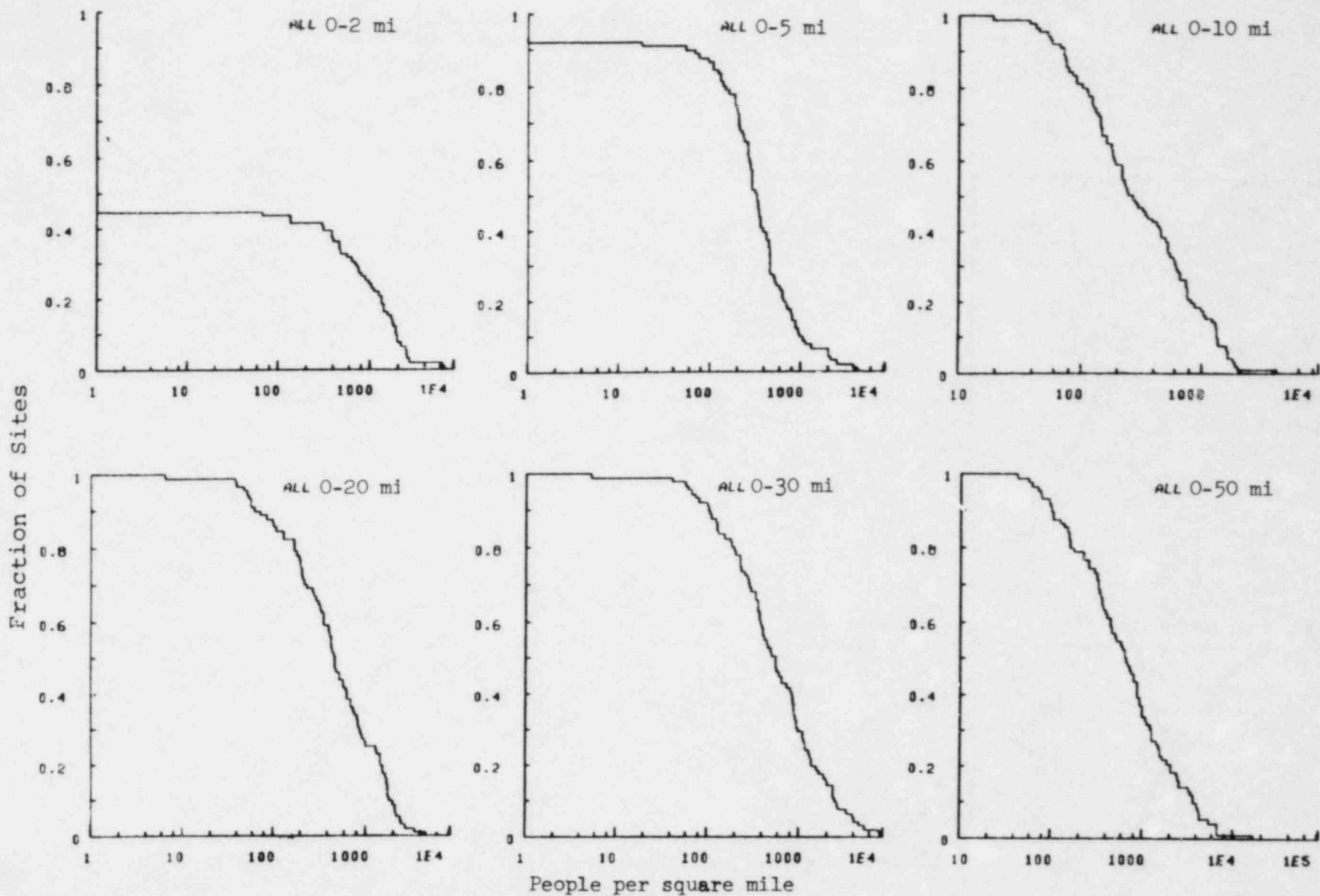


Figure 3-6. CCDFs of Population Density ( $\text{People}/\text{Mile}^2$ ) in the Most Populated 22.5 Degree Sector at 91 Sites for Six Radial Distances.

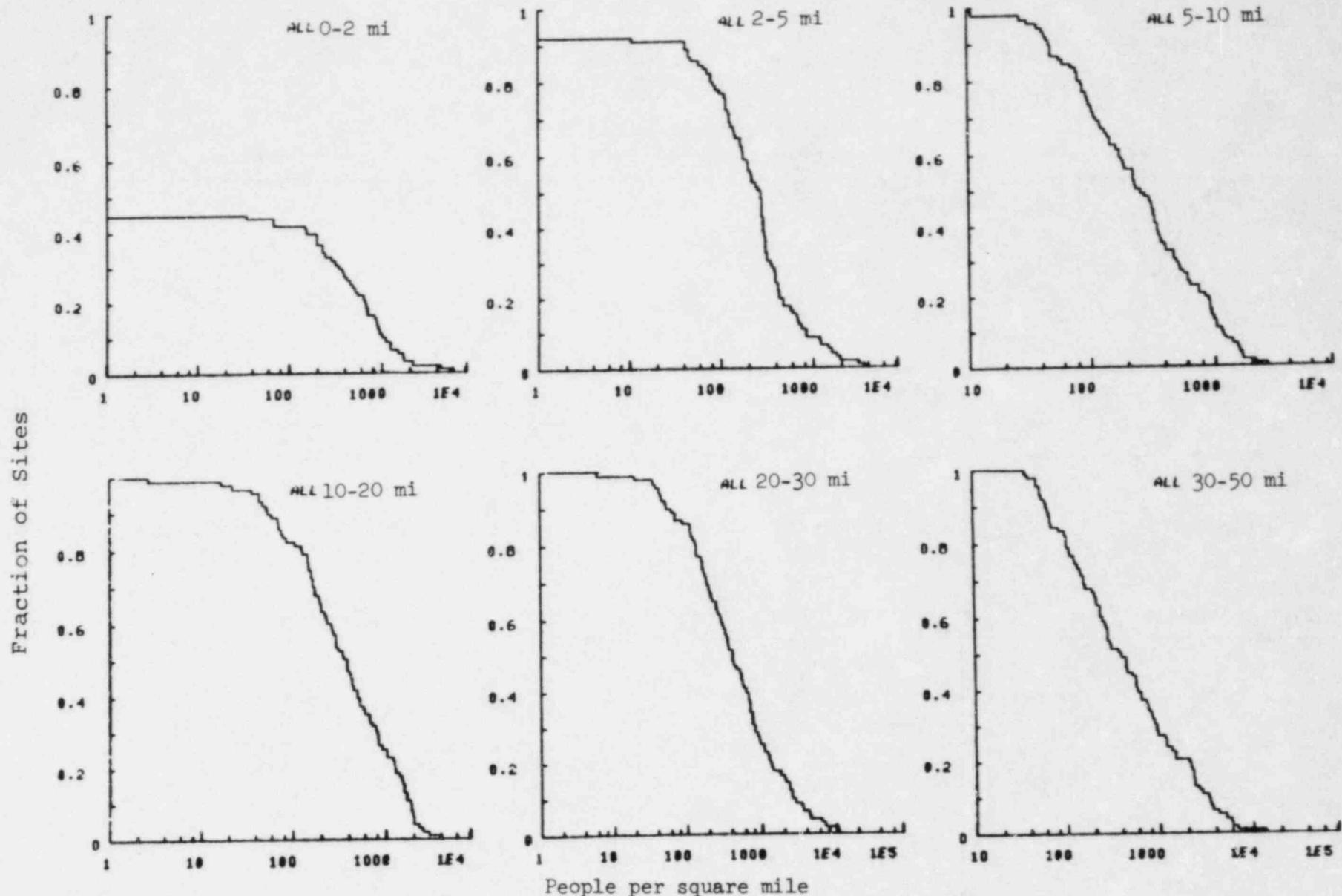


Figure 3-7. CCDFs of Population Density ( $\text{People}/\text{Mile}^2$ ) in the Most Populated 45 Degree Sector at 91 Sites for Six Radial Annuli.

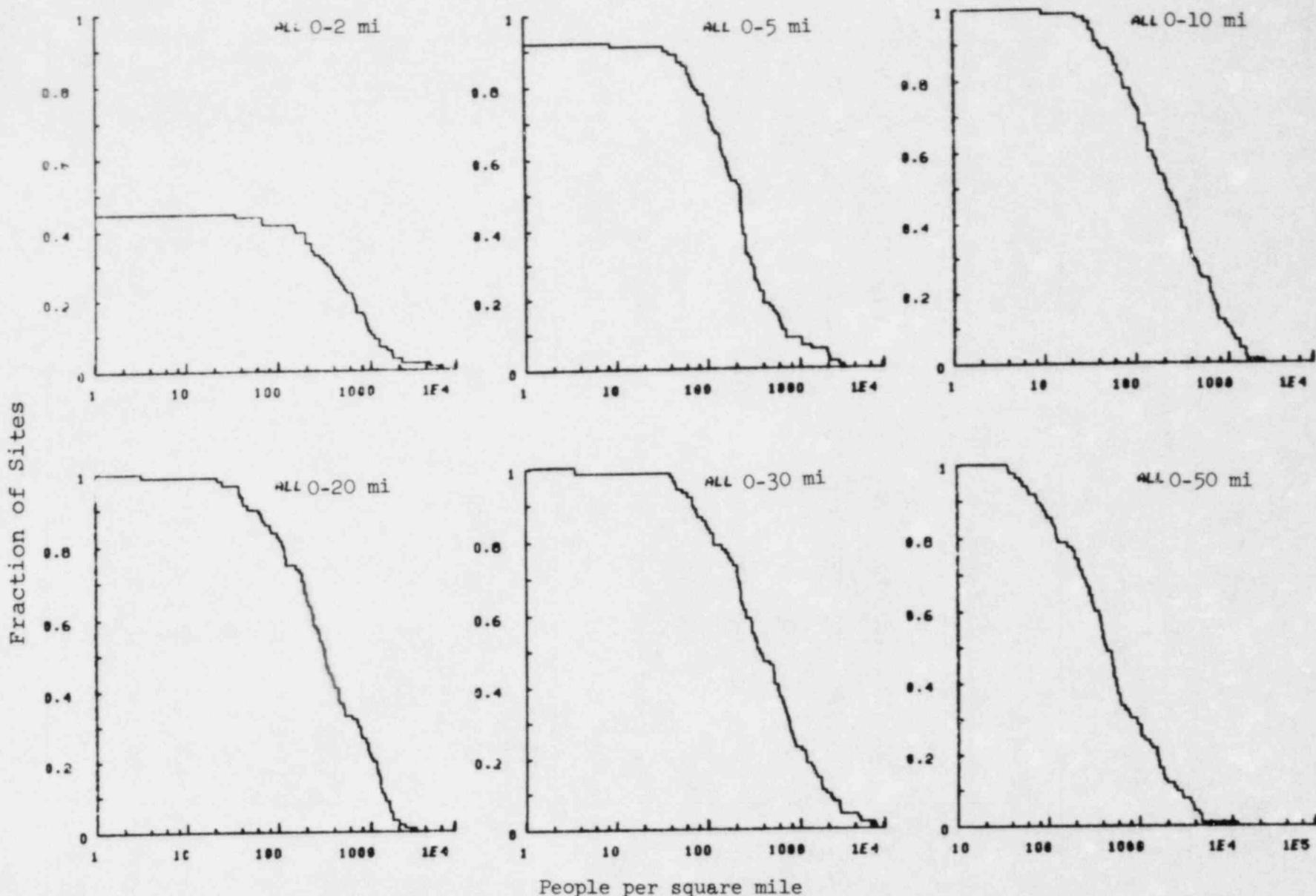


Figure 3-8. CCDFs of Population Density ( $\text{People}/\text{Mile}^2$ ) in the Most Populated 45 Degree Sector at 91 Sites for Six Radial Distances.

Table 3-2. Maximum, 90th Percentile, Median, and Minimum Population Densities (people/sq mi) for Seven Radial Annuli by Geographic Region and for All Regions Combined.

CCDF Value	Maximum						90th Percentile					
Region	NE	MW	S	W	SW	All	NE	MW	S	W	SW	All
Interval (mi)												
0-5	790	540	180	100	30	790	740	270	100	100	30	190
5-10	620	700	250	100	40	700	550	280	180	200	40	260
10-20	730	530	510	180	150	730	670	340	300	180	150	380
20-30	2000	1300	490	490	230	2000	1800	620	200	490	240	490
30-50	2500	1200	210	630	290	2500	770	940	160	620	280	660
50-100	880	440	180	310	90	880	820	430	110	310	90	420
100-200	350	190	160	150	40	350	280	170	110	150	40	190
CCDF Value	Median						Minimum					
Region	NE	MW	S	W	SW	All	NE	MW	S	W	SW	All
Interval (mi)												
0-5	100	60	30	20	10	40	0	8	0	0	0	0
5-10	130	60	80	30	20	80	6	4	8	2	7	2
10-20	170	90	70	60	30	90	40	9	10	0	7	0
20-30	180	120	100	50	40	110	50	9	8	2	7	2
30-50	400	100	80	40	130	110	50	20	10	20	30	10
50-100	360	130	80	50	40	90	20	10	30	10	20	10
100-200	170	110	70	30	30	80	20	30	8	9	6	6

Table 3-3. Maximum, 90th Percentile, Median, and Minimum Population Densities (people/sq mi) for Seven Radial Distances by Geographic Region and for All Regions Combined.

CCDF Value	Maximum						90th Percentile					
	Region						NE	MW	S	W	SW	All
<u>Interval (mi)</u>												
0-5	790	540	180	100	30	790	740	270	100	100	30	190
0-10	650	660	200	170	30	660	470	270	150	170	280	230
0-20	710	470	410	160	110	710	630	340	250	160	110	380
0-30	1500	850	380	320	180	1500	1300	460	290	330	180	420
0-50	2100	890	210	460	200	2100	880	830	200	460	200	530
0-100	760	370	170	350	100	760	750	350	130	360	100	440
0-200	350	210	160	120	50	350	340	200	100	120	50	290
<u>CCDF Value</u>	<u>Median</u>						<u>Minimum</u>					
<u>Region</u>	NE	MW	S	W	SW	All	NE	MW	S	W	SW	All
<u>Interval (mi)</u>												
0-5	100	60	30	20	10	40	0	8	0	0	0	0
0-10	120	60	70	30	20	70	4	10	6	3	7	3
0-20	210	90	60	50	30	90	30	10	20	1	8	1
0-30	230	120	100	50	30	110	50	20	10	2	7	2
0-50	320	120	90	50	90	120	50	20	20	10	20	10
0-100	330	120	80	70	70	90	80	10	40	10	30	10
0-200	290	130	80	40	40	90	50	30	20	20	10	10

Table 3-4. Maximum, 90th Percentile, Median, and Minimum Population Densities (people/sq mi) for the Most Populated 22.5° Sector within Four Radial Distances by Geographic Region and for All Regions Combined.

CCDF Value		Maximum				90th Percentile							
Region	NE	MW	S	W	SW	All	NE	MW	S	W	SW	All	
Interval (mi)													
0-5	4200	2000	950	450	320	4200	3500	2000	510	460	310	950	
0-10	2000	3800	1300	1600	140	3800	1300	1400	1000	1500	140	1000	
0-20	4500	3400	2600	800	860	4500	2000	2100	210	780	860	1800	
0-30	8700	5200	4000	1800	1600	8700	3700	3200	1300	1800	1600	2500	
CCDF Value	Median				Minimum								
Region	NE	MW	S	W	SW	All	NE	MW	S	W	SW	All	
Interval (mi)													
0-5	630	350	240	280	170	330	0	50	0	0	0	0	
0-10	750	220	280	150	70	270	40	40	60	20	50	20	
0-20	880	620	360	430	150	480	170	40	50	6	40	6	
0-30	940	800	430	290	120	550	110	60	40	5	70	5	

by administrative region of the site specific population data for population density seven annuli and seven radial distances, and for four radial distances of the most populated 22.5° sector.

In Section 2.7.4 the sensitivity of consequences to population distribution was examined using a number of hypothetical population distributions, all of which had average densities within 30 miles of the reactor of 750 people per square mile. Figure 3-4 shows that, within 30 miles of the reactor, only 4 of the 91 sites (4%) have population densities within that distance which exceed 750 people per square mile. Figure 3-8 shows that for the most populated 45° sector 30 of the 91 sites (33%) have population densities that exceed 750 people per square mile. Finally, Figure 3-6 and Table D1.4 show that for the most populated 22.5° sector 38 of the 91 sites (42%) have densities greater than 750 people per square mile.

Examination of the reactor site population density scatter plots for the five NRC administrative regions presented in Figures 3-9 through 3-11 shows that the densities within any region range across approximately two orders of magnitude and that between regions there is substantial overlap of ranges. Densities are largest in the Northeast and lowest in the Southwest; qualitatively the densities are ordered from largest to smallest: NE, MW, S, W, SW. Tables 3-2 through 3-4 confirm this qualitative ordering, although there are a number of exceptions (S and W are often inverted).

### 3.4 Time Dependent Trends

Figure 3-12 presents scatter plots by region of the year of site selection (the year in which a construction permit was granted was used as a surrogate for the actual year of site selection) for the 91 reactor sites examined in this study. Only four sites were selected prior to 1960, two each in the Northeast and the Midwest. Not until 1973 was a reactor site selected in the Southwest.

Because the years during which sites were selected are distributed over time quite differently by region, trends by selection year in the density of the population distributions surrounding reactor sites were also examined both by region and for all regions combined. Figure 3-13 presents plots of population density within

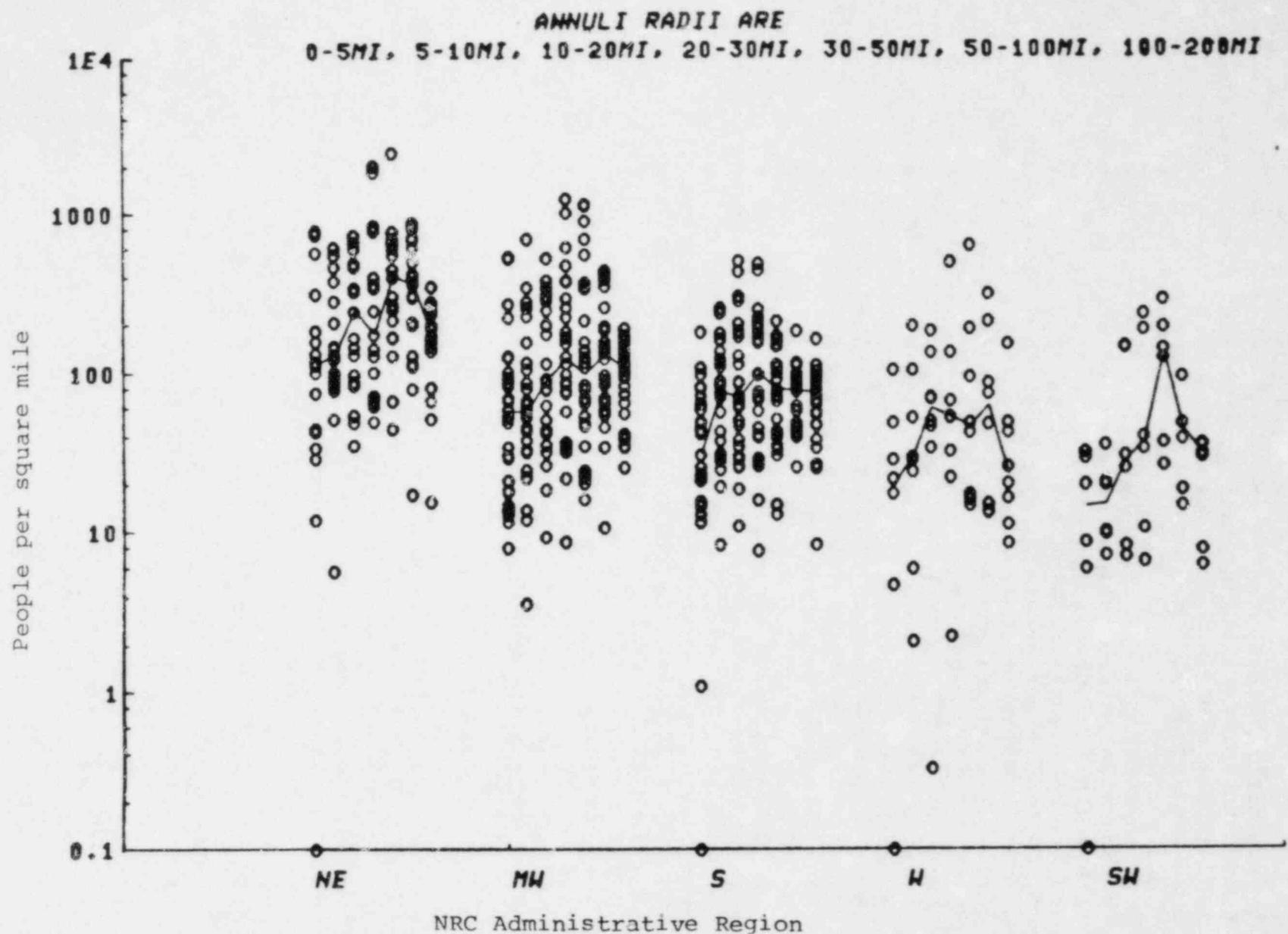


Figure 3-9. Population Density (People/Mile<sup>2</sup>) at 91 Reactor Sites by Geographic Region for 7 Radial Annuli: 0-5, 5-10, 10-20, 20-30, 30-50, 50-100, and 100-200 Miles (the Lines Connect Median Values).

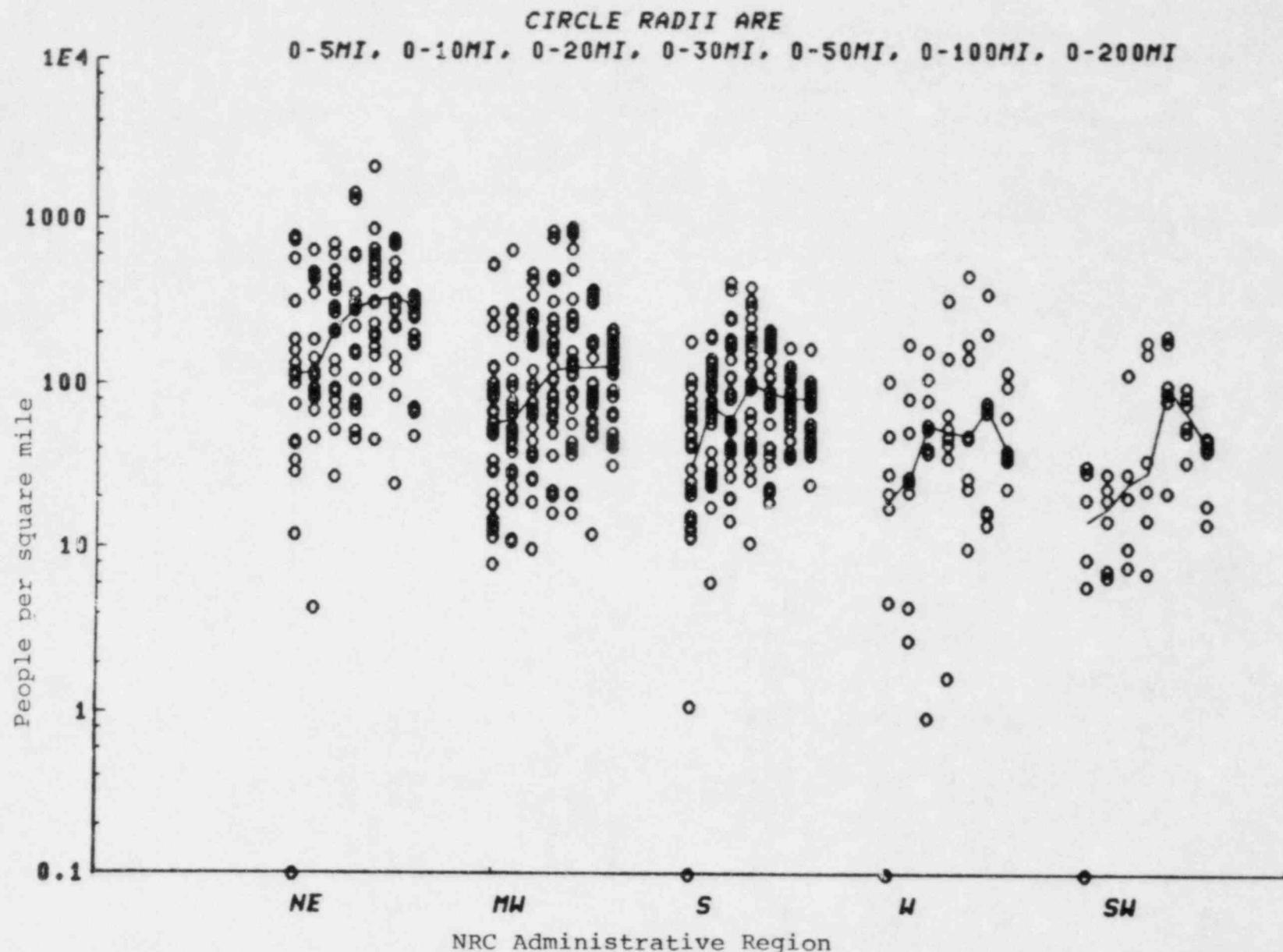


Figure 3-10. Population Density (People/Mile<sup>2</sup>) at 91 Reactor Sites by Geographic Region for 7 Radial Distances.  
0-5, 0-10, 0-20, 0-30, 0-50, 0-100, and 0-200 Miles.

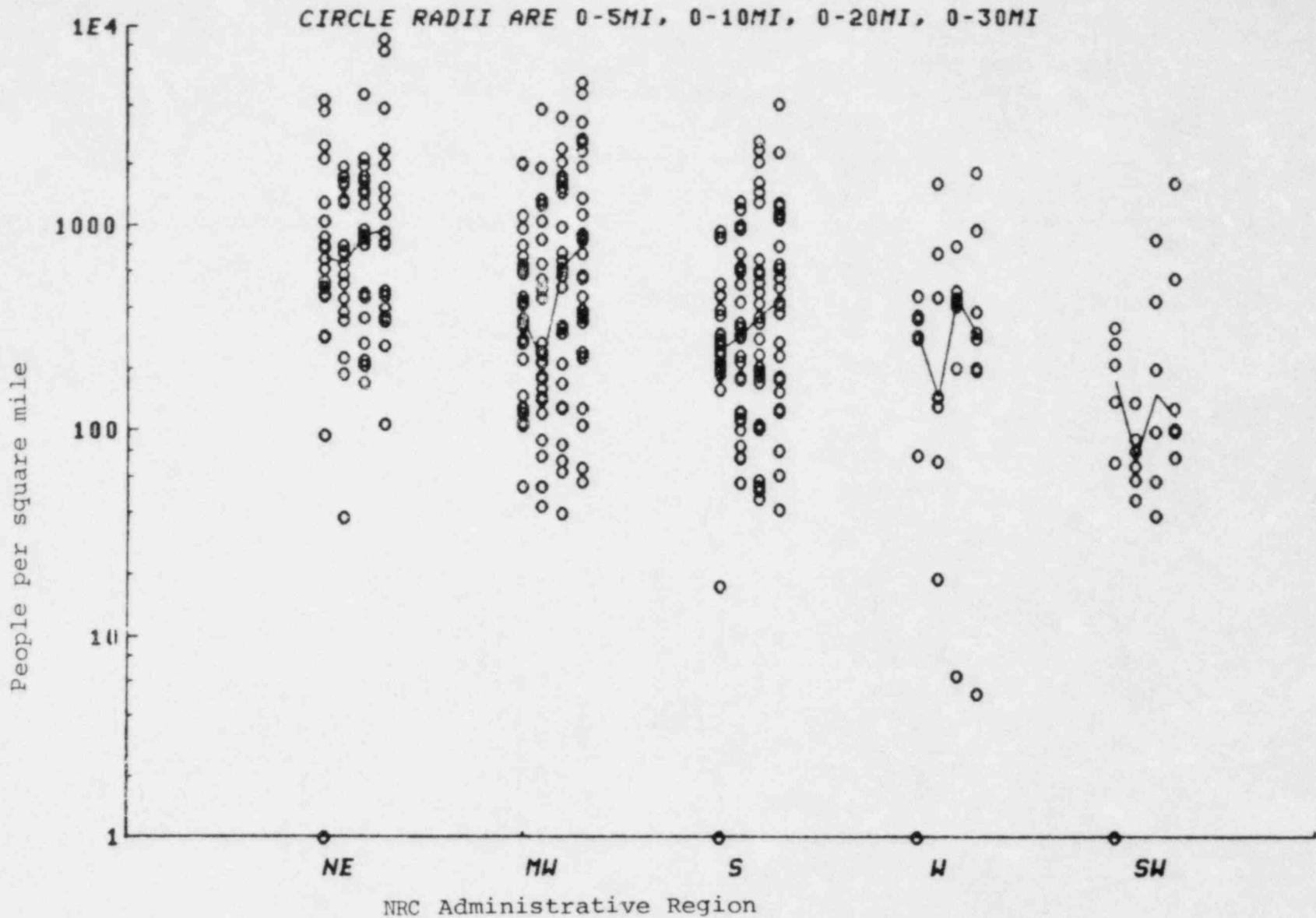


Figure 3-11. Population Density (People/Mile<sup>2</sup>) in the Most Populated 22.5 Degree Sector at 91 Sites by Geographic Region for 4 Radial Intervals: 0-5, 0-10, 0-20, and 0-30 Miles. (The lines connect median values).

30 miles of the site versus year of site selection, for each region and for all regions combined. The line on each plot is the least squares linear fit of the data. The slope of the line is the change in the logarithm of 30-mile population density with time. The lines for the Northeast, Midwest, and South have slopes which, given the scatter in the data points, are little different from zero (NE = -0.04, MW = -0.01, S = 0.03). Given the narrow time span and considerable scatter of the five Southwest site selection years, the slope of that plot (SW = 0.7), though substantial, is of no importance. Only for the West (W = -0.23) and to a lesser degree for all regions combined (All = -0.08) do the slopes of the plots seem important.

To better define the significance of the time trends displayed in Figure 3-13, an analysis of variance [3] of the logarithm-transformed population density data was performed. The analysis partitioned the variability in the data among four terms: one for the common time trend of all regions combined, one for unique time trends within each region, one for regional differences corrected for regional time trends, and a residual term for variability not attributable to either regional differences or time trends. The results of this analysis are presented in Table 3-5. In the table, the mean square value is obtained by dividing the sum of squares value by its number of degrees of freedom (number of independent terms in the sum of squares). Comparison of the magnitude of the mean square values indicates the relative importance of the three terms (mean square values large by comparison to the residual mean square value are useful in explaining the observed variability).

Table 3-5 Analysis of Variance

Source of Variation	Sum of Squares	Degrees of Freedom	Mean Square
Common time trend	11.2	1	11.2
Regional time trend	18.4	4	4.6
Regional differences corrected for regional time trends	7.1	4	1.8
Residual	<u>82.0</u>	<u>81</u>	1.0
TOTAL	118.7	90	

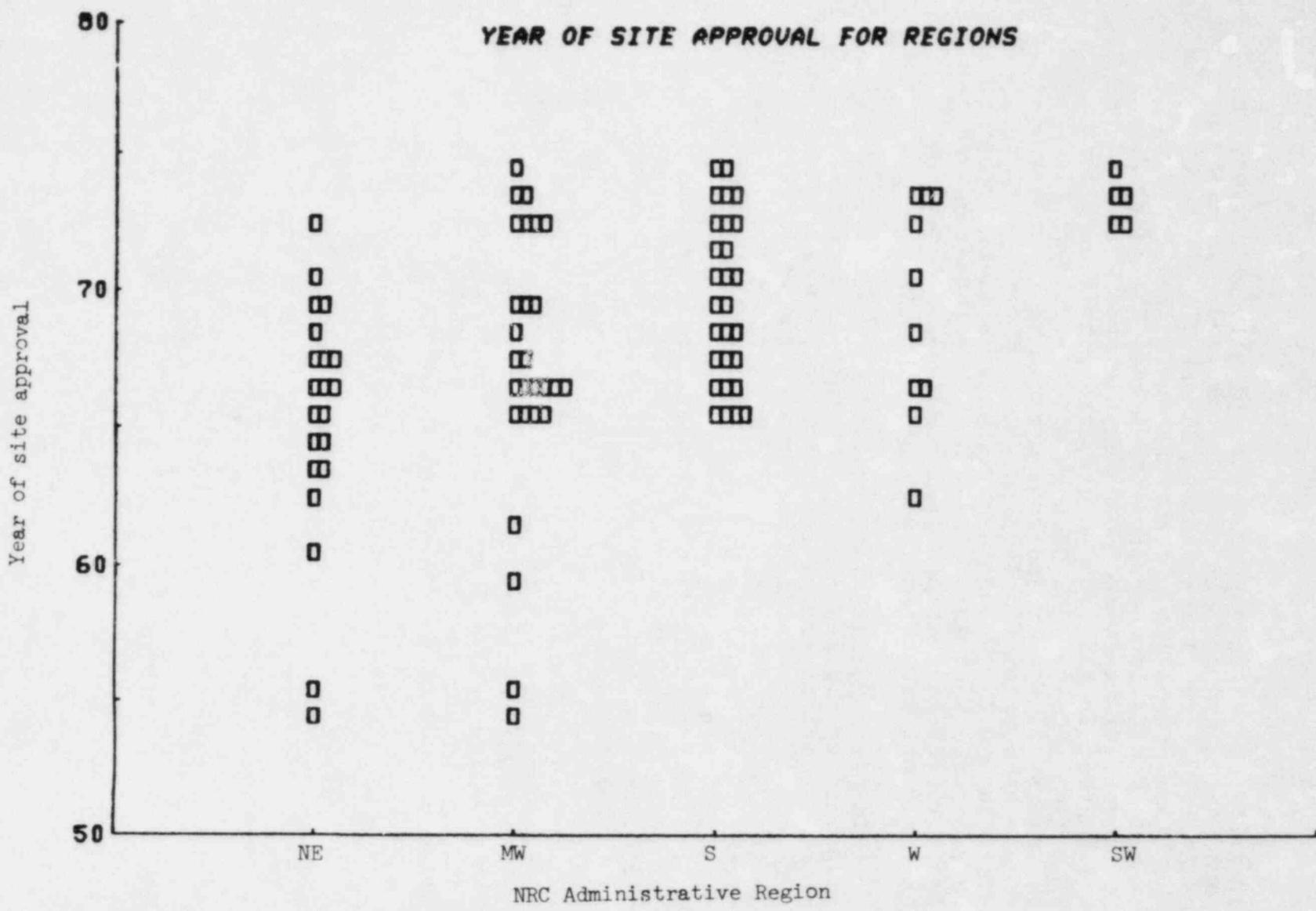


Figure 3-12. Scatter Plot by Region of Year of Site Approval.

3-23

POP DENSITY VS APPROVAL YEAR  
FOR 91 REACTOR SITES BY REGION

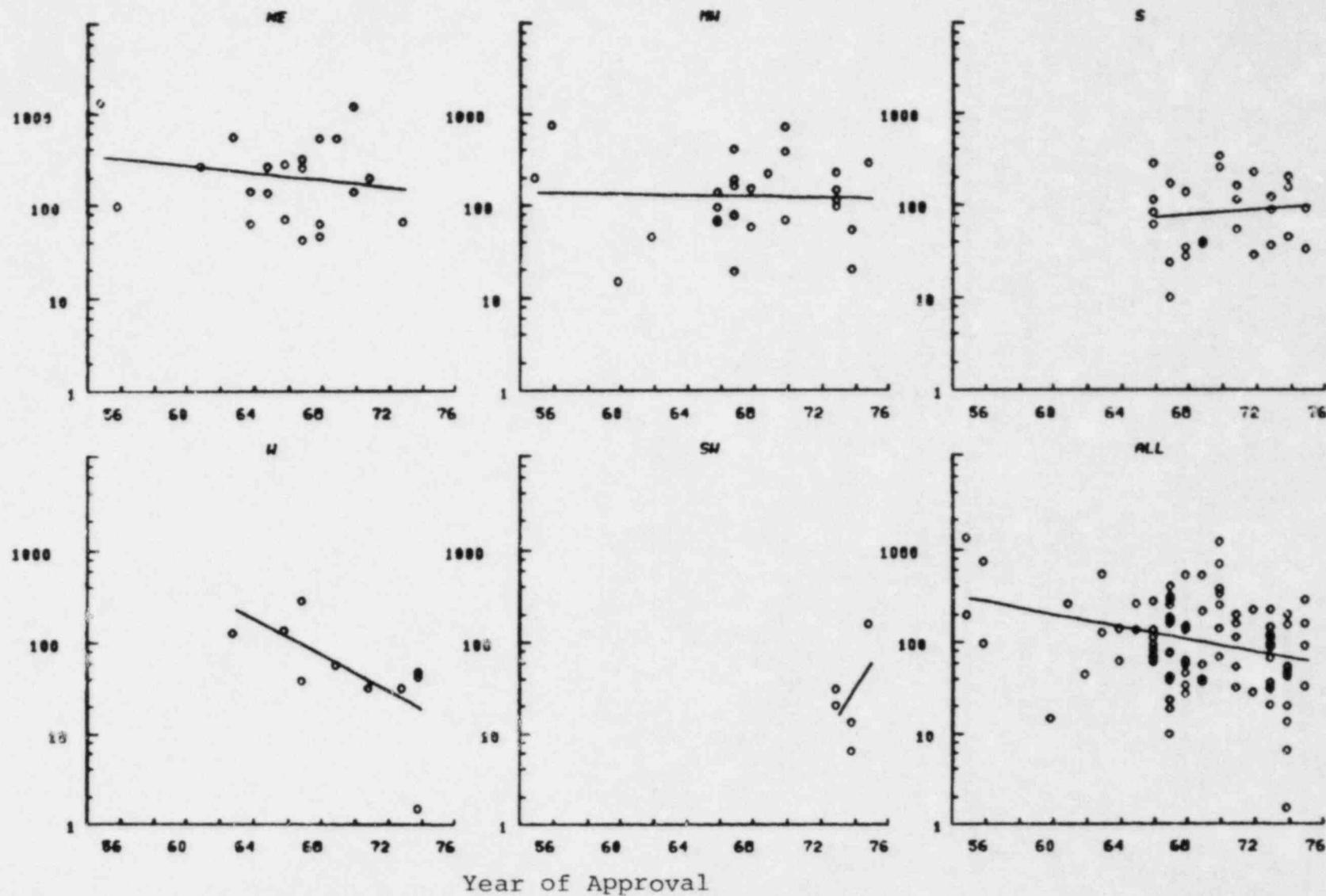


Figure 3-13. Plots of 30-Mile Population Density vs. Year of Site Approval. The lines are the least squares linear fit to the data. The slope of the lines are: NE = -0.04, MW = -0.01, S = 0.03, W = -0.23, SW = 0.7 and All = -0.08.

Table 3-5 suggests that the variability in logarithm-transformed site population data results principally from a common trend with time. Since this common trend is not strong (the slope of the linear correlation for all regions combined is only -0.08), its importance is unclear. It is possible that the trend toward less dense siting with time is (1) real, or (2) an artifact of the data. If the trend is real, it may result from some factor not addressed by this analysis (e.g., with the passage of time, suitable sites near cities become unavailable, so more remote sites are selected, which are necessarily less densely populated).

#### References for Chapter 3

1. Demographic Statistics Pertaining to Nuclear Power Reactor Sites, U. S. Nuclear Regulatory Commission, NUREG-0348, October 1979.
2. J. E. Kohler, A. P. Kenneke, and B. K. Grimes, The Site Population Factor: A Technique For Consideration of Population In Site Comparison, U. S. Atomic Energy Commission, WASH-1235, October 1974.
3. P. W. M. John, Statistical Design and Analysis of Experiments, Macmillan, New York, NY, 1971.

## 4. Site Availability Impacts

### 4.1 Introduction

The previous chapters of this report have examined the potential consequences of accidents at nuclear power reactors and the relationship of site population distribution to consequences. In addition, the population characteristics of current sites were examined. In order to reduce societal risk from siting, it is desirable to locate reactors in areas of low population density. This, of course, forces a trade-off between reduced risk and site availability. To evaluate more precisely the implications of this trade-off, this chapter reports on work performed by Dames and Moore, under contract to Sandia, to study the impacts of siting criteria alternatives on land availability. The study included consideration of the impacts on site availability of environmental factors (seismicity, topographic character, surface and groundwater availability, and restrictions due to regulations (wetlands, National parks, etc.)) as well as population.

### 4.2 Methodology

The study was performed in three steps: identification of issues affecting site availability, data collection, and analysis and display of data. The final step was performed iteratively, using Dames and Moore's Geographic Information Management System (GIMS), which manipulates geographical data in a grid cell format.

#### 4.2.1 Issues of Concern

A set of general siting issues was defined and used to identify and discriminate more suitable siting areas from less suitable ones. These issues cover a variety of demographic considerations and a diverse set of environmental siting criteria relating normally to costs.

Three issues were defined for population criteria. These are:

1. Stand-off Zones -- restrictions imposed by distance from urban centers of a particular size;
2. Population Density -- a measure of population density within a specified (circular) area; and
3. Angular Population Distribution -- a measure of the uniformity of population distribution within a specified (circular) area.

Four issues were defined for environmental criteria. These are:

1. Restricted Lands -- those areas in which the development of a nuclear power plant is difficult due to legal constraints or the predominance of wetlands;
2. Seismic Hardening -- the additional cost or difficulty of compliance with seismic design criteria; assumed to be measured by the maximum expected (50 year) horizontal ground acceleration expressed in fractions of gravity (g);
3. Site Preparation -- A relative measure of the ruggedness or topographic character expressed as an index which indicates the percentage of land with access and construction difficulty; and
4. Water Availability -- an index reflecting the relative cost of obtaining water for cooling.

The latter three issues were further combined to define an overall environmental suitability measure.

It is necessary to keep in mind that the goal of this study was to provide information regarding land availability and not to select sites on which to construct nuclear power plants. The defined issues were

analyzed on a nationwide basis to yield trends and indicate areas on a regional basis that could be considered for selection of power plant sites. Site selection analyses are generally conducted at a more specific scale and level of resolution. This is especially true for environmental criteria. Many site selection issues are related to physical features that are not geographically extensive, or consider factors that are important in the site planning process (which includes the precise location of the reactor and other plant facilities within the site). While these factors are important for specific site identification, they are not considered here.

#### 4.2.2 Data Structure Diagram

A data structure diagram describing the flow of data and information through the Dames and Moore study is presented in Figure 4-1. The diagram shows the sources and flow of information on the demographic and environmental issues as well as how these issues are combined to provide assessments of land availability for various siting criteria.

The data structure diagram is principally an aid to help conceptualize the entire impact analysis. For the most part, each box on the diagram represents a map that was created or a data file that could be displayed as a map.

#### 4.2.3 Display of Results

Results are presented as maps which display the impact of a criterion, which when printed on a transparent medium, can be overlaid on other maps to see the effect of composite criteria. Many of the results are displayed for the whole U.S. as well as for the northeastern section of the U.S. (the most populous region of the country).

In addition to maps, results are presented as tabulations of statistics for each state for various categories of information. Most of this statistical work was performed for comparisons of impacts of environmental suitability and population criteria and is described in Section 4.6.

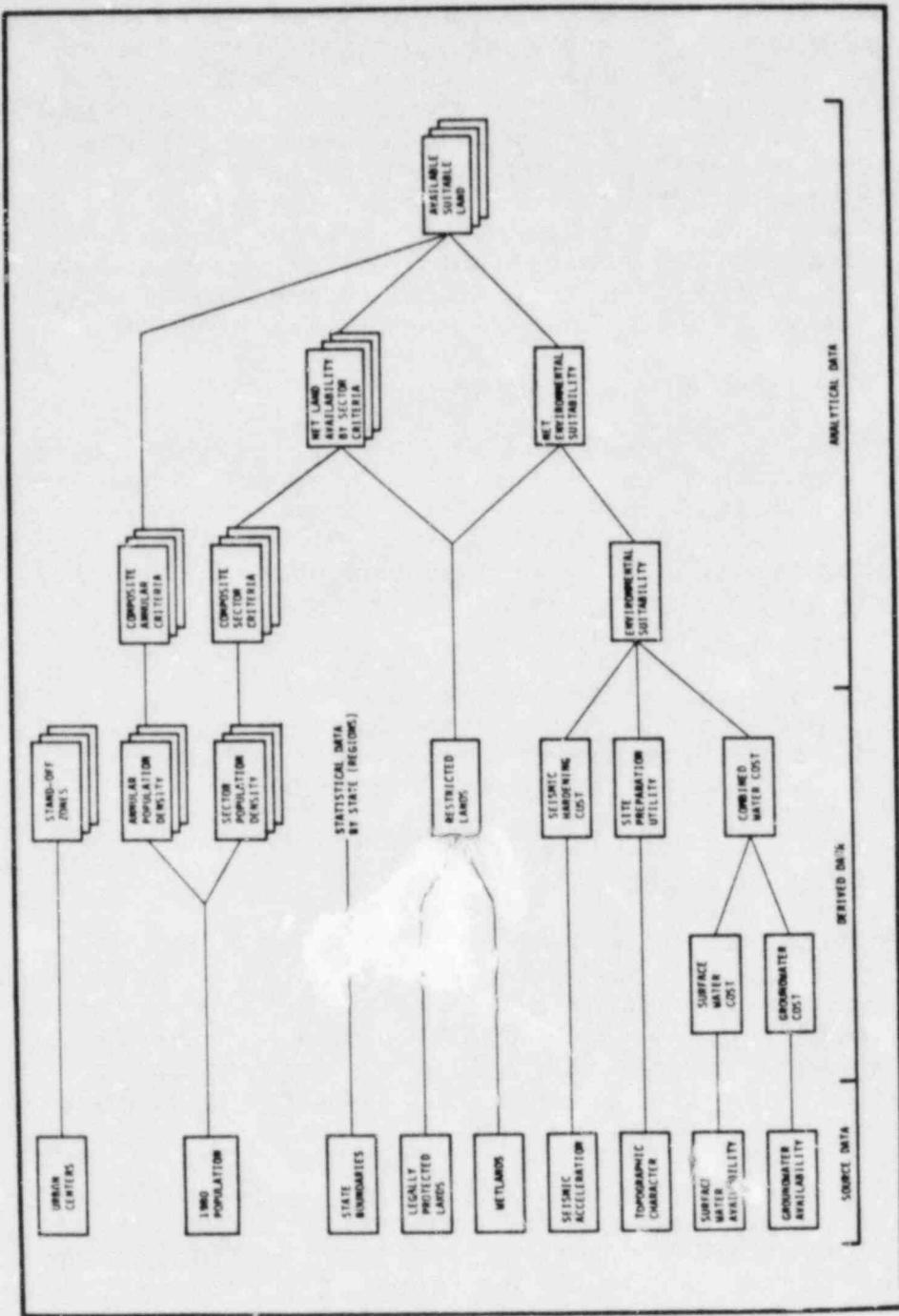


Figure 4-1 Data Structure Diagram for the Dames and Moore Study

#### 4.2.4 Geographic Information Management System

The Dames and Moore Geographic Information Management System (GIMS) was employed for this study. GIMS is a computerized system that provides planners with a comprehensive approach to recording, storing, manipulating and displaying the mappable information used in studies of this nature. The system provides a data base which can be readily updated, and allows evaluation of many alternative criteria that would otherwise be explored by time-consuming manual procedures.

#### 4.2.5 Mapping Approach

The mapping approach is a function of four related factors: (1) characteristics of the study area; (2) nature of the input data; (3) analysis methodology; and (4) desired output or display products. All of these factors are important in determining the base map and grid cell size and shape. Based on these considerations, the Albers Equal Area projection was chosen at a scale of 1:3,168,000 (1 inch = 50 miles) for digitizing input data and displaying output results. In addition, it was decided that data would be analyzed using a grid system consisting of square cells 5 km on a side (each cell represents 25 km<sup>2</sup> or 9.65 square miles). An artificial equal-area grid was placed on the base map by converting longitude and latitude coordinates into X and Y coordinates given in meters on the ground from an origin in the southwest corner of the map. Using grid cells of this size and shape and the Albers projection ensures that any maps produced from the analysis have several important characteristics:

1. Format is consistent with map projection and level of detail of input data;
2. A reliable sampling of population data (especially for the smaller area annuli) is maintained;
3. Computer time and cost are at an efficient level;
4. Maps are of manageable size while retaining important visible regional patterns;

5. Directional bias of analysis is minimal; and
6. Line printer graphics show area relationships truly, and thus, do not distort the implied impacts of criteria.

#### 4.3 Data Base

The data base consists of those data necessary for analyzing both demographic criteria and net environmental suitability. It includes:

1. Demographic Data
  - o Location and population of urban centers
  - o 1980 population estimated for enumeration districts
2. State and national boundaries
3. Restricted lands
  - o Legally protected
  - o Major wetlands
4. Seismic hardening
  - o Seismic acceleration
5. Site preparation
  - o Topographic character
6. Water availability
  - o Surface water availability
  - o Groundwater availability

##### 4.3.1 Demographic Data

Site availability impacts based on demographic characteristics considered both standoff distances from urban centers and surrounding population density and angular distribution. These analyses required two types of data.

#### 4.3.1.1 Urban Centers

Data concerning urban centers were extracted from NUREG-0348 [1]. This publication categorizes urban centers into three groups: those centers with population in excess of 25,000 people, greater than 100,000 people, and greater than 200,000 people. The data were updated with information provided by the NRC to include population figures for urban centers greater than 250,000 people, greater than 500,000 people and greater than 1,000,000 people.

Populations for these urban centers were identified geographically by latitude and longitude coordinates. The degrees of longitude and latitude were converted into X and Y coordinates which corresponded to the same geographic grid that was applied to the Albers base map as discussed in Section 4.2.5. This conversion prepared the data for eventual use in the production of maps showing how much land would be available after imposing population center standoff zone criteria. The analysis of standoff zones is discussed in Section 4.5.1.

#### 4.3.1.2 Population Density

To calculate population density, analyze various criteria, and ensure that the results are reliable in the face of changing national population trends, it was necessary to obtain the most up-to-date and detailed population figures. Figures from the 1980 decennial census were not available in time for use in this study. In their place, estimates for 1980 population were used. Data were supplied by the National Planning Data Corporation (Ithaca, New York). While it is difficult to give an estimate for the percent error, it is believed that the data are quite reliable, especially when individual data points (which correspond to centroids of enumeration districts or block groups) are taken in groups of 4 or 5. This is typically the case in this study. It is especially true for all areas except the most remote and rural. Thus, the data are considered reliable for its intended function, the analysis of population data around the more urbanized areas of the country.

The 1980 population estimates were obtained formatted on magnetic tapes with population figures geographically referenced by latitude and longitude. As with urban center data, the degrees of longitude and latitude were converted into X and Y coordinates on the Albers grid system. This process prepared the demographic data base for analysis of population density. The analysis is discussed in Section 4.5.2.

#### 4.3.2 State Boundaries

Using the Albers base map at 1:3,168,000 scale, all coastlines, international boundaries, and state boundaries were digitized. The area within each state was assigned a unique code to identify it for further use. The state boundaries map file allows analysis or display of results on an individual state basis or by any group of states.

#### 4.3.3 Restricted Lands

The nature of certain areas of the country causes them to be protected or restricted from development. Two types of lands were considered as restricted: legally protected lands and existing wetlands.

##### 4.3.3.1 Protected Lands

The Energy Reorganization Act of 1974 (Section 207) states that national forests, national parks, national historic monuments and national wilderness areas should be excluded from consideration as potential nuclear energy center sites. While this study did not deal with nuclear energy centers, it is reasonable to consider such lands as protected from the siting of a single nuclear power plant, regardless of a national policy on this matter. Utility industries tend to avoid such areas because of the possibility of time consuming and costly legal battles. The following areas were considered to be protected:

- o National Parks
- o National Forests
- o National Monuments
- o National Wilderness Areas

- o National Grasslands
- o National Wildlife (Game) Refuges
- o National Recreation Areas
- o National Seashores
- o State Parks
- o State Forests
- o State Reserves/Refuges
- o State Recreation Areas
- o Military Reservations
- o Indian Reservations

Three different map sources were used to obtain the locations of these protected lands. The United States base map utilized in this study (compiled by the U.S. Geological Survey, 1965) was used to extract the location of national parks, forests, monuments, wildlife refuges, and Indian reservations. Sectional sheets at a scale of 1:2,000,000 from the National Atlas [2] were used to update the boundary information for the above protected lands as well as to obtain the location of national recreational areas. Because of the relatively small size of protected state areas and some protected national areas, a screening process was used for certain types of land, rather than identifying and digitizing every one. Because this study dealt not with site selection but with the general patterns of land availability, a minimum size screen of 100 square miles was used for the following types of areas: military reservations, national grasslands, state parks, state forests, state monuments, state reserves and refuges, and state recreational areas. Information for these types of lands was obtained from the 1980 Rand McNally Atlas, as this was the most detailed, up-to-date and uniform source of information.

#### 4.3.3.2 Wetlands

Besides the above legally protected lands which would be restricted from development either on the basis of national policy or avoidance on the part

of the utility industry, certain types of environmental restrictions might be imposed as well. For this study, one such environmental constraint was applied -- namely, the location of major wetlands. It is the policy of the Water Resources Council to ensure the protection of wetlands from adverse impacts and degradation [3].

No uniform nationwide data base exists regarding the location of major wetlands. After consideration of several approaches to defining the extent of wetlands in an efficient manner, a source was found to satisfy the needs of this study. The 1:2,000,000 scale sectional sheets of the National Atlas [2] were used to outline the extent of major wetlands. At this scale, only major wetlands can be shown. A comparison of these source data with more detailed map data shows that some of the wetland boundaries have been generalized and most wetlands less than 60 square miles were probably not shown on the sectional sheets.

The locations of both protected lands and wetlands were digitized into separate map files. Each of the 15 different types of protected lands was given a unique identifying code to allow individual consideration of each type of protected land. The two map files were added together to produce a map file called restricted lands (Figure F1 in Appendix F). The restricted lands file was later added to the individual environmental issue map files as well as the environmental suitability map file to produce maps showing the location of restricted lands, and conversely, the net availability of land.

#### 4.3.4 Seismic Hardening

There are generally three major factors to be considered in the seismic evaluation of a nuclear power plant site:

1. Fault Rupture Hazard -- primarily a siting problem;
2. Dynamic Soil Stability (liquefaction) -- both a siting and a design problem; and
3. Strong Ground Motion (vibratory) -- both a siting and design issue.

Siting requirements are specified by the NRC [4] and the evaluation of a site (for design purposes) is based on the additional cost imposed by the site-related conditions. Although a detailed site qualifications study would require the careful consideration of all three factors, their evaluation generally requires effort far beyond the scope of this study. However, after careful consideration of their overall impact, a methodology was developed for a coarse screening process which reflects the overall impact of these factors. The data necessary to evaluate the potential problem from the standpoints of rupture hazard and dynamic soil stability were not uniformly available throughout the United States. For this reason, seismic hardening was evaluated solely on the basis of vibratory ground motion.

Strong ground motion criteria are determined by the postulated Safe Shutdown Earthquake (SSE) which is the largest possible event on the controlling seismogenic feature, which could be a capable fault (not necessarily the closest one) or a tectonic province. The SSE is determined on the basis of historical earthquake data (seismicity) and detailed investigation of the length and capability of nearby faults, according to procedures specified by the NRC [5]. The plant must be able to survive such an earthquake in a manner which will not result in the release of radioactivity in excess of stated limits. An additional design requirement is imposed by the Operating Basis Earthquake (OBE) which is commonly defined as having a peak acceleration equal to 1/2 that of the SSE. The plant must be designed so that it can continue to operate during and after an OBE: alternatively, none of the structural or mechanical components may be stressed beyond their elastic limit by the OBE.

While the detailed investigations required for the determination of the SSE for each 5 km by 5 km grid cell were clearly beyond the scope of this study, it was possible using available data to probabilistically evaluate the relative severity of the strong ground motion hazard in the study area and consider costs of seismic hardening. This was accomplished using probabilistic studies of seismic risk prepared by Algermissen and Perkins [6] and the Applied

Technology Council (ATC) [7] and supplemented with information from a U.S. Geological Survey professional paper [8]. The ATC map represents an adaptation of a comprehensive analysis by Algermissen and Perkins. The map shows accelerations in bedrock expressed as a fraction of gravity. The combination of these three sources resulted in the seismic acceleration source data map illustrated in Figure F2, Appendix F.

The map shows the horizontal acceleration (expressed as a fraction of gravity) in rock with a 90 percent probability of not being exceeded in 50 years. According to Algermissen and Perkins:

"Certain facilities such as nuclear power plants may require design adequate for accelerations with exceedance probability no larger than 0.5 percent in 50 years. For structures for which very low exceedance probabilities are appropriate, it is clear that this source map indicates only a relative idea of the hazards -- the design motions will be high for much smaller exceedance probabilities. In those regions where seismicity is lower than in California, the accelerations shown on this map vary with return period according to the very approximate rule: the level of motion doubles as the return period increases by 5 (exceedance probability decreases by 5)."

This rule was used to modify the values on the source data map. The exceedance probability was decreased by a factor of 5 -- from 10 percent to 2 percent -- and the acceleration values were doubled. Another iteration of this process decreased the exceedance probability from 2 to 0.4 percent and again doubled the acceleration values. The new values were then considered to be four times the values expressed in Figure F2. Thus, the data in the modified map file became consistent with the notion of using a 0.5 percent exceedance probability for nuclear power plants (as suggested by Algermissen and Perkins).

The seismic risk source data file was further adapted by interpolating between the contour levels to develop a more continuous distribution of seismic risk (horizontal acceleration). The continuous distribution was desirable from a siting standpoint,

so that sites falling on either side of a dividing contour would not appear to have greatly differing seismic requirements. (The contours of the source map do not generally have any geological significance which would warrant such sharp distinctions.) It is still recognized that the absolute resolution of the source data map is probably no more precise than the contour intervals given. However, the relative ranking of areas for reactor sites is probably representable to the finer resolution implied by the interpolation.

The general impact of seismic design requirements is assumed to be proportional to the specific cost of the additional design and construction features required to satisfy the seismic design requirements. In NUREG/CR-1508 [9], seismic hardening costs were calculated and shown on a graph relating the Safe Shutdown Earthquake expressed as a fraction of gravity to the estimated cost differential in millions of dollars. The cost curve used in this study is shown in Figure 4-2.

The map shown in Figure F2, Appendix F, indicates that the lowest acceleration contour is equal to 0.05g. Remembering that the exceedance probability was twice decreased by a factor of 5 (thereby twice doubling the ground motion), the lowest acceleration contour may now be considered equivalent to 0.2g. By applying Stevenson's cost information to the modified probabilistic seismic acceleration information, a cost surface that shows the additional cost of seismic hardening was generated.

Using the curve shown in Figure 4-2, acceleration values between 0.2g and 0.6g (0.05 and 0.15 on the source map) were assigned costs ranging from \$23.7 million to \$55.5 million. Acceleration values of less than 0.2g were assigned a cost of \$23.7 million (the same as for 0.2g). This was because nuclear power plants in the U.S. are designed for an SSE of 0.2g, although it may be possible to build them more cheaply. For acceleration values greater than 0.6g, it was felt that there is no reasonable way to accurately estimate the increased costs of seismic hardening. Rather than assign a cost, they were labeled "inestimably high".

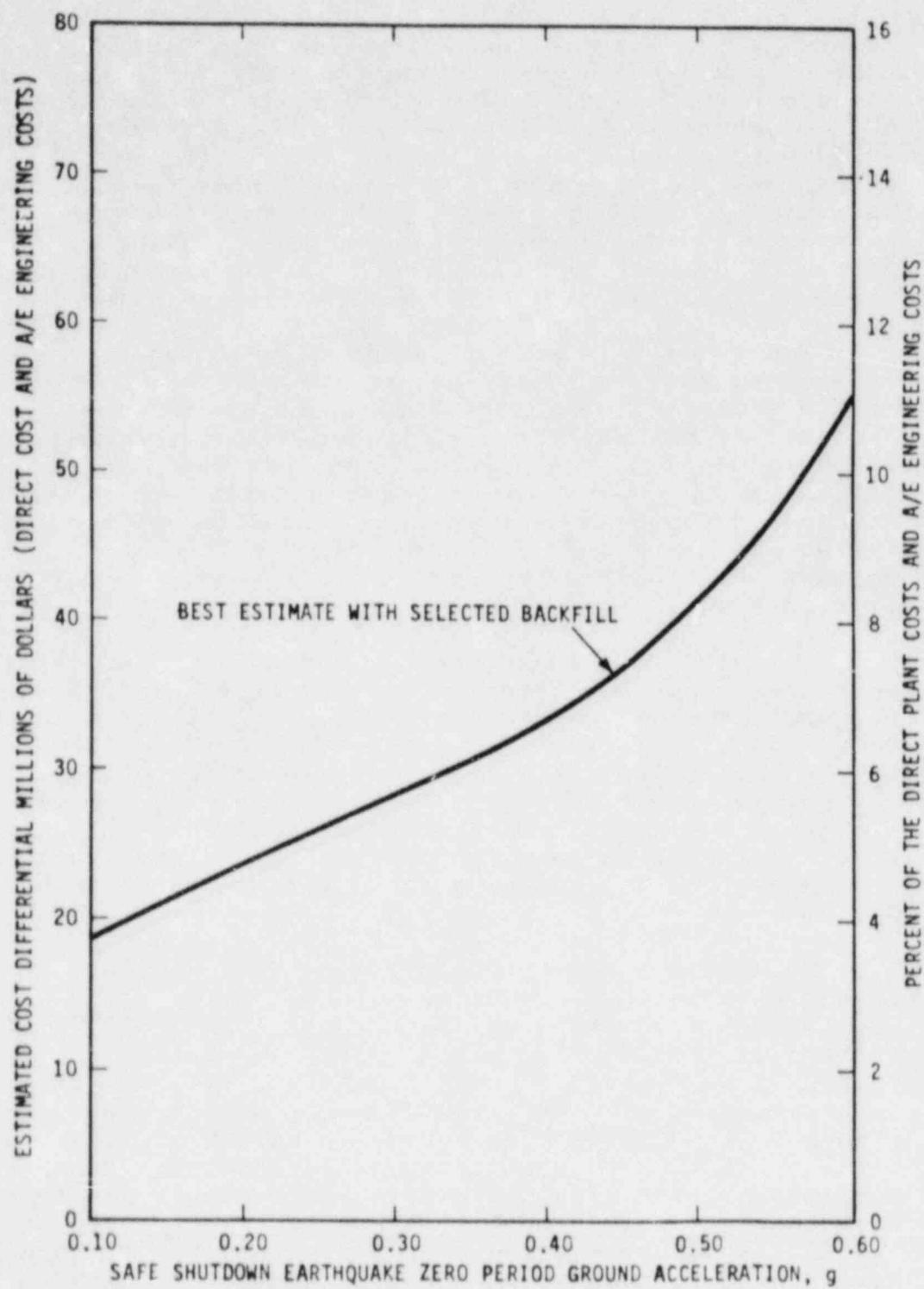


Figure 4-2. Cost Increase As A Function Of Seismic Load for Nominal 1100-MWe Nuclear Power Plant (1977 Dollars)

The costs derived from Figure 4-2 (1977 dollars) were next converted to 1980 dollars yielding a low of \$31.5 million and a high of \$73.9 million. To calculate the cost of seismic hardening that was considered as "additional", the design-basis value of \$31.5 million was subtracted from all the costs. This resulted in a range of costs of 0.0 to \$42.4 million. The graphic display of seismic hardening cost is shown in Figure F3, Appendix F.

#### 4.3.5 Site Preparation

An increase in slope or ruggedness of terrain translates directly into increased cost for construction. This includes the difficulty that may be encountered in excavation for foundations, construction of access roads where low grades are required (due to the transport of large components such as the turbine or pressure vessel), and finally, measures that must be taken to mitigate environmental disturbances such as control of run-off and erosion from cut slopes.

To evaluate the impact of topographic character on site preparation cost over a large regional area, a general index that indicates both the steepness of slopes and the areal extent of such slopes was sought. Such data was found in a paper by E. H. Hammond [10] and his map which was adapted and found in the National Atlas [2]. Regions on the map are characterized by the percentage of their area which is classified with a topographical gradient of less than 8 percent slope (gently sloping). The 8 percent slope is not a critical threshold value for land utilization. It does, however, indicate a value beyond which movement of vehicles becomes impeded, and in general, construction and operation becomes more difficult.

The smallest region delimited and given a classification has an area of about 800 square miles. Smaller areas are omitted or absorbed into the adjacent region that they most resemble. With this level of resolution, it is possible that sites suitable for building a nuclear power plant exist within the area characterized by even the highest proportion of rugged terrain. However, at this regional level of analysis, these special conditions are not practically observed. Because not only site ruggedness but the ruggedness of the access route

for implacement of heavy components affects the construction costs, the analysis of site preparation costs is based solely on the general indication of topographic character, as defined by the data. Figure F4, Appendix F, is a map showing the source data with grey tones implying preparation costs. Four terrain classifications are shown: regions with less than 20%, 20 to 50%, 50 to 80%, and greater than 80% gently sloping (less than 8% slope).

#### 4.3.6 Water Availability

Cooling system cost has become a major component of total power plant cost. Several factors are involved in determining cooling system cost: the type of cooling system -- mechanical draft wet towers, natural draft wet towers, cooling ponds, or once through cooling; climatic temperature distributions; existing priorities for use of available water; and restrictions such as wild and scenic rivers. While a detailed analysis of these factors is beyond the scope of this study, a methodology was developed to present a general picture of water availability and the cost involved in its use. Sources of both surface water and groundwater were mapped and costs were determined for each. The two map files were then overlaid, and a map was produced showing the least cost of available water.

##### 4.3.6.1 Surface Water

Hydrological implications of water consumption by nuclear power plants have been discussed by Giusti and Meyer [11]. Many existing power plants are located on sites next to streams and draw their water directly from those streams without provisions for significant storage. In siting plants along rivers one must consider the periods of low flow when the impact on the water resources of total water consumed in the cooling process is at a maximum. This consideration is especially significant for plants that do not use cooling ponds with a large amount of storage capacity. In light of this, it is important to have reliable estimates of the low flow of streams from which plants can draw cooling water. According to Giusti and Meyer there are several reasons for estimating these flows:

1. Safety -- the regulatory staff of the U.S. Atomic Energy Commission (1972) in reference to a safety analysis report for nuclear power plants states:  
"Estimate the probable minimum flow rate resulting from the most severe drought considered reasonably possible in the region as such conditions may affect the ability of the ultimate heat sink to perform adequately.";
2. Standards -- most states have issued standards regarding the maximum permissible mineral concentration in surface water to be used for cooling. As is well known, this concentration is at a maximum at a low flow period because the flow consists of groundwater discharge which is normally more concentrated mineralogically than surface water. Additional concentration of the stream flow mineral content is brought about by transpiration which is also at a maximum during low flow periods;
3. Ecology -- maximum ecological impact on fresh water biota can occur on some streams during low flow periods if the mineral concentration exceeds certain limits or if the flow is abruptly reduced by withdrawal at power plants. Furthermore, the withdrawal entails loss of biota by physical entrainment on the intake screens or by physical injury on passage through the water pumps; and
4. Plant Operation -- the conditions described above may be such as to force the shutdown of the plant, with contingent costs and loss of revenue to plant operators and loss of service to consumers. While this may be considered an acceptable operational rule under exceptional circumstances, say once in 10 years, it becomes a serious problem of misdesign when recurring more often, say once every year.

Stankowski, Limerinos, and Euell [12] have examined the low water flow in the United States to provide information regarding potential sources of cooling

water. They have prepared a map which identifies those streams for which the average 7-day low flow with a recurrence interval of 10 years is at least 300 cubic feet per second (cfs). (The 7-day, 10-year low flow or 7Q10, is the average low flow that occurs over 7 consecutive days with a probable recurrence of 10 years.) Their map shows those stream reaches that: (1) have a 7Q10 of at least 300 cfs, or (2) could furnish a sustained flow of at least 300 cfs if storage were provided. For their study, 300 cfs was selected as the needed flow in the stream on the assumption that many states will not permit more than 10 percent of the dependable flow to be withdrawn for a consumption use. Ten percent of 300 cfs equals 30 cfs which is the amount of water that might be considered necessary to cool a 1,000 MWe nuclear power plant if cooling towers, sprays, or ponds are used. The requirement of 30 cfs for cooling is in agreement with the information produced by Giusti and Meyer [11]. The Stankowski, et al., map was digitized and used as a source map to show surface water availability.

To extend the utility of surface water information, the map file showing surface water availability was converted into a map showing surface water cost. First, an estimate was made of the dollar per mile pumping cost to move surface water. These costs were estimated for each of the four terrain types characterized for site preparation (Section 4.3.5). Both an initial capital cost and a 30-year operating and maintenance cost were estimated. In addition to the pumping cost, a penalty cost was added for those streams that required the use of reservoirs in order to sustain a 7Q10 of 300 cfs. Based on this information, a computer model was used to calculate, for each cell, the cost of obtaining surface water as a function of pumping costs over a variety of terrain and the potential use of a reservoir. The model determined the least of the cost alternatives for supplying surface water to a cell. The cost information was mapped and is shown in Figure F5, Appendix F. There are eight equal interval levels between zero and \$300 million. Costs above \$300 million were grouped together -- amounting to about 10 percent of the study area. This grouping at the high cost end allows regional differences in the more reasonable range of costs to be displayed.

#### 4.3.6.2 Groundwater Availability

Groundwater is an important source of cooling water in many parts of the country. Characteristics of groundwater can vary quite dramatically within a small region. Despite this, an attempt was made to locate a source of information that would satisfy the broad scale requirements of this study. Using the USGS Water Supply Paper 1800 [13], and supplementing this with such maps as the Hydrologic Investigations Atlas [14], Tectonic Map of North America [15], and Shaded Relief of U.S. [16], major regions and subregions of the country were mapped as source data. Although variability exists within any one of the regions or sub-regions, regions do show differences regarding their characteristics of quality, quantity, depth to water, and required well field size.

Based on these characteristics, cost information was applied to the map data. Both capital costs and 30-year operating and maintenance costs were calculated for each of the delimited areas on the basis of dollars per well. To obtain the equivalent of 30 cfs from any of the generalized aquifers, it would be necessary to sink several wells. The required number of wells was calculated by dividing 30 cfs by the expected yield per well of the given aquifer. Multiplying this number of wells by the cost per well resulted in the cost associated with bringing 30 cfs to the surface from any of the generalized aquifers. It was observed that several of the generalized aquifer areas require well fields which are too large for practical use. For these areas, groundwater was considered to be unavailable in a practical sense. For reasonably sized well field areas, the cost of collecting the water from numerous wells and bringing it to a single point was estimated. For each of the groundwater regions, the two costs -- that of bringing the water to the surface, and that of collecting the water from a well field, were added together. The cost data were then mapped as is shown in Figure F6, Appendix F.

#### 4.3.6.3 Combined Water Costs

Using the cost information for both surface water and groundwater, a map file was created which indicated

the cost of obtaining cooling water using the least expensive alternative. To do this, the two map files -- surface water costs and groundwater costs, were compared on a cell-by-cell basis. For every cell, the lowest cost value was saved and placed into another map file. This was called "combined water cost" and the map is shown in Figure F7, Appendix F.

#### 4.4 Environmental Suitability Analysis

In order to evaluate the impact of demographic criteria on land availability it was necessary to first establish a base of available land. This base was constructed from the protected area and environmental consideration data bases. The environmental factors were combined by dividing utility functions for each factor, and then summing the utility values within each cell. The protected areas were then overlaid on this data.

##### 4.4.1 Individual Site Availability Issue Assessments (Utility Functions)

To evaluate the suitability of each potential site area, each of the siting issues was first evaluated independently. This evaluation was accomplished by defining a utility function for each issue such that the characteristics of a specific site area could be translated into a value on a defined suitability scale. This was a numeric scale ranging from 1 to 9, where 1 was the lowest level of suitability and 9 was the highest.

###### 4.4.1.1 Seismic Hardening Cost Utility Function

The issue of seismic hardening was assigned a utility function on the basis of additional hardening costs as discussed in Section 4.3.4. Table 4-1 shows the data categories of seismic hardening costs and their corresponding utility value.

A map of the seismic hardening utility function was produced and is shown in Figure F3, Appendix F. (This is the same map used to show the cost of seismic hardening.)

TABLE 4-1  
SEISMIC HARDENING UTILITY FUNCTION

<u>Cost in Millions of 1980 Dollars</u>	<u>Utility Value</u>
0.0 to 6.1	8 (high suitability)
6.1 to 12.1	7
12.1 to 18.2	6
18.2 to 24.1	5
24.1 to 30.3	4
30.3 to 36.4	3
36.4 to 42.4	2
No reasonable estimate	1 (low suitability)

#### 4.4.1.2 Site Preparation Utility Function

Actual dollar costs associated with site preparation could not be located as source data. However, discussions with authorities in the construction of nuclear power plants as to how the topographic character of the landscape might affect the site preparation costs have allowed for the assignment of the utility values to terrain classifications which were discussed in Section 4.3.5. These are shown in Table 4-2.

A map of the site preparation utility function was created and is shown in Figure F4, Appendix F. (This is the same map used to show the site preparation source data.)

TABLE 4-2  
SITE PREPARATION UTILITY FUNCTION

<u>Topographic Character (percent of area that is gently sloping*)</u>	<u>Utility Value</u>
>80 percent	8 (high suitability)
50 to 80 percent	5
20 to 50 percent	2
<20 percent	1 (low suitability)

\*Gently sloping means 8 percent slope.

#### 4.4.1.3 Water Availability Utility Function

Utility values have also been assigned to data representing the cost of obtaining cooling water. Based on this cost information (described in Section 4.3.6), costs in excess of \$300 million were grouped together and assigned the lowest utility value. For costs less than \$300 million utility values were assigned on the basis of 8 equal intervals as shown in Table 4-3.

TABLE 4-3  
WATER AVAILABILITY UTILITY FUNCTION

Combined Water Cost (in millions of 1980 dollars)	Utility Value
0 to 37.5	9 (high suitability)
37.5 to 75.0	8
75.0 to 112.5	7
112.5 to 150.0	6
150.0 to 187.5	5
187.5 to 225.0	4
225.0 to 262.5	3
262.5 to 300.0	2
>300.0	1 (low suitability)

A map was prepared showing the water availability utility function and is shown in Figure F7, Appendix F. (This is the same map used to show the combined water cost.)

#### 4.4.2 Site Availability Issue Overlay

Using the utility functions, each issue map was translated into a partial suitability map where each potential site area was represented by a utility value. These individual suitability maps are represented in Figures F3, F4 and F7. They are considered partial suitability maps because each includes only one siting issue. They were combined into a composite suitability map by adding the individual map files together. It was felt that the reconnaissance nature of this study, as well as the broad scale representation of environmental data, did not justify a more sophisticated manipulation of the files. For this reason, the three maps were overlaid -- each with an equal importance weighting.

The addition of the three utility value map files resulted in a map file with values ranging from 4 through 25 -- each value having a different frequency of occurrence. Maintaining the relationship that high values represented the most suitable land, the distribution of the composited utility values was divided into five intervals. The intervals were selected to include equal land areas. This resulted in five categories or levels of environmental suitability -- each level representing 20 percent of the data base. The restricted lands file was then added to the composite utility value file. A color-coded version of a map produced from this combined file was supplied to NRC.

#### 4.4.3 Environmental Statistics

Analysis of the impact of various siting criteria on land availability was accomplished in two ways: creation of maps to visually show these impacts and production of statistics to quantify the impacts. The maps concerning environmental factors have been presented elsewhere in this section. To quantify the impacts of various siting criteria, tables were prepared which used the data files created during the visual or map analysis. Statistics regarding the amount of area in each data category were computed for each of the 48 states.

For each of the three environmental issues -- seismic hardening costs, site preparation costs, and water availability costs -- a table was prepared that shows the amount of land in each of the categories that was represented by a utility value. Two additional tables were produced: one for surface water cost and one showing the five different levels of composite environmental suitability. These statistics are shown in Tables F1.1 through F1.5, Appendix F. The numbers in each column indicate the amount of land in the specified category. The area is shown in square miles as well as percent of the total state area.

## 4.5 Demographic Analysis

As discussed in Section 4.2, three issues were defined as relevant to population criteria - stand-off zones, population density, and angular distribution. Stand-off zones are restrictions on distances from urban centers to nuclear plant sites. Population density is a measure of the persons per square mile within a specified (circular) area surrounding a site. The population density calculations were mapped as single data files or in combination with other annular densities to produce composite population criteria maps. Angular distribution restrictions are limitations on the permissible population within one or more  $22\frac{1}{2}^{\circ}$  sectors surrounding a site.

### 4.5.1 Stand-off Zones

To study the impact of restrictions imposed by distance from urban centers, stand-off zone maps were prepared. As discussed in Section 4.3.1, populations and locations were provided for urban centers of a variety of sizes. The location of these urban centers was indicated by a single latitude/longitude coordinate which was converted to a Y and X coordinate corresponding to grid cells on the Albers base map. Urban centers were grouped according to their size: greater than 25,000, 100,000, 200,000, 250,000, 500,000 and 1,000,000 people. For each grid cell in the study area, its distance from the nearest urban center of a particular size was computed. This resulted in six separate data files. These files were converted into maps by specifying a threshold distance at which a cell would be considered either suitable or unsuitable for siting a nuclear plant. Based on the above data, thirteen such stand-off maps were produced. The maps produced are indicated in Table 4-4 and presented in Figures F8.1 through F8.13, Appendix F. The maps illustrating stand-off zones from the three largest cities were created only for the northeastern U.S.

Maps of stand-off zones are quite self-explanatory. There is a direct relationship between the stand-off distance and the amount of area that is constrained by the specified criteria.

TABLE 4-4  
STAND-OFF ZONES

<u>Size of Urban Center</u>	<u>Mapped Stand-Off Distance (in miles)</u>
25,000	5, 10
100,000	10, 15, 25
200,000	25, 30, 40, 50, 100
250,000	12.5
500,000	18
1,000,000	25

An example of these maps is shown in Figure 4-3.

#### 4.5.2 Population Density

A wide variety of population distribution criteria based on density surrounding a prospective site were studied for their impact on land availability. Densities were calculated for both circular areas and annular areas. As described in Section 6.3.1, population source data was identified by a latitude and longitude coordinate system. These coordinates were converted into the Y and X coordinates compatible with the Albers grid base map. This raw data were then converted into a set of map files giving the population density of an area a given radius centered on each cell. Maps of varying thresholds were produced from these files. The matrix shown in Figure 4-4 indicates all of the map files that were produced regarding population density. An "X" in a box means that the map files were produced for both the total US and the northeastern window. An "NE" in a box means that the map file was produced only for the northeast. An example of these maps is shown in Figure 4-5. Maps representative of the variety of population densities are shown in Figures F9.1 through F9.26, Appendix F.

An understanding of the spatial relationships produced by various criteria can be gained by comparing some of the maps. Figure F9.5 shows the areas constrained by a density threshold of 100 people per square mile in the 0-5 mile circle. Figure F9.8, concerning the same circle employs a density threshold of 500 persons per square mile. It is obvious that

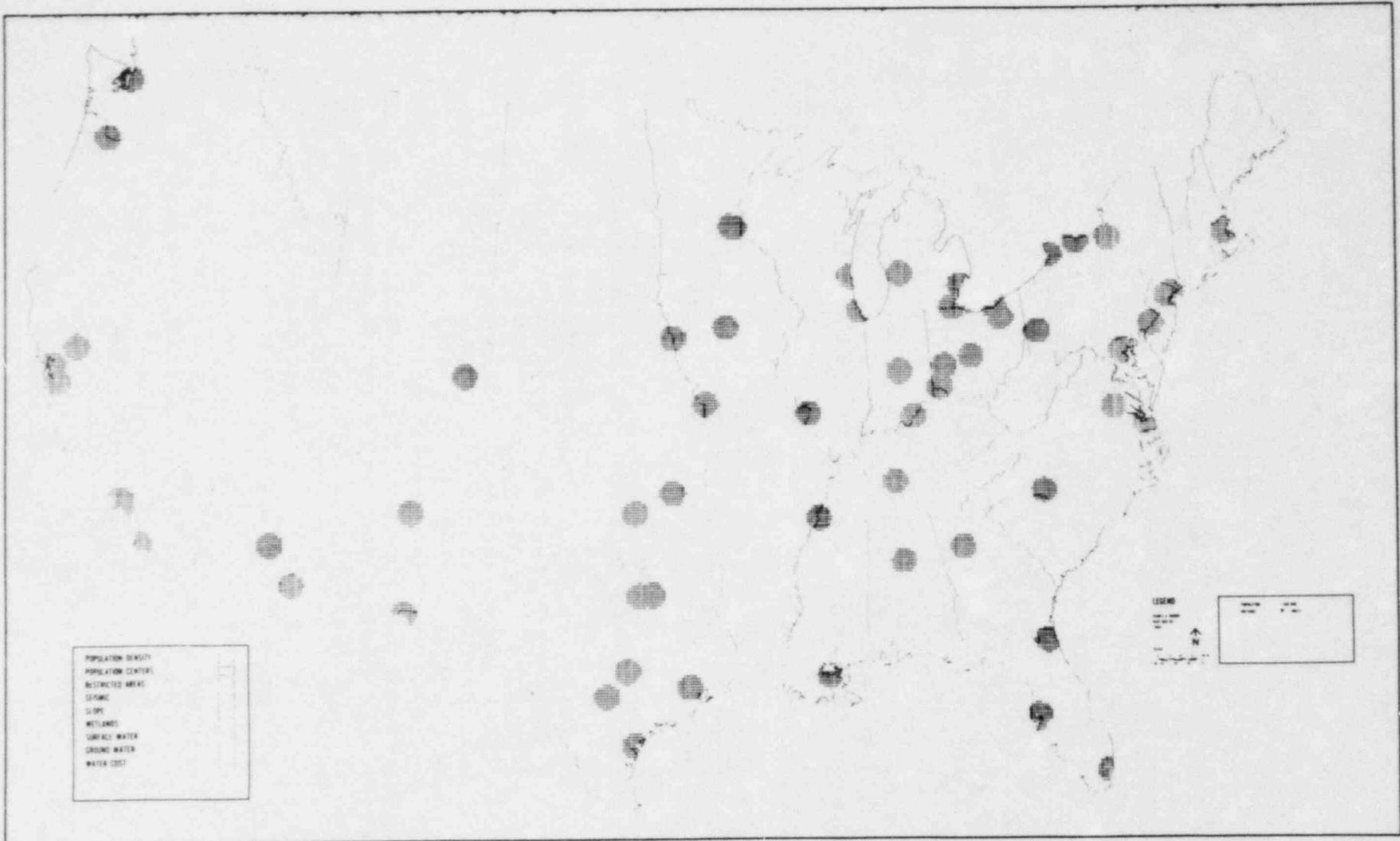


Figure 4-3. Example of Standoff Zone Maps.

ANNULAR RADIUS IN MILES	DENSITY PEOPLE/SQUARE MILE						
	$\geq 100$	$\geq 150$	$\geq 200$	$\geq 250$	$\geq 350$	$\geq 400$	$\geq 500$
0-2	x		x				NE
0-5	x	x	x	x	x		
0-10	x	x	x	x	x	x	
2-20		x					
0-30				x	x	x	
5-10	x	x	x	x	x	x	
5-20					x		
10-20			x	x	x	x	
20-30				x	x	x	
30-50			x	x	x	x	

Figure 4-4. Annular Population Density Data Files



Figure 4-5. Example of Annular Population Density Data Maps.

every area constrained in Figure F9.8 is also constrained in Figure F9.5. If the size of the annulus remains constant, the area constrained using a higher density threshold is always completely contained within the area constrained by a lower threshold. In addition, the use of a lower density threshold as in Figure F9.5, constrains a much greater portion of the suburban and rural land areas.

Spatial differences are also noted through a comparison of circle size while maintaining a constant density threshold. For example, compare Figure F9.8, which shows the areas constrained by a 500 people per square mile density threshold within the 0-5 mile circle, with Figure F9.14 which applies the same threshold to the 0-30 mile circle. Use of the larger radius tends to constrain only the urban and some suburban areas of major cities. None of the rural or smaller urban areas are constrained and the impacts look similar to those which result from stand-off zone criteria.

Another interesting spatial effect of the demographic criteria can be seen on any of the maps in which the annulus is defined using an inner radius greater than zero. In these cases, the annulus surrounding a prospective site is shaped like a ring rather than a circle and the effect is that the shape of some of the constrained areas is also that of a ring. The occurrence of this type of pattern depends upon the specified density threshold in conjunction with the limits of the annulus and the population data itself. For example, Figure F9.23 indicates the amount of land constrained if a criterion of 500 people per square mile in the 20-30 mile annulus were applied. Note that in the St. Louis area the land surrounding the city would be constrained -- but not the land comprising the city. St. Louis' land area is small enough so that a relatively small population is located between 20 and 30 miles of the city center, and yet the city population is large enough to cause the density threshold to be exceeded in the surrounding areas. Chicago, on the other hand, occupies an area large enough so that grid cells at the city center are within 20 to 30 miles of significant population and the pattern of constrained land is solid.

A comparison of the St. Louis area between Figure F9.23 and Figure F9.21, which employs the same density threshold within the 10 to 20 mile annulus indicates not only the absence of a ring structure but also a shrinking of the extent to which land is constrained using the smaller annulus. The pattern of the area constrained near Chicago remains solid in both figures; however, both the extent and amount of land increase with increasing annular radii. Thus, if the density threshold remains constant, the extent from the central city of the criterion's effect increases with increasing annular radius. However, the total amount of land constrained may not increase accordingly due to the possible elimination from constraint of the central city.

#### 4.5.3 Composite Population Densities

When using a criterion of the form of less than 500 people per square mile from 2 to 30 miles, it is possible for a cell to satisfy that criterion, while it doesn't satisfy a 500 people per square mile criterion out to only 15 miles. This occurs when there is a dense population pocket surrounded by low density areas. In order to pinpoint areas for which this occurs, a new set of criteria were developed which restricted population to a given density for all radii from an inner radius to an outer radius. Thus, for the example of 500 people per square mile from 2 to 30 miles, the new criterion is satisfied only if the population density is less than 500 people per square mile from 2 to R miles, where R takes every value from 2 to 30.

Evaluating population density for every radius from the inner radius to the outer radius is impractical in practice, so an approximation is used.

Using the example of mapping any cells that exceed the 500 persons per square mile threshold for the 2-30 mile annulus, density calculations were made for 6 portions of the 2-30 mile annulus and were then composited. First, any cell that exceeded the 500 persons per square mile threshold within the 2-3 mile annulus was recorded. Next, unsuitable cells in the 2-4 mile annulus were recorded and unsuitable cells

in the 2-5 mile annulus were recorded. This process was repeated for the 2-10 mile annulus, 2-20 mile annulus, and the large 2-30 mile annulus. These 6 individual files were then added together, creating a file in which a cell that was shown to be unsuitable in any of the 6 was also considered unsuitable for the 2-30 mile composite annulus. In this manner, data files were created for the 2-30 mile composite annulus for the following densities.

250 persons/square mile  
500 persons/square mile  
750 persons/square mile  
1,000 persons/square mile  
1,500 persons/square mile

Example maps for the northeast are shown in Figures F10.1 thru F10.4, Appendix F.

Besides creating a composite map file for a particular annulus (such as 2-30 miles) and a particular density (such as 500 persons per square mile), another type of composite was created. This consisted of two separate annuli -- each with its own given population density threshold. For example, as discussed above, 6 individual data files were added together to create the 2-30 mile composite annulus. Now, a different annulus with a different population density threshold was added to the 2-30 mile composite annulus. Two maps were created in this manner and are shown in Figures F11 and F12, Appendix F. Each map shows cells that are considered unsuitable for the 2-30 mile composite annulus (with density of 500 persons per square mile) as well as for the 0-2 mile annulus for population densities of either 100 persons per square mile or 250 persons per square mile. In addition to these two mapped data files, other complex composite files were created. Some of these were used for statistical analyses in combination with the environmental criteria. (These statistics are discussed in Section 4.6). The six complex composite data files which were created are indicated in Table 4-5. The numbers in the columns underneath the two annuli represent population density figures (persons/mile<sup>2</sup>).

TABLE 4-5  
COMPLEX COMPOSITE POPULATION DENSITIES

<u>0-2 Miles</u>	<u>2-30 Miles (Composite)</u>
(1) 100	250
(2) 100	500
(3) 250	500
(4) 250	750
(5) 500	750
(6) 500	1500

#### 4.5.4 Sector Population Density

To this point in the chapter, any potential demographic criteria addressing population density were analyzed using what might be termed a uniform density distribution. Criteria were stated in terms of the number of persons that would be allowed in an area of a given size -- that is, population density -- and the shape of the area was always circular. Using a circular area allowed relatively dense concentrations of population to exist provided that the total number of people within the circle did not exceed a stated limit.

Results of reactor accident consequence calculations indicate that certain risk characteristics depend strongly on the maximum number of persons within any given direction sector (see Section 2.7.4). Therefore, criteria regarding the maximum allowable population within sectors in addition to total population surrounding a site were considered. The impact on land availability was examined for alternative sector criteria and compared to the impact of uniform density criteria.

Sector criteria were stated in terms of allowing up to a fraction of the allowed number of people to be located in any sector of a particular width. For example, a sector criteria might be stated: no more than 1/6 of the people allowed by a uniform density of 500 persons per square mile can be located in any 45 degree sector at distances within 3 miles of a site.

The impact of sector criteria was investigated with regard to several variables. The parameters were:

- o Distance: Radii of 2, 5, 10, 20, and 30 miles
- o Sector width: 22.5, 45.0, 90.0 degrees, and 360 degrees (for uniform density)
- o Fraction: 1/16, 1/8, 1/4, 1/3, and 1/2 the population allowed by uniform density
- o Density: 250, 500, 750, and 1500 persons per square mile

Population counts were determined within 2, 5, 10, 20, and 30 miles of potential sites (grid cells) and within sector widths of 22.5, 45.0, and 90.0 degrees. The maximum number of persons found in a sector of a stated width and for a particular radius was recorded. For example, investigating a circle of radius 10 miles and using a sector width of 22.5 degrees, the circle was divided into 16 sectors. The number of people was determined within each sector and the maximum of the 16 counts was recorded. This procedure of determining the maximum count was undertaken 15 times -- once for every combination of sector width (3) and radius (5).

Alternative criteria were then applied to the count data on the basis of allowing a certain fraction of the total number of people allowed within the circle to be located in any sector. The total number of people allowed in a circle is dependent upon the radius (for area) and the density that is allowed. For this sector analysis, the previously established densities were analyzed -- 250, 500, 750, and 1500 persons per square mile and five radii were used -- 2, 5, 10, 20, and 30 miles. For 0-2 miles, only one density was used as a part of every criteria -- namely, 250 persons per square mile. To calculate the allowable population threshold out to 5, 10, 20, and 30 miles for each of the densities, the area from 2 miles to r miles (radius) was multiplied by the density and the product added to the threshold for 0-2 miles with its 250 persons per square mile density. For example, at 20 miles using density 750 persons per square mile, the threshold equals:

$$\begin{aligned}
 & (\text{Area of 0-2}) \times 250 + (\text{Area of 2-20}) \times 750 \\
 & = (12.57 \times 250) + (\text{Area of 2-20}) \times 750 \\
 & = (3142 + 933075) \\
 & = 936,217 \text{ people}
 \end{aligned}$$

Using only one density (250 persons per square mile) for 0-2 miles and four densities for the other four distances resulted in 17 separate thresholds. These thresholds were used not only for uniform density criteria analyses but also for calculating the fractional thresholds applied to sector population distributions. Thus, if a criterion was stated that no more than 1/4 of the people allowed by a uniform density of 750 persons per square mile within 20 miles would be allowed in a sector, the threshold would be  $936,217 \times 1/4 = 234,054$  people.

Being consistent with previously computed impacts, the impacts for sector criteria for any particular density or fraction were composited to 30 miles. That is, sites exceeding a threshold at 2 miles were recorded and saved into a map file. Sites exceeding a threshold at 5 miles were also recorded and stored into a map file, as were all sites for 10, 20, and 30 miles. Finally, all five map files were merged resulting in a file that showed sites constrained by any one or more of the thresholds. Spatially, it was found that any criteria at smaller radii tended to eliminate sites in rural areas as well as in cities but only out to their edge. Criteria applied at larger radii tended to eliminate cities and large areas around their edges (similar to a "standoff" criteria) but allow local population concentrations in rural areas. By compositing criteria for all five radii, both urban and rural population concentrations were evaluated for their impact on availability of potential nuclear sites. Additionally, it was found that the effects of sector criteria occurred in the same areas as affected by annular density criteria.

Sector criteria were of interest in regard to their impact on land availability above and beyond that already affected by uniform density criteria. To depict and quantify this information, tables were created to show the amount of land available for siting

in each state if a particular sector criterion was established. The information is shown in Tables F2.1 through F2.24, Appendix F. Each table shows the impact of alternative fractional criteria along with the uniform density criteria on land availability. All of the fractional and uniform density criteria have been composited to 30 miles by adding the individual impacts of a criterion at 2, 5, 10, 20, and 30 miles.

Each table considers a unique combination of allowable annulus population density and sector width. The four population densities and three sector widths resulted in 12 combinations. Twenty-four tables were created as each of the 12 combinations was tabulated using two different formats. Tables F2.1 through F2.12 are formatted such that the numbers in the columns represent the amount of land that is uniquely constrained by the specified criteria.

The columns are arranged so that total magnitude of constrained land decreases from left to right. As an example, Table F2.1 indicates the impacts of alternative fractional criteria applied to 22.5 degree sectors using a density threshold of 250 people per square mile for both the 0-2 mile and 2-30 mile annulus. The leftmost column "Available Land," shows the amount of land available for siting if the criterion stated in the adjacent column is applied; that is, no more than 1/16 of the population allowed in the annulus at a density of 250 people per square mile can be located in any 22.5 degree sector of the annulus. The criterion stated in the second column of these 12 tables always represents the most constraining fractional criterion.

The rightmost column, "Restricted Lands," shows the amount of land that is constrained because it is either legally protected or a major wetland. No demographic criteria affect these numbers.

The numbers in each of the middle columns show the amount of land that is uniquely constrained by the specified criterion which is above the total amount previously constrained by the criteria in all of the columns to the right. In Table F2.1, for example, the column labeled "Uniform Density" shows for Alabama values of 5,703 square miles or 11.0 percent of the state area. This is the area that would be constrained

by applying a uniform (annular) density criterion and it is additional to the area already constrained by restricted lands (2,075 square miles or 4.0 percent). Thus, the application of this particular uniform density criterion in Alabama would constrain a total of 7,778 square miles or 15.0 percent of the state area if no sector criteria were applied. The next column to the left, "1/2 Allowable Pop.," would add another 2,355 square miles or 4.5 percent of constraint if a sector criterion were stated that no more than 1/2 of the total population allowed by a density threshold of 250 people per square mile in both the 0-2 mile and 2-30 mile annuli could be located in any 22.5 degree sector. Similarly, using a criterion of allowing up to 1/3 of the allowable uniform density population to be located in a single sector, would constrain an additional 6,388 square miles or 12.3 percent of the land area. The total constrained land in this case would be 16,521 square miles or 31.8 percent of the state area. Conversely, 68.2 percent (100 minus 31.8) of the land would be available for siting.

To more clearly summarize the information that shows the availability of land when specific sector criteria are applied, Tables F2.13 through F2.24 were created in a different format than the previous 12 tables. On these tables, the numbers in the columns show the amount of land available for siting if the specified criterion is applied. For example, Table F2.13 indicates that 68.2 percent of the land in Alabama would be available for siting if a criterion of allowing up to 1/3 of the population (allowed by a uniform density criteria using a density threshold of 250 people per square mile in both the 0-2 mile and 2-30 mile annuli) to be located in any 22.5 degree sector. This number agrees with the one produced in the above example regarding Table F2.1. The column labeled "Uniform Density" indicates land availability when no sector criteria are applied. The column "No Pop. Criteria" shows the amount of land available when only restricted lands are considered a constraint.

#### 4.6 Impact Analysis

Analysis of the impact of various siting criteria on land availability was accomplished in two ways: creation of maps to visually show these impacts, and production of statistics to quantify the impacts. Many

of the maps produced have already been reviewed in other sections of this chapter. All maps were produced on a transparent base enabling them to be overlaid. This capability allows creation of complex composite population criteria maps. In addition, these population criteria maps can be overlaid on the color-coded environmental suitability map.

To quantify the impacts of various siting criteria, tables were prepared which used the data files created during the visual or map analysis. For a particular subject, whether environmental or demographic, statistics regarding the amount of area impacted were computed for each of the 48 states. Fifteen tables were produced which were grouped into three different types: environmental criteria, environmental suitability levels versus selected population cases, and population criteria versus individual environmental suitability levels.

#### 4.6.1 Environmental Statistics

For each of the three environmental issues -- seismic hardening costs, site preparation costs, and water availability costs -- a table was prepared that showed the amount of land in each of the categories that was represented by a utility value (see Section 4.4). Two additional tables were produced: one for the surface water cost, and one showing the five different levels of composite environmental suitability. As discussed earlier, these statistics are shown in Tables F1.1 through F1.5, Appendix F.

#### 4.6.2 Impact Comparisons

The overlay of transparent maps provided a quick look at potential land availability. A map containing five levels of environmental suitability along with a sixth level showing restricted lands, when overlaid with a variety of population criteria, produces numerous groupings of data. To present these data in statistical form, a method was devised to keep each table simple enough to be understood, while retaining a large amount of information.

First, five population cases were defined on the basis of complex composite criteria. These population cases are shown in Table 4-6. The numbers in the columns underneath the 0-2 and 2-30 mile annuli represent population density figures.

TABLE 4-6

<u>Population Case</u>	<u>0-2 Miles</u>	<u>2-30 Miles (Composite)</u>
1	100	250
2	250	500
3	250	750
4	500	750
5	500	1500

Five tables were produced -- one for each population case -- which compared the environmental suitability levels to an individual population case. These statistics indicate the amount of land in each of the environmental suitability levels that is available for siting nuclear power plants if a given set of population criteria (a population case) is applied. These statistics are shown in Tables F3.1 to F3.5.

To illustrate the effect of applying different population criteria (the five population cases) on land availability in a particular environmental suitability class, five more tables were produced. In these tables, the statistics represent the amount of land available for siting nuclear power plants in a given environmental suitability class as well as the amount of land uniquely constrained by each of the five population cases. These statistics are shown in Tables F3.6 through F3.10. The columns representing population cases have been arranged such that in moving from left to right, the stringency decreases. The leftmost column of the table -- available land -- shows land that is available for the given environmental suitability class even if the most stringent population criterion (population case 1) is applied. The second column -- population case 1 -- represents an additional amount of land considered available if population case 1 were relaxed. The next column -- population case 2 -- represents the additional increment of available land if the criteria for population case 2 were also relaxed. It follows that if no population criteria were established, the amount of land available in a particular environmental suitability class would be equal to the total of the first six columns in the table; the only land considered constrained would be that by a restricted land designation.

#### 4.7 Summary

The analytical methods used in this study were designed to explore the impact of various demographic siting criteria on the availability of land considered suitable for the siting of nuclear power plants. Maps were created so that impacts could be easily visualized and tabular statistics were prepared to allow a more rigorous analysis.

The determination of land considered suitable for siting was accomplished through a multi-objective environmental suitability analysis. The analysis was performed using factors generally related to engineering costs as well as conservation of specific resources. Because this investigation concerned the entire 48 contiguous United States and was not a site selection project, environmental factors were analyzed at a relatively general level of detail and were each considered to be of equal importance. The most suitable areas were characterized by an adequate water supply, low seismicity and gentle topography as well as an absence of protected resources. Although the map of environmental suitability (Figure F8) shows the eastern one-half of the country to be more suitable than the western, it is felt that there are numerous suitable sites available in the western portion.

Three types of population criteria were investigated: stand-off zones, annular density and sector density. The effects of stand-off zone criteria are straightforward. There is a direct relationship between the stand-off distance and the amount of land area constrained.

The analysis of annular density thresholds showed that the use of smaller radii to define the annulus resulted in constraints on sites near both large and small urban populations as well as sites near some locally dense rural areas. Larger radii tended to constrain a greater amount of area near suburban population but only around major cities; small urban and rural areas were not constrained.

Because results of reactor accident consequence calculations indicated (Section 2.7.4, Chapter 2) that certain risk characteristics depended strongly on the maximum number of persons within any given direction

sector, sector population criteria were designed. Their impacts were investigated to determine the amount of land area that would be constrained additional to that affected by annular density criteria. It was found that sector criteria affected the same areas and those adjacent to the areas affected by annular densities. Also, the area of impact responded to changes in annular radius in the same manner as for annular density criteria.

Transparent overlay maps and tabular statistics were provided to NRC for use in establishing siting criteria which would be numerically based upon population density, distribution and exclusion distance. Tabular statistics were used to quantify the impacts on a state-by-state basis. The use of transparent overlays provides a means not only to see the impacts of the generated criteria but also to create and view the effects of complex criteria by overlaying any combination of maps. Maps showing demographic criteria were also overlain onto the map of environmental suitability to visualize the potentially available suitable land. Through both the overlay procedure and a comparison of statistics, it was found that the greatest impacts of demographic criteria occur in the areas of high environmental suitability (i.e., Northeast).

### References for Chapter 4.0

1. Demographic Statistics Pertaining to Nuclear Power Reactor Sites, U. S. Nuclear Regulatory Commission, NUREG-0348, October 1979.
2. National Atlas of the United States of America, U.S. Geological Survey, compiled 1967, revised 1972-73.
3. Federal Register, Volume 45, #226, November 20, 1980, pages 76682-76684.
4. 10 CFR, Part 100: Reactor Site Criteria.
5. 10 CFR, Part 100: Appendix A -- Seismic and Geologic Criteria for Nuclear Power Plants.
6. S. T. Algermissen and D. M. Perkins, "A Probabilistic Estimate of Maximum Acceleration in Rock in the Contiguous United States," U. S. Geological Survey Open File Report, 76-416, 1976.
7. R. V. Whitman, N. C. Donovan, B. Boalt, and S. T. Algermissen, Effective Peak Velocity Related Acceleration Map, Applied Technology Council of Structural Engineers Association of California, 1977.
8. G. A. Bollinger, "Reinterpretation of the Intensity Effect of the 1886 Charleston, South Carolina Earthquake," U. S. Geological Survey Professional Paper 1028, 1977.
9. J. D. Stevenson, "Evaluation of the Cost Effects on Nuclear Power Plant Construction Resulting From the Increase in Seismic Design Level," Office of Nuclear Reactor Regulation, NUREG/CR-1508, 1981.
10. E. H. Hammond, "Analysis of Properties in Land Form Geography: An Application to Broad-Scale Land Form Mapping," Annals of the Association of American Geographers, Volume 54, Number 1, 1964.
11. E. V. Giusti and E. L. Meyer, "Water Consumption by Nuclear Power Plants and Some Hydrological Implications," U. S. Geological Survey Circular 745, 1977.

12. S. J. Stankowski, J. T. Limerinos, S. E. Buell, Stream Reaches In The Conterminous United States Where Dependable Flows of 300 Cubic Feet per Second Are Available Or Could Be Provided, U. S. Geological Survey Open File Report 77-646, 1977.
13. The Role of Groundwater in the National Water Situation, U. S. Geological Survey Water Supply Paper 1800, 1963.
14. C. L. McGuinness, Generalized Map Showing Annual Runoff and Productive Aquifers in the Conterminous U. S., U. S. Geological Survey Hydrologic Investigations Atlas HA-194, 1964.
15. P. E. King, Tectonic Map of North America, U. S. Geological Survey 1:500,000; 1969.
16. R. E. Harrison, Shaded Relief of U. S., U. S. Geological Survey 1:7,500,000; 1969.

## 5. Socioeconomic Impacts

### 5.1 Introduction

Because the construction and operation of a nuclear power plant can have social and economic impacts on nearby communities, the dependence of socioeconomic impacts on site location was examined by the Battelle Human Affairs Research Centers (Battelle-HARC) under contract to Sandia National Laboratories. The Battelle-HARC study (1) developed a classification scheme for the remoteness of light water reactor (LWR) site locations; (2) calculated average growth rates for several demographic and economic variables during the period of plant construction for two groups of LWR sites of differing remoteness, (3) examined the dependence of transmission line costs on site remoteness; and (4) discussed the significance of these results in the light of previous studies of the socioeconomic impacts of rural industrialization projects, boom towns, and nuclear power plants. This chapter presents a summary of the Battelle-HARC study. Full details are reported in the final report of that study [1].

### 5.2 Site Remoteness

Conceptually, the degree of remoteness of a nuclear power plant site depends upon both population density (the more sparse the population the more remote the site) and proximity to major population centers (nearby cities of significant size decrease remoteness). To capture this dual dependence, two measures were developed to define the degree of site remoteness, one of population sparseness and the other of proximity to urban centers.

Sparseness was defined in terms of total population and number of communities of population 25,000 or more within 20 miles of the site. Four sparseness categories were defined as follows:

### Sparseness Measure

<u>Category</u>	<u>Definition</u>
Most Sparse	<ol style="list-style-type: none"><li>1. Less than 50,000 persons and no community with more than 25,000 persons within 20 miles.</li><li>2. From 50,000 to 74,999 persons and no community with more than 25,000 persons within 20 miles.</li><li>3. From 75,000 to 149,999 persons or less than 75,000 persons but with at least one community with more than 25,000 persons within 20 miles.</li></ol>
Least Sparse	<ol style="list-style-type: none"><li>4. 150,000 or more persons within 20 miles.</li></ol>

Proximity was defined in terms of total population and the presence of cities with population  $\geq 100,000$  within 50 miles of the site. Four proximity categories were defined as follows:

### Proximity Measure

<u>Category</u>	<u>Definition</u>
Not in Close Proximity	<ol style="list-style-type: none"><li>1. No city with more than 100,000 persons and less than 400,000 persons within 50 miles.</li><li>2. No city with more than 100,000 persons and between 400,000 and 1,499,999 persons within 50 miles.</li><li>3. One or more large cities with more than 100,000 persons and less than 1,500,000 persons within 50 miles.</li></ol>
In Close Proximity	<ol style="list-style-type: none"><li>4. 1,500,000 or more persons within 50 miles.</li></ol>

The distance of 20 miles and a community size of 25,000 (sparseness measure) were chosen because the NRC Siting Policy Task Force [2] recommended that population densities around sites be limited out to a distance of 20 miles and because current siting practice requires that the nearest town of 25,000 persons be at least more distant than one and one-third times the distance to the outer boundary of the low population zone surrounding the plant site. The distance of 50 miles (proximity measure) was chosen because workforce commuting distances, which strongly affect the degree of population increase during construction periods and thus the magnitude of socioeconomic impacts, are usually limited to about a one-hour commute [3], or about 50 miles at current speed limits.

Table 5-1 presents the cross-classification by sparseness and proximity of 84 LWR sites in the U.S., where reactors are currently operating or under construction.

Table 5-1. Site Remoteness Matrix

		<u>Proximity</u>				Total
		1	2	3	4	
<u>Sparseness</u>	1	11	1	3	0	15
	2	3	1	4	2	10
	3	4	7	10	4	25
	4	0	0	11	23	34
	Total	18	9	28	29	84

Within this matrix remoteness decreases as one moves from cell (1,1) to cell (4,4) and sites in cells with indices that sum to the same total [e.g., cells (3,1), (2,2), and (1,3)] should be similar in degree of remoteness. By

summing the numbers of sites having similar degrees of remoteness, the distribution of remoteness over the 84 sites is obtained. Table 5-2 displays this distribution.

Table 5-2. Distribution of Remoteness

Group	Cell	Number of Sites
1. Most Remote Sites	(1,1)	11
2.	(2,1), (1,2)	4
3.	(3,1), (2,2), (1,3)	8
4.	(4,1), (3,2), (2,3), (1,4)	11
5.	(4,2), (3,3), (2,4)	12
6.	(4,3), (3,4)	15
7. Least Remote Sites	(4,4)	23
		84

Tables 5-1 and 5-2 show that, of the 84 sites, only 15 are not located within 20 miles of a town of 25,000 or within 50 miles of a city of 100,000. By contrast, 23 of the 84 sites have populations of 150,000 within 20 miles of the site and 1,500,000 within 50 miles. Thus, Tables 1 and 2 show that most current U.S. LWRs are not remotely sited.

### 5.3 Growth Rates

The socioeconomic impacts of large industrial projects usually depend on the size of the project workforce. Since the peak construction workforce ( $\geq 2000$ ) for a nuclear power plant is substantially larger than the plant's operational staff ( $\sim 200$ ), the socioeconomic impacts of nuclear power plants should be largest during the plant's construction phase. A measure of the magnitude of these impacts can be obtained by calculating average growth rates for population and economic activity in the areas surrounding nuclear power plants during their preconstruction (baseline) and construction periods. Variation of impacts with remoteness can be examined by performing these calculations for two groups of sites, a non-remote group and a remote group, and comparing the results.

Time series data for population, employment (total, retail trade, and construction), payroll (total, retail

trade, and construction), and government revenues (property tax per capita) and expenditures (total, education, highway, health, and welfare) were obtained for the preconstruction and construction periods at 21 nuclear power plant sites. Cross-classification of the 21 sites, according to the sparseness and proximity measures previously defined, yields Table 5-3. Table 5-3 shows that 7 of the sites are relatively remote and the other 14 are nonremote.

Table 5-3. Cross-classification Remoteness Matrix for 7 Remote and 14 Non-Remote Sites.

Sparseness	Category	<u>Proximity</u>				Total
		1	2	3	4	
	1	4	-	-	-	4
	2	1	-	-	-	1
	3	2	-	2	1	5
	4	-	-	5	6	11
	Total	7	0	7	7	21

Population data were available in census publications [4] for the years 1960, 1966, and 1970 through 1978. Employment and payroll data were obtained for the years 1959, 1962, and 1964 through 1978 from County Business Patterns [5]. Government revenue and expenditure data were collected from the County and City Data Book [6] for 1962, 1967, and 1972, and from the Census of Governments [7] for 1977.

Average yearly values of government revenues and expenditures for the preconstruction (baseline) and construction periods for the non-remote group of 14 sites and the remote group of 7 sites are presented in Table 5-4. Table 5-4 also presents the percentage

Table 5-4. Average Yearly Government Revenue and Expenditures  
for Remote and Non-Remote Groups\*

Variable	Remote			Non-Remote		
	Baseline Period	Construction Period	Percentage Increase†	Baseline Period	Construction Period	Percentage Increase†
Property Tax Per Capita	71	88	24	112	139	24
Total Government Expenditures	7,658	12,567	64	78,582	115,478	47
Education Expenditures	3,852	6,566	70	30,274	57,159	89
Highway Expenditures	684	909	33	5,677	6,383	12
Health Expenditures	792	1,687	113	3,626	5,657	56
Public Welfare Expenditures	174	200	15	5,275	9,787	85

\*Property tax per capita in dollars, expenditures in thousands of dollars.

†[(Construction Period Value/Baseline Period Value)-1]100.

increase of each variable for the construction period relative to the baseline period. Table 5-4 shows that the percentage increases in total government, highway, and health expenditures were greater at remote than non-remote sites, that the converse is true for education and welfare expenditures, and that the increase in per capita property tax was the same for both site groups. Therefore, because these data showed no consistent variation and because the amount of data was scant (data were available for only 4 years), average yearly growth rates were not calculated for these government variables.

The exponential growth of the variable  $X$  at a rate  $k$  per year over the time period  $t$  is given by

$$x_t = x_{t_0} e^{kt} \quad (1)$$

Average growth rates for a group of sites can be obtained by linear regression analysis after recasting equation 1 as follows, where  $k$  is the yearly average growth rate of the variable  $X$  for the site group,  $i$  is a site index, and  $w_{eww_i}$  is a site specific difference term.

$$\ln x_{i,t} = \ln x_{i,t_0} + \bar{k}t + \epsilon_i \quad (2)$$

Average growth rates were calculated for both site groups for the preconstruction (baseline) and construction periods for 7 variables (population, and total, retail, and construction employment and payroll). Table 5-5 presents the results of these linear regression analyses.

Examination of Table 5-5 reveals a consistent pattern. For each of the 7 variables and for both periods (baseline and construction), growth rates are higher for the remote site group than for the non-remote group. On the average, during the baseline period growth rates at remote sites exceed those at non-remote sites by about 50 percent. During the construction period growth rates at remote sites are 2 to 3 times larger than are growth rates at non-remote sites. As would be expected, growth rates are largest for construction payroll and employment. In addition, because of the increased demand for labor, the average number of hours worked also increases and therefore payroll growth exceeds employment growth.

Table 5-5. Average Growth Rates for Population, Employment, and Payroll at Remote and Non-Remote Sites.

	Average Yearly Growth Rates (%) <sup>a</sup>				Construction Impacts (%) <sup>b</sup>		Impact Differences (%) <sup>c</sup>
	Preconstruction		Construction		Remote	Non-Remote	
	Remote	Non-Remote	Remote	Non-Remote	Remote	Non-Remote	
Population	1.7 <sub>+0.2</sub>	1.4 <sub>+0.2</sub>	6.1 <sub>+0.8</sub>	1.6 <sub>+0.6</sub>	4.3 <sub>+1.0</sub> <sup>d</sup>	0.2 <sub>+1.4</sub>	4.1 <sub>+2.4</sub> <sup>d</sup>
Total Employment	5.7 <sub>+0.4</sub>	3.9 <sub>+0.2</sub>	12.0 <sub>+1.5</sub>	4.4 <sub>+0.9</sub>	7.1 <sub>+1.9</sub> <sup>d</sup>	0.5 <sub>+1.1</sub>	6.5 <sub>+3.0</sub> <sup>d</sup>
Total Payroll	8.4 <sub>+0.3</sub>	5.7 <sub>+0.3</sub>	18.9 <sub>+2.4</sub>	7.3 <sub>+1.5</sub>	10.5 <sub>+2.7</sub> <sup>d</sup>	1.6 <sub>+1.8</sub>	8.9 <sub>+4.5</sub> <sup>d</sup>
Retail Employment	5.5 <sub>+0.3</sub>	3.8 <sub>+0.3</sub>	9.8 <sub>+1.0</sub>	4.3 <sub>+0.6</sub>	3.4 <sub>+1.3</sub> <sup>d</sup>	0.5 <sub>+0.9</sub>	2.8 <sub>+2.2</sub> <sup>d</sup>
Retail Payroll	8.1 <sub>+0.2</sub>	5.0 <sub>+0.3</sub>	9.9 <sub>+1.0</sub>	4.5 <sub>+0.6</sub>	1.7 <sub>+1.2</sub>	-0.5 <sub>+0.9</sub>	2.2 <sub>+2.1</sub> <sup>e</sup>
Construction Employment	8.3 <sub>+0.8</sub>	3.9 <sub>+0.5</sub>	33.3 <sub>+3.5</sub>	11.8 <sub>+2.2</sub>	24.9 <sub>+4.3</sub> <sup>d</sup>	7.9 <sub>+2.7</sub> <sup>d</sup>	17.1 <sub>+7.0</sub> <sup>d</sup>
Construction Payroll	10.8 <sub>+1.0</sub>	7.2 <sub>+0.6</sub>	45.9 <sub>+5.0</sub>	17.2 <sub>+3.1</sub>	35.1 <sub>+6.0</sub> <sup>d</sup>	10.0 <sub>+3.7</sub> <sup>d</sup>	25.1 <sub>+9.7</sub> <sup>d</sup>

a. All values are significant at the 0.01 level by f-test

b. (Construction Growth Rate) - (Preconstruction Growth Rate)

c. (Remote Impact) - (Non-Remote Impact)

d. Significant at the 0.01 level by t-test

e. Significant at the 0.05 level by t-test

By subtracting baseline period growth rates from construction period growth rates, estimates of the growth rates due only to nuclear power plant construction (construction impact) are obtained. Table 5-5 shows that for the non-remote group of sites, construction impacts were significant only for construction payroll and employment. However, for the remote group of sites, impacts were significant for all variables, being largest for construction payroll (35%) and employment (25%) and substantial for total payroll (10%). Finally, the last column of Table 5-5 shows that, for all variables except retail payroll, impact differences (remote site construction impact minus non-remote site construction impact) are all statistically significant at the 0.01 level.

#### 5.4 Transmission Line Costs

Transmission line costs are comprised of installation and operating costs. Installation costs depend on (1) the length of the right-of-way along which the lines will be strung in order to connect the power plant to the existing national power grid; (2) right-of-way acquisition costs; (3) the number and size (conductor rating) of the lines installed; and (4) installation labor costs (right-of-way preparation, construction of line towers and substations, stringing of lines). Operating costs consist principally of the cost of line losses during transmission and maintenance costs.

Transmission losses are less for shorter line lengths and larger conductors. Larger conductors cost more than smaller conductors, require a wider right-of-way (125 ft wide for 230 kV cable; 200 ft for 500 kV [8]), and are more costly to install. Despite these higher costs, EPRI projections [9] predict an increasing use of higher rated (larger) conductors through the year 2000. This agrees with the findings by Power Transmission, Inc. [10] that utilities currently prefer to minimize future transmission losses by installation of larger conductors.

Unit costs for labor (hourly wages) in suburban areas were found by an EPRI study [11] to exceed those in rural areas by about 25%. Unit costs for the acquisition of land for right-of-way are also likely to be lower in rural areas than in suburban areas. In contrast to this, total costs due to acquisition of right-of-way, purchase of materials and equipment, payment of labor, and transmission line losses all increase with increasing line length. Therefore, since remote siting would seem to require longer transmission lines, remote siting would appear to

entail higher transmission line installation and operating costs. This is not always the case, however.

Maps of the existing national transmission grid show that, except for the more remote regions of the Rocky Mountains, grid transmission lines pass through all regions (both remote and non-remote) of the U.S. [12]. Although consideration of environmental, social, and asthetic issues as required by NEPA has tended to somewhat lengthen line right-of-ways, the factor that dominates the length of new transmission lines is the gross distance of the power plant site from the nearest leg of the national transmission grid. Because this grid runs through both remote and non-remote areas, remote siting does not necessarily mean a lengthy transmission line. Table 5-6 presents data in support of this conclusion.

Table 5-6 presents data on the conductor rating, length, and acreage of the transmission lines which connect 29 power plant sites (those with all facilities operating as of 1978) of varying remoteness to the national power grid. Examination of the right-of-way lengths, which were drawn from DOE maps [12], shows that for existing sites right-of-way lengths do not correlate with remoteness. Some remote sites are closer to the national grid than are some less remote sites. Thus, it is distance from the national transmission grid and not distance from major population centers (remoteness) that principally determines the costs of transmission line installation and operation.

## 5.5 Discussion

Major construction projects have large workforce requirements. In rural settings, when workforce requirements can not be met locally or by commuting from nearby cities, in-migration of workers occurs. If this in-migration is substantial, "boomtown" conditions may result and the host area may experience significant socioeconomic impacts. This scenario has been the subject of considerable study. Rural industrial development studies [13,14] have examined the impacts of industrial projects upon small, rural communities. Boomtown studies [15-18] have examined the local impacts of rapid, large-scale energy development projects, located primarily in remote farming and ranching areas of the Rocky Mountains. The impacts of nuclear power plant construction have also been examined by several previous studies [19-21].

Table 5-6. Power Transmission Line Data for 29 Operating Nuclear Sites

Remoteness Index	Total Miles of Right-of-Way	Estimated Acres of Right-of-Way	Average Kilovolts Per Mile of Line
1-1	230	4,182	345
2-1	266	4,030	230
2-1	38	800	399
3-2	52	661	156
3-2	230	4,061	301
3-2	102	1,855	345
3-2	179	2,670	206
2-4	30	545	345
2-4	151	2,655	309
3-3	118	2,675	418
3-3	85	1,370	267
3-3	5	91	345
3-3	95	1,803	316
3-3	84	1,273	230
3-3	123	2,236	337
3-4	17	309	345
3-4	124	2,255	345
3-4	24	291	115
4-3	170	3,576	423
4-3	85	1,455	304
4-3	25	358	198
4-3	67	1,218	345
4-4	409	8,291	147
4-4	60	758	134
4-4	4	61	230
4-4	104	2,545	485
4-4	90	1,636	345
4-4	217	4,561	378
4-4	29	527	345

Significant in-migration to a construction project's host area occurs only if workforce requirements can not be met locally or by commuting from nearby population centers (generally, those located within about a one-hour commute of the site [3]). Even when substantial in-migration does occur, a boomtown can be avoided, if the resulting population growth is spread over several nearby communities [22]. In general, adverse socio-economic impacts are not observed until the rate of population growth of a single community exceeds 10 to 15 percent per year [23,24]. Under these conditions institutional breakdowns may occur in the labor and housing markets and in the provision of government services (education, health care, recreational facilities, police and fire protection) [23].

The small sizes, undiversified economies, small tax bases, homogeneous populations, and traditional life styles of rural communities tend to increase their susceptibility to socioeconomic impacts resulting from rapid population growth. Mortgage investors tend to find small, economically undiversified, rural communities unattractive investment locales. Lack of mortgage money combined with shortages of building materials and housing construction workers can produce a serious housing shortage. Because of their limited tax bases and because the project under construction generally yields little tax revenue until nearly completed, rural communities are often unable to finance the increased load of government services needed to accommodate rapid population growth. Finally, rural communities having a homogeneous population and life style may be less willing or able to welcome newcomers having different ideas, ways of doing business, and life styles and to accept the changes in personal, social, business, and institutional interactions that incorporation of the newcomers into their communities would entail [16-18,25].

The willingness of rural communities to accept change depends upon community perception of the benefits (and risks) that will accompany the changes, and upon the degree of community involvement in the decisions which determine the nature and rate of the changes. Because the construction of a large industrial or energy facility promises increased tax revenues, new jobs, more retail trade, and therefore improved government services, an end to out-migration of children and friends [14,15], and a higher standard of living [21], many rural communities welcome these projects (at least initially).

However, community resistance may develop, if the economic benefits are unevenly distributed (e.g., business men and land owners profit while the poor, the elderly, and minorities suffer), if the project is perceived to benefit principally distant cities (e.g., electric generating stations [19,25]), if project decisions affecting the community are made without community involvement, and if there are concerns about the safety of the facility (e.g., nuclear power plants [21]).

The degree to which the socioeconomic impacts, characteristic of rural industrialization and boom-towns, have occurred as the result of nuclear power plant siting was examined by gathering data about peak construction employment, number of in-migrants, and socioeconomic impacts at 12 remote nuclear power plant sites. The data, which were extracted from Environmental Impact Statements and post-licensing case studies (where available), are presented in Table 5-7. For the 12 sites listed in Table 5-7, peak construction employment was approximately 2200 (+700), or 5 percent of the surrounding population to 20 miles. For the 9 sites where in-migration data were available, peak construction in-migration (workers plus families) on an average represented only 3 percent of the surrounding population to 20 miles. Examination of the last column in Table 5-7 shows that with scattered exceptions (crowded classrooms, Yellow Creek; stressed government services, Hatch; wage inflation, St. Lucie; safety controversy, Diablo Canyon) the socioeconomic impacts at the 12 sites were largely beneficial (significantly increased tax revenues, increased retail trade). Given the modest increases in total population in the regions surrounding the sites, it is not surprising that detrimental impacts were minimal, while economic impacts were favorable.

Since socioeconomic impacts depend principally on the rate of population growth, which scales with construction workforce growth, additional data on construction workforce growth were developed for 19 non-remote construction projects including 15 nuclear power plants and for 28 remote construction projects including one nuclear power plant. The data are presented in Table 5-8, which shows that an average remote site experiences twice as much in-migration as a non-remote site.

Table 5-7. Socioeconomic Impacts at Selected Remote Sites

Site (Projected Year of Completion for Each Reactor at a Site) <sup>1</sup>	Utility (Total Megawatts at Site)	Remoteness Index <sup>2</sup>	Estimated Peak Construction Employment (Workers)	Total Popula- tion Within 20 Miles (1980 Projected, provided by James & Moore)	Estimated Number of Immigrants at Peak of Construction	Overall Assessment Social and Economic Impacts	
						Social	Economic
YELLOW CREEK <sup>3</sup> 1985, 1988 (Luray, MS)	TVA 2,370 MWe	(1,1) Most Sparse Least Proximate	2,600	55,430	780 workers (470 with families, 310 without fami- lies)	Increase in students will require seventeen classrooms and teachers; classroom space is currently scarce.	
GRAND GULF <sup>4</sup> 1982, 1986 (Port Gibson, MS)	Mississippi Power & Light 2,500 MWe	(1,1)	Up to 2,600	27,092	Not provided	1. More electrical power available. 2. Dramatically increases the tax base. 3. Significant direct and indirect increases in employment and income. (p. 8-16)	
SOUTH TEXAS <sup>5</sup> PROJECT 1984, 1986 (Palacios, TX)	Houston Lighting and Power Company 2,500 MWe	(1,1)	2,100	32,307	2,000 persons	Similar to Grand Gulf.	
HATCH <sup>6,7</sup> 1975, 1979 (Baxley, GA)	Georgia Power Company 1,572 MWe	(1,1)	2,300	49,808	920 to 1,150 workers	Some growth impacts on schools, housing, and public services but not serious. No unmanageable strains on community infrastructure. Plant's economic benefits (reduced tax rate, growth and employment) were viewed very positively by host area.	
VOOTLE <sup>8</sup> 1985, 1988 (Waynesboro, GA)	Georgia Power Company 1,200 MWe	(1,2)	3,800	26,170	815 workers	Construction of the proposed nuclear plant will slow, but not halt, the current trend in population migration from this rural area. For the effects of construction to be most beneficial, efforts to attract new and related commercial activity should continue. (p. 27)	
CLINTON <sup>9</sup> 1982 (Clinton, IL)	Illinois Power Company 1,900 MWe	(2,2)	1,200	47,792	418 persons (191 workers, 121 adults, 106 children)	Minimal impacts anticipated due to close proximity (approximately 60 miles) of large urban areas.	
ARKANSAS <sup>10</sup> 1973, 1976 (Russellville, AR)	Arkansas Power and Light Company 1,748 MWe	(1,1)	973	59,322	200 persons	1. Stabilize area's construction workers. 2. Increases in direct and indirect employment and income. 3. Expansion of electric power provisions to the service area. 4. Increase in property tax payments which aided in reversal of school overcrowding and financial difficulties.	
ST. LUCIE <sup>11</sup> 1976, 1983 (Hutchinson Island, FL)	Florida Power and Light Company 1,554 MWe	(3,1)	1,847	121,542	Not provided	1. Increased tax base by approximately 35%. 2. Public construction projects in the county had to be delayed or cancelled due to inflated wage rates resulting from construction of the plant.	
CRYSTAL RIVER <sup>12</sup> 1977 (Crystal River, FL)	Florida Power Corporation 825 MWe	(1,1)	1,790	38,705	Not provided	1. Increased tax base. 2. 50% (85) of operating workforce relocated to Crystal River. 3. Retail sales in area increased due to relocation of non-local construction workforce.	
DIABLO <sup>13</sup> CANYON 1981, 1981 (Avila Beach, CA)	Pacific Gas and Electric 2,190 MWe	(2,1)	2,470	101,151	3,308 persons <sup>17</sup>	1. Divisiveness of entire Diablo Canyon issue among community residents (not necessarily due to workforce in-migration). Operation of facilities held up due to environmentalists' concerns regarding geologic fault at site.	
FARLEY <sup>14</sup> 1977, 1980 (Dothan, AL)	Alabama Power Company 1,720 MWe	(3,1)	2,250 <sup>18</sup>	93,185	1,057 workers <sup>19</sup>	1. Increase in direct and indirect employment and income.	
SURREY <sup>15</sup> 1972, 1973 (Gravel Neck, VA)	Virginia Electric and Power Company 1,550 MWe	(4,4) <sup>20</sup>	1,934	284,669	102 persons <sup>16</sup>	1. Increase in tax base. 2. Increased employment, business income, tourism, traffic and land cost during construction in Surrey and Isle of Wight Counties.	

Table 5-7. Footnotes

<sup>1</sup>\*Commercial Nuclear Power Stations in the United States--Operable, Under Construction or Ordered--August 1, 1980, Wallchart, published by Nuclear News, La Grange Park, Illinois.

<sup>2</sup>The remoteness index as defined by sparseness and proximity measures (see text).

<sup>3</sup>Tennessee Valley Authority, Final Environmental Statement, Yellow Creek Nuclear Plant Units 1 and 2, Vol. 1, Vol. 2., January 1978.

<sup>4</sup>Mississippi Power & Light Company, Final Environmental Statement Related to Construction of Grand Gulf Nuclear Stations Units 1 and 2, Sec. 8.2, August 1973.

<sup>5</sup>Houston Lighting & Power Company, "Benefits and Costs" Chapter 8 and "Summary Benefit-Cost Analysis" Chapter 11 of South Texas Project-Environmental Report, Vol. 1, amended June 1975.

<sup>6</sup>Altameda Area Planning and Development Commission, Impact of the Georgia Power Company Nuclear Plant on Community Facilities in the Toombs-Appling BiCounty Area, Georgia Institute of Technology, Winter 1969.

<sup>7</sup>Shields, M. A., et al., Socioeconomic Impacts of Nuclear Power Plants: A Paired Comparison of Operating Facilities, NUREG/CR-0916, Oak Ridge, TN: Oak Ridge National Laboratory, July 1979.

<sup>8</sup>Central Savannah Area Planning and Development Commission, Impact of the Georgia Power Company Vogtle Nuclear Power Plant on the Central Savannah River Area, Appendix A, Georgia Institute of Technology, Spring 1972.

<sup>9</sup>Illinois Power Company, "Economic and Social Effects of Plant Construction and Operation," Chapter 8 of Environment Report--Construction Permit Stage for the Clinton Power Station, September 1974.

<sup>10</sup>Pijawka, D., Arkansas Nuclear One, Preliminary Site Report, Washington: U.S. Nuclear Regulatory Commission, February 1979.

<sup>11</sup>Pijawka, D., St. Lucie, Units 1 and 2, Preliminary Site Visit Report, Washington: U.S. Nuclear Regulatory Commission, February 1979.

<sup>12</sup>Pijawka, D., Crystal River, Unit 3, Preliminary Site Visit Report, Washington: U.S. Nuclear Regulatory Commission, February 1979.

<sup>13</sup>York, M. N., Diablo Canyon, Units 1 and 2, Preliminary Site Visit Report, Washington: U.S. Nuclear Regulatory Commission, February 1979.

<sup>14</sup>Alabama Power Company, Final Environmental Statement Related to Operation of Joseph M. Farley Nuclear Plant, Units 1 and 2, December 1974.

<sup>15</sup>Flynn, J., Surrey Nuclear Plant, Units 1 and 2, Preliminary Site Visit Report, Washington: U.S. Nuclear Regulatory Commission, February 1979.

<sup>16</sup>Flynn J., Socioeconomic Impacts of Nuclear Generating Stations, Surry Case Study, Washington: U.S. Nuclear Regulatory Commission, November 1980.

<sup>17</sup>Pijawka, D., and Yoquinto, G., Socioeconomic Impacts of Nuclear Generating Stations, Diablo Canyon Case Study, Washington: U.S. Nuclear Regulatory Commission, December 1980.

<sup>18</sup>Alabama Power Company, Estimate, February 1979.

<sup>19</sup>Based on percentages from a survey at Joseph M. Farley #2. Malhotra, S., Manninen, D., Migration and Residential Location of Workers at Nuclear Power Plant Construction Sites, Vol. 11, Profile Analysis of Worker Survey, Final Report. BHARC-100/80/030, Seattle, WA: Battelle Human Affairs Research Centers, September 1980.

<sup>20</sup>Based on population size within 20 and 50 miles of the site, Surrey is classified as non-remote. However when natural barriers are taken into consideration the population of the area within 20 miles of the site which has easy access to the site is considerably less. The figure for 50 miles is still appropriate as a representation of the population within commuting distance of the site.

Table 5-8. Variation in Migrant Proportion by Location

<u>Location*</u>	<u>Number of Sites</u>	<u>Migrant Proportion (%) Construction Workers</u>	
		<u>Average</u>	<u>Range</u>
<u>Remote</u>			
Bureau of Reclamation Water Development Projects <sup>1,2</sup>	10	59	40-89
Old West Regional Commission Study, Coal-fired Power Plants <sup>3</sup>	14	60	21-97
North Dakota State University Leland Olds and Square Butte <sup>4</sup> Coal Creek <sup>5</sup>	2 1	50 39	**
NRC Labor Migration Study <sup>6,7</sup>	1	47	—
—	—	—	—
	N = 28	Weighted Average = 58	
<u>Non-remote</u>			
NRC Labor Migration Study <sup>6,7</sup> (excluding TVA)	8	29	15-49
TVA Sites <sup>8</sup> Nuclear <sup>9</sup>	7	26	11-40
Non-nuclear <sup>9</sup>	2	34	29-47
Bureau of Reclamation Water Development Projects <sup>2</sup>	2	17	12-22
—	—	—	—
	N = 19	Weighted Average = 27	

\*Remoteness assignments were made using the sparseness and proximity measures described in the text.

\*\*Migrant proportions were not provided separately for these sites in the reference document.

Table 5-8. Footnotes

<sup>1</sup>J. A. Chalmers, Bureau of Reclamation Construction Worker Survey, Bureau of Reclamation, Engineering and Research Center, October 1977.

<sup>2</sup>In general the Bureau of Reclamation Water Development Projects were constructed in sparsely settled regions of the western United States. Two sites, however, were located in the Phoenix area and are included in the nonremote group.

<sup>3</sup>Mountain West Research, Inc., Construction Worker Profile, Final Report, prepared for the Old West Regional Commission, 1975.

<sup>4</sup>A. G. Leholm, F. L. Leistritz and J. S. Wieland, Profile of Electric Power Plant Construction Work Force, Agricultural Economics Statistical Series, Issue No. 22, Department of Agricultural Economics, North Dakota State University, July 1976.

<sup>5</sup>J. S. Wieland and F. L. Leistritz, Profile of the Coal Creek Project Construction Work Force. Agricultural Economics Miscellaneous Report No. 33, Department of Agricultural Economics, North Dakota State University, February 1978.

<sup>6</sup>S. Malhotra and D. Manninen, Migration and Residential Location of Workers at Nuclear Power Plant Construction Sites, Vol. II Profile Analysis of Worker Surveys, Battelle Human Affairs Research Centers, September 1980.

<sup>7</sup>The NRC labor migration study included only one remote site.

<sup>8</sup>TVA has published numerous reports containing the results of construction worker surveys conducted at TVA sites. For example see Tennessee Valley Authority, Hartsville Nuclear Plants Socioeconomic Monitoring and Mitigation Report, March 31, 1978, Knoxville, Tennessee, Tennessee Valley Authority, 1978.

<sup>9</sup>Multiple surveys were conducted at the TVA sites. The average and range of migrant proportions shown are for 35 surveys conducted at the nine TVA sites.

### 5.6 Conclusions

Classification of current nuclear power plant sites according to remoteness shows that most sites are nonremote, while few are truly remotely sited. In fact, although half of the current sites are located in nonmetropolitan counties, a majority are within 60 miles of [19] and few are more than 100 miles from a major metropolitan area.

The data on growth rates (Table 5-5) and construction workforce in-migration proportions (Table 5-8) show that population and economic growth rates are higher at more remote as opposed to less remote sites. Impacts do increase with site remoteness. However, although the differences in growth rates between more and less remote sites presented in Table 5-5 are all statistically significant, the 6 percent growth rate in total population observed for the more remote sites is significantly below the rate of 10 to 15 percent needed to produce boomtown conditions and thus adverse socioeconomic impacts. This conclusion is supported by the data presented in Table 5-7, which showed that 12 somewhat remotely sited nuclear power plants produced principally favorable socioeconomic impacts (much increased tax revenues, increased retail trade, some strains on government services, stabilization of population) on nearby communities.

Finally, it seems clear (1) that should future nuclear power plants be sited no more remotely than are current plants, then they will have few if any adverse socioeconomic impacts and (2) should they be sited in truly remote locations, then the potential for adverse impacts on nearby small rural communities can be substantially reduced by advance planning.

### References for Chapter 5

1. C. Cluett, S. Malhotra, and D. Manninen, Socio economic Impacts of Remote Nuclear Power Plant Siting, BHARC-400/81/002, Battelle Human Affairs Research Centers, Seattle, Washington, May 1981.
2. U.S. Nuclear Regulatory Commission, Report of the Siting Policy Task Force, NUREG-0625, August 1979.
3. W. P. Freudenburg, "The Social Impact of Energy Boom Development on Rural Communities: A Review of Literatures and Some Predictions," Department of Sociology, Yale University, August 1976.
4. U.S. Bureau of the Census, Current Population Reports, Series P-25 and P-26, Washington: Government Printing Office, 1966 and 1971-1978.
5. U.S. Bureau of the Census, County Business Patterns (for individual states), Washington: Government Printing Office, 1959 and 1962 and 1964-1978.
6. U.S. Bureau of the Census, County and City Data Book, Washington: Government Printing Office, 1967 and 1972 and 1977.
7. U.S. Bureau of the Census, Census of Governments, Washington: Government Printing Office, 1977.
8. EPRI Planning and Evaluation Staff, "Technical Assessment Guide," EPRI Rept. No. PS 1201-SR, July 1979, Exhibit 9-1, p. 9-2.
9. Ibid, p. 9-3.
10. I. S. Grant and V. J. Longo, "Economic Incentives for Larger Transmission Conductors," Power Transmission, Inc., Schenectady, NY, p. 1.
11. Commonwealth Associates, Inc., "Cost Components of High Capacity Transmission Options," EPRI Rept. No. EL-1065, Vol. 1, May 1979, p. 2-35.
12. Principal Electric Facilities (Map Series), Department of Energy, Energy Information Administration, 1979.

13. Frankena, F., Community Impacts of Rapid Growth in Nonmetropolitan Areas, East Lansing, MI: Michigan State University, Department of Sociology, June 1980.
14. Summers, G. F., et al., Industrial Invasion of Nonmetropolitan America: A Quarter Century of Experience, New York, NY, Praeger Publishers, 1976.
15. Freudenburg, W. R., "The Social Impact of Energy Boom Development on Rural Communities: A Review of Literatures and Some Predictions," Department of Sociology, Yale University, August 1976.
16. C. F. Cortese and B. Jones, "The Sociological Analysis of Boom Towns," Western Sociological Review, 8:76-90, 1977.
17. C. F. Cortese, "The Social Impacts of Energy Development in the West: An Introduction," The Social Science Journal, 16:1-7, April 1979.
18. C. F. Cortese, "Rapid Growth and Social Change in Western Communities," Social Impact Assessment, No. 40/41, April-May 1979.
19. U.S. Nuclear Regulatory Commission, Office of Nuclear Regulatory Research, Socioeconomic Impacts: Nuclear Power Station Siting, NUREG-0150, June 1977.
20. E. Peelle, "Social Effects of Nuclear Power Plants," in C. P. Wolf, ed., 2: Social Impact Assessment, Milwaukee, WI: Environmental Design Research Association, Inc., 1974, p. 114.
21. M. A. Shields et al., Socioeconomic Impacts of Nuclear Power Plants: A Paired Comparison of Operating Facilities, NUREG/CR-0916, Oak Ridge, TN: Oak Ridge National Laboratory, July 1979.
22. Conversation with John Gilmore and Dean Coddington, Denver Research Institute, November 26, 1980. Case studies reviewed by DRI suggest that communities with a population of less than 1,000 are likely to be by-passed by in-migrants looking for housing, shopping facilities, and schools.

23. J. S. Gilmore, "Boom Towns May Hinder Energy Resource Development," Science, 191:535-540, February 13, 1976.
24. U.S. Department of Housing and Urban Development, Office of Community Planning and Development, Rapid Growth From Energy Projects: Ideas for State and Local Action, a Program Guide, 1976, p. 2.
25. U.S. Congress, Senate, Statement of S. H. Murdock in Hearing Before the Subcommittee on Rural Development, Committee on Agriculture, Nutrition, and Forestry, The Socioeconomic Effects of a Nuclear Waste Storage Site on Rural Areas and Small Communities, 96th Cong., 2d Sess., 1980, p. 67.

## Appendix A: Site Data

A large body of site-related data was collected for use in performing the consequence calculations discussed in Chapter 2 of this report. These data are summarized in the following sections of this appendix as listed below.

<u>Section</u>	<u>Data Description</u>
A.1	General Site and Reactor Data
A.2	Site Population Data
A.3	Weather Data
A.4	Site Wind Rose Data
A.5	Economic Data

### A.1 General Site and Reactor Data

Calculations were performed for 91 sites where reactors are currently operating, are under construction, or have been assigned a construction permit. Table A.1-1 lists the site locations (county/state) and the power level (MWe), type, supplier, and date of startup (actual or expected) for the reactors located at these sites. Table A.1-2 gives the latitude and longitude of each site,\* as well as the meteorological station and sheltering region assigned for performing site consequence calculations. The meteorological data used in this study are further described in Section A.3. The sheltering region is based on housing types and is used to determine external exposure shielding factors when population sheltering is assumed to be an emergency protective measure. The important housing characteristics and assumed shielding factors for the seven regions used in this study are described in Table A.1-3. For further information on sheltering regions and shielding factors, see reference [2].

\*Latitudes and longitudes were taken from reference [1].

Table A.1-1 General Site and Reactor Data

Plant	Location (County/State)	Power Level (MWe)	Type	Reactor Supplier	Actual or Expected Date of Startup
Allens Creek	Austin, TX	1200	BWR	GE	/87
Arkansas 1,2	Pope, AR	836	PWR	B&W	12/74
		912	PWR	C-E	3/80
Bailly	Porter, IN	645	BWR	GE	6/87
Beaver Valley 1,2	Beaver, PA	833	PWR	W	4/77
		833	PWR	W	5/86
Bellefonte 1,2	Jackson, AL	1213	PWR	B&W	9/83
		1213	PWR	B&W	6/84
Big Rock Pt.	Charlevoix, MI	63	BWR	GE	12/62
Black Fox 1,2	Rogers, OK	1150	BWR	GE	7/85
		1150	BWR	GE	7/88
Braidwood 1,2	Will, IL	1120	PWR	W	10/85
		1120	PWR	W	10/86
Browns Ferry 1,2,3	Limestone, AL	1067	BWR	GE	8/74
		1067	BWR	GE	8/75
		1067	BWR	GE	3/77
Brunswick 1,2	Brunswick, NC	790	BWR	GE	3/77
		790	BWR	GE	11/75
Byron 1,2	Ogle, IL	1120	PWR	W	10/83
		1120	PWR	W	10/84
Callaway 1,2	Callaway, MO	1150	PWR	W	10/82
		1150	PWR	W	4/87
Calvert Cliffs 1,2	Calvert, MD	850	PWR	C-E	5/75
		850	PWR	C-E	5/77
Catawba 1,2	York, SC	1145	PWR	W	7/83
		1145	PWR	W	1/85
Cherokee 1,2,3	Cherokee, SC	1280	PWR	C-E	1/90
		1280	PWR	C-E	1/92
		1280	PWR	GE	Indef.
Clinton 1,2	Dewitt, IL	950	BWR	GE	12/82
		950	BWR	GE	Indef.
Comanche Peak 1,2	Somervell, TX	1150	PWR	W	/81
		1150	PWR	W	/83
Cooper	Nemaha, NB	778	BWR	GE	7/74
Crystal River 3	Citrus, FL	825	PWR	B&W	3/77
Davis-Besse	Ottawa, OH	906	PWR	B&W	11/77
Diablo Canyon 1,2	San Luis Obispo, CA	1084	PWR	W	/81
		1106	PWR	W	/81
E. Donald C. Cook 1,2	Berrien, MI	1054	PWR	W	8/75
		1094	PWR	W	6/78
Dresden 1,2,3	Grundy, IL	200	BWR	GE	8/60
		800	BWR	GE	8/70
		800	BWR	GE	10/71
Duane Arnold	Linn, IA	545	BWR	GE	5/74
Fermi 2	Monroe, MI	1100	BWR	GE	3/82
Fitzpatrick*	Oswego, NY	821	BWR	GE	7/75
Forked River **	Ocean, NJ	1120	PWR	C-E	5/86
Ft. Calhoun	Washington, NB	457	PWR	C-E	9/73
Ft. St. Vrain	Weld, CO	330	HTGR	GA	1/79
Ginna (Brookwood)	Wayne, NY	490	PWR	W	3/70
Grand Gulf 1,2	Clairborne, MS	1250	BWR	GE	4/82
		1250	BWR	GE	9/86
Haddam Neck	Middlesey, CT	575	PWR	W	1/68
Hartsville A1,A2, B1,B2	Troydale & Smith, TN	1233	BWR	GE	7/86
		1233	BWR	GE	7/87
		1233	BWR	GE	Indef.
		1233	BWR	GE	Indef.

\*Same site as Nine Mile Point

\*\*Same site as Oyster Creek

Table A.1-1 General Site and Reactor Data (cont)

Plant	Location (County/State)	Power Level (MWe)	Type	Reactor Supplier	Actual or Expected Date of Startup
Hatch 1,2	Appling, GA	786	BWR	GE	12/75
		786	BWR	GE	8/79
Hope Creek 1,2*	Salem, NJ	1070	BWR	GE	12/86
		1070	BWR	GE	12/89
Indian Point 2,3	Westchester, NY	873	PWR	W	7/74
		965	PWR	W	8/76
Joseph M. Farley 1,2	Houston, AL	860	PWR	W	12/77
		860	PWR	W	11/80
Keweenaw	Keweenaw, WI	535	PWR	W	6/74
LaCross	Monroe, WI	50	BWR	Allis	11/69
LeSalle 1,2	LaSalle, IL	1078	BWR	GE	6/81
		1078	BWR	GE	6/82
Limerick 1,2	Montgomery, PA	1055	BWR	GE	4/85
		1055	BWR	GE	4/87
Maine Yankee	Lincoln, ME	790	PWR	C-E	12/72
Marble Hill 1,2	Jefferson, IN	1130	PWR	W	/86
		1130	PWR	W	/87
McGuire 1,2	Mecklenberg, NC	1180	PWR	W	8/80
		1180	PWR	W	4/82
Midland 1,2	Midland, MI	530	PWR	B&W	7/84
		805	PWR	B&W	12/83
Millstone 1,2,3	New London, CT	660	BWR	GE	12/70
		870	PWR	C-E	12/75
		1150	PWR	W	5/86
Monticello	Wright, MN	536	BWR	GE	7/71
Nine Mile Pt. 1,2**	Oswego, NY	610	BWR	GE	12/69
		1080	BWR	GE	10/86
North Anna 1,2,3,4	Louisa, VA	850	PWR	W	6/78
		850	PWR	B&W	8/80
		934	PWR	B&W	4/87
		934	PWR	B&W	4/88
Oconee 1,2,3	Oconee, SC	860	PWR	B&W	7/73
		860	PWR	B&W	9/74
		860	PWR	B&W	12/74
Oyster Creek ***	Ocean, NJ	620	BWR	GE	12/69
Palisades	VanBuren, MI	740	PWR	C-E	12/71
Palo Verde 1,2,3	Manicopa, AZ	1270	PWR	C-E	5/83
		1270	PWR	C-E	5/84
		1270	PWR	C-E	5/86
Peach Bottom 2,3	York, PA	1065	BWR	GE	7/74
		1065	BWR	GE	12/74
Pebble Springs 1,2	Gilliam, OR	1260	PWR	B&W	9/88
		1260	PWR	B&W	9/90
Perkins 1,2,3	Davie, NC	1280	PWR	C-E	Indef.
		1280	PWR	C-E	Indef.
		1280	PWR	C-E	Indef.
Perry 1,2	Lake, OH	1205	BWR	GE	5/84
		1205	BWR	GE	5/88
Phipps Bend 1,2	Hawkins, TN	1233	BWR	GE	Indef.
		1233	BWR	GE	Indef.
Pilgrim 1,2	Plymouth, MA	670	BWR	GE	12/72
		1150	PWR	C-E	Indef.
Pt. Beach 1,2	Manitowoc, WI	497	PWR	W	12/70

\*Same site as Salem

\*\*Same site as Fitzpatrick

\*\*\*Same site as Forked River

Table A.1-1 General Site and Reactor Data (cont)

<u>Plant</u>	<u>Location (County/State)</u>	<u>Power Level (MWe)</u>	<u>Type</u>	<u>Reactor Supplier</u>	<u>Actual or Expected Date of Startup</u>
Prairie Island 1,2	Goodhue, MN	520	PWR	W	12/73
		520	PWR	W	12/74
Quad Cities 1,2	Rock Island, IL	800	BWR	GE	8/72
		800	BWR	GE	10/72
Rancho Seco	Sacramento, CA	913	PWR	B&W	4/75
River Bend 1,2	West Feliciani, LA	940	BWR	GE	4/84
		940	BWR	GE	Indef
Robinson 2	Darlington, SC	665	PWR	W	3/71
St. Lucie 1,2	St. Lucie, FL	777	PWR	C-E	12/76
		777	PWR	C-E	5/83
Salem 1,2*	Salem, NJ	1090	PWR	W	6/77
		1115	PWR	W	1/81
San Onofre 1,2,3	San Diego, CA	436	PWR	W	1/68
		1100	PWR	C-E	12/81
		1100	PWR	C-E	2/83
Seabrook 1,2	Rockingham, NH	1150	PWR	W	12/83
		1150	PWR	W	/85
Sequoyah 1,2	Hamilton, TN	1148	PWR	W	/80
		1148	PWR	W	6/81
Shearon Harris 1,2, 3,4	Wake & Chatham, NC	900	PWR	W	3/85
		900	PWR	W	3/88
		900	PWR	W	3/94
		900	PWR	W	3/92
Shoreham	Suffolk, NY	820	BWR	GE	3/83
Skagit 1,2	Skagit, WA	1288	BWR	GE	Indef
		1288	BWR	GE	Indef
South Texas 1,2	Matagorda, TX	1250	PWR	W	4/84
		1250	PWR	W	4/86
Surry 1,2	Surry, VA	775	PWR	W	12/72
		775	PWR	W	5/73
Susquehanna 1,2	Luzerne, PA	1050	BWR	GE	1/82
		1050	BWR	GE	1/83
Three Mile Island 1,2	Dauphin, PA	792	PWR	GE	9/74
		880	PWR	W	12/78
Trojan	Columbia, OR	1130	PWR	W	5/76
Turkey Pt. 3,4	Dade, FL	666	PWR	W	12/72
		666	PWR	W	9/73
Vermont Yankee	Windham, VT	514	BWR	GE	11/72
Virgil Summer	Fairfield, SC	900	PWR	W	6/81
Vogtle 1,2	Burke, GA	1100	PWR	W	/85
		1100	PWR	W	/88
WPPSS 1,2,4	Benton, WA	1250	PWR	B&W	6/85
		1100	BWR	GE	1/83
		1250	PWR	B&W	6/86
WPPSS 3,5	Grays Harbor, WA	1240	PWR	C-E	6/86
		1240	PWR	C-E	6/87
Waterford 3	St. Charles, LA	1165	PWR	C-E	/82
Watts Bar 1,2	Rhea, TN	1177	PWR	W	9/81
		1177	PWR	W	6/82
Wolf Creek	Coffey, KS	1150	PWR	W	4/83
Yankee Rowe	Franklin, MA	175	PWR	W	6/61
Yellow Creek 1,2	Tishomingo, MS	1285	PWR	C-E	11/85
		1285	PWR	C-E	4/88
Zimmer	Clermont, OH	810	BWR	GE	/81
Zion 1,2	Lake, IL	1100	PWR	W	6/73
		1100	PWR	W	12/73

\*Same site as Hope Creek

Table A.1-2 General Site Data

Plant	Number	Site	Latitude	Longitude	Meteorological Station	Sheltering Region	State
Allens Creek	1		29-40-43	96-06-15	Fort Worth (14)	3	TX
Arkansas	2		35-18-42	93-13-15	Columbia (10)	7	AR
Bailly	3		41-38-30	87-07-30	Chicago (9)	2	IN
Beaver Valley	4		40-37-18	80-26-06	Washington, DC (29)	1	PA
Bellefonte	5		34-42-32	85-55-36	Nashville (23)	7	AL
Big Rock Point	6		45-21-32	85-11-45	Milwaukee (21)	2	MI
Black Fox	7		36-07-01	95-32-54	Columbia (10)	3	OK
Braidwood	8		41-14-37	89-13-44	Moline (22)	4	IL
Browns Ferry	9		34-42-13	87-07-16	Nashville (23)	7	AL
Brunswick	10		33-57-32	78-01-15	Cape Hatteras (6)	6	NC
Byron	11		42-04-30	89-16-55	Moline (22)	4	IL
Callaway	12		38-45-42	91-46-52	Columbia (10)	4	MO
Calvert Cliffs	13		38-25-39	76-25-35	Washington, DC (29)	6	MD
Catawba	14		35-03-05	81-04-10	Nashville (23)	6	SC
Cherokee	15		35-02-12	81-30-43	Nashville (23)	6	SC
Clinton	16		40-10-19	88-50-03	Moline (22)	4	IL
Comanche Peak	17		32-17-49	97-47-07	Ft. Worth (14)	3	TX
Cooper	19		40-21-41	95-38-17	Omaha (25)	4	NB
Crystal River	20		28-57-26	82-41-56	Apalachicola (2)	7	FL
Davis-Besse	21		41-35-42	83-05-11	Chicago (9)	2	OH
Diablo Canyon	22		35-12-41	120-51-08	Santa Maria (27)	5	CA
Donald C. Cook	18		41-58-44	86-33-43	Chicago (9)	2	MI
Dresden	23		41-23-23	88-16-17	Moline (22)	4	IL
Duane Arnold	24		42-05-54	91-46-21	Omaha (25)	4	IA
Fermi	26		41-58-41	83-15-34	Chicago (9)	2	MI
Fitzpatrick*	27		43-31-19	76-23-54	Milwaukee (21)	1	NY
Forked River**	28		39-48-36	74-12-36	New York (24)	1	NJ
Ft. Calhoun	29		41-31-12	96-04-50	Omaha (25)	4	NB
Ft. St. Vrain	30		40-14-40	104-52-27	Dodge City (11)	4	CO
Ginna	31		43-16-39	77-18-30	Milwaukee (21)	1	NY
Grand Gulf	32		32-00-27	91-02-53	Lake Charles (17)	7	MS
Haddem Neck	33		41-28-56	72-29-57	New York (24)	1	CT
Hartsville	34		36-21-15	86-05-10	Nashville (23)	7	TN
Hatch	35		31-56-05	82-20-40	Charleston (8)	6	CA
Hope Creek***	92		39-27-46	75-32-08	Washington, DC (29)	1	NJ
Indian Point	36		41-15-57	73-56-06	New York (24)	1	NY
Joseph M. Farley	25		31-13-21	85-06-42	Lake Charles (17)	7	AL
Keweenaw	37		44-19-34	87-31-27	Milwaukee (21)	2	WI
LaCrosse	39		43-33-36	91-13-42	Madison (18)	2	WI
LaSalle	38		41-14-24	88-40-12	Moline (22)	4	IL
Limerick	40		40-13-12	75-35-24	Washington, DC (29)	1	PA
Maine Yankee	42		43-57-02	69-41-48	Caribou (7)	1	ME
Marble Hill	41		38-26-00	85-26-53	Moline (22)	2	IN
McGuire	43		35-25-59	80-56-55	Nashville (23)	6	NC
Midland	44		43-35-10	84-13-08	Milwaukee (21)	2	MI

\*Same site as Nine Mile Point

\*\*Same site as Oyster Creek

\*\*\*Same site as Salem

Table A.1-2 General Site Data (cont)

Plant	Number	Site	Latitude	Longitude	Meteorological Station	Sheltering Region	State
Millstone	45		41-18-32	72-10-04	Boston (4)	1	CT
Monticello	46		45-20-03	93-50-55	Madison (18)	2	MN
Nine Mile Point*	47		43-31-19	76-23-54	Milwaukee (21)	1	NY
North Anna	48		38-03-48	77-47-13	Washington, DC (29)	6	VA
Oconee	49		34-47-40	82-53-55	Nashville (23)	6	SC
Oyster Creek**	50		39-48-50	74-12-41	New York (24)	1	NJ
Palisades	51		42-19-24	86-18-52	Chicago (9)	2	MI
Palo Verde	52		33-23-25	112-51-45	Phoenix (26)	3	AZ
Peach Bottom	53		39-45-33	76-16-08	Washington, DC (29)	1	PA
Pebble Springs	54		45-42-05	120-08-17	Medford (19)	5	OR
Perkins	55		35-50-53	80-27-10	Nashville (23)	6	NC
Perry	56		41-48-03	81-08-36	Chicago (9)	2	OH
Phipps Bend	57		36-27-47	82-48-32	Nashville (23)	7	TN
Pilgrim	58		41-56-40	70-34-41	Boston (4)	1	MA
Point Beach	59		44-16-35	87-31-08	Milwaukee (21)	2	WI
Prairie Island	60		44-37-25	92-38-04	Madison (18)	2	MN
Quad Cities	61		41-43-38	90-20-30	Moline (22)	4	IL
Rancho Seco	62		38-21-00	121-07-12	Fresno (15)	5	CA
River Bend	63		30-45-26	91-19-54	Lake Charles (17)	7	LA
Robinson	64		34-24-12	80-09-30	Nashville (23)	6	SC
St. Lucie	65		27-20-55	80-14-47	Miami (20)	7	FL
Salem †	66		39-27-46	75-32-08	Washington, DC (29)	1	NJ
San Onofre	67		33-2-53	117-31-17	Santa Maria (27)	5	CA
Seabrook	68		42-53-53	70-51-05	Boston (4)	1	NH
Sequoiah	69		35-13-31	85-05-13	Nashville (23)	7	TN
Shearon Harris	70		35-38-00	78-57-22	Nashville (23)	6	NC
Shoreham	72		40-57-30	72-52-00	New York (24)	1	NY
Skagit	71		48-32-00	122-07-26	Seattle (28)	5	WA
South Texas	73		28-47-42	96-02-53	Brownsville (5)	3	TX
Surry	75		37-10-00	76-41-50	Washington, DC (29)	6	VA
Susquehanna	76		41-06-00	76-09-00	Washington, DC (29)	1	PA
Three Mile Island	77		40-09-12	76-43-37	Washington, DC (29)	1	PA
Trojan	78		46-02-24	122-52-06	Medford (19)	5	OR
Tuckey Point	79		25-26-02	80-19-54	Miami (20)	7	FL
Vermont Yankee	80		42-46-49	72-30-57	Caribou (7)	1	VT
Virgil Summer	74		34-17-54	81-18-55	Nashville (23)	6	SC
Vogtle	81		33-08-31	81-45-53	Charleston (8)	6	CA
WPSS 1,2,4††	84		46-28-03	119-18-51	Medford (19)	5	WA
WPSS 3,5	85		46-57-11	123-28-11	Medford (19)	5	WA
Waterford	82		30-00-00	90-28-12	Lake Charles (17)	7	LA
Watts Bar	83		35-36-10	84-47-25	Nashville (23)	7	TN
Wolf Creek	87		38-14-20	95-41-20	Omaha (25)	4	KN
Yankee Rowe	88		42-43-41	72-55-29	New York (24)	1	MA
Yellow Creek	89		34-57-24	88-12-57	Nashville (23)	7	MS
Zimmer	90		38-51-55	84-13-45	Nashville (23)	2	OH
Zion	91		42-27-34	87-48-23	Chicago (9)	4	IL

\*Same site as Fitzpatrick

\*\*Same site as Forked river

†Same site as Hope Creek

††Same site as Skagit

Table A.1-3 Sheltering Regions

<u>Region Number</u>	<u>Location</u>	<u>% Brick Housing Units</u>	<u>% Homes With Basements</u>	<u>Shielding Factor*</u>	
				<u>Cloud</u>	<u>Ground</u>
1	Northeast	47	87	0.5	0.08
2	Great Lakes	36	77	0.6	0.1
3	Southwest	40	13	0.7	0.3
4	Midwest	35	71	0.5	0.09
5	Pacific Coast	27	23	0.7	0.3
6	Atlantic Coast	45	51	0.6	0.2
7	Southeast	59	16	0.7	0.2

A-7

\*The ratio of dose received when sheltered to the dose that would be received if outdoors. Cloud refers to gamma exposure from radionuclides dispersed in the atmosphere. Ground refers to gamma exposure from ground-deposited radionuclides.

## A.2 Population Data

CRAC2 requires a description of the population distribution surrounding the reactor site being evaluated. Distributions are input as population counts for individual spatial elements. These elements are the cells in a polar grid consisting of up to 34 annuli and 16 sectors (each 22 1/2° in width). This study used 34 annuli, with radii of 0.5, 1, 1.5, 2, 2.5, 3, 3.5, 4, 4.5, 5, 6, 7, 8.5, 10, 12.5, 15, 17.5, 20, 25, 30, 35, 40, 45, 50, 55, 60, 65, 70, 85, 100, 150, 200, 350, and 500 miles. The population distribution for each site was derived from 1970 census data using a program called SECPOP which was developed by the Office of Radiation Programs, Environmental Protection Agency.\* SECPOP constructs a polar grid from user-specified annular radii and number of sectors. This grid is centered on a location specified by latitude and longitude. A data file containing census data is then scanned to determine which enumeration district centroids fall into each spatial element. The population of each enumeration district is considered to be wholly within the spatial element in which its centroid falls. While this is an approximation, especially in sparsely populated areas for which the centroids are widely dispersed, it has an accuracy comparable to much of the other data used as input to CRAC2. In addition, the nature of the inaccuracy is such that it should have a very limited impact on conclusions drawn from exercising the model. The latitudes and longitudes for the 91 sites are provided in Table A.1-2. Summary population statistics for each site are provided in Chapter 3 and Appendix E.

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\*Technical Memorandum 73-146, U.S. Department of Commerce, Office of Telecommunications.

### A.3 Weather Data

CRAC2 requires an input file containing 8760 hourly weather observations (one year). The hourly observations consist of wind speed, wind direction, stability class, and precipitation. These data are used in the dispersion/ deposition submodel to determine the rate at which the radioactive plume travels, disperses, and is depleted.

Past studies have typically employed data gathered by a licensee over a one-year period at a proposed site, usually as part of the license application. For this study we have selected 29 National Weather Service (NWS) stations as the sources of meteorological data. NWS data are available for a large number of sites, cover long periods of time, are generally of higher quality, and are more detailed than actual reactor site data. Each of the NWS stations selected has approximately 25 years of available data. Therefore, rather than select a single year at random, a Typical Meteorological Year (TMY) [3] was used to represent the long-term average behavior of the weather at a station. The technique used to determine a TMY involves comparing the distribution of certain weather characteristics for a given month over the entire period of record. Using statistical techniques described in reference [3], the one month "most typical" of the period is selected as part of the TMY. This procedure was performed for each of the twelve calendar months to obtain the TMY. In addition, a small amount of smoothing is performed at the boundaries between months to avoid abrupt changes in weather conditions.

The criteria used to generate the TMYs were selected based on their relevance to solar heating simulations and include temperature, wind speed, and insolation. Since these parameters are correlated to the data required for the CRAC2 input, the TMYs are considered to be reasonably representative years to use as input to the consequence model. These data are probably better than the single year weather data used in the past which are of uncertain quality and are subject to the anomalies of a single year's weather.

The TMYs are available from the National Climatic Center (NCC), Asheville, NC. The data tapes supplied by the NCC are not compatible with CRAC2 requirements. In addition, these tapes do not contain a classification of stability class. A conversion program, METDAT, was

developed by Science Applications, Inc. (SAI) under contract to Sandia. This program uses CRSTER [4], developed by the National Oceanic and Atmospheric Administration (NOAA), to generate the stability class using the insolation and wind speed data available in the TMY tapes.

CRAC2 requires rainfall intensity data for each hourly observation. Like atmospheric stability, rainfall data are not available on the TMY tapes. Therefore, rainfall statistics were gathered from other NWS data and were merged with the TMY information using the METDAT program.

The diffusion model used in CRAC2 also takes into account mixing height during dispersion calculations. The mixing height can affect the vertical diffusion of the radionuclide plume because mixing is essentially terminated at these levels. The mixing heights used for the 29 NWS stations were determined from the Holzworth isopleths of mean annual afternoon mixing height [5] (see Figure A.3-1). Table A.3-1 lists the 29 NWS stations with the assigned mixing heights. Figure A.3-2 shows the location of these stations in addition to the locations of the 91 reactor sites.

The meteorological data used for each of these 29 stations are summarized in Table A.3-2 in terms of the weather bin categories described in Appendix F. Additional rainfall data for the 29 stations are included in Table A.3-3.

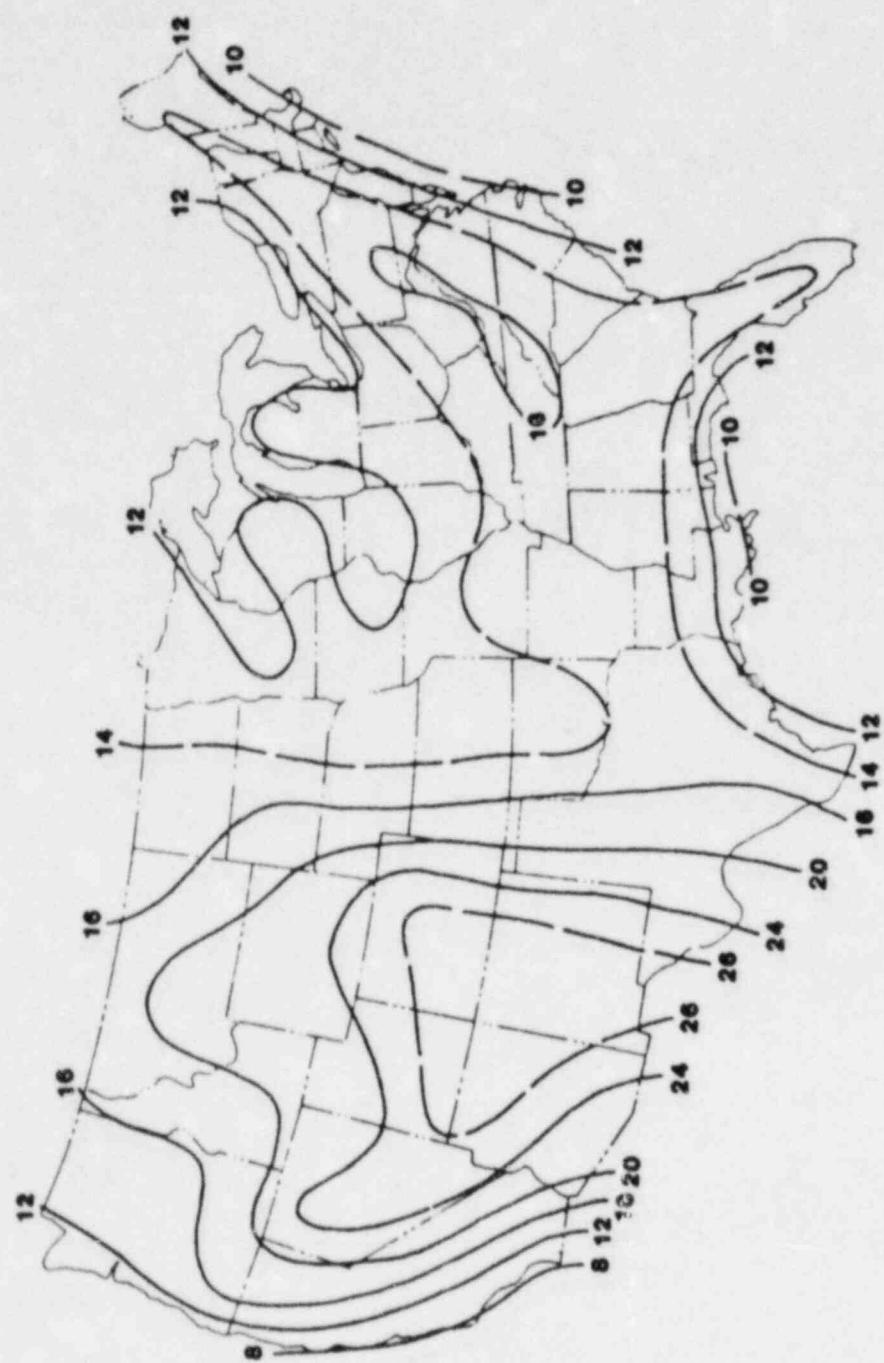


Figure A.3-1 Isopleths ( $m \times 10^{-2}$ ) of Mean Annual Afternoon Mixing Heights. From reference [5].

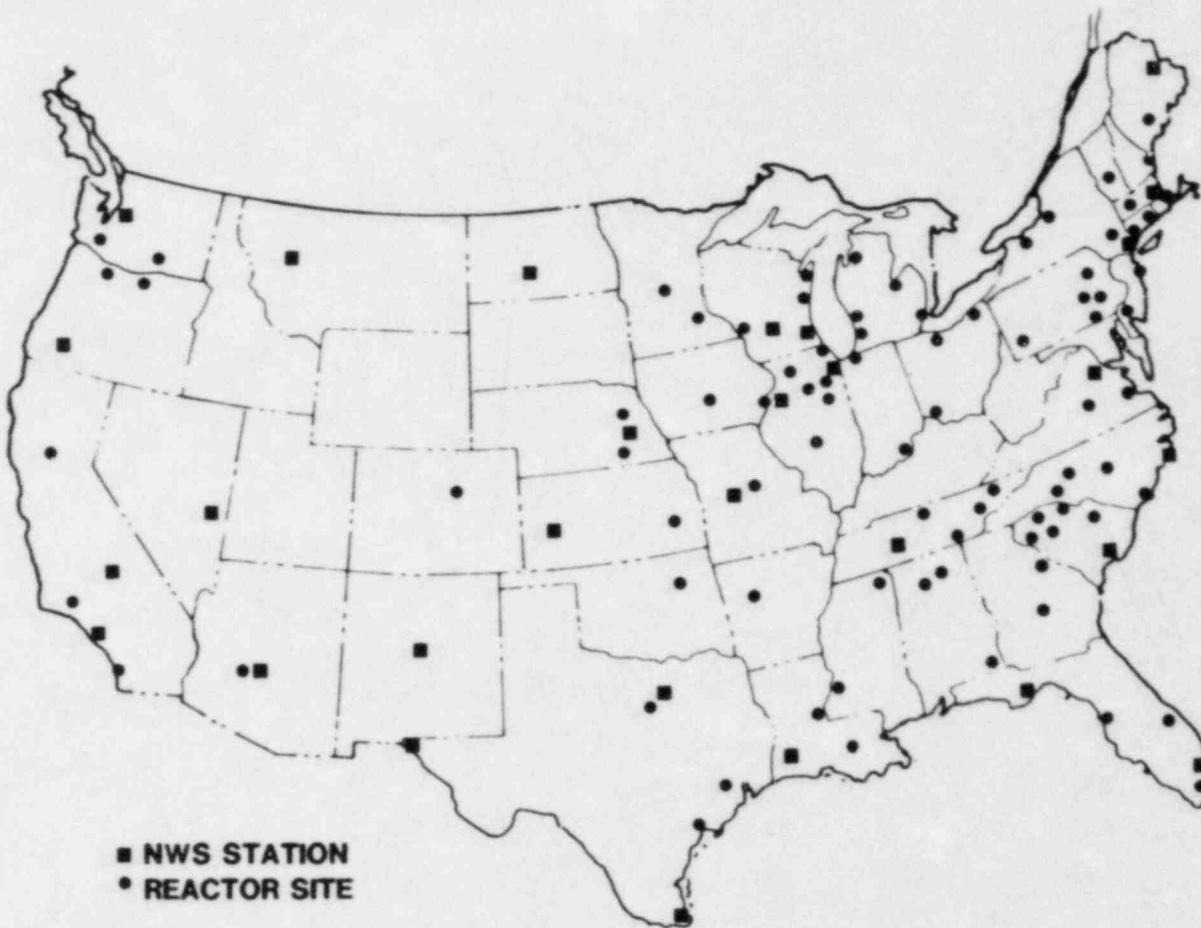


Figure A.3.2 Geographic location of the 29 NWS Stations and the 91 Reactor sites.

Table A.3-1 NWS Station Locations and Mixing Heights

No.	Station	Mixing Height (m)	No.	Station	Mixing Height (m)
1	Albuquerque, NM	2600	16	Great Falls, MT	2000
2	Apalachicola, FL	1200	17	Lake Charles, LA	1100
3	Bismarck, ND	1500	18	Madison, WI	1200
4	Boston, MA	1100	19	Medford, OR	1600
5	Brownsville, TX	1300	20	Miami, FL	1200
6	Cape Hatteras, NC	1000	21	Milwaukee, WI	1200
7	Caribou, ME	1300	22	Moline, IL	1200
8	Charleston, SC	1300	23	Nashville, TN	1600
9	Chicago, IL	1200	24	New York, NY	1200
10	Columbia, MO	1200	25	Omaha, NB	1300
11	Dodge City, KS	1600	26	Phoenix, AZ	2400
12	El Paso, TX	2600	27	Santa Maria, CA	800
13	Ely, NV	2400	28	Seattle, WA	1200
14	Fort Worth, TX	1500	29	Washington, DC	1500
15	Fresno, CA	1600			

Table A.3-2 Meteorological Data for 29 NWS Stations Summarized  
Using Weather Bin Categories

Weather Bin Definitions

R - Rain starting within indicated interval (miles).

S - Slowdown occurring within indicated interval (miles).

A-C D E F - Stability categories

1(0-1), 2(1-2), 3(2-3), 4(3-5), 5(GT 5) - Wind speed intervals (m/s).

Percent of Weather Sequences

Weather Bin	Albuquerque (1)	Apalachicola (2)	Bismarck (3)	Boston (4)	Brownsville (5)	Cape Hatteras (6)	Caribou (7)	Charleston (8)	Chicago (9)	Columbia (10)	Dodge City (11)	El Paso (12)
1 R (0)	1.46	4.50	3.94	8.89	2.25	6.69	10.14	5.87	6.19	6.26	3.69	1.30
2 R (0-5)	0.09	0.70	0.15	0.17	0.06	0.11	0.38	0.29	0.15	0.11	0.11	0.06
3 R (5-10)	0.31	1.14	0.40	0.79	0.39	0.75	1.26	0.88	0.68	0.75	0.27	0.26
4 R (10-15)	0.55	1.34	0.67	1.24	0.49	1.12	1.60	1.32	1.21	0.91	0.58	0.51
5 R (15-20)	0.33	1.11	0.76	0.82	0.54	1.02	1.28	0.81	0.87	0.91	0.37	0.34
6 R (20-25)	0.37	0.99	0.55	0.90	0.53	0.83	1.12	0.87	0.68	0.76	0.55	0.32
7 R (25-30)	0	0.26	0.66	0.94	0.42	0.83	1.29	0.99	0.86	0.76	0.50	0.34
8 S (0-10)	2.00	1.36	1.02	0.55	0.34	0.14	0.53	0.51	0.51	0.53	0.24	0.98
9 S (10-15)	2.01	1.02	0.90	0.43	0.27	0.08	0.42	0.43	0.41	0.42	0.25	0.96
10 S (15-20)	1.78	1.04	0.63	0.50	0.27	0.09	0.40	0.33	0.35	0.39	0.14	0.91
11 S (20-25)	1.55	1.02	0.73	0.37	0.21	0.07	0.29	0.39	0.38	0.32	0.15	0.71
12 S (25-30)	1.62	1.19	0.88	0.45	0.31	0.14	0.33	0.39	0.28	0.45	0.18	0.89
13 A-C 1,2,3	12.97	6.44	4.22	1.51	1.18	1.66	4.29	3.05	2.66	3.32	2.48	11.08
14 A-C 4,5	11.08	15.70	7.11	7.52	11.46	12.48	5.48	13.11	10.98	13.53	13.03	14.74
15 D 1	0.00	0.00	0.00	0.00	0.00	0.00	0.00	0.00	0.00	0.00	0.00	0.00
16 D 2	1.51	2.19	1.71	0.74	0.59	0.21	1.82	1.06	1.02	0.92	0.43	1.31
17 D 3	3.07	2.81	3.18	1.77	1.95	1.67	4.49	3.41	3.62	3.05	1.61	2.91
18 D 4	4.81	7.72	8.56	9.63	7.33	8.50	10.92	12.45	11.90	11.18	7.39	5.89
19 D 5	19.29	12.31	35.99	45.75	43.07	38.66	31.10	19.92	32.15	27.92	49.13	20.50
20 E 1	0.00	0.00	0.00	0.00	0.00	0.00	0.00	0.00	0.00	0.00	0.00	0.00
21 E 2	1.26	1.85	1.11	0.23	0.54	0.26	0.53	0.83	0.48	0.50	0.09	1.53
22 E 3	3.15	2.48	1.91	0.79	2.44	1.23	2.43	4.01	2.20	2.00	0.67	3.15
23 E 4	7.87	5.34	6.21	6.36	7.28	9.68	6.71	7.57	7.25	9.06	7.68	6.45
24 E 5	2.35	1.85	1.67	3.13	2.69	3.01	2.09	1.80	2.84	2.23	3.74	2.51
25 F 1	0.00	0.00	0.00	0.00	0.00	0.00	0.00	0.00	0.00	0.00	0.00	0.00
26 F 2	6.94	14.51	7.71	1.13	3.69	1.56	3.11	8.17	2.75	2.32	0.72	9.59
27 F 3	7.50	6.46	5.48	1.80	6.40	4.20	4.75	6.92	4.93	4.73	2.24	8.32
28 F 4	5.78	4.01	3.85	3.58	5.30	5.00	3.28	4.61	4.60	6.74	3.74	4.42
29 F 5	0.00	0.00	0.00	0.00	0.00	0.00	0.00	0.00	0.00	0.00	0.00	0.00

Table A.3-2 Meteorological Data for 29 NWS Stations Summarized  
Using Weather Bin Categories (cont)

Weather Bin Definitions

R - Rain starting within indicated interval (miles).

S - Slowdown occurring within indicated interval (miles).

A-C D E F - Stability categories

1(0-1), 2(1-2), 3(2-3), 4(3-5), 5(GT 5) - Wind speed intervals (m/s).

Percent of Weather Sequences

Weather Bin	Ely (13)	Fort Worth (14)	Presno (15)	Great Falls (16)	Lake Charles (17)	Madison (18)	Medford (19)	Miami (20)	Milwaukee (21)	Moline (22)	Nashville (23)	New York (24)
1 R (0)	3.06	3.97	2.09	5.56	3.73	6.08	4.61	4.37	6.12	5.84	6.60	7.96
2 R (0-5)	0.36	0.10	0.11	0.40	0.32	0.24	1.37	0.32	0.18	0.11	0.18	0.14
3 R (5-10)	0.65	0.47	0.56	0.94	1.00	0.98	1.56	1.14	0.66	0.79	0.79	0.71
4 R (10-15)	0.65	0.66	0.49	1.11	0.98	1.28	1.59	1.34	1.20	1.03	1.04	1.16
5 R (15-20)	0.66	0.45	0.32	0.82	0.68	1.03	1.13	1.15	0.84	0.83	0.90	0.86
6 R (20-25)	0.57	0.45	0.40	0.59	0.76	0.94	1.13	1.02	0.71	0.66	0.01	0.76
7 R (25-30)	0.51	0.48	0.39	0.76	0.66	0.98	1.19	1.31	0.88	0.80	0.73	0.70
8 S (0-10)	0.86	0.49	0.90	0.59	0.51	0.94	1.47	0.62	0.59	0.47	0.73	0.27
9 S (10-15)	0.32	0.33	0.81	0.39	0.43	0.73	1.37	0.50	0.40	0.32	0.66	0.18
10 S (15-20)	0.73	0.25	0.70	0.40	0.35	0.75	1.20	0.49	0.34	0.35	0.65	0.21
11 S (20-25)	0.28	0.33	0.62	0.34	0.38	0.58	1.27	0.41	0.32	0.41	0.68	0.16
12 S (25-30)	0.64	0.33	0.78	0.33	0.42	0.68	1.29	0.53	0.43	0.35	0.70	0.21
13 A-C 1,2,3	9.60	4.12	16.69	4.49	3.97	3.38	15.49	3.46	2.25	3.50	4.40	1.92
14 A-C 4,5	13.70	14.92	7.45	8.12	11.58	8.64	6.06	15.70	9.68	10.73	11.18	10.18
15 D 1	0.00	0.00	0.00	0.00	0.00	0.00	0.00	0.00	0.00	0.00	0.00	0.00
16 D 2	1.54	0.67	4.65	1.36	1.35	2.40	10.54	0.95	1.26	1.71	2.23	0.70
17 D 3	3.12	2.35	5.91	2.92	4.87	3.90	7.31	2.39	2.53	4.68	3.86	2.58
18 D 4	8.57	9.57	4.94	8.64	13.79	11.86	4.50	8.89	10.61	10.82	9.66	10.82
19 D 5	25.41	31.63	7.21	42.24	19.93	29.43	5.27	17.64	36.80	29.33	19.65	37.96
20 E 1	0.00	0.00	0.00	0.00	0.00	0.00	0.00	0.00	0.00	0.00	0.00	0.00
21 E 2	0.59	0.43	2.40	0.55	0.75	1.26	2.93	1.16	0.78	1.63	1.36	0.31
22 E 3	1.78	2.10	3.85	2.34	3.89	1.97	3.26	3.73	0.70	2.56	3.36	1.91
23 E 4	10.75	8.80	6.37	6.28	6.29	5.40	2.11	8.20	6.90	5.74	6.06	7.79
24 E 5	3.78	2.88	2.39	2.79	0.99	1.24	0.45	1.97	2.11	1.47	1.07	3.08
25 F 1	0.00	0.00	0.00	0.00	0.00	0.00	0.00	0.00	0.00	0.00	0.00	0.00
26 F 2	2.82	2.90	13.63	2.32	6.95	8.12	13.89	8.06	5.22	8.24	7.25	1.32
27 F 3	4.29	5.14	11.28	3.09	9.62	4.32	7.65	8.54	3.78	5.32	8.26	3.54
28 F 4	4.81	6.18	5.07	2.64	5.75	2.96	1.26	6.12	3.71	3.49	4.41	4.59
29 F 5	0.00	0.00	0.00	0.00	0.00	0.00	0.00	0.00	0.00	0.00	0.00	0.00

Table A.3-2 Meteorological Data for 29 NWS Stations Summarized  
Using Weather Bin Categories (cont)

Weather Bin Definitions

R - Rain starting within indicated interval (miles).

S - Slowdown occurring within indicated interval (miles).

A-C D E F - Stability categories

1(0-1), 2(1-2), 3(2-3), 4(3-5), 5(GT 5) - Wind speed intervals (m/s).

Percent of Weather Sequences

<u>Weather Bin</u>	<u>Omaha (25)</u>	<u>Phoenix (26)</u>	<u>Santa Maria (27)</u>	<u>Seattle (28)</u>	<u>Washington (29)</u>
1 R (0)	5.43	1.00	2.24	8.72	5.79
2 R (0-5)	0.13	0.08	0.19	0.42	0.39
3 R (5-10)	0.62	0.31	0.40	1.87	1.28
4 R (10-15)	0.89	0.25	0.62	2.12	1.14
5 R (15-20)	0.70	0.23	0.41	1.90	0.88
6 R (20-25)	0.51	0.24	0.32	1.53	0.87
7 R (25-30)	0.59	0.22	0.43	1.77	0.86
8 S (0-10)	1.16	1.27	2.41	1.36	0.71
9 S (10-15)	0.90	1.21	1.84	1.44	0.67
10 S (15-20)	0.75	1.20	1.63	1.02	0.48
11 S (20-25)	0.67	0.91	1.45	0.98	0.63
12 S (25-30)	0.86	1.13	1.77	1.21	0.63
13 A-C 1,2,3	3.79	16.02	7.96	5.15	7.33
14 A-C 4,5	12.36	15.92	12.53	6.87	11.30
15 D 1	0.00	0.00	0.00	0.00	0.00
16 D 2	1.26	1.52	11.16	2.95	2.98
17 D 3	3.23	3.18	8.66	6.55	6.08
18 D 4	8.87	6.69	6.97	16.12	10.64
19 D 5	30.39	6.30	13.40	19.46	16.20
20 E 1	0.00	0.00	0.00	0.00	0.00
21 E 2	0.99	1.96	2.44	0.72	1.85
22 E 3	2.24	3.57	2.41	2.07	3.52
23 E 4	6.53	6.35	2.42	4.82	5.27
24 E 5	1.77	0.92	0.81	1.02	1.23
25 F 1	0.00	0.00	0.00	0.00	0.00
26 F 2	7.63	11.20	11.16	3.46	9.81
27 F 3	4.17	12.09	4.81	3.80	6.38
28 F 4	3.56	6.22	1.54	2.68	3.09
29 F 5	0.00	0.00	0.00	0.00	0.00

Table A.3-3 Summary of Rainfall Data for  
29 NWS Station TMYs

<u>Station</u>	<u>Hours of Observed Rainfall</u>	<u>Annual Rain (inches)</u>
Albuquerque (1)	128	7
Apalachicola (2)	394	65
Bismarck (3)	345	16
Boston (4)	779	41
Brownsville (5)	197	16
Cape Hatteras (6)	586	49
Caribou (7)	888	31
Charleston (8)	514	52
Chicago (9)	542	37
Columbia (10)	548	37
Dodge City (11)	323	26
El Paso (12)	114	6
Ely (13)	268	10
Fort Worth (14)	348	33
Fresno (15)	183	7
Great Falls (16)	487	16
Lake Charles (17)	327	41
Madison (18)	533	29
Medford (19)	404	17
Miami (20)	383	53
Milwaukee (21)	536	27
Moline (22)	512	37
Nashville (23)	578	49
New York (24)	697	49
Omaha (25)	476	30
Phoenix (26)	88	4
Santa Maria (27)	196	10
Seattle (28)	764	40
Washington (29)	507	32

#### A.4 Site Wind Rose Data

CRAC2 uses a straight-line trajectory model for plume movement, employing the wind speeds in the weather sequence to determine the rate of travel. To calculate the effects of the accident in different directions, CRAC2 uses the wind rose as an empirical distribution for the probability that the plume trajectory will be in a given direction. All consequences are calculated assuming that the plume follows each of the 16 directions, and the results are weighted by the frequency of wind travel in that direction.

The wind rose data for the 91 sites were taken from either the Environmental Reports or the Preliminary or Final Safety Analysis Reports submitted to the Nuclear Regulatory Commission. The site wind roses used in this study are presented in Table A.4-1. A summary histogram of peak to mean wind rose probability ratios for the 91 sites is presented in Figure A.4-1. This histogram illustrates the importance of wind rose probabilities to reactor accident consequence calculations. (The mean wind rose probability is 1/16.)

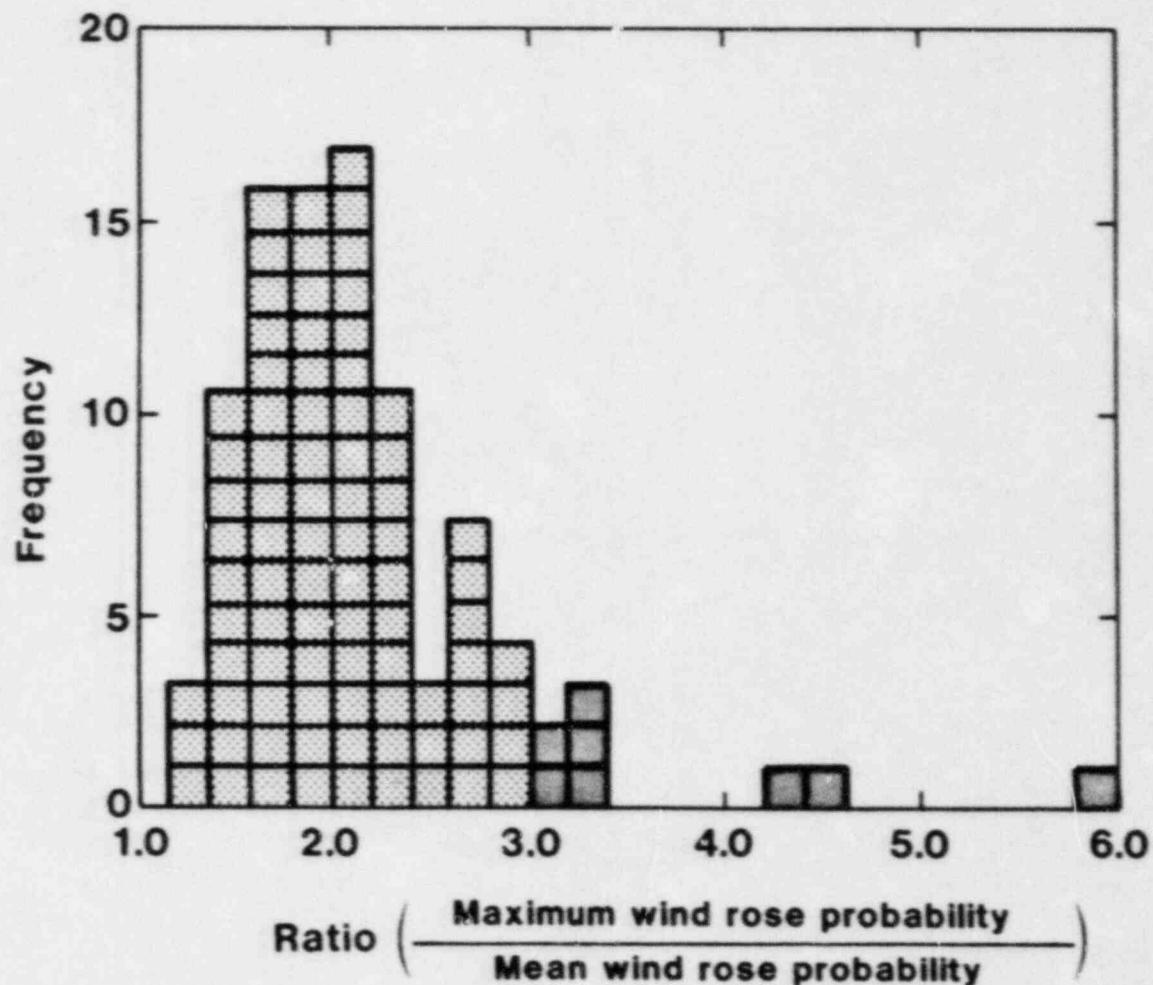
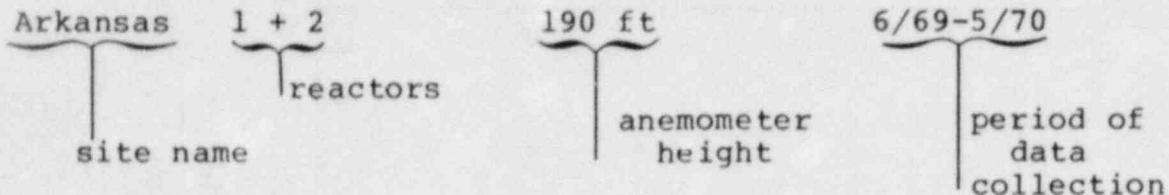


Figure A.4-1 Summary Histogram of Peak to Mean Wind Rose Probability Ratios for the 91 Sites

Table A.4-1  
Site Wind Rose Data

Explanation of Titles:

  
site name      1 + 2 reactors      190 ft      6/69-5/70  
                  reactors                 ft            period of  
  height       data  
   collection

Note: All wind roses in Table A.4-1 are presented as the probability of wind blowing toward the sector indicated. This is the opposite of the conventional definition used by meteorologists.

**Table A.4-1 Site Wind Rose Data**  
**Probability of Wind Blowing Towards Sector**

Station	N S	NEE SWW	NE SW	ESE WNW	E W	ESE WNW	SE NW	SSE NNW
Allens Creek		<u>10 m</u>			<u>8/1/1972 - 7/31/1973</u>			
	.123	.073	.043	.024	.022	.021	.027	.069
	.107	.075	.062	.050	.046	.055	.101	.104
Arkansas 1, 2		<u>190 ft</u>			<u>6/69 - 5/70</u>			
	.103	.074	.052	.074	.126	.087	.053	.021
	.025	.015	.037	.056	.096	.077	.057	.042
Bailly		<u>230 ft</u>			<u>12/4/51 - 12/3/57</u>			
	.064	.105	.095	.086	.069	.056	.040	.038
	.064	.068	.069	.063	.038	.028	.053	.066
Beaver Valley 1, 2		<u>150 ft</u>			<u>9/15/69 - 9/3/70</u>			
	.087	.078	.051	.041	.083	.137	.123	.050
	.055	.023	.021	.023	.040	.059	.067	.064
Bellevue 1		<u>54 ft</u>			<u>1971</u>			
	.064	.075	.092	.082	.071	.067	.060	.076
	.069	.066	.031	.040	.037	.053	.064	.053
Big Rock Point		<u>250 ft</u>			<u>2/63 - 2/63</u>			
	.112	.075	.071	.081	.099	.058	.057	.065
	.056	.039	.034	.046	.088	.037	.037	.045
Black Fox		<u>33 ft</u>			<u>12/73 - 11/74</u>			
	.180	.055	.026	.026	.022	.030	.051	.059
	.067	.064	.056	.045	.034	.046	.079	.160
Braidwood 1		<u>30 ft</u>			<u>11/1/73 - 10/31/74</u>			
	.105	.113	.077	.065	.061	.070	.065	.045
	.052	.048	.048	.045	.043	.044	.056	.063
Browns Ferry 1, 2, 3		<u>300 ft</u>			<u>2/11/67 - 12/31/68</u>			
	.072	.066	.058	.058	.052	.067	.055	.054
	.052	.047	.056	.038	.032	.072	.101	.099
Brunswick 1, 2		<u>350 ft</u>			<u>9/25/70 - 1/5/73</u>			
	.055	.077	.145	.088	.053	.037	.036	.041
	.059	.065	.084	.078	.053	.044	.047	.038
Byron 1		<u>30 ft</u>			<u>6/11/73 - 5/31/74</u>			
	.097	.089	.081	.065	.075	.063	.076	.057
	.053	.037	.048	.058	.049	.044	.039	.069
Callaway		<u>10 m</u>			<u>5/4/73 - 5/4/74</u>			
	.126	.094	.074	.043	.058	.070	.058	.050
	.051	.040	.026	.028	.036	.046	.083	.116
Calvert Clift 1, 2		<u>33 ft</u>			<u>5/25/73 - 5/25/73</u>			
	.116	.089	.070	.045	.064	.081	.103	.078
	.084	.058	.038	.024	.035	.028	.028	.082
Catawba 1		<u>30 ft</u>			<u>6/30/71 - 6/30/72</u>			
	.023	.056	.207	.087	.043	.024	.036	.026
	.075	.079	.179	.060	.033	.025	.040	.017
Cherokee		<u>30 ft</u>			<u>9/11/73 - 9/31/74</u>			
	.036	.048	.124	.104	.094	.081	.114	.052
	.064	.059	.075	.055	.029	.022	.036	.019
Clinton		<u>10 m</u>			<u>5/72 - 6/73</u>			
	.104	.093	.086	.054	.042	.041	.042	.052
	.070	.068	.071	.056	.054	.038	.049	.067
Commerce Peak		<u>10 m</u>			<u>5/12/72 - 5/14/76</u>			
	.151	.084	.041	.025	.024	.029	.067	.076
	.060	.040	.029	.025	.032	.060	.105	.149

Table A.4-1 Site Wind Rose Data (cont)  
Probability of Wind Blowing Towards Sector

Station	N S	NNE SSW	NE SW	ENE WSW	E W	ESE WNW	SE NW	SSE NNW
Cook DC 1, 2		200 ft				1967		
	.091	.105	.055	.045	.051	.069	.057	.062
	.078	.042	.042	.050	.040	.063	.072	.073
Cooper		338 ft				3/70 - 3/71		
	.116	.117	.079	.037	.030	.041	.060	.100
	.094	.061	.025	.031	.027	.034	.058	.090
Crystal River		33 ft				1/1/75 - 12/31/75		
	.043	.048	.051	.048	.082	.057	.043	.030
	.062	.047	.098	.121	.111	.064	.061	.034
Davis-BE 1		35 ft				8/4/74 - 8/3/75		
	.064	.116	.130	.102	.081	.039	.053	.037
	.030	.019	.058	.057	.077	.041	.038	.039
Diablo Canyon 1, 2		250 ft				10/69 - 9/70		
	.031	.012	.014	.015	.026	.045	.363	.128
	.059	.029	.055	.017	.014	.015	.103	.075
Dresden 2, 3		300 ft						
	.088	.090	.096	.067	.101	.085	.080	.056
	.049	.031	.039	.033	.036	.033	.060	.055
Duane Arnold		165 ft				1971		
	.129	.073	.053	.036	.051	.062	.083	.095
	.075	.040	.032	.034	.039	.060	.061	.076
Farley 1, 2		33 ft						
	.073	.070	.064	.044	.044	.045	.067	.090
	.097	.083	.086	.062	.044	.035	.040	.056
Fermi 2						9/1/73 - 8/31/74		
	.041	.088	.089	.102	.083	.086	.063	.047
	.026	.025	.059	.063	.069	.050	.058	.058
Fitzpatrick		200 ft				1963 - 1964		
	.087	.059	.102	.132	.115	.056	.053	.035
	.040	.047	.033	.014	.018	.037	.101	.068
Forked River		400 ft				2/66 - 2/67		
	.075	.096	.087	.068	.087	.093	.075	.063
	.044	.037	.052	.055	.039	.040	.047	.040
Fort Calhoun		40 ft				10/68 - 9/70		
	.093	.059	.034	.021	.042	.079	.113	.098
	.071	.018	.017	.022	.028	.064	.115	.126
Fort St. Vrain		205 ft				1967 - 1968		
	.063	.069	.076	.057	.040	.029	.035	.039
	.164	.085	.076	.064	.058	.043	.051	.049
Ginnia R.E.		50 ft				1966 - 1967		
	.090	.081	.102	.097	.112	.101	.079	.044
	.030	.032	.031	.038	.045	.036	.030	.052
Grand Gulf 1						1951 - 1960		
	.101	.074	.062	.043	.036	.043	.070	.064
	.065	.060	.061	.040	.040	.044	.080	.117
Haddam Neck		129 ft				1963		
	.048	.046	.043	.038	.070	.160	.265	.052
	.011	.006	.009	.013	.035	.092	.055	.055
Hartsville		33 ft				2/1/73 - 1/31/74		
	.045	.058	.048	.056	.151	.034	.044	.025
	.045	.113	.175	.063	.150	.074	.069	.051

Table A.4-1 Site Wind Rose Data (cont)

Probability of Wind Blowing Towards Sector

Station	N S	NNE SSW	NE SW	ENE NW	E W	ESE WNW	SE NW	SSE NNW
Hatch, E.I. 1, 2		<u>150 ft</u>				<u>6/1/70 - 8/31/74</u>		
	.955 .040	.069 .038	.082 .051	.073 .067	.075 .081	.077 .068	.072 .057	.049 .044
Indian Point 2, 3		<u>100 ft</u>				<u>1/1/71 - 12/31/71</u>		
	.076 .124	.055 .135	.038 .066	.039 .027	.053 .019	.079 .019	.077 .041	.070 .063
Keweenaw		<u>180 ft</u>				<u>8/31/68 - 3/25/70</u>		
	.032 .066	.090 .055	.064 .042	.075 .030	.094 .022	.117 .023	.082 .028	.080 .050
LaSalle 1, 2		<u>300 ft</u>						
	.088 .049	.090 .031	.096 .039	.067 .033	.101 .036	.085 .033	.080 .060	.056 .055
La Crosse		<u>350 ft</u>				<u>1968 - 1970</u>		
	.194 .125	.139 .101	.084 .048	.018 .011	.051 .022	.026 .010	.076 .025	.062 .033
Limerick 1		<u>270 ft</u>				<u>1/72 - 12/74</u>		
	.071 .054	.068 .039	.052 .035	.051 .046	.090 .0670	.150 .040	.109 .037	.059 .040
Marble Hill		<u>33 ft</u>				<u>1/74 - 12/74</u>		
	.058 .045	.141 .044	.124 .063	.074 .060	.062 .047	.060 .030	.044 .030	.037 .041
Me Yankee		<u>149 ft</u>				<u>7/67 - 6/68</u>		
	.118 .075	.124 .068	.082 .064	.041 .030	.041 .024	.055 .027	.088 .031	.089 .044
McGuire 1, 2		<u>130 ft</u>				<u>10/17/70 - 10/16/71</u>		
	.070 .057	.090 .068	.122 .113	.062 .078	.054 .056	.042 .037	.042 .038	.040 .030
Midland 2						<u>1962 - 1966</u>		
	.060 .045	.082 .046	.123 .061	.106 .043	.124 .045	.066 .024	.064 .028	.051 .032
Hillstone 1, 2		<u>152 ft</u>				<u>8/65 - 9/67</u>		
	.038 .066	.060 .060	.076 .036	.170 .035	.078 .058	.070 .035	.078 .025	.073 .041
Monticello		<u>140 ft</u>				<u>2/9/67 - 2/10/68</u>		
	.089 .036	.091 .041	.063 .029	.055 .051	.030 .031	.089 .055	.104 .052	.119 .065
Nine M. Ft. 1, 2		<u>204 ft</u>				<u>1963 - 1964</u>		
	.082 .041	.060 .048	.104 .034	.131 .013	.118 .018	.059 .037	.054 .097	.037 .069
North Anna 1, 2, 3		<u>150 ft</u>				<u>9/16/71 - 9/15/72</u>		
	.141 .100	.095 .048	.058 .044	.047 .035	.055 .041	.047 .035	.074 .042	.084 .054
Oconee 1, 2, 3						<u>6/19/68 - 6/19/69</u>		
	.021 .174	.036 .084	.075 .100	.051 .058	.062 .060	.043 .038	.061 .036	.081 .019
Oyster Creek		<u>400 ft</u>				<u>2/66 - 2/67</u>		
	.075 .044	.096 .037	.087 .052	.068 .055	.087 .039	.093 .040	.075 .047	.063 .040
Palisade		<u>55 ft</u>				<u>9/67 - 8/68</u>		
	.204 .060	.113 .033	.027 .013	.030 .012	.058 .052	.046 .038	.072 .049	.081 .093

Table A.4-1 Site Wind Rose Data (cont)  
Probability of Wind Blowing towards Sector

Station	N S	NNE SSW	NE SW	ENE WSW	E W	ESE WNW	SE NW	SSE NNW
Palo Verde 1		<u>200 ft</u>				<u>8/13/73 - 8/13/74</u>		
	.055	.073	.144	.082	.068	.047	.052	.035
	.048	.059	.068	.048	.073	.059	.056	.041
Peach Bottom 2, 3		<u>120 ft</u>				<u>8/67 - 7/69</u>		
	.085	.064	.046	.052	.069	.095	.115	.109
	.060	.043	.031	.032	.034	.046	.054	.064
Pebbin Springs		<u>30 ft</u>				<u>1/74 - 12/74</u>		
	.017	.039	.075	.201	.313	.094	.021	.009
	.012	.019	.050	.055	.035	.028	.020	.014
Perkins		<u>30 ft</u>				<u>10/12/73 - 10/11/74</u>		
	.036	.067	.125	.066	.058	.047	.064	.053
	.068	.066	.104	.067	.063	.037	.044	.034
Perry 1		<u>200 ft</u>				<u>5/1/72 - 4/30/73</u>		
	.105	.095	.092	.084	.081	.054	.057	.042
	.045	.030	.057	.045	.048	.037	.054	.073
Phipps Bend		<u>33 ft</u>				<u>2/1/74 - 1/31/75</u>		
	.037	.054	.107	.106	.053	.071	.053	.120
	.054	.110	.112	.045	.020	.018	.021	.019
Pilgrim 1		<u>72 ft</u>						
	.051	.185	.118	.085	.094	.060	.053	.046
	.051	.038	.042	.035	.048	.031	.033	.030
Point Beach 1, 2		<u>150 ft</u>				<u>4/67 - 4/68</u>		
	.088	.122	.087	.048	.081	.097	.075	.056
	.096	.070	.055	.022	.020	.018	.031	.036
Prairie 1, 2		<u>140 ft</u>				<u>6/1/71 - 5/31/72</u>		
	.065	.031	.025	.031	.073	.102	.125	.065
	.046	.023	.019	.019	.055	.108	.134	.080
Quad Cities 1, 2		<u>400 ft</u>				<u>4/68 - 9/69</u>		
	.072	.128	.090	.049	.045	.069	.083	.067
	.068	.051	.042	.028	.037	.033	.075	.063
Rancho Seco		<u>50 ft</u>				<u>1967 - 1969</u>		
	.066	.073	.069	.107	.114	.078	.100	.074
	.049	.034	.029	.021	.029	.039	.057	.062
Riverbend 1		<u>135 ft</u>				<u>10/1/72 - 9/30/73</u>		
	.057	.058	.048	.048	.054	.048	.061	.066
	.069	.066	.066	.060	.076	.082	.072	.067
H. B. Robinson 2		<u>120 ft</u>				<u>4/14/67 - 4/19/68</u>		
	.045	.074	.072	.081	.071	.037	.036	.043
	.141	.114	.093	.050	.040	.035	.038	.029
Saint Lucie 1		<u>50 ft</u>				<u>11/1/72 - 12/31/72</u>		
	.062	.056	.063	.046	.030	.041	.053	.029
	.045	.038	.070	.088	.121	.093	.098	.067
Salem 1, 2		<u>300 ft</u>				<u>6/69 - 5/71</u>		
	.067	.062	.060	.056	.073	.095	.132	.094
	.062	.046	.049	.037	.028	.023	.042	.074
San Onofre		<u>10 m</u>				<u>1/25/73 - 1/24/76</u>		
	.066	.061	.054	.065	.088	.109	.060	.031
	.034	.111	.134	.028	.016	.022	.049	.070
Seabrook 1		<u>30 ft</u>				<u>11/71 - 10/72</u>		
	.030	.040	.069	.089	.110	.167	.145	.049
	.039	.024	.033	.046	.038	.041	.043	.037

Table A.4-I Site Wind Rose Data (cont)  
Probability of Wind Blowing Towards Sector

Station	N S	NNE SSW	NE SW	ENE WSW	E W	ESE NNW	SE NW	SSE NNW
Sequoia 1, 2		<u>33 ft</u>			<u>4/21/72 - 3/31/72</u>			
	.066	.151	.161	.48	.024	.024	.035	.070
	.058	.169	.116	.026	.011	.008	.013	.019
Shearon Harris		<u>10 m</u>			<u>1/76 - 12/76</u>			
	.079	.107	.098	.079	.053	.054	.057	.062
	.083	.067	.063	.047	.033	.031	.035	.053
Skagit		<u>10 m</u>						
	.014	.011	.021	.037	.128	.109	.085	.062
	.037	.071	.041	.028	.109	.058	.039	.020
Shoreham		<u>150 ft</u>			<u>10/1/73 - 9/30/74</u>			
	.060	.129	.095	.050	.079	.103	.094	.066
	.050	.045	.049	.043	.032	.028	.034	.041
South Texas		<u>33 ft</u>			<u>7/20/73 - 7/20/74</u>			
	.148	.046	.029	.010	.015	.014	.020	.037
	.075	.078	.080	.047	.053	.059	.137	.153
Virgin C. Summer		<u>202 ft</u>			<u>1975</u>			
	.068	.090	.118	.087	.064	.046	.055	.043
	.029	.042	.080	.070	.059	.041	.052	.056
Surry St 1, 2		<u>150 ft</u>			<u>11/67 - 12/69</u>			
	.064	.082	.083	.062	.059	.061	.087	.081
	.072	.051	.046	.045	.057	.052	.055	.043
Susquehanna 1					<u>1956 - 1960</u>			
	.037	.070	.125	.126	.044	.059	.100	.090
	.046	.039	.049	.054	.040	.062	.031	.029
Three Mile Island		<u>100 ft</u>			<u>4/71 - 3/72</u>			
	.054	.045	.054	.059	.001	.092	.091	.070
	.040	.027	.036	.057	.085	.082	.062	.057
Trojan		<u>30 ft</u>			<u>9/1/71 - 8/31/72</u>			
	.203	.070	.026	.013	.022	.037	.070	.132
	.172	.054	.016	.006	.007	.009	.046	.120
Turkey Point 1, 2		<u>30 ft</u>			<u>1969</u>			
	.038	.041	.047	.027	.027	.047	.051	.077
	.035	.028	.048	.100	.136	.135	.100	.062
Vermont Yankee 1		<u>140 ft</u>			<u>8/67 - 7/68</u>			
	.072	.027	.018	.023	.069	.086	.117	.196
	.070	.025	.017	.019	.024	.066	.085	.086
Vogtle		<u>30 ft</u>			<u>12/73 - 12/74</u>			
	.064	.062	.074	.079	.084	.075	.056	.031
	.043	.043	.072	.065	.069	.060	.063	.060
Waterford 3		<u>30 ft</u>			<u>5/72 - 4/73</u>			
	.042	.053	.045	.047	.049	.056	.064	.072
	.046	.092	.088	.059	.029	.100	.083	.077
Watts Bar 1, 2		<u>300 ft</u>			<u>7/1/73 - 6/30/75</u>			
	.033	.109	.183	.089	.040	.031	.035	.037
	.053	.106	.132	.059	.041	.019	.014	.019
WPPS 1, 4		<u>33 ft</u>			<u>4/74 - 3/75</u>			
	.100	.062	.063	.052	.061	.099	.107	.085
	.164	.045	.036	.031	.022	.026	.040	.075
WPPS 2		<u>33 ft</u>			<u>4/74 - 3/75</u>			
	.100	.082	.063	.052	.061	.098	.107	.085
	.164	.045	.036	.031	.022	.026	.040	.075

Table A.4-1 Site Wind Rose Data (cont)  
Probability of Wind Blowing Towards Sector

Station	S E	SE SW	NE NW	ENE WNW	E W	ESE WNW	SE NW	S-E N-W
WPPS 3, 5			60 m			5/23 - 6/24		
	.071	.092	.124	.170	.125	.031	.015	.040
	.018	.019	.062	.074	.047	.052	.050	.027
Polf Creek			10 m			5/21/73 - 5/31/75		
	.080	.100	.040	.024	.030	.041	.064	.069
	.164	.058	.039	.035	.039	.046	.061	.111
Yankee Howe			30 ft			10/71 - 9/72		
	.101	.080	.052	.037	.039	.041	.072	.086
	.086	.054	.065	.063	.047	.036	.052	.061
Yellow Creek			33 ft			7/1/74 - 8/30/75		
	.142	.097	.049	.039	.040	.050	.057	.087
	.037	.079	.049	.019	.021	.046	.066	.110
Zimmer			36 ft			3/1/72 - 2/28/74		
	.108	.076	.068	.056	.051	.058	.047	.062
	.062	.013	.027	.011	.030	.054	.127	.129
Zion			35 ft			10/70		
	.01	.078	.079	.113	.069	.176	.046	.071
	.046	.059	.037	.078	.035	.075	.160	.096

## A.5 Economic Data

The input data to the economic model in CRAC2 can be divided into two groups: those which are national in character and those which are applicable to individual states. Appendix VI of WASH-1400 [6] contains a detailed discussion of these parameters.

The national data can be further divided into data which measure costs on a per capita basis, and data which measure costs on a per acre basis. Decontamination costs for business, residential, and public areas, relocation costs, consumed dairy products, and consumed nondairy products, are all measured in dollars per person. The decontamination cost for farm land is measured in dollars per acre. Table A.5-1 lists current figures for these cost parameters and in addition compares these costs with those contained in Appendix VI of WASH-1400.

WASH-1400 Appendix VI describes some of the decontamination techniques considered when the original costs estimates were made. It does not, however, give a detailed breakdown of costs. As an approximation, the decontamination costs were broken down into labor, energy, and durable goods (equipment) components. The breakdown of costs was assumed to be 40% labor, 50% energy, and 10% durable goods for farmland decontamination and 60%, 30%, and 10% for decontamination of public areas. Using data contained in the Statistical Abstract of the US [7], the change in the Consumer Price Index (CPI) from 1972 to 1979 was calculated for each of these areas. These factors are 1.69 for labor, 2.66 for energy, and 1.55 for durable goods. The updated decontamination costs were obtained by multiplying the original WASH-1400 cost figures by the appropriately weighted combinations of these CPI factors.

Relocation costs were calculated in Appendix VI as a combination of lost income, both individual and corporate, and moving costs. These costs, which were calculated on a per capita basis, are \$1,100 for lost individual income, \$940 for lost corporate income, and \$1,300 for transportation expenses. Based on data from the Statistical Abstract, the employee compensation rate has increased by a factor of 1.44 between 1973 and 1978. The nonfarm business gross national product (GNP) has increased by a factor of 1.54 and transportation services by a factor of 1.53 in the same period. The updated relocation cost was obtained by summing the products of each of the three costs and the appropriate factor.

The revised per capita value of residential, business, and public areas, and annual per capita dairy and nondairy consumption costs were derived from data contained in the Statistical Abstract of the U.S. The net value of residential, business, and public assets, less farm assets, was divided by the US population to obtain the updated per capita value of nonfarm assets. The updated agricultural consumption figures were obtained by dividing the total annual market value of these commodities by the US population. Per capita agricultural consumption figures are used by CRAC2 to determine radiation exposure through dairy and nondairy product ingestion.

The data, which are supplied on a state-by-state basis, all relate to farm costs and values. The input parameters are fraction of state area devoted to farming, average annual sale of farm products in dollars per acre, the fraction of farm sales resulting from dairy products, the average value of farmland in dollars per acre, and the major farming season. Table A.5-2 lists the values for all of these fields. The Statistical Abstract of the United States is the source for farmland value and farmland fraction. Farm sales and dairy share are found in reference [8]. The farming seasons are the same as the WASH-1400 figures.

Table A.5-1 National Economics Data

<u>Description</u>	<u>WASH-1400 Data</u>	<u>Current Data</u>
Decontamination cost for farmland (\$/acre)	230	500*
Decontamination cost for residential, business, and public property (\$/person)	1,700	4,400*
Value of residential, business, and public property (\$/person)	17,000	32,000*
Depreciation rate for improvements ( $\text{yr}^{-1}$ )	0.2	0.2
Relocation cost (\$/person)	2,900	4,300**
Annual cost of dairy product consumption (\$/person)	--	135**
Annual cost of non-dairy product consumption (\$/person)	--	690**

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\*Represents 1979 statistics

\*\*Represents 1978 statistics

Table A.5-2 Agricultural Land Use Characteristics

State	Fraction of State Used as Farm Land*,**	Average Annual Sale of Farm Products† (\$/acre-year)	Average Share of Dairy Products‡ (\$ dairy/\$ products)	Average Value of Farmland† (\$/acre)	Major Farming Season
Maine	0.077	250	0.182	485	May-Sept
New Hampshire	0.097	150	0.444	802	May-Sept
Vermont	0.283	177	0.791	657	May-Sept
Massachusetts	0.123	372	0.283	1366	May-Sept
Rhode Island	0.081	476	0.220	2133	May-Sept
Connecticut	0.140	500	0.313	2158	May-Sept
New York	0.315	188	0.579	642	May-Sept
New Jersey	0.197	376	0.162	2222	May-Sept
Pennsylvania	0.307	239	0.413	669	May-Sept
Ohio	0.618	183	0.153	1516	May-Sept
Indiana	0.728	206	0.067	1498	May-Sept
Illinois	0.795	213	0.041	1786	May-Sept
Michigan	0.285	197	0.238	955	May-Sept
Wisconsin	0.520	194	0.598	807	May-Sept
Minnesota	0.563	160	0.185	854	May-Sept
Iowa	0.944	242	0.050	1458	May-Sept
Missouri	0.724	111	0.079	674	May-Sept
North Dakota	0.922	45	0.047	306	May-Sept
South Dakota	0.922	46	0.074	257	May-Sept
Nebraska	0.967	99	0.027	470	May-Sept
Kansas	0.915	92	0.034	437	May-Sept
Delaware	0.471	508	0.046	1725	April-Oct
Maryland	0.414	273	0.227	1799	April-Oct
Virginia	0.371	126	0.171	864	April-Oct
West Virginia	0.270	44	0.203	472	April-Oct
North Carolina	0.368	261	0.056	819	April-Oct
South Carolina	0.327	148	0.063	635	April-Oct
Georgia	0.417	164	0.058	609	April-Oct
Florida	0.368	233	0.077	930	April-Oct
Kentucky	0.557	141	0.117	792	April-Oct
Tennessee	0.507	118	0.140	669	April-Oct
Alabama	0.400	144	0.041	515	April-Oct
Mississippi	0.475	135	0.047	520	April-Oct
Arkansas	0.494	158	0.030	691	April-Oct
Louisiana	0.332	137	0.087	763	April-Oct
Oklahoma	0.782	68	0.051	442	April-Oct
Texas	0.811	54	0.053	354	April-Oct
Montana	0.658	20	0.026	186	May-Sept
Idaho	0.894	93	0.114	485	May-Sept
Wyoming	0.560	15	0.024	119	May-Sept
Colorado	0.570	69	0.039	332	April-Oct
New Mexico	0.600	21	0.056	100	April-Oct
Arizona	0.556	36	0.069	134	April-Oct
Utah	0.236	36	0.215	265	April-Oct
Nevada	0.127	19	0.117	104	April-Oct
Washington	0.369	132	0.138	586	May-Sept
Oregon	0.300	68	0.093	330	May-Sept
California	0.318	316	0.119	936	April-Oct

\*Fraction of total state area (including water areas) devoted to agricultural use

\*\*Reflect 1979 statistics

†Reflect 1978 statistics

## References for Appendix A

1. Demographic Statistics Pertaining to Nuclear Power Reactor Sites, NUREG-0348, U.S. Nuclear Regulatory Commission, October 1979.
2. D. C. Aldrich, D. M. Ericson, and J. D. Johnson, Public Protection Strategies for Potential Nuclear Accidents: Sheltering Concepts with Existing Public and Private Structures, SAND77-1725, Sandia Laboratories, Albuquerque, NM, February 1978.
3. I. Hall, R. Prairie, and E. Boes, Generation of Typical Meteorological Years for 26 SOLMET Stations, SAND78-1601, Sandia Laboratories, August 1978.
4. User's Manual for Single Source (CRSTER) Model, US Environmental Protection Agency, Office of Air and Waste Management, Research Triangle Park, NC, July 1977.
5. G. C. Holzworth, Mixing Heights, Wind Speeds, and Potential for Urbain Air Pollution Throughout the Contiguous United States, Publ. No. AP-101, US Environmental Protection Agency, Office of Air Programs, Research Triangle Park, NC, 1972.
6. Reactor Safety Study, Appendix VI: Calculation of Reactor Accident Consequences, WASH-1400 (NUREG 75/014), US Nuclear Regulatory Commission, October 1975.
7. US Department of Commerce, 1979, Statistical Abstract of the United States.
8. US Department of Agriculture, Agricultural Statistics, 1979.

## Appendix B: Reactor Core Radionuclide Inventories

### B.1 Core Radionuclide Inventory

Reactor accident consequence calculations are often performed using the Reactor Safety Study [1] radionuclide inventory for a 3200 Mwt Westinghouse pressurized water reactor (PWR). This inventory, calculated for an end-of-cycle equilibrium core, has been used to represent both boiling water reactor (BWR) and PWR cores. Recently, however, an end-of-cycle equilibrium inventory for a 3412 Mwt Westinghouse PWR was calculated using the SANDIA-ORIGEN computer code [2]. This inventory, which was calculated using a 25% greater fuel burnup than used for the WASH-1400 inventory, was used to perform the reactor consequence calculations discussed in Chapter 2. (A spent fuel burnup of 26,400 MWd/MTU was assumed to calculate the WASH-1400 inventories.)

The 3412 Mwt PWR inventory was calculated by assuming that the three regions of the reactor core (each initially loaded with uranium enriched to 3.3% U-235) were operated at a constant specific power density of 38.3 MW/MTU charged. A three year refueling cycle and an 80% capacity factor were also assumed. This inventory is representative of an equilibrium core at a time when the three regions have average burnups of 11,000, 22,000, and 33,000 MWd/MTU charged (end-of-cycle).

The SANDIA-ORIGEN code calculates the time dependent activities of approximately 500 radionuclides; including activation products, fission products, and actinides. Of this number, only 54 radionuclides are expected to significantly impact reactor accident consequence calculations and as a result, are input to the CRAC2 code. The elimination of radionuclides from consideration was based on a number of parameters, such as quantity (curies), release fraction, radioactive half-life, dosimetry, and chemical characteristics [1]. Table B.1-1 lists the 54 radionuclides used to perform the consequence calculations. Also given is the activity of each radionuclide at the time the accident is assumed to occur. The reactor core inventories used to perform the power level sensitivity calculations discussed in Chapter 2 were obtained by linearly scaling (by thermal power level) the inventories presented in Table B.1-1.

Table B.1-1 Inventory of Radionuclides in the 3412 Mwt PWR Core

No.	Radionuclide	Radioactive Inventory Source (curies x 10 <sup>-6</sup> )	Half-Life (days)
1	Cobalt-58	0.0075	71.0
2	Cobalt-60	0.000045	1,920
3	Krypton-85	0.0066	3,950
4	Krypton-85m	0.31	0.183
5	Krypton-87	0.57	0.0528
6	Krypton-88	0.77	0.117
7	Rubidium-86	0.00048	18.7
8	Strontium-89	0.96	52.1
9	Strontium-90	0.052	10,300
10	Strontium-91	1.2	0.403
11	Yttrium-90	0.055	2.67
12	Yttrium-91	1.2	59.0
13	Zirconium-95	1.5	65.2
14	Zirconium-97	1.6	0.71
15	Niobium-95	1.4	35.0
16	Molybdenum-99	1.7	2.8
17	Technetium-99m	1.4	0.25
18	Ruthenium-103	1.2	39.5
19	Ruthenium-105	0.82	0.185
20	Ruthenium-106	0.29	366
21	Rhodium-105	0.56	1.50
22	Tellurium-127	0.075	0.391
23	Tellurium-127m	0.0098	109
24	Tellurium-129	0.25	0.048
25	Tellurium-129m	0.067	34.0
26	Tellurium-131m	0.13	1.25
27	Tellurium-132	1.3	3.25
28	Antimony-127	0.077	3.88
29	Antimony-129	0.27	0.179
30	Iodine-131	0.87	8.05
31	Iodine-132	1.3	0.0958
32	Iodine-133	1.8	0.875
33	Iodine-134	2.0	0.0366
34	Iodine-135	1.7	0.280
35	Xenon-133	1.8	5.28
36	Xenon-135	0.38	0.384
37	Cesium-134	0.13	750
38	Cesium-136	0.039	13.0
39	Cesium-137	0.065	11,000
40	Barium-140	1.7	12.8
41	Lanthanum-140	1.7	1.67
42	Cerium-141	1.5	32.3
43	Cerium-143	1.5	1.38
44	Cerium-144	0.92	284
45	Praseodymium-143	1.5	13.7
46	Neodymium-147	0.65	11.1
47	Neptunium-239	19.0	2.35
48	Plutonium-238	0.0012	32,500
49	Plutonium-239	0.00026	8.9 x 10 <sup>6</sup>
50	Plutonium-240	0.00029	2.5 x 10 <sup>6</sup>
51	Plutonium-241	0.054	5,350
52	Americium-241	0.000036	1.6 x 10 <sup>5</sup>
53	Curium-242	0.014	163
54	Curium-244	0.00084	6,630

## B.2 Radionuclide Inventory Impacts on Reactor Accident Consequences

The potential impacts of different radionuclide inventories on predicted accident consequences, and the appropriateness of inventory scaling, were examined using the CRAC2 code [3]. Consequence calculations were performed using end-of-cycle equilibrium inventories for the WASH-1400 3200 Mwt Westinghouse PWR, the 3412 Mwt Westinghouse PWR, a 3578 Mwt General Electric (GE) BWR, and a 1518 Mwt Westinghouse PWR. Calculations were also performed for the 3412 Mwt PWR at 1/3 and 2/3 of the way through the annual operating cycle. (The 3578 Mwt BWR and 1518 Mwt PWR inventories, like those for the 3412 Mwt PWR, were generated with the SANDIA-ORIGEN computer code.) The operating characteristics for the four reactors are summarized in Table B.2-1. The 3412 Mwt and 1518 Mwt PWRs and the 3578 Mwt BWR are considered to be representative of current reactor designs and compositions.

Table B.2-2 summarizes the four reactor inventories for selected radionuclides. In general, inventories of short-lived radionuclides are proportional to reactor thermal power level, while inventories of long-lived radionuclides are proportional to burnup; both are influenced by in-core fuel management plans.

Consequences were calculated assuming (1) an SST1 release (large-scale core melt with uncontrolled release directly to the atmosphere), (2) Indian Point population and wind-rose data, (3) New York City weather data, and (4) a distribution of evacuations within 10 miles of the reactor.\* Table B.2-3 summarizes the consequence calculation results from which the following observations can be made.

- 1) The 3412 Mwt PWR land interdiction and decontamination results are approximately 30% larger than those for the WASH-1400 PWR. Differences for other consequences are somewhat less (10-17%).

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\*Consists of a 30%, 40%, 30% weighting of a 10 mile per hour evacuation after 1, 3, and 5 hour delays, respectively.

Table B.2-1 Reactor Operating Characteristics

Characteristic	WASH-1400	3412 Mwt PWR	3578 Mwt <sup>a</sup> BWR	1518 Mwt PWR
Total Uranium in Fresh Core (MT)	---	89.1	136.7	47.5
Initial U-235 Enrichment (percent)	3.3	3.3	2.66, 2.83	3.2
Refueling Cycle	annually	annually	annually	annually
Number of Years an Element Spends in Core (years)	3	3	3,4	3
Capacity Factor (Percent of time at Full Power)	---	80	80	80
Average Fuel Burnup at discharge (MWd/MTU)	26,400	33,600	---	28,000
Average Power Density (MW/MTU)	40	38.3	26.1	32.0

<sup>a</sup>The SANDIA-ORIGEN BWR calculations were performed on a per fuel assembly basis. The code generated radionuclide inventories by blending individual assembly results.

Table B.2-2 Inventory of Selected Radionuclides for the Reactors Studied.

Radionuclide	Half-Life (days)	End-of-Cycle 3412 Mwt PWR (Ci)	(Designated Inventory) ÷ (3412 Mwt PWR Inventory)					
			WASH-1400					
			End-of-Cycle 3200 Mwt PWR	End-of-Cycle 3578 Mwt BWR	End-of-Cycle 1518 Mwt PWR	1/3 Cycle 3412 Mwt PWR	2/3 Cycle 3412 Mwt PWR	
Kr-85	0.117	$6.64 \times 10^5$	1.03	1.36	0.44	0.68	0.84	
Mo-99	2.8	$1.66 \times 10^8$	0.94	1.05	0.45	1.02	1.01	
Tc-99m	0.25	$1.43 \times 10^8$	1.00	1.05	0.45	1.03	1.01	
Ru-103	39.5	$1.25 \times 10^8$	0.85	1.06	0.44	0.87	0.96	
Ru-105	0.185	$8.22 \times 10^7$	0.88	1.07	0.43	0.86	0.94	
Ru-106	366	$2.90 \times 10^7$	0.86	1.24	0.42	0.66	0.83	
Te-129m	0.34	$6.70 \times 10^6$	0.79	1.06	0.44	0.86	0.96	
Te-131m	1.25	$1.28 \times 10^7$	1.00	1.07	0.44	0.97	0.98	
Te-132	3.25	$1.27 \times 10^8$	0.92	1.06	0.45	1.00	1.00	
Sb-129	0.179	$2.72 \times 10^7$	1.22	1.06	0.44	0.93	0.97	
I-131	8.05	$8.74 \times 10^7$	0.98	1.06	0.45	0.99	1.00	
I-132	0.096	$1.29 \times 10^8$	0.92	1.05	0.44	0.99	1.00	
I-133	0.875	$1.84 \times 10^8$	0.94	1.05	0.45	1.02	1.01	
I-134	0.037	$2.02 \times 10^8$	0.95	1.05	0.45	1.02	1.01	
I-135	0.28	$1.73 \times 10^8$	0.88	1.06	0.45	1.02	1.01	
Cs-134	750	$1.26 \times 10^7$	0.60	1.20	0.38	0.55	0.76	
Cs-136	13.0	$3.91 \times 10^6$	0.77	1.04	0.41	0.67	0.84	
Cs-137	11,000	$6.54 \times 10^6$	0.72	1.39	0.44	0.67	0.83	
Ba-140	12.8	$1.68 \times 10^8$	0.94	1.05	0.45	1.02	1.01	
Ce-144	284	$9.15 \times 10^7$	0.92	1.14	0.45	0.77	0.90	

Table B.2-3 Summary of CRAC2 Consequence Predictions.

Consequence	End-of-Cycle 3412 Mwt PWR	WASH-1400			Scaled <sup>1</sup>		End-of-Cycle 1518 Mwt PWR	End-of-Cycle 1518 Mwt PW
		End-of-Cycle 3200 Mwt PWR	End-of-Cycle 3578 Mwt BWR	1/3 Cycle 3412 Mwt PWR	2/3 Cycle 3412 Mwt PWR			
Mean Early Fatalities	800	690	890	750	780	150	150	150
Mean Early Injuries	3600	3000	4100	3400	3500	960	970	970
Mean Latent Cancer Fatalities	7800	7000	8400	6800	7300	5300	5400	5400
Mean Land Interdiction Area (km <sup>2</sup> )	200	140	280	130	160	92	97	97
Mean Land Decontamination Area (km <sup>2</sup> )	3800	2800	5900	2800	3100	2000	2100	2100

<sup>1</sup>Inventory = (1518 Mwt/3412 Mwt) x (3412 Mwt PWR inventory).

- 2) The 3578 Mwt BWR land decontamination and interdiction consequences are approximately 50% larger than those for the 3412 Mwt PWR. Again, differences for other consequences are on the order of 10%.
- 3) Comparison of the 3412 and 1518 Mwt PWR results indicate reductions in reactor size result in proportionately larger reductions in early consequences.
- 4) Comparison of the 1/3, 2/3, and end-of-cycle 3412 Mwt PWR results indicate that differences in radionuclide inventory during the annual operating cycle have little influence on early consequences. However, time of the accident during the cycle does significantly influence predicted latent cancer fatalities and areas of land interdiction and decontamination.
- 5) There is essentially no difference in consequences for the 1518 Mwt PWR predicted by using either the calculated or scaled inventories.

Differences in latent cancer fatality, land interdiction, and land decontamination consequences largely result from long-lived radionuclide inventory differences (e.g., Cs-137). Differences in early consequences are primarily due to differences in short-lived radionuclide inventories.

In summary, the results presented above indicate that reactor accident consequences are sensitive to differences in radionuclide inventories due to reactor size and design. Because of in-core fuel management plans, boiling water reactors will likely have larger inventories of long-lived radionuclides than a pressurized water reactor of the same size. Therefore, using PWR inventories for BWR consequence calculations could underestimate latent consequences. The time of a reactor accident during the annual operating cycle has little influence on early consequences; however, it can significantly influence latent effects. Reductions in reactor size will lead to substantial reductions in early consequences, more so than would be expected based on differences in reactor power levels. In addition, linear scaling of radionuclide inventories by thermal power level is adequate for consequence calculations, provided that the reactor of interest has operating and design characteristics similar to those of the reactor from which the inventories are scaled.

References for Appendix B

1. "Reactor Safety Study Appendix VI: Calculation of Reactor Accident Consequences," WASH-1400 (NUREG 75/014), U.S. Nuclear Regulatory Commission, October 1975.
2. D. E. Bennett, Radionuclide Core Inventories for Standard PWR and BWR Fuel Management Plans, SAND82-1111, NUREG/CR-2724, Sandia National Laboratories, Albuquerque, NM, to be published.
3. R. M. Ostmeyer, "Radionuclide Inventory Impacts on Reactor Accident Consequences," Transactions of the American Nuclear Society (November 1981).

## Appendix C: Site Specific Consequence Estimates

This appendix presents the consequence estimates for each of the 91 sites analyzed in Chapter 2. It is important to note that in each case the calculations assumed (1) that the site contained an 1120 MWe PWR, (2) meteorology based on the most appropriate regional National Weather Service Station (from among the 29 detailed in Appendix A), (3) actual site wind rose and population, (4) a summary evacuation (all persons within 10 miles evacuate at 10 mph after delays of 1, 3, or 5 hours, with probability .3, .4, .3, respectively) and (5) hypothetical releases of radioactive materials (see Section 2.3, Chapter 2). Thus the estimates presented in this appendix are only a guide to the impact of site characteristics (principally population distribution) on predicted consequences. In no way are these to be taken as estimates of existing/reactor combinations.

Table C.1 provides a summary of the mean early fatalities, early injuries, and latent cancer fatalities for SST1, SST2, and SST3. Figures C-1 through C-18 contain early fatality, early injury, and latent cancer fatality CCDFs for each of the 91 sites, conditional on an SST1 release and assuming the 1120 MWe PWR, summary evacuation, regional meteorology, and actual site population and wind rose. Since some of these characteristics do not exactly duplicate the characteristics of the actual reactor/site combinations, the CCDFs are not to be used in place of actual risk estimates for existing reactor/site combinations.

Table C-1. Mean Number (Per Reactor-Year) of Early Fatalities, Early Injuries and Latent Cancer Fatalities for each of 91 Sites, for SST1, SST2, or SST3 Accident Source Terms.

Assumptions:

- (1) Standard 1120 MWe PWR
- (2) Summary Evacuation
- (3) Actual Site Population and Wind rose
- (4) Best Estimate Meteorology

	Mean Early Fatalities*			Mean Early Injuries*			Mean Latent Cancer Fatalities*		
	SST1	SST2	SST3	SST1	SST2	SST3	SST1	SST2	SST3
Allens Creek	31xP <sub>1</sub>	0xP <sub>2</sub>	0xP <sub>3</sub>	93xP <sub>1</sub>	0.9xP <sub>2</sub>	0xP <sub>3</sub>	620xP <sub>1</sub>	49xP <sub>2</sub>	0.2xP <sub>3</sub>
Arkansas	17xP <sub>1</sub>	0xP <sub>2</sub>	0xP <sub>3</sub>	150xP <sub>1</sub>	0.2xP <sub>2</sub>	0xP <sub>3</sub>	950xP <sub>1</sub>	82xP <sub>2</sub>	0.3xP <sub>3</sub>
Bailly	58xP <sub>1</sub>	0xP <sub>2</sub>	0xP <sub>3</sub>	1200xP <sub>1</sub>	0.5xP <sub>2</sub>	0xP <sub>3</sub>	3300xP <sub>1</sub>	260xP <sub>2</sub>	0.9xP <sub>3</sub>
Beaver Valley	150xP <sub>1</sub>	0xP <sub>2</sub>	0xP <sub>3</sub>	1200xP <sub>1</sub>	0.4xP <sub>2</sub>	0xP <sub>3</sub>	3400xP <sub>1</sub>	200xP <sub>2</sub>	0.6xP <sub>3</sub>
Bellefonte	6.3xP <sub>1</sub>	0.08xP <sub>2</sub>	0xP <sub>3</sub>	110xP <sub>1</sub>	5.6xP <sub>2</sub>	0xP <sub>3</sub>	1000xP <sub>1</sub>	70xP <sub>2</sub>	0.3xP <sub>3</sub>
Big Rock Pt.	15xP <sub>1</sub>	0xP <sub>2</sub>	0xP <sub>3</sub>	90xP <sub>1</sub>	0.5xP <sub>2</sub>	0xP <sub>3</sub>	680xP <sub>1</sub>	53xP <sub>2</sub>	0.2xP <sub>3</sub>
Black Fox	13xP <sub>1</sub>	0xP <sub>2</sub>	0xP <sub>3</sub>	220xP <sub>1</sub>	0.01xP <sub>2</sub>	0xP <sub>3</sub>	780xP <sub>1</sub>	69xP <sub>2</sub>	0.3xP <sub>3</sub>
Braidwood	160xP <sub>1</sub>	0.05xP <sub>2</sub>	0xP <sub>3</sub>	420xP <sub>1</sub>	10xP <sub>2</sub>	0xP <sub>3</sub>	3200xP <sub>1</sub>	240xP <sub>2</sub>	0.9xP <sub>3</sub>
Browns Ferry	25xP <sub>1</sub>	0xP <sub>2</sub>	0xP <sub>3</sub>	220xP <sub>1</sub>	0.03xP <sub>2</sub>	0xP <sub>3</sub>	970xP <sub>1</sub>	69xP <sub>2</sub>	0.3xP <sub>3</sub>
Brunswick	12xP <sub>1</sub>	0xP <sub>2</sub>	0xP <sub>3</sub>	120xP <sub>1</sub>	0.01xP <sub>2</sub>	0xP <sub>3</sub>	890xP <sub>1</sub>	98xP <sub>2</sub>	0.4xP <sub>3</sub>

\*Detailed Probabilistic Risk Assessments (PRAs) have not been performed for all reactors. Therefore, consequence calculations were performed in this study using Siting Source Terms (SSTs) defined by NRC (see Section 2.3.1, Chapter 2). By adjusting the probabilities associated with each of the source terms, the set can be made to approximately represent any current LWR design. Based on currently available PRAs, NRC has suggested that representative probabilities for the SSTs are:  $P_1$  for SST1 =  $1 \times 10^{-5}$ ,  $P_2$  for SST2 =  $2 \times 10^{-5}$ , and  $P_3$  for SST3 =  $1 \times 10^{-4}$ . There are very large variations (factors of 10 to 100) in the accident probabilities associated with a specific design.

Caution should be used when applying these numbers. Probability times consequence is not an adequate representation of risk; it provides only a common measure for comparative purposes (i.e., rank ordering). The Complementary Cumulative Distribution Functions (shown in Figure C-1 through C-18) are a better representation of risk.

Table C-1. (continued)

	Mean Early Fatalities*			Mean Early Injuries*			Mean Latent Cancer Fatalities*		
	SST1	SST2	SST3	SST1	SST2	SST3	SST1	SST2	SST3
Byron	54xP <sub>1</sub>	0.09xP <sub>2</sub>	0xP <sub>3</sub>	330xP <sub>1</sub>	4.3xP <sub>2</sub>	0xP <sub>3</sub>	2500xP <sub>1</sub>	190xP <sub>2</sub>	0.7xP <sub>3</sub>
Callaway	10xP <sub>1</sub>	0xP <sub>2</sub>	0xP <sub>3</sub>	100xP <sub>1</sub>	0.04xP <sub>2</sub>	0xP <sub>3</sub>	1200xP <sub>1</sub>	97xP <sub>2</sub>	0.3xP <sub>3</sub>
Calvert Cliffs	18xP <sub>1</sub>	0xP <sub>2</sub>	0xP <sub>3</sub>	170xP <sub>1</sub>	0.08xP <sub>2</sub>	0xP <sub>3</sub>	2400xP <sub>1</sub>	120xP <sub>2</sub>	0.4xP <sub>3</sub>
Catawba	100xP <sub>1</sub>	0xP <sub>2</sub>	0xP <sub>3</sub>	710xP <sub>1</sub>	0.2xP <sub>2</sub>	0xP <sub>3</sub>	1500xP <sub>1</sub>	110xP <sub>2</sub>	0.4xP <sub>3</sub>
Cherokee	27xP <sub>1</sub>	0xP <sub>2</sub>	0xP <sub>3</sub>	250xP <sub>1</sub>	0.1xP <sub>2</sub>	0xP <sub>3</sub>	1200xP <sub>1</sub>	76xP <sub>2</sub>	0.3xP <sub>3</sub>
Clinton	16xP <sub>1</sub>	0xP <sub>2</sub>	0xP <sub>3</sub>	130xP <sub>1</sub>	0.7xP <sub>2</sub>	0xP <sub>3</sub>	2300xP <sub>1</sub>	170xP <sub>2</sub>	0.7xP <sub>3</sub>
Comanche Peak	1.3xP <sub>1</sub>	0xP <sub>2</sub>	0xP <sub>3</sub>	37xP <sub>1</sub>	0xP <sub>2</sub>	0xP <sub>3</sub>	640xP <sub>1</sub>	49xP <sub>2</sub>	0.2xP <sub>3</sub>
Cooper	4.7xP <sub>1</sub>	0xP <sub>2</sub>	0xP <sub>3</sub>	47xP <sub>1</sub>	0.09xP <sub>2</sub>	0xP <sub>3</sub>	900xP <sub>1</sub>	81xP <sub>2</sub>	0.3xP <sub>3</sub>
Crystal River	21xP <sub>1</sub>	0xP <sub>2</sub>	0xP <sub>3</sub>	88xP <sub>1</sub>	0.9xP <sub>2</sub>	0xP <sub>3</sub>	590xP <sub>1</sub>	66xP <sub>2</sub>	0.3xP <sub>3</sub>
Davis-Besse	21xP <sub>1</sub>	0xP <sub>2</sub>	0xP <sub>3</sub>	420xP <sub>1</sub>	0.6xP <sub>2</sub>	0xP <sub>3</sub>	2600xP <sub>1</sub>	160xP <sub>2</sub>	0.5xP <sub>3</sub>
Diablo Canyon	4.7xP <sub>1</sub>	0xP <sub>2</sub>	0xP <sub>3</sub>	50xP <sub>1</sub>	0xP <sub>2</sub>	0xP <sub>3</sub>	1200xP <sub>1</sub>	98xP <sub>2</sub>	0.4xP <sub>3</sub>
Donald C. Cook	55xP <sub>1</sub>	0.04xP <sub>2</sub>	0xP <sub>3</sub>	590xP <sub>1</sub>	2.2xP <sub>2</sub>	0xP <sub>3</sub>	2500xP <sub>1</sub>	120xP <sub>2</sub>	0.4xP <sub>3</sub>
Dresden	42xP <sub>1</sub>	0xP <sub>2</sub>	0xP <sub>3</sub>	540xP <sub>1</sub>	0.3xP <sub>2</sub>	0xP <sub>3</sub>	3300xP <sub>1</sub>	260xP <sub>2</sub>	0.9xP <sub>3</sub>
Duane Arnold	21xP <sub>1</sub>	0xP <sub>2</sub>	0xP <sub>3</sub>	380xP <sub>1</sub>	0.4xP <sub>2</sub>	0xP <sub>3</sub>	1700xP <sub>1</sub>	190xP <sub>2</sub>	0.8xP <sub>3</sub>
Fermi	160xP <sub>1</sub>	0.08xP <sub>2</sub>	0xP <sub>3</sub>	970xP <sub>1</sub>	7.1xP <sub>2</sub>	0xP <sub>3</sub>	3000xP <sub>1</sub>	200xP <sub>2</sub>	0.6xP <sub>3</sub>
Fitzpatrick	5.0xP <sub>1</sub>	0xP <sub>2</sub>	0xP <sub>3</sub>	110xP <sub>1</sub>	0.06xP <sub>2</sub>	0xP <sub>3</sub>	1200xP <sub>1</sub>	57xP <sub>2</sub>	0.2xP <sub>3</sub>
Forked River	84xP <sub>1</sub>	0xP <sub>2</sub>	0xP <sub>3</sub>	530xP <sub>1</sub>	0.8xP <sub>2</sub>	0xP <sub>3</sub>	4400xP <sub>1</sub>	200xP <sub>2</sub>	0.6xP <sub>3</sub>
Fort Calhoun	50xP <sub>1</sub>	0.1xP <sub>2</sub>	0xP <sub>3</sub>	440xP <sub>1</sub>	3.0xP <sub>2</sub>	0xP <sub>3</sub>	1100xP <sub>1</sub>	110xP <sub>2</sub>	0.4xP <sub>3</sub>
Pt. St. Vrain	15xP <sub>1</sub>	0xP <sub>2</sub>	0xP <sub>3</sub>	220xP <sub>1</sub>	0xP <sub>2</sub>	0xP <sub>3</sub>	810xP <sub>1</sub>	82xP <sub>2</sub>	0.3xP <sub>3</sub>
Ginna	11xP <sub>1</sub>	0xP <sub>2</sub>	0xP <sub>3</sub>	370xP <sub>1</sub>	0.1xP <sub>2</sub>	0xP <sub>3</sub>	1900xP <sub>1</sub>	89xP <sub>2</sub>	0.3xP <sub>3</sub>
Grand Gulf	14xP <sub>1</sub>	0xP <sub>2</sub>	0xP <sub>3</sub>	73xP <sub>1</sub>	0.7xP <sub>2</sub>	0xP <sub>3</sub>	700xP <sub>1</sub>	60xP <sub>2</sub>	0.3xP <sub>3</sub>
Haddem Neck	110xP <sub>1</sub>	0xP <sub>2</sub>	0xP <sub>3</sub>	890xP <sub>1</sub>	1.2xP <sub>2</sub>	0xP <sub>3</sub>	2100xP <sub>1</sub>	160xP <sub>2</sub>	0.5xP <sub>3</sub>

\*See footnote, page C-2.

Table C-1. (continued)

	Mean Early Fatalities*			Mean Early Injuries*			Mean Latent Cancer Fatalities*		
	SST1	SST2	SST3	SST1	SST2	SST3	SST1	SST2	SST3
Hartsville	19xP <sub>1</sub>	0xP <sub>2</sub>	0xP <sub>3</sub>	140xP <sub>1</sub>	0.04xP <sub>2</sub>	0xP <sub>3</sub>	970xP <sub>1</sub>	64xP <sub>2</sub>	0.2xP <sub>3</sub>
Hatch	4xP <sub>1</sub>	0xP <sub>2</sub>	0xP <sub>3</sub>	62xP <sub>1</sub>	0.04xP <sub>2</sub>	0xP <sub>3</sub>	770xP <sub>1</sub>	64xP <sub>2</sub>	0.2xP <sub>3</sub>
Hope Creek	120xP <sub>1</sub>	0xP <sub>2</sub>	0xP <sub>3</sub>	440xP <sub>1</sub>	0xP <sub>2</sub>	0xP <sub>3</sub>	3000xP <sub>1</sub>	160xP <sub>2</sub>	0.5xP <sub>3</sub>
Indian Pt.	830xP <sub>1</sub>	0.08xP <sub>2</sub>	0xP <sub>3</sub>	3600xP <sub>1</sub>	18xP <sub>2</sub>	0xP <sub>3</sub>	8100xP <sub>1</sub>	590xP <sub>2</sub>	1.8xP <sub>3</sub>
Joseph M. Farley	12xP <sub>1</sub>	0xP <sub>2</sub>	0xP <sub>3</sub>	85xP <sub>1</sub>	0.03xP <sub>2</sub>	0xP <sub>3</sub>	670xP <sub>1</sub>	56xP <sub>2</sub>	0.2xP <sub>3</sub>
Kewaunee	1.2xP <sub>1</sub>	0xP <sub>2</sub>	0xP <sub>3</sub>	78xP <sub>1</sub>	0xP <sub>2</sub>	0xP <sub>3</sub>	1200xP <sub>1</sub>	70xP <sub>2</sub>	0.3xP <sub>3</sub>
LaCrosse	32xP <sub>1</sub>	0xP <sub>2</sub>	0xP <sub>3</sub>	200xP <sub>1</sub>	1.8xP <sub>2</sub>	0xP <sub>3</sub>	850xP <sub>1</sub>	58xP <sub>2</sub>	0.2xP <sub>3</sub>
La Salle	26xP <sub>1</sub>	0xP <sub>2</sub>	0xP <sub>3</sub>	180xP <sub>1</sub>	0.6xP <sub>2</sub>	0xP <sub>3</sub>	2800xP <sub>1</sub>	200xP <sub>2</sub>	0.7xP <sub>3</sub>
Limerick	970xP <sub>1</sub>	2.2xP <sub>2</sub>	0xP <sub>3</sub>	2800xP <sub>1</sub>	6.6xP <sub>2</sub>	0xP <sub>3</sub>	5400xP <sub>1</sub>	370xP <sub>2</sub>	1.3xP <sub>3</sub>
Maine Yankee	4.1xP <sub>1</sub>	0xP <sub>2</sub>	0xP <sub>3</sub>	34xP <sub>1</sub>	0xP <sub>2</sub>	0xP <sub>3</sub>	770xP <sub>1</sub>	29xP <sub>2</sub>	0.1xP <sub>3</sub>
Marble Hill	28xP <sub>1</sub>	0xP <sub>2</sub>	0xP <sub>3</sub>	420xP <sub>1</sub>	0xP <sub>2</sub>	0xP <sub>3</sub>	2400xP <sub>1</sub>	180xP <sub>2</sub>	0.7xP <sub>3</sub>
McGuire	130xP <sub>1</sub>	0xP <sub>2</sub>	0xP <sub>3</sub>	680xP <sub>1</sub>	0xP <sub>2</sub>	0xP <sub>3</sub>	1600xP <sub>1</sub>	130xP <sub>2</sub>	0.5xP <sub>3</sub>
Midland	320xP <sub>1</sub>	0.2xP <sub>2</sub>	0xP <sub>3</sub>	1100xP <sub>1</sub>	1.3xP <sub>2</sub>	0xP <sub>3</sub>	2200xP <sub>1</sub>	130xP <sub>2</sub>	0.5xP <sub>3</sub>
Millstone	240xP <sub>1</sub>	0.02xP <sub>2</sub>	0xP <sub>3</sub>	990xP <sub>1</sub>	4.5xP <sub>2</sub>	0xP <sub>3</sub>	3200xP <sub>1</sub>	160xP <sub>2</sub>	0.6xP <sub>3</sub>
Monticello	12xP <sub>1</sub>	0xP <sub>2</sub>	0xP <sub>3</sub>	200xP <sub>1</sub>	0.08xP <sub>2</sub>	0xP <sub>3</sub>	1100xP <sub>1</sub>	98xP <sub>2</sub>	0.4xP <sub>3</sub>
Nine Mile Pt.	5.2xP <sub>1</sub>	0xP <sub>2</sub>	0xP <sub>3</sub>	110xP <sub>1</sub>	0.06xP <sub>2</sub>	0xP <sub>3</sub>	1200xP <sub>1</sub>	58xP <sub>2</sub>	0.2xP <sub>3</sub>
North Anna	14xP <sub>1</sub>	0xP <sub>2</sub>	0xP <sub>3</sub>	92xP <sub>1</sub>	0.08xP <sub>2</sub>	0xP <sub>3</sub>	1800xP <sub>1</sub>	75xP <sub>2</sub>	0.3xP <sub>3</sub>
Oconee	2.0xP <sub>1</sub>	0xP <sub>2</sub>	0xP <sub>3</sub>	240xP <sub>1</sub>	0.03xP <sub>2</sub>	0xP <sub>3</sub>	1100xP <sub>1</sub>	70xP <sub>2</sub>	0.3xP <sub>3</sub>
Oyster Creek	84xP <sub>1</sub>	0xP <sub>2</sub>	0xP <sub>3</sub>	530xP <sub>1</sub>	0.8xP <sub>2</sub>	0xP <sub>3</sub>	4400xP <sub>1</sub>	200xP <sub>2</sub>	0.6xP <sub>3</sub>
Palisades	37xP <sub>1</sub>	0.02xP <sub>2</sub>	0xP <sub>3</sub>	250xP <sub>1</sub>	1.3xP <sub>2</sub>	0xP <sub>3</sub>	1700xP <sub>1</sub>	90xP <sub>2</sub>	0.3xP <sub>3</sub>
Palo Verde	5.8xP <sub>1</sub>	0xP <sub>2</sub>	0xP <sub>3</sub>	59xP <sub>1</sub>	0.2xP <sub>2</sub>	0xP <sub>3</sub>	450xP <sub>1</sub>	26xP <sub>2</sub>	0.09xP <sub>3</sub>
Peach Bottom	97xP <sub>1</sub>	0xP <sub>2</sub>	0xP <sub>3</sub>	400xP <sub>1</sub>	0.02xP <sub>2</sub>	0xP <sub>3</sub>	2800xP <sub>1</sub>	140xP <sub>2</sub>	0.4xP <sub>3</sub>

\*See footnote, page C-2.

Table C-1. (continued)

	Mean Early Fatalities*			Mean Early Injuries*			Mean Latent Cancer Fatalities*		
	SST1	SST2	SST3	SST1	SST2	SST3	SST1	SST2	SST3
Pebble Springs	0.41xP <sub>1</sub>	0xP <sub>2</sub>	0xP <sub>3</sub>	3.7xP <sub>1</sub>	0xP <sub>2</sub>	0xP <sub>3</sub>	230xP <sub>1</sub>	18xP <sub>2</sub>	0.07xP <sub>3</sub>
Perkins	98xP <sub>1</sub>	0xP <sub>2</sub>	0xP <sub>3</sub>	520xP <sub>1</sub>	2.1xP <sub>2</sub>	0xP <sub>3</sub>	1500xP <sub>1</sub>	120xP <sub>2</sub>	0.5xP <sub>3</sub>
Perry	95xP <sub>1</sub>	0.07xP <sub>2</sub>	0xP <sub>3</sub>	520xP <sub>1</sub>	4.2xP <sub>2</sub>	0xP <sub>3</sub>	2500xP <sub>1</sub>	160xP <sub>2</sub>	0.6xP <sub>3</sub>
Phipps Bed	170xP <sub>1</sub>	0.3xP <sub>2</sub>	0xP <sub>3</sub>	300xP <sub>1</sub>	16xP <sub>2</sub>	0xP <sub>3</sub>	1300xP <sub>1</sub>	82xP <sub>2</sub>	0.3xP <sub>3</sub>
Pilgrim	71xP <sub>1</sub>	0.02xP <sub>2</sub>	0xP <sub>3</sub>	300xP <sub>1</sub>	2.4xP <sub>2</sub>	0xP <sub>3</sub>	1500xP <sub>1</sub>	85xP <sub>2</sub>	0.3xP <sub>3</sub>
Pt. Beach	7.7xP <sub>1</sub>	0xP <sub>2</sub>	0xP <sub>3</sub>	110xP <sub>1</sub>	0.3xP <sub>2</sub>	0xP <sub>3</sub>	1400xP <sub>1</sub>	77xP <sub>2</sub>	0.3xP <sub>3</sub>
Prairie Is.	56xP <sub>1</sub>	0xP <sub>2</sub>	0xP <sub>3</sub>	260xP <sub>1</sub>	2.4xP <sub>2</sub>	0xP <sub>3</sub>	1400xP <sub>1</sub>	110xP <sub>2</sub>	0.4xP <sub>3</sub>
Quad Cities	17xP <sub>1</sub>	0xP <sub>2</sub>	0xP <sub>3</sub>	290xP <sub>1</sub>	0.04xP <sub>2</sub>	0xP <sub>3</sub>	1900xP <sub>1</sub>	170xP <sub>2</sub>	0.7xP <sub>3</sub>
Rancho Seco	15xP <sub>1</sub>	0xP <sub>2</sub>	0xP <sub>3</sub>	110xP <sub>1</sub>	0.02xP <sub>2</sub>	0xP <sub>3</sub>	870xP <sub>1</sub>	87xP <sub>2</sub>	0.3xP <sub>3</sub>
River Bend	31xP <sub>1</sub>	0xP <sub>2</sub>	0xP <sub>3</sub>	200xP <sub>1</sub>	0.2xP <sub>2</sub>	0xP <sub>3</sub>	750xP <sub>1</sub>	60xP <sub>2</sub>	0.2xP <sub>3</sub>
Robinson	16xP <sub>1</sub>	0xP <sub>2</sub>	0xP <sub>3</sub>	170xP <sub>1</sub>	0.01xP <sub>2</sub>	0xP <sub>3</sub>	880xP <sub>1</sub>	59xP <sub>2</sub>	0.2xP <sub>3</sub>
St. Lucie	77xP <sub>1</sub>	0xP <sub>2</sub>	0xP <sub>3</sub>	310xP <sub>1</sub>	0.6xP <sub>2</sub>	0xP <sub>3</sub>	700xP <sub>1</sub>	69xP <sub>2</sub>	0.4xP <sub>3</sub>
Salem	120xP <sub>1</sub>	0xP <sub>2</sub>	0xP <sub>3</sub>	440xP <sub>1</sub>	0xP <sub>2</sub>	0xP <sub>3</sub>	3000xP <sub>1</sub>	160xP <sub>2</sub>	0.5xP <sub>3</sub>
San Onofre	11xP <sub>1</sub>	0xP <sub>2</sub>	0xP <sub>3</sub>	150xP <sub>1</sub>	0xP <sub>2</sub>	0xP <sub>3</sub>	1800xP <sub>1</sub>	150xP <sub>2</sub>	0.5xP <sub>3</sub>
Seabrook	13xP <sub>1</sub>	0xP <sub>2</sub>	0xP <sub>3</sub>	210xP <sub>1</sub>	0.04xP <sub>2</sub>	0xP <sub>3</sub>	1000xP <sub>1</sub>	54xP <sub>2</sub>	0.2xP <sub>3</sub>
Sequoyah	110xP <sub>1</sub>	0xP <sub>2</sub>	0xP <sub>3</sub>	690xP <sub>1</sub>	0.6xP <sub>2</sub>	0xP <sub>3</sub>	1300xP <sub>1</sub>	95xP <sub>2</sub>	0.3xP <sub>3</sub>
Shearon Harris	40xP <sub>1</sub>	0xP <sub>2</sub>	0xP <sub>3</sub>	260xP <sub>1</sub>	0.4xP <sub>2</sub>	0xP <sub>3</sub>	1300xP <sub>1</sub>	110xP <sub>2</sub>	0.4xP <sub>3</sub>
Shoreham	140xP <sub>1</sub>	0xP <sub>2</sub>	0xP <sub>3</sub>	870xP <sub>1</sub>	1.9xP <sub>2</sub>	0xP <sub>3</sub>	3400xP <sub>1</sub>	170xP <sub>2</sub>	0.5xP <sub>3</sub>
Skagit	50xP <sub>1</sub>	0xP <sub>2</sub>	0xP <sub>3</sub>	370xP <sub>1</sub>	0.4xP <sub>2</sub>	0xP <sub>3</sub>	500xP <sub>1</sub>	49xP <sub>2</sub>	0.2xP <sub>3</sub>
South Texas	5.2xP <sub>1</sub>	0xP <sub>2</sub>	0xP <sub>3</sub>	32xP <sub>1</sub>	0xP <sub>2</sub>	0xP <sub>3</sub>	610xP <sub>1</sub>	43xP <sub>2</sub>	0.2xP <sub>3</sub>
Surry	65xP <sub>1</sub>	0xP <sub>2</sub>	0xP <sub>3</sub>	330xP <sub>1</sub>	0xP <sub>2</sub>	0xP <sub>3</sub>	1700xP <sub>1</sub>	95xP <sub>2</sub>	0.3xP <sub>3</sub>
Susquehanna	180xP <sub>1</sub>	0xP <sub>2</sub>	0xP <sub>3</sub>	700xP <sub>1</sub>	0.2xP <sub>2</sub>	0xP <sub>3</sub>	3300xP <sub>1</sub>	150xP <sub>2</sub>	0.5xP <sub>3</sub>

\*See footnote, page C-2.

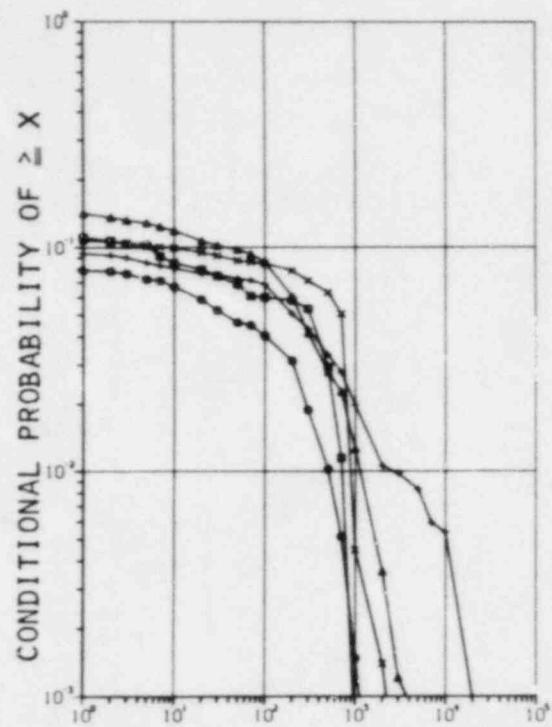
Table C-1. (continued)

	Mean Early Fatalities*			Mean Early Injuries*			Mean Latent Cancer Fatalities*		
	SST1	SST2	SST3	SST1	SST2	SST3	SST1	SST2	SST3
Three Mile Island	240xP <sub>1</sub>	0xP <sub>2</sub>	0xP <sub>3</sub>	1200xP <sub>1</sub>	4.5xP <sub>2</sub>	0xP <sub>3</sub>	3500xP <sub>1</sub>	170xP <sub>2</sub>	0.6xP <sub>3</sub>
Trojan	46xP <sub>1</sub>	0.1xP <sub>2</sub>	0xP <sub>3</sub>	350xP <sub>1</sub>	3.8xP <sub>2</sub>	0xP <sub>3</sub>	1100xP <sub>1</sub>	73xP <sub>2</sub>	0.3xP <sub>3</sub>
Turkey Pt.	31xP <sub>1</sub>	0xP <sub>2</sub>	0xP <sub>3</sub>	460xP <sub>1</sub>	0xP <sub>2</sub>	0xP <sub>3</sub>	690xP <sub>1</sub>	83xP <sub>2</sub>	0.4xP <sub>3</sub>
Vermont Yankee	130xP <sub>1</sub>	0xP <sub>2</sub>	0xP <sub>3</sub>	320xP <sub>1</sub>	4.4xP <sub>2</sub>	0xP <sub>3</sub>	1800xP <sub>1</sub>	72xP <sub>2</sub>	0.3xP <sub>3</sub>
Virgil Summer	12xP <sub>1</sub>	0xP <sub>2</sub>	0xP <sub>3</sub>	120xP <sub>1</sub>	0xP <sub>2</sub>	0xP <sub>3</sub>	1000xP <sub>1</sub>	63xP <sub>2</sub>	0.2xP <sub>3</sub>
Vogtle	0.07xP <sub>1</sub>	0xP <sub>2</sub>	0xP <sub>3</sub>	85xP <sub>1</sub>	0xP <sub>2</sub>	0xP <sub>3</sub>	900xP <sub>1</sub>	70xP <sub>2</sub>	0.3xP <sub>3</sub>
WPPSS 1,4	0.1xP <sub>1</sub>	0xP <sub>2</sub>	0xP <sub>3</sub>	110xP <sub>1</sub>	0xP <sub>2</sub>	0xP <sub>3</sub>	310xP <sub>1</sub>	37xP <sub>2</sub>	0.2xP <sub>3</sub>
WPPSS 2	1.0xP <sub>1</sub>	0xP <sub>2</sub>	0xP <sub>3</sub>	120xP <sub>1</sub>	0xP <sub>2</sub>	0xP <sub>3</sub>	720xP <sub>1</sub>	53xP <sub>2</sub>	0.2xP <sub>3</sub>
WPPSS 3,5	0.1xP <sub>1</sub>	0xP <sub>2</sub>	0xP <sub>3</sub>	110xP <sub>1</sub>	0xP <sub>2</sub>	0xP <sub>3</sub>	310xP <sub>1</sub>	37xP <sub>2</sub>	0.2xP <sub>3</sub>
Waterford	170xP <sub>1</sub>	0.2xP <sub>2</sub>	0xP <sub>3</sub>	580xP <sub>1</sub>	8.3xP <sub>2</sub>	0xP <sub>3</sub>	990xP <sub>1</sub>	93xP <sub>2</sub>	0.4xP <sub>3</sub>
Watts Bar	13xP <sub>1</sub>	0xP <sub>2</sub>	0xP <sub>3</sub>	110xP <sub>1</sub>	0.02xP <sub>2</sub>	0xP <sub>3</sub>	1000xP <sub>1</sub>	66xP <sub>2</sub>	0.3xP <sub>3</sub>
Wolf Creek	2.4xP <sub>1</sub>	0xP <sub>2</sub>	0xP <sub>3</sub>	34xP <sub>1</sub>	0.04xP <sub>2</sub>	0xP <sub>3</sub>	760xP <sub>1</sub>	70xP <sub>2</sub>	0.3xP <sub>3</sub>
Yankee Rowe	18xP <sub>1</sub>	0xP <sub>2</sub>	0xP <sub>3</sub>	180xP <sub>1</sub>	0.05xP <sub>2</sub>	0xP <sub>3</sub>	2300xP <sub>1</sub>	100xP <sub>2</sub>	0.2xP <sub>3</sub>
Yellow Creek	5.6xP <sub>1</sub>	0xP <sub>2</sub>	0xP <sub>3</sub>	68xP <sub>1</sub>	0xP <sub>2</sub>	0xP <sub>3</sub>	850xP <sub>1</sub>	63xP <sub>2</sub>	0.3xP <sub>3</sub>
Zimmer	46xP <sub>1</sub>	0xP <sub>2</sub>	0xP <sub>3</sub>	670xP <sub>1</sub>	0.4xP <sub>2</sub>	0xP <sub>3</sub>	2400xP <sub>1</sub>	170xP <sub>2</sub>	0.6xP <sub>3</sub>
Zion	520xP <sub>1</sub>	4.1xP <sub>2</sub>	0xP <sub>3</sub>	1600xP <sub>1</sub>	32xP <sub>2</sub>	0xP <sub>3</sub>	4000xP <sub>1</sub>	330xP <sub>2</sub>	1.2xP <sub>3</sub>

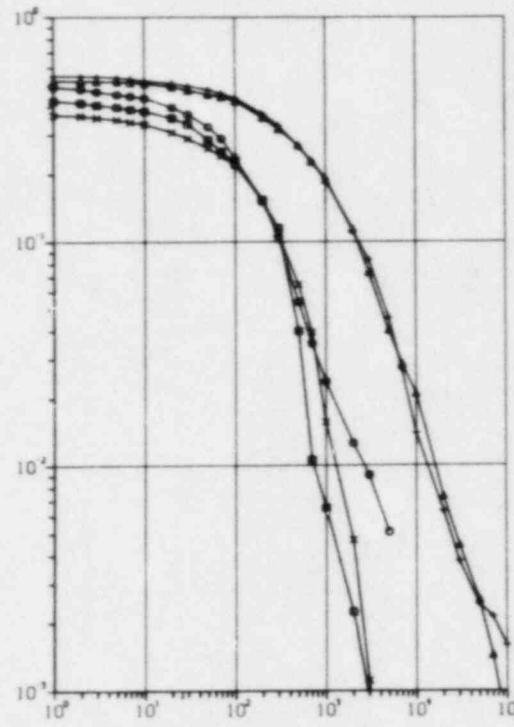
\*See footnote, page C-2.

Note: These CCDFs do not represent effects from existing reactor/site combinations, all assume an 1120 MWe reactor. In addition, these results are conditional on the occurrence of a hypothetical SST1 release. Recent evidence suggests that the source term magnitude assumed for SST1 may be overestimated by a factor of 10 or more (see section 2.3.2).

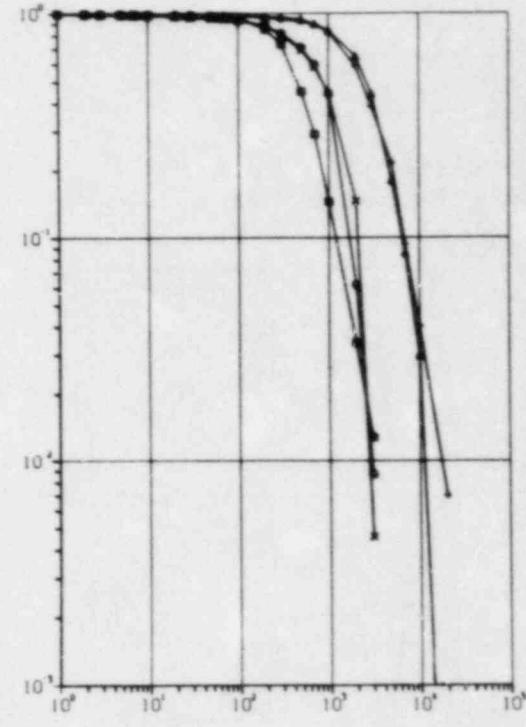
C-7



EARLY FATALITIES,  $X$ ,  
CONDITIONAL ON SST1\*



EARLY INJURIES,  $X$ ,  
CONDITIONAL ON SST1\*



LATENT CANCER FATALITIES,  $X$ ,  
CONDITIONAL ON SST1\*

#### KEY

- ALLENS CREEK
- ARKANSAS
- △ BAILEY
- BEAVER VALLEY
- × BELLEVILLE

Figure C-1: Early fatality, early injury, and latent cancer fatality CCDFs at named sites, conditional on an SST1 release.

Assumptions: 1120 MWe reactor, summary evacuation, representative meteorology (see Appendix A), and actual site population and windrose.

\*See footnote, page C-2.

Note: These CCDFs do not represent effects from existing reactor/site combinations, all assume an 1120 MWe reactor. In addition, these results are conditional on the occurrence of a hypothetical SST1 release. Recent evidence suggests that the source term magnitude assumed for SST1 may be overestimated by a factor of 10 or more (see section 2.3.2).

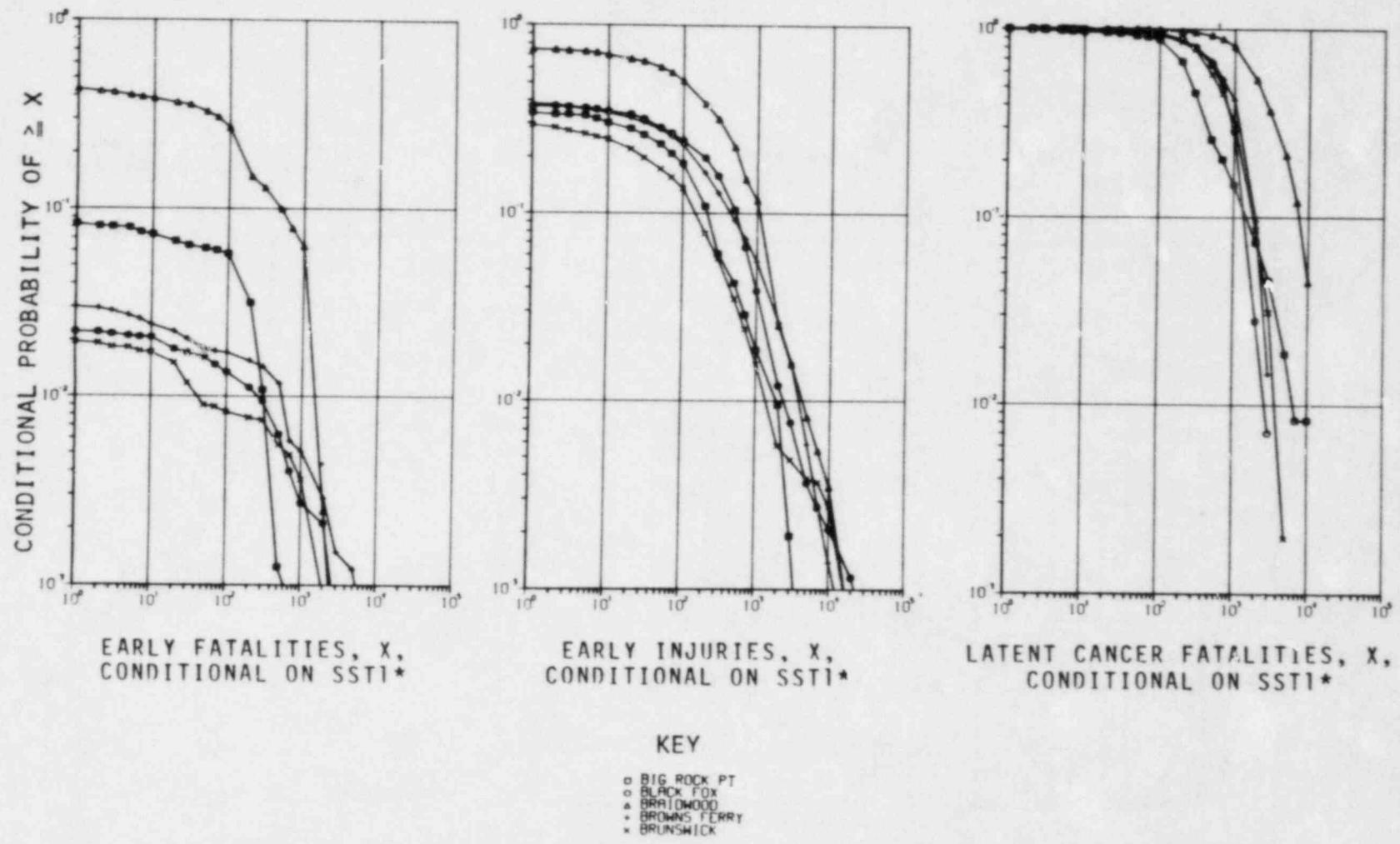


Figure C-2: Early fatality, early injury, and latent cancer fatality CCDFs at named sites, conditional on an SST1 release.

Assumptions: 1120 MWe reactor, summary evacuation, representative meteorology (see Appendix A), and actual site population and windrose.

\*See footnote, page C-2.

Note: These CCDFs do not represent effects from existing reactor/site combinations, all assume an 1120 MWe reactor. In addition, these results are conditional on the occurrence of a hypothetical SST1 release. Recent evidence suggests that the source term magnitude assumed for SST1 may be overestimated by a factor of 10 or more (see section 2.3.2).

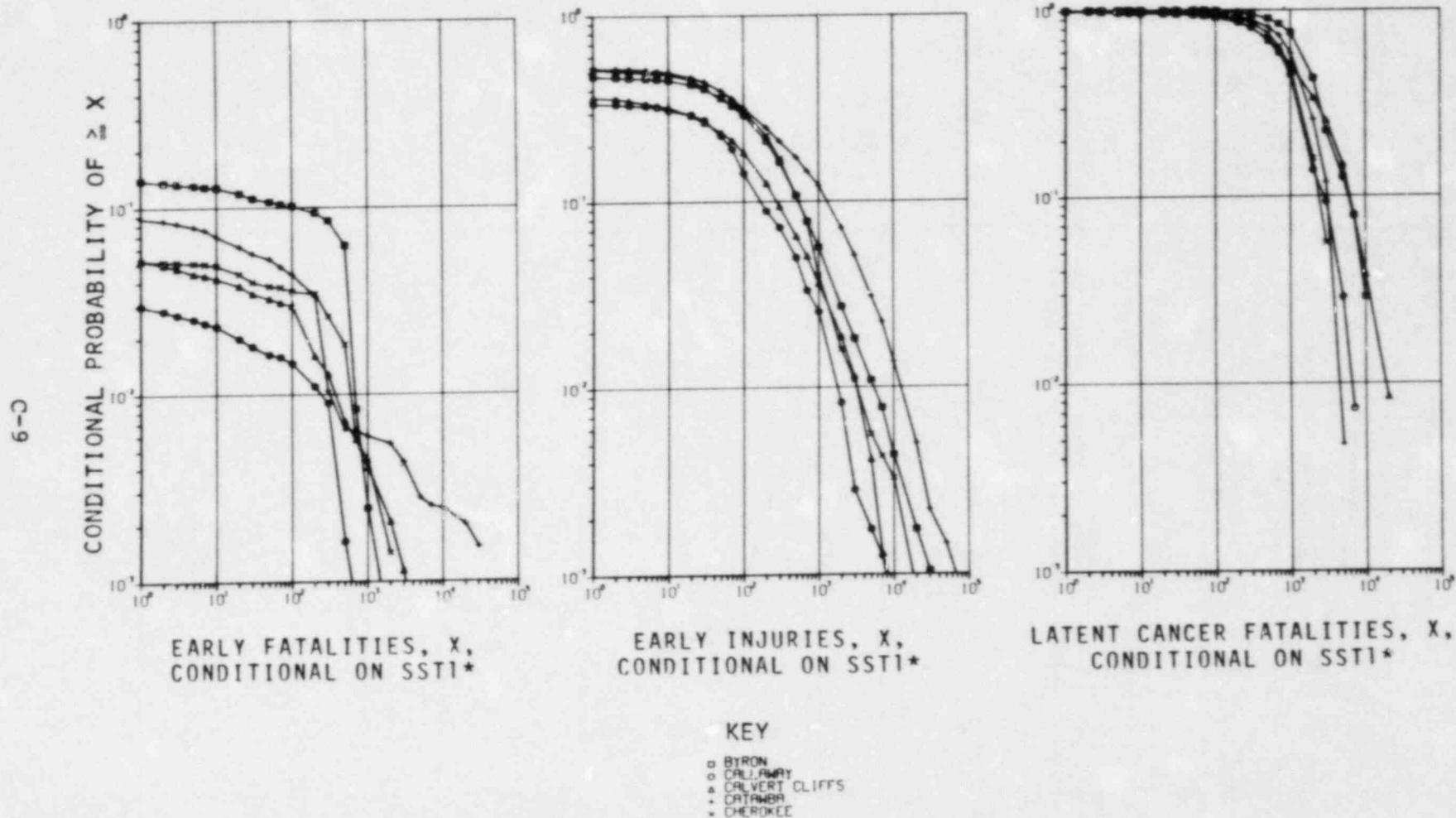
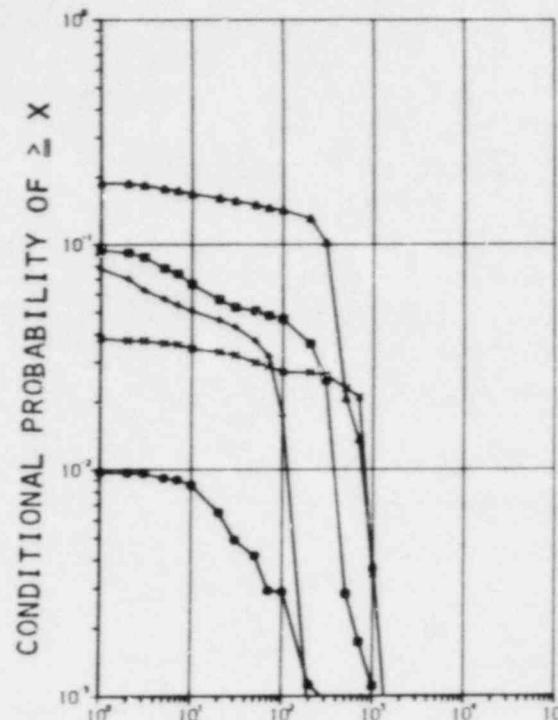


Figure C-3: Early fatality, early injury, and latent cancer fatality CCDFs at named sites, conditional on an SST1 release.  
Assumptions: 1120 MWe reactor, summary evacuation, representative meteorology (see Appendix A), and actual site population and windrose.

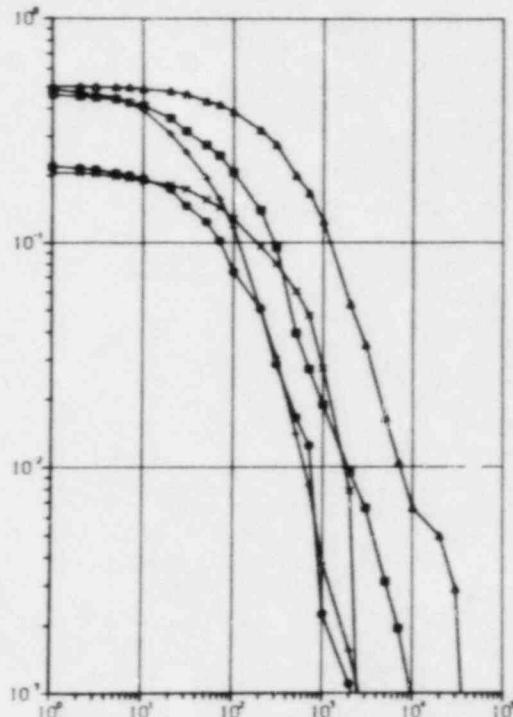
\*See footnote, page C-2.

Note: These CCDFs do not represent effects from existing reactor/site combinations, all assume an 1120 MWe reactor. In addition, these results are conditional on the occurrence of a hypothetical SST1 release. Recent evidence suggests that the source term magnitude assumed for SST1 may be overestimated by a factor of 10 or more (see section 2.3.2).

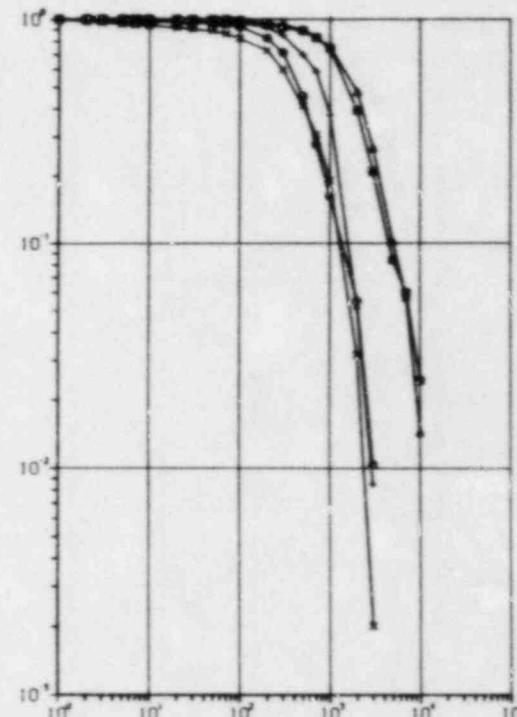
C-10



EARLY FATALITIES,  $X$ ,  
CONDITIONAL ON SST1\*



EARLY INJURIES,  $X$ ,  
CONDITIONAL ON SST1\*



LATENT CANCER FATALITIES,  $X$ ,  
CONDITIONAL ON SST1\*

#### KEY

- CLINTON
- COMMENCHE PERK
- ▲ D.C. COOK
- COOPER
- × CRYSTAL RIVER

Figure C-4: Early fatality, early injury, and latent cancer fatality CCDFs at named sites, conditional on an SST1 release.

Assumptions: 1120 MWe reactor, summary evacuation, representative meteorology (see Appendix A), and actual site population and windrose.

\*See footnote, page C-2.

Note: These CCDFs do not represent effects from existing reactor/site combinations, all assume an 1120 MWe reactor. In addition, these results are conditional on the occurrence of a hypothetical SST1 release. Recent evidence suggests that the source term magnitude assumed for SST1 may be overestimated by a factor of 10 or more (see section 2.3.2).

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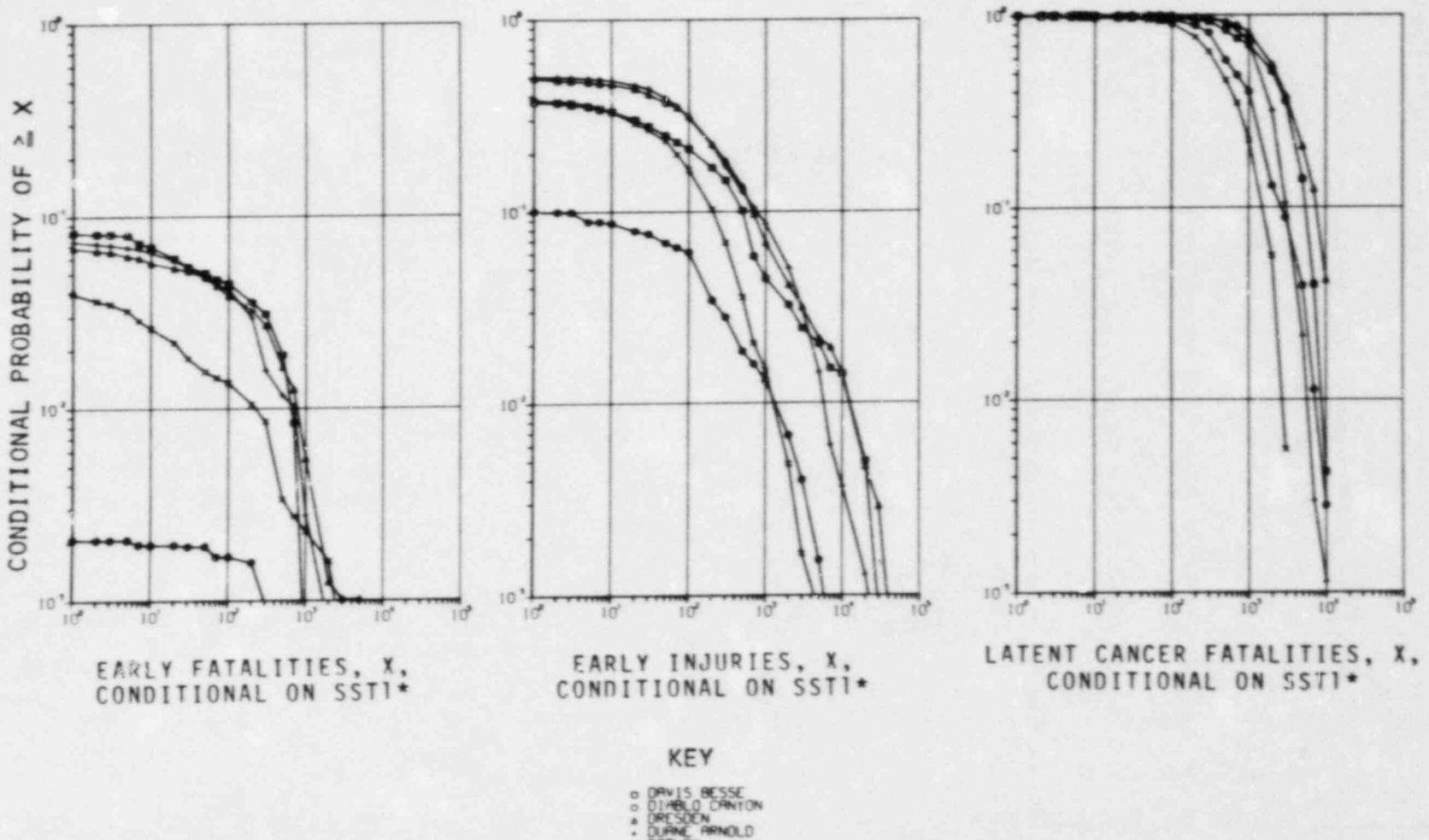


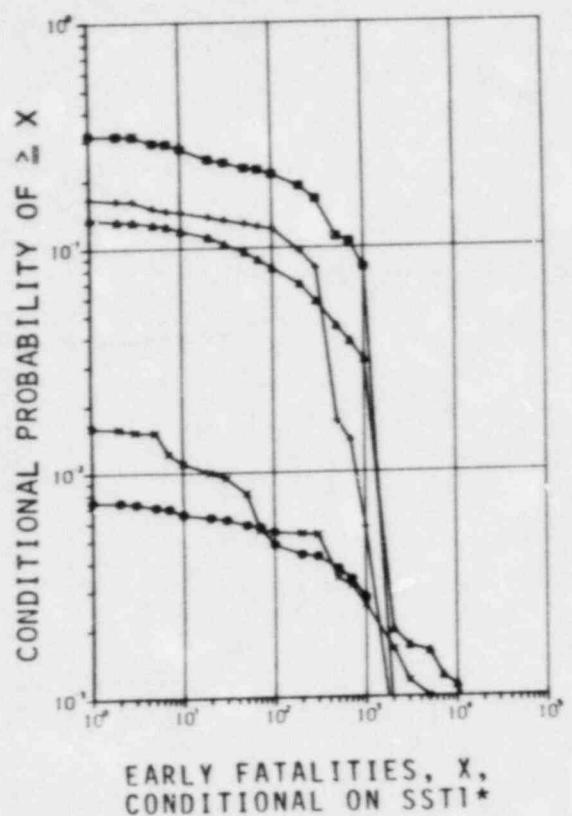
Figure C-5: Early fatality, early injury, and latent cancer fatality CCDFs at named sites, conditional on an SST1 release.

Assumptions: 1120 MWe reactor, summary evacuation, representative meteorology (see Appendix A), and actual site population and windrose.

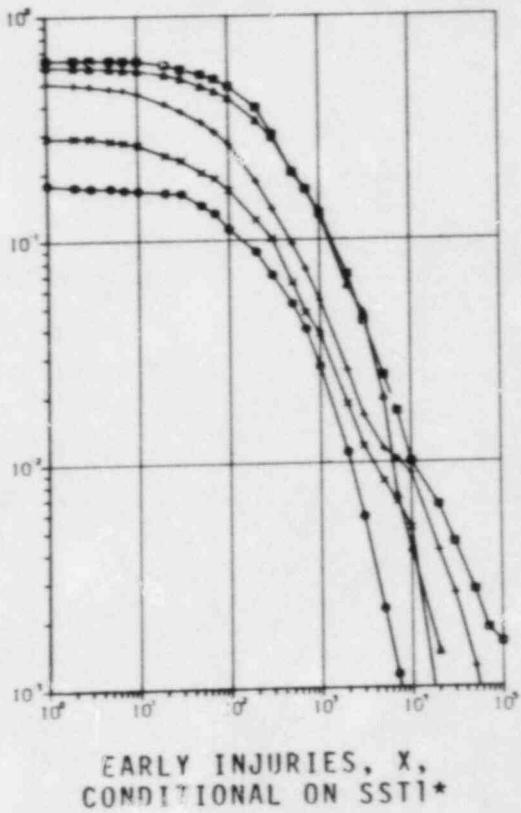
\*See footnote, page C-2.

Note: These CCDFs do not represent effects from existing reactor/site combinations, all assume an 1120 MWe reactor. In addition, these results are conditional on the occurrence of a hypothetical SST1 release. Recent evidence suggests that the source term magnitude assumed for SST1 may be overestimated by a factor of 10 or more (see section 2.3.2).

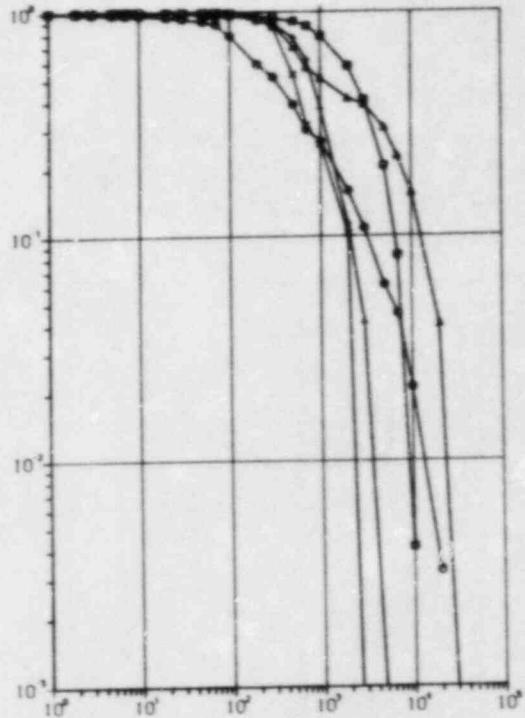
C-12



EARLY FATALITIES,  $X$ ,  
CONDITIONAL ON SST1\*



EARLY INJURIES,  $X$ ,  
CONDITIONAL ON SST1\*



LATENT CANCER FATALITIES,  $X$ ,  
CONDITIONAL ON SST1\*

#### KEY

- FERM
- FITZPATRICK
- △ FORKED RIVER
- + FORT CALHOUN
- × FORT ST. VRIN

Figure C-6: Early fatality, early injury, and latent cancer fatality CCDFs at named sites, conditional on an SST1 release.  
Assumptions: 1120 MWe reactor, summary evacuation, representative meteorology (see Appendix A), and actual site population and windrose.

\*See footnote, page C-2.

Note: These CCDFs do not represent effects from existing reactor/site combinations, all assume an 1120 MWe reactor. In addition, these results are conditional on the occurrence of a hypothetical SST1 release. Recent evidence suggests that the source term magnitude assumed for SST1 may be overestimated by a factor of 10 or more (see section 2.3.2).

C-13

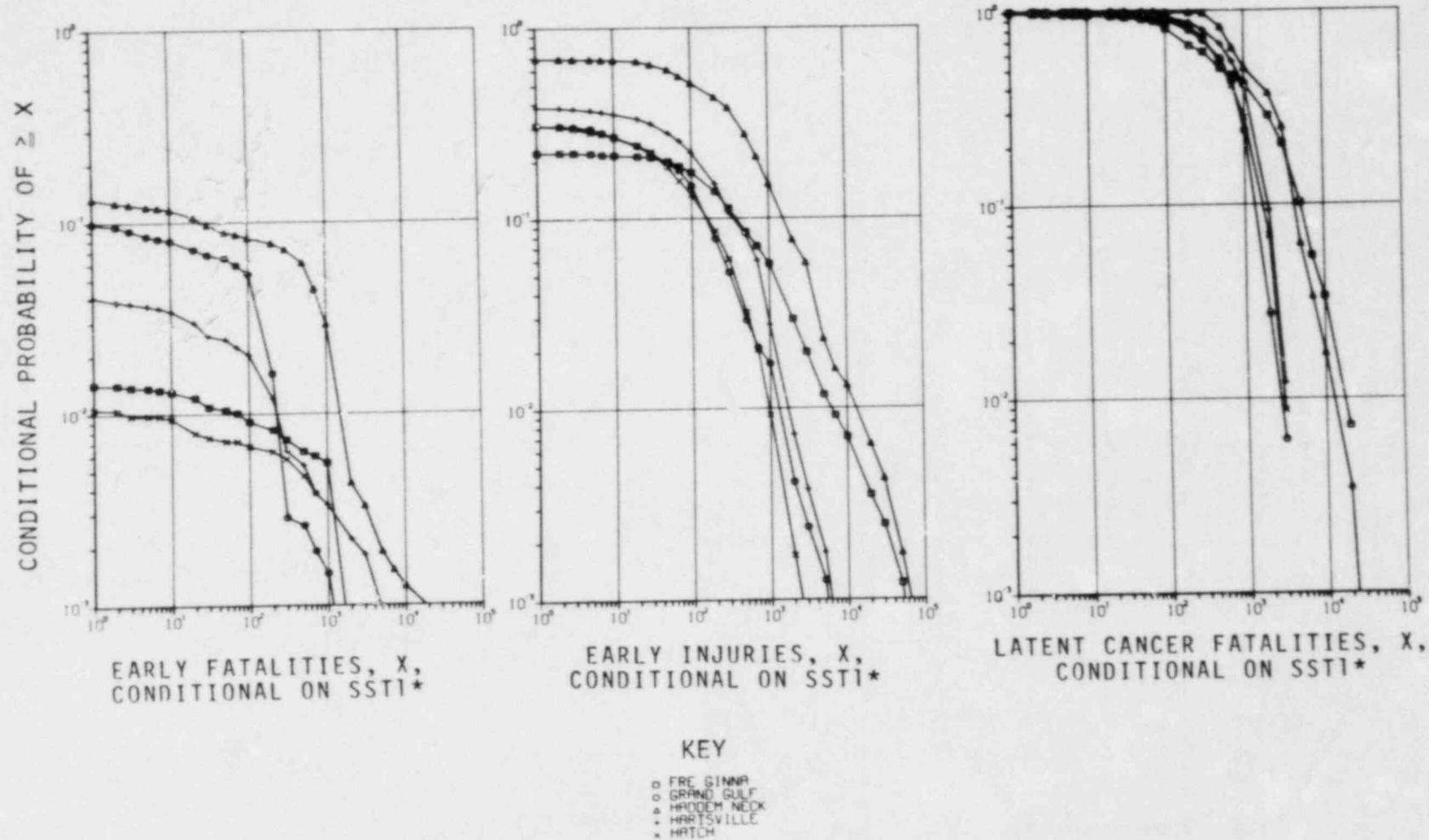


Figure C-7: Early fatality, early injury, and latent cancer fatality CCDFs at named sites, conditional on an SST1 release.  
Assumptions: 1120 MWe reactor, summary evacuation, representative meteorology (see Appendix A), and actual site population and windrose.

\*See footnote, page C-2.

Note: These CCDFs do not represent effects from existing reactor/site combinations, all assume an 1120 MWe reactor. In addition, these results are conditional on the occurrence of a hypothetical SST1 release. Recent evidence suggests that the source term magnitude assumed for SST1 may be overestimated by a factor of 10 or more (see section 2.3.2).

C-14

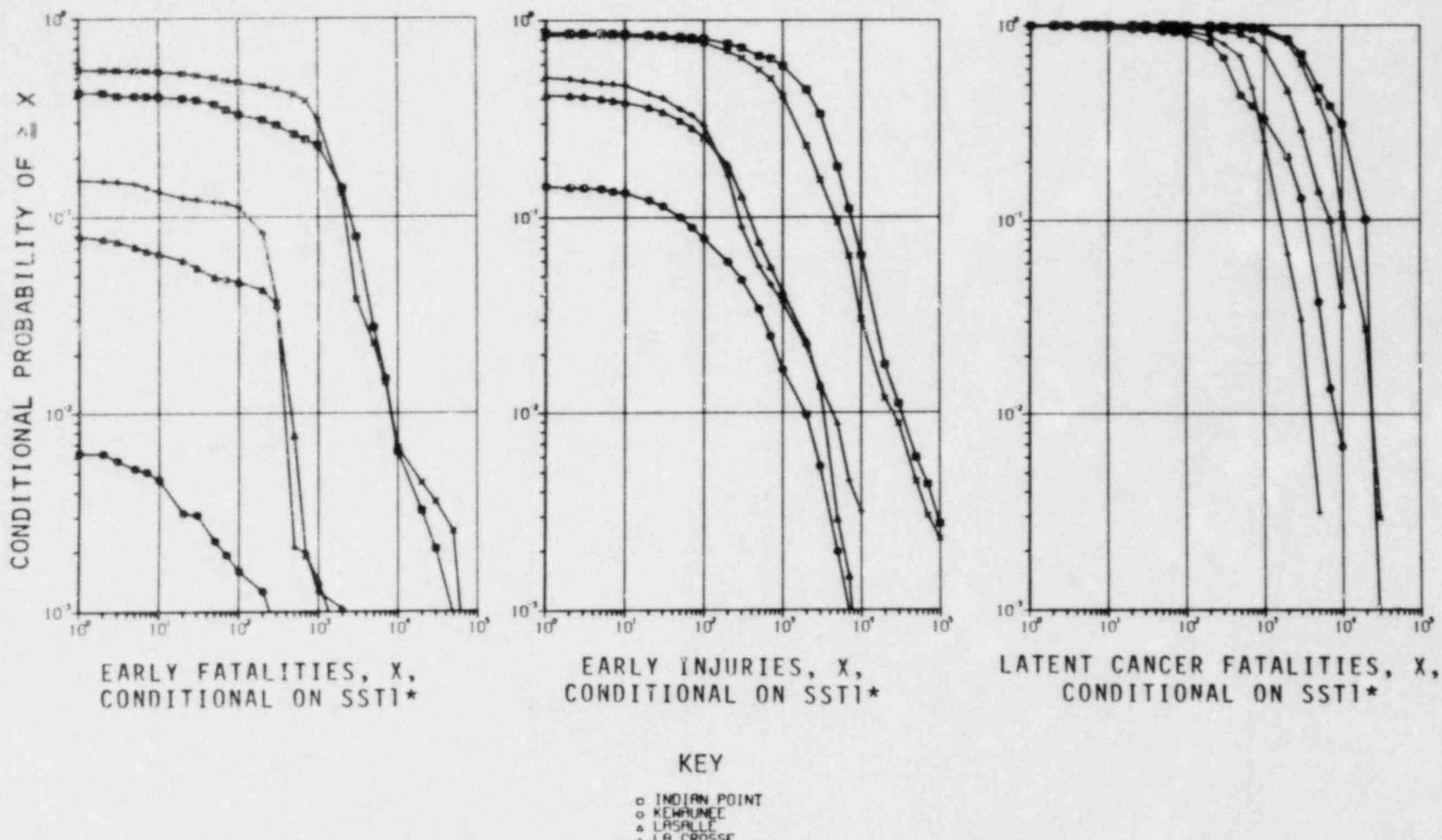


Figure C-8: Early fatality, early injury, and latent cancer fatality CCDFs at named sites, conditional on an SST1 release.

Assumptions: 1120 MWe reactor, summary evacuation, representative meteorology (see Appendix A), and actual site population and windrose.

\*See footnote, page C-2.

Note: These CCDFs do not represent effects from existing reactor/site combinations, all assume a 1120 MWe reactor. In addition, these results are conditional on the occurrence of a hypothetical SSTI release. Recent evidence suggests that the source term magnitude assumed for SSTI may be overestimated by a factor of 10 or more (see section 2.3.2).

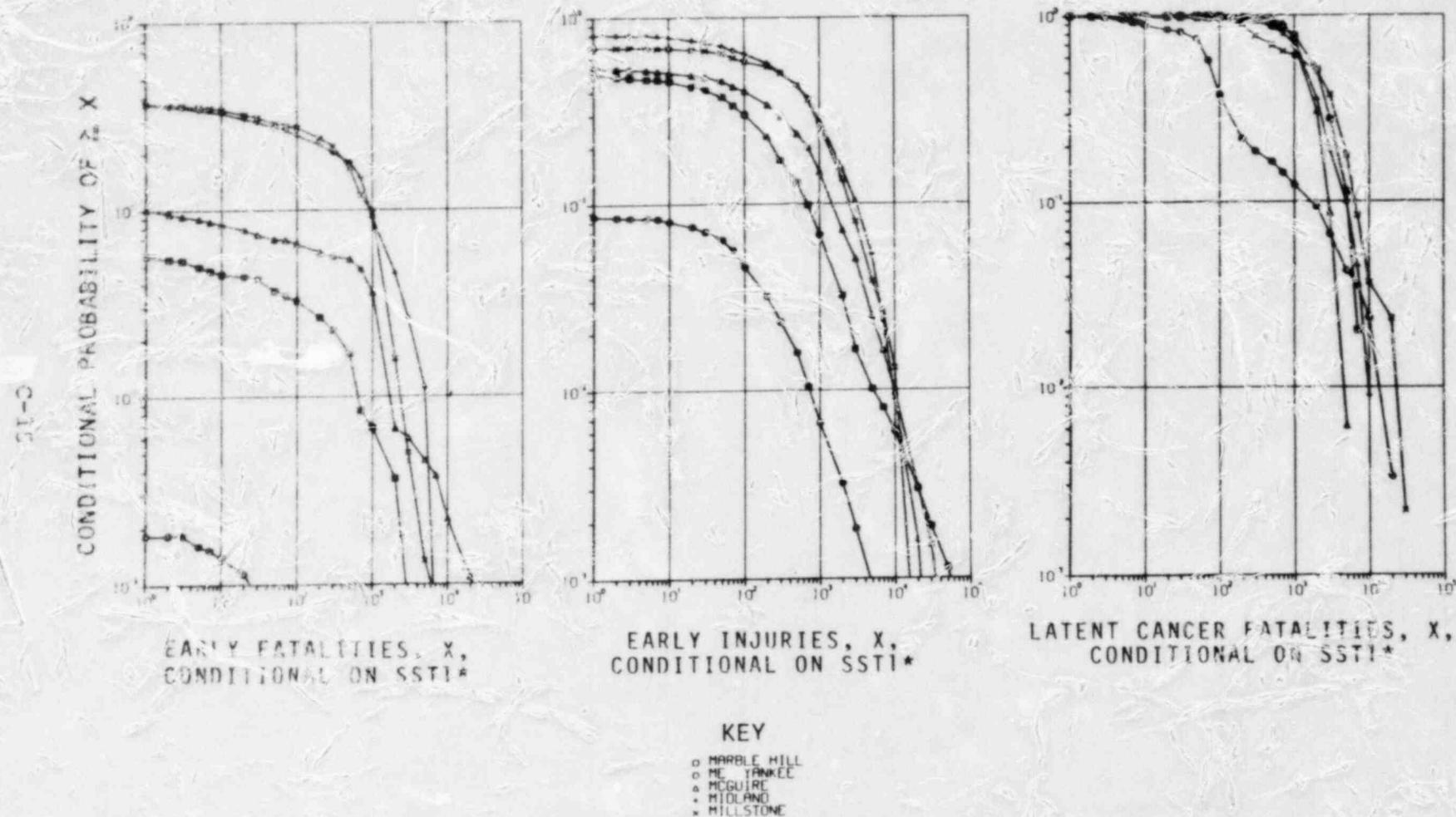


Figure C-9: Early fatality, early injury, and latent cancer fatality CCDFs at named sites, conditional on an SSTI release.  
Assumptions: 1120-MWe reactor, summary evacuation, representative meteorology (see Appendix A), and actual site population and windrose.

\*See footnote, page C-2.

Note: These CCDFs do not represent effects from existing reactor/site combinations, all assume an 1120 MWe reactor. In addition, these results are conditional on the occurrence of a hypothetical SST1 release. Recent evidence suggests that the source term magnitude assumed for SST1 may be overestimated by a factor of 10 or more (see section 2.3.2).

C-16

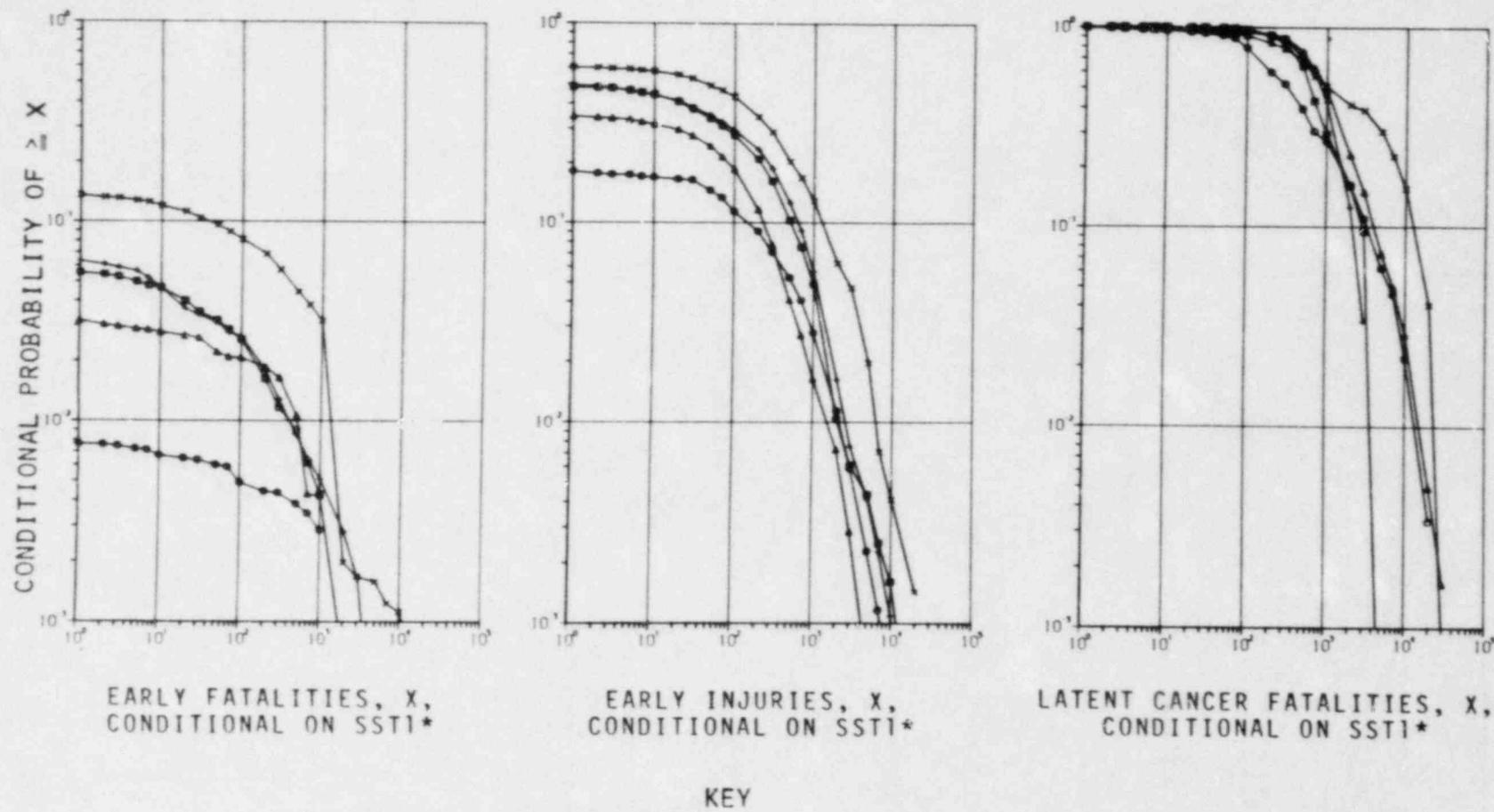


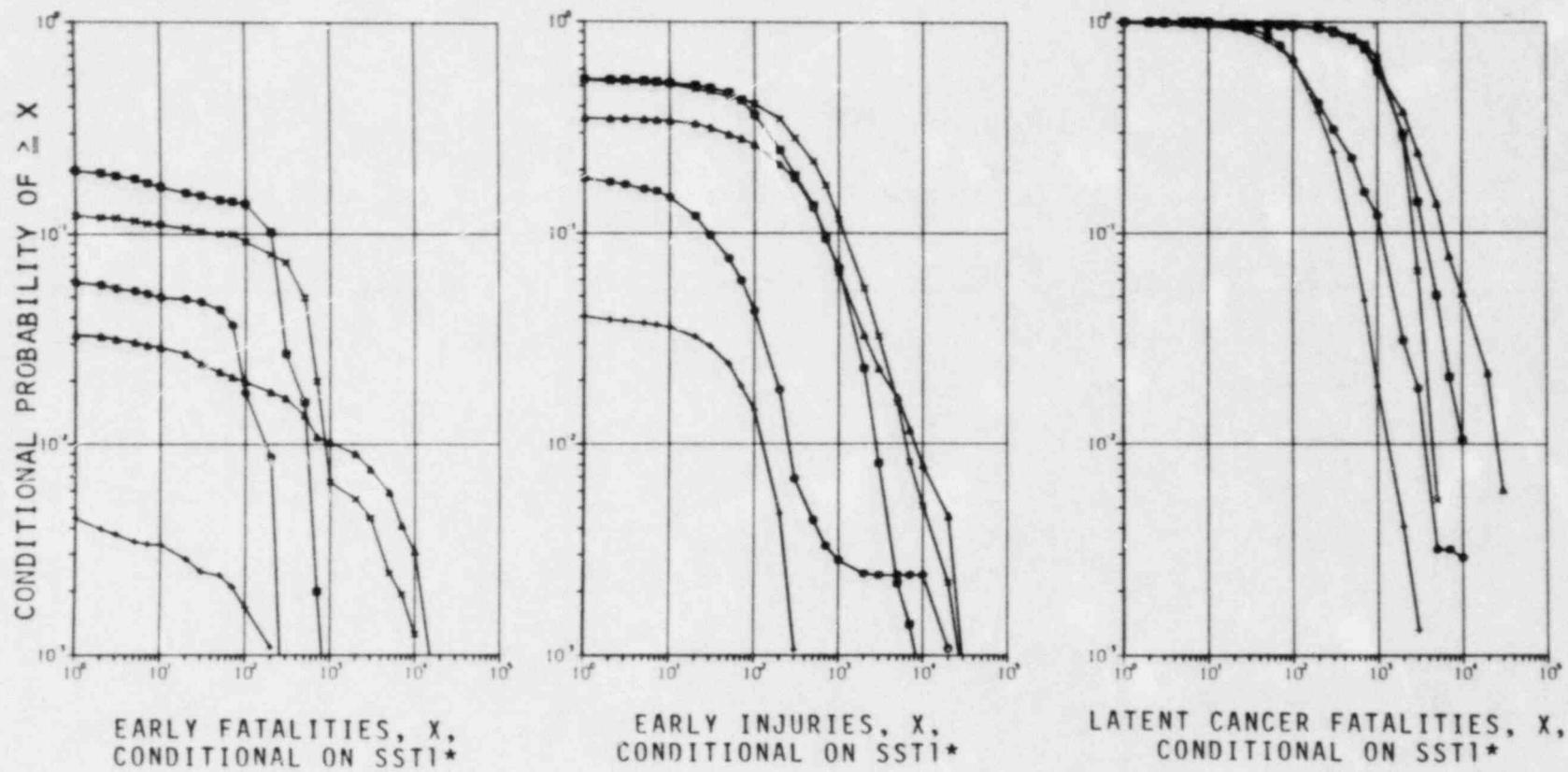
Figure C-10: Early fatality, early injury, and latent cancer fatality CCDFs at named sites, conditional on an SST1 release.

Assumptions: 1120 MWe reactor, summary evacuation, representative meteorology (see Appendix A), and actual site population and windrose.

\*See footnote, page C-2.

Note: These CCDFs do not represent effects from existing reactor/site combinations, all assume an 1120 MWe reactor. In addition, these results are conditional on the occurrence of a hypothetical SST1 release. Recent evidence suggests that the source term magnitude assumed for SST1 may be overestimated by a factor of 10 or more (see section 2.3.2).

C-17



#### KEY

- PALISADE
- PALO VERDE
- △ PEACH BOTTOM
- PEBBLE SPRINGS
- ✖ PERKINS

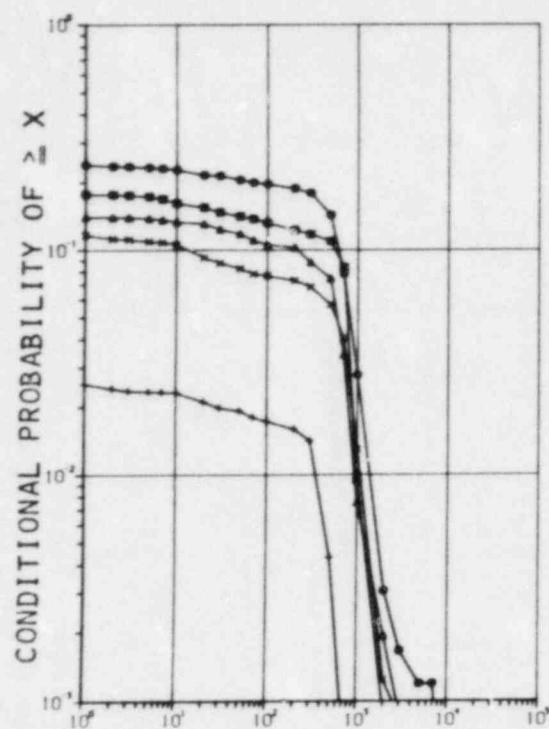
Figure C-11: Early fatality, early injury, and latent cancer fatality CCDFs at named sites, conditional on an SST1 release.

Assumptions: 1120 MWe reactor, summary evacuation, representative meteorology (see Appendix A), and actual site population and windrose.

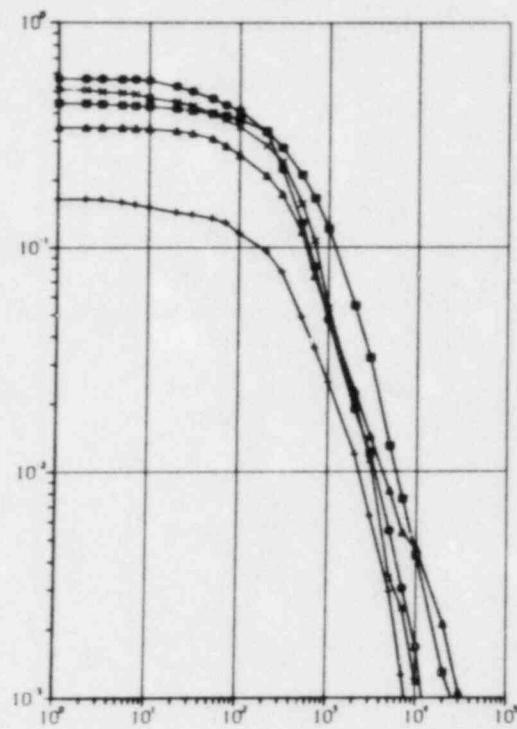
\*See footnote, page C-2.

Note: These CCDFs do not represent effects from existing reactor/site combinations, all assume an 1120 MWe reactor. In addition, these results are conditional on the occurrence of a hypothetical SST1 release. Recent evidence suggests that the source term magnitude assumed for SST1 may be overestimated by a factor of 10 or more (see section 2.3.2).

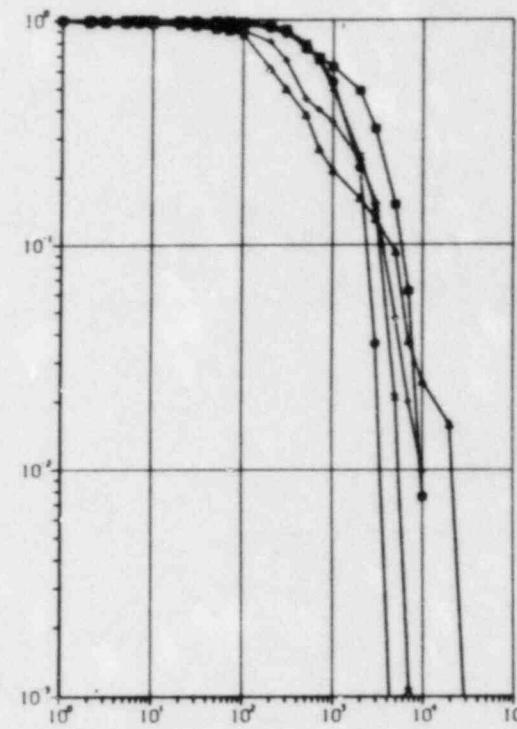
C-18



EARLY FATALITIES,  $X$ ,  
CONDITIONAL ON SST1\*



EARLY INJURIES,  $X$ ,  
CONDITIONAL ON SST1\*



LATENT CANCER FATALITIES,  $X$ ,  
CONDITIONAL ON SST1\*

KEY

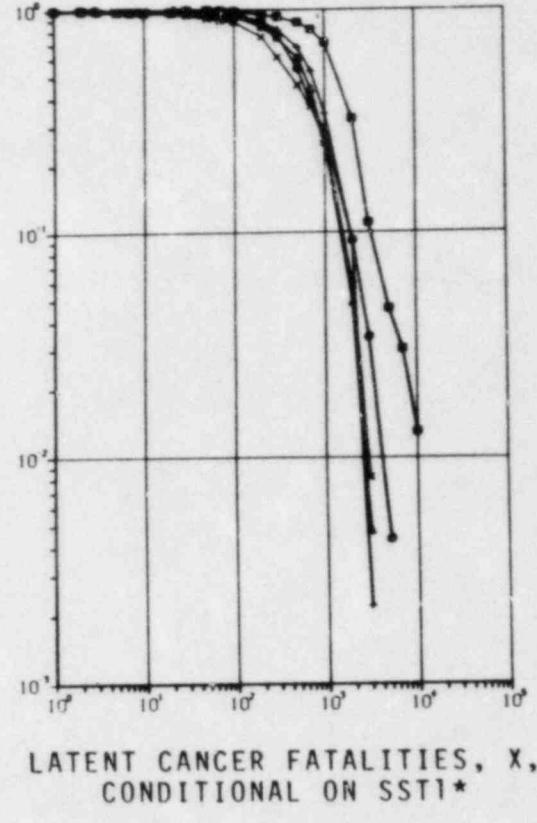
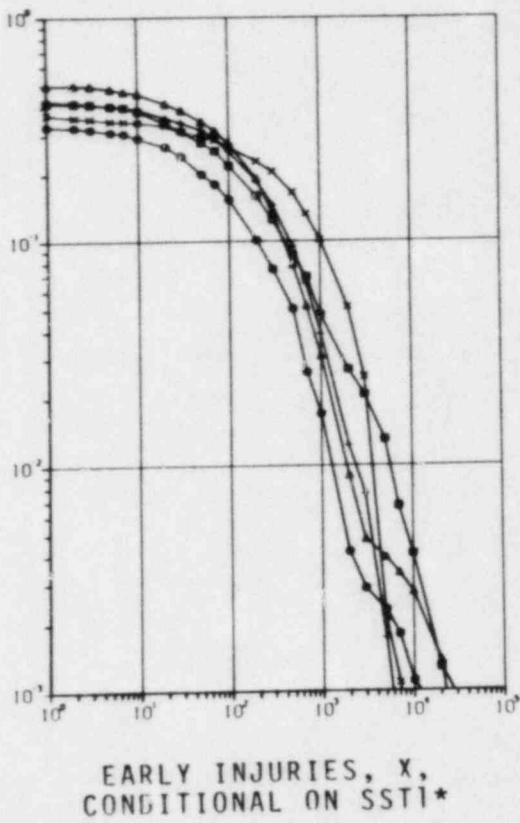
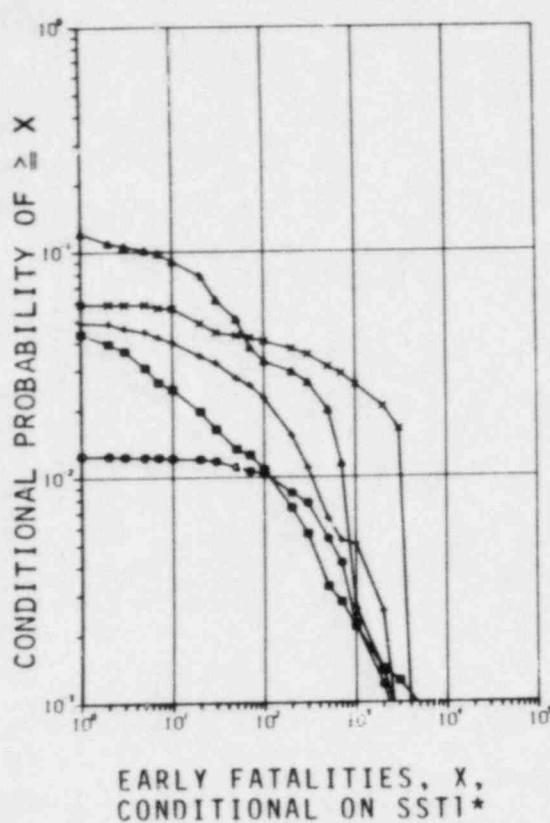
- PERRY
- PHIPPS
- ▲ PILGRIM
- POINT BEACH
- × PRAIRIE

Figure C-12: Early fatality, early injury, and latent cancer fatality CCDFs at named sites, conditional on an SST1 release.  
Assumptions: 1120 MWe reactor, summary evacuation, representative meteorology (see Appendix A), and actual site population and windrose.

\*See footnote, page C-2.

Note: These CCDFs do not represent effects from existing reactor/site combinations, all assume an 1120 MWe reactor. In addition, these results are conditional on the occurrence of a hypothetical SST1 release. Recent evidence suggests that the source term magnitude assumed for SST1 may be overestimated by a factor of 10 or more (see section 2.3.2).

C-19



#### KEY

- QUAD CITIES
- △ RANCHO SECO
- ▲ RIVERBEND
- ROBINSON
- × SAINT LUCIE

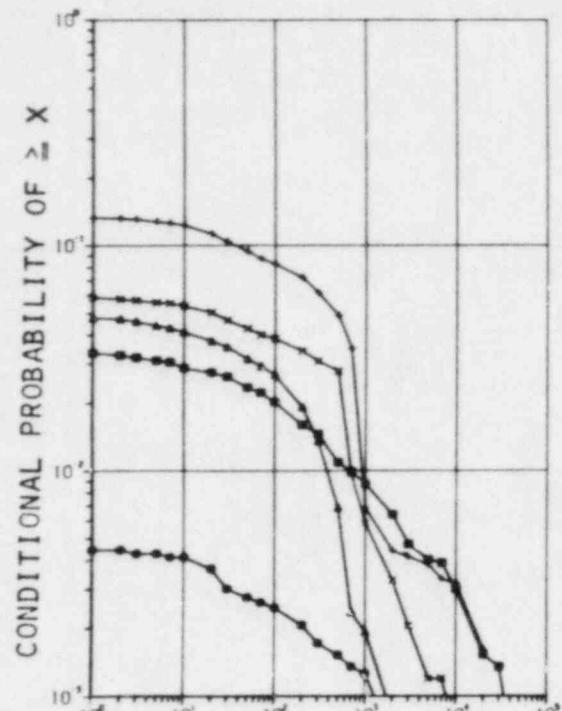
Figure C-13: Early fatality, early injury, and latent cancer fatality CCDFs at named sites, conditional on an SST1 release.

Assumptions: 1120 MWe reactor, summary evacuation, representative meteorology (see Appendix A), and actual site population and windrose.

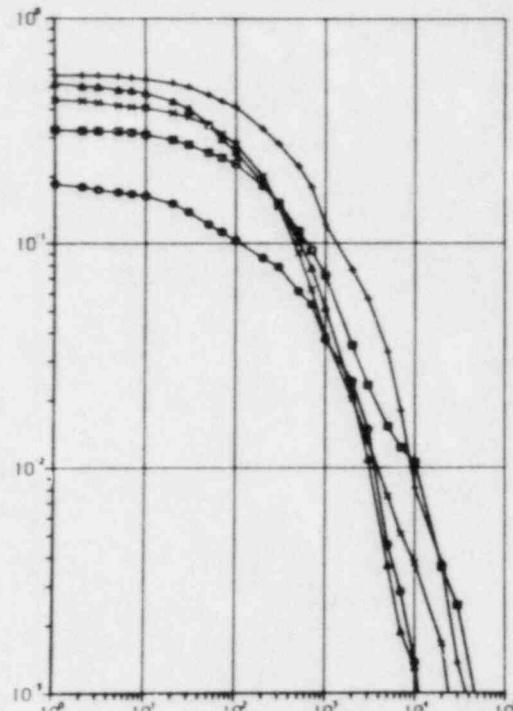
\*See footnote, page C-2.

Note: These CCDFs do not represent effects from existing reactor/site combinations, all assume an 1120 MWe reactor. In addition, these results are conditional on the occurrence of a hypothetical SST1 release. Recent evidence suggests that the source term magnitude assumed for SST1 may be overestimated by a factor of 10 or more (see section 2.3.2).

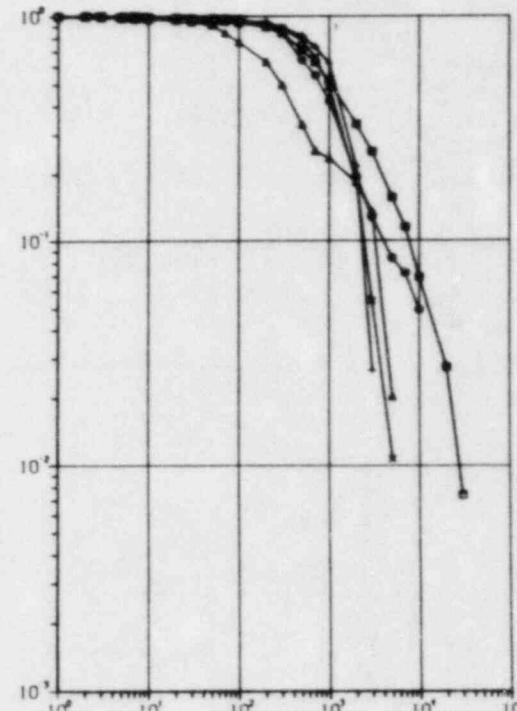
C-20



EARLY FATALITIES,  $X$ ,  
CONDITIONAL ON SST1\*



EARLY INJURIES,  $X$ ,  
CONDITIONAL ON SST1\*



LATENT CANCER FATALITIES,  $X$ ,  
CONDITIONAL ON SST1\*

#### KEY

- SALEM
- SAN ONOFRE
- ▲ SEABROOK
- SEQUOYAH
- × SHEARON HARRIS

Figure C-14: Early fatality, early injury, and latent cancer fatality CCDFs at named sites, conditional on an SST1 release.

Assumptions: 1120 MWe reactor, summary evacuation, representative meteorology (see Appendix A), and actual site population and windrose.

\*See footnote, page C-2.

Note: These CCDFs do not represent effects from existing reactor/site combinations, all assume an 1120 MWe reactor. In addition, these results are conditional on the occurrence of a hypothetical SST1 release. Recent evidence suggests that the source term magnitude assumed for SST1 may be overestimated by a factor of 10 or more (see section 2.3.2).

C-21

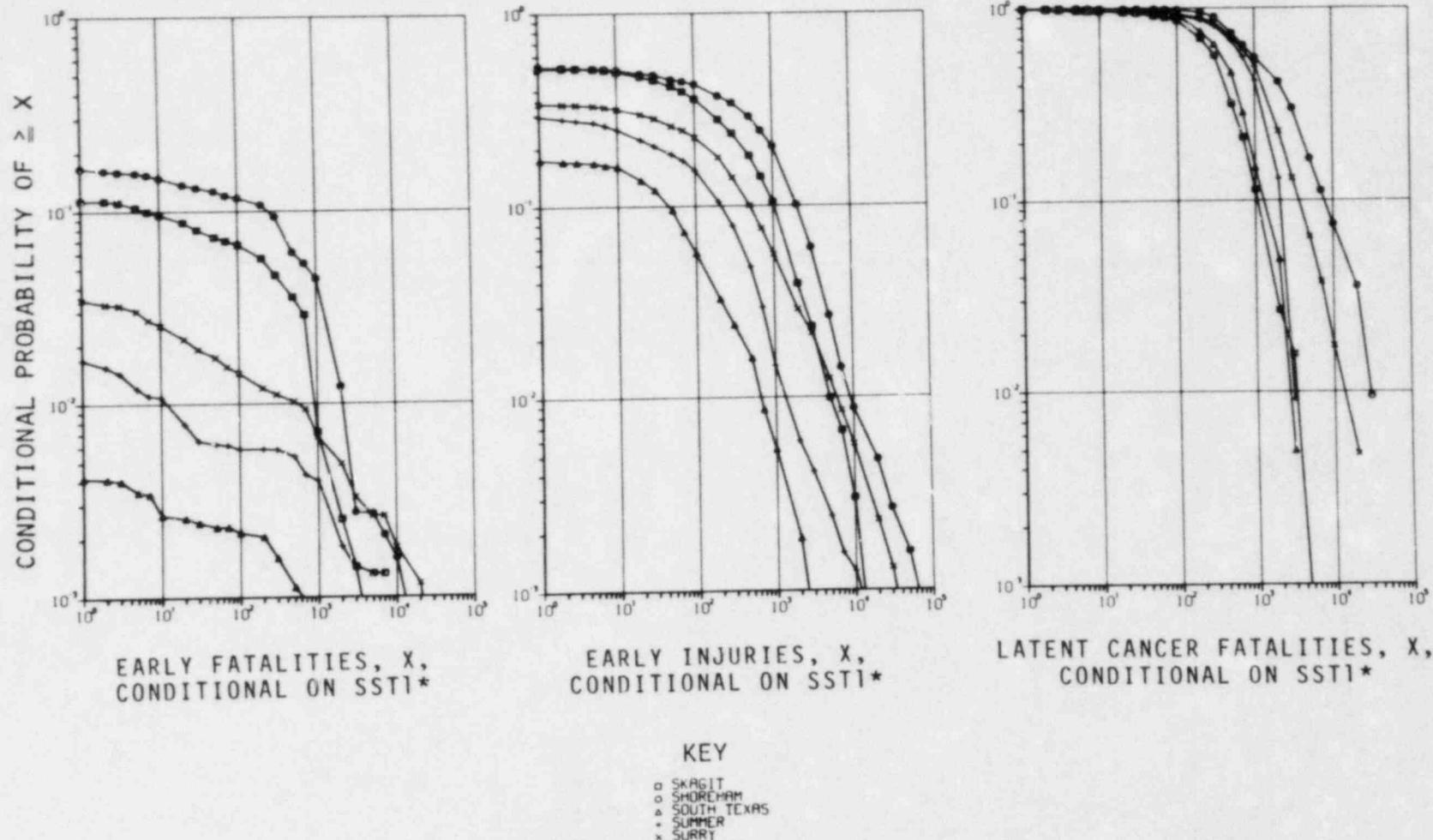


Figure C-15: Early fatality, early injury, and latent cancer fatality CCDFs at named sites, conditional on an SST1 release.

Assumptions: 1120 MWe reactor, summary evacuation, representative meteorology (see Appendix A), and actual site population and windrose.

\*See footnote, page C-2

Note: These CCDFs do not represent effects from existing reactor/site combinations, all assume an 1120 MWe reactor. In addition, these results are conditional on the occurrence of a hypothetical SST1 release. Recent evidence suggests that the source term magnitude assumed for SST1 may be overestimated by a factor of 10 or more (see section 2.3.2).

C-22

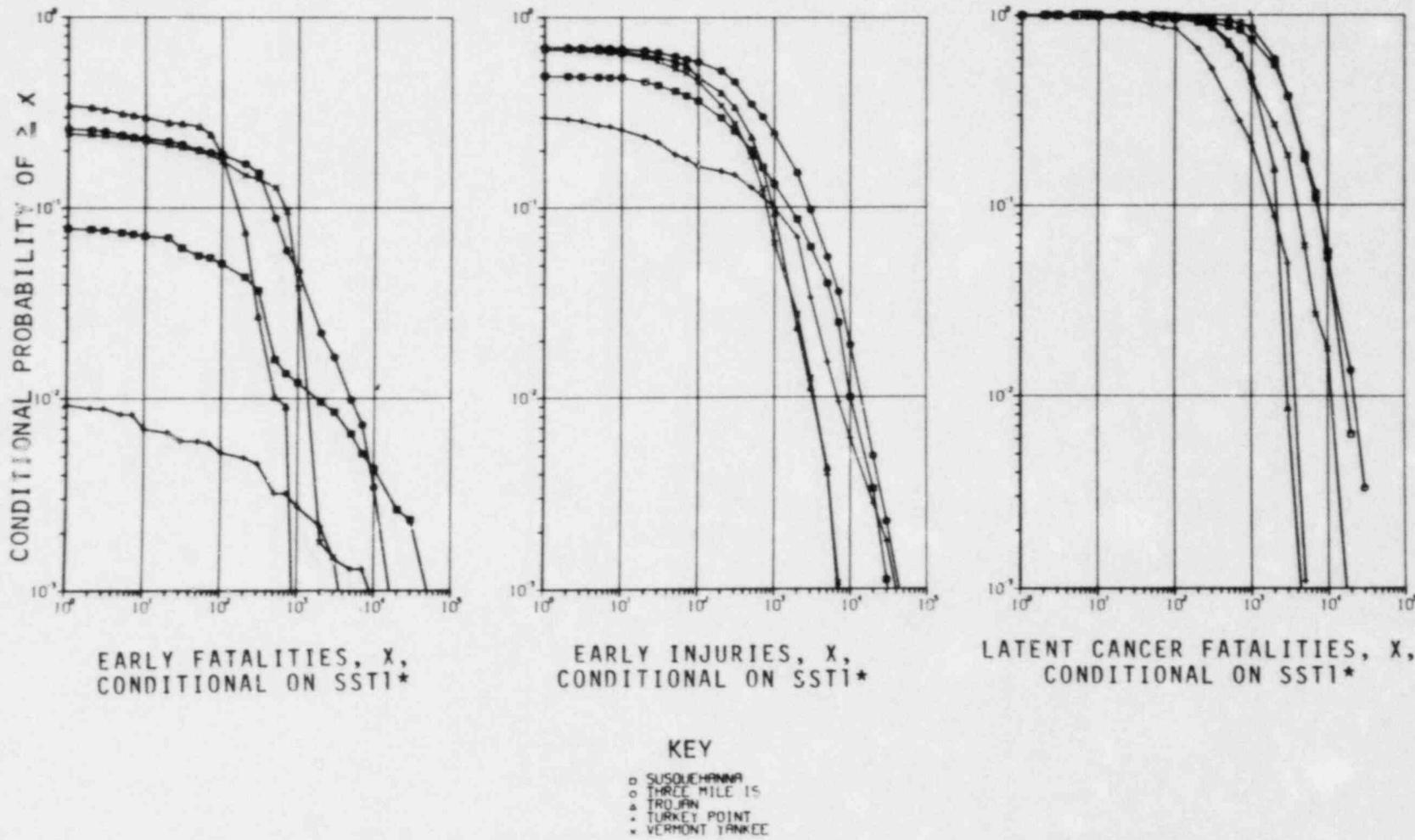


Figure C-16: Early fatality, early injury, and latent cancer fatality CCDFs at named sites, conditional on an SST1 release.

Assumptions: 1120 MWe reactor, summary evacuation, representative meteorology (see Appendix A), and actual site population and windrose.

\*See footnote, page C-2.

Note: These CCDFs do not represent effects from existing reactor/site combinations, all assume an 1120 MWe reactor. In addition, these results are conditional on the occurrence of a hypothetical SST1 release. Recent evidence suggests that the source term magnitude assumed for SST1 may be overestimated by a factor of 10 or more (see section 2.3.2).

C-23

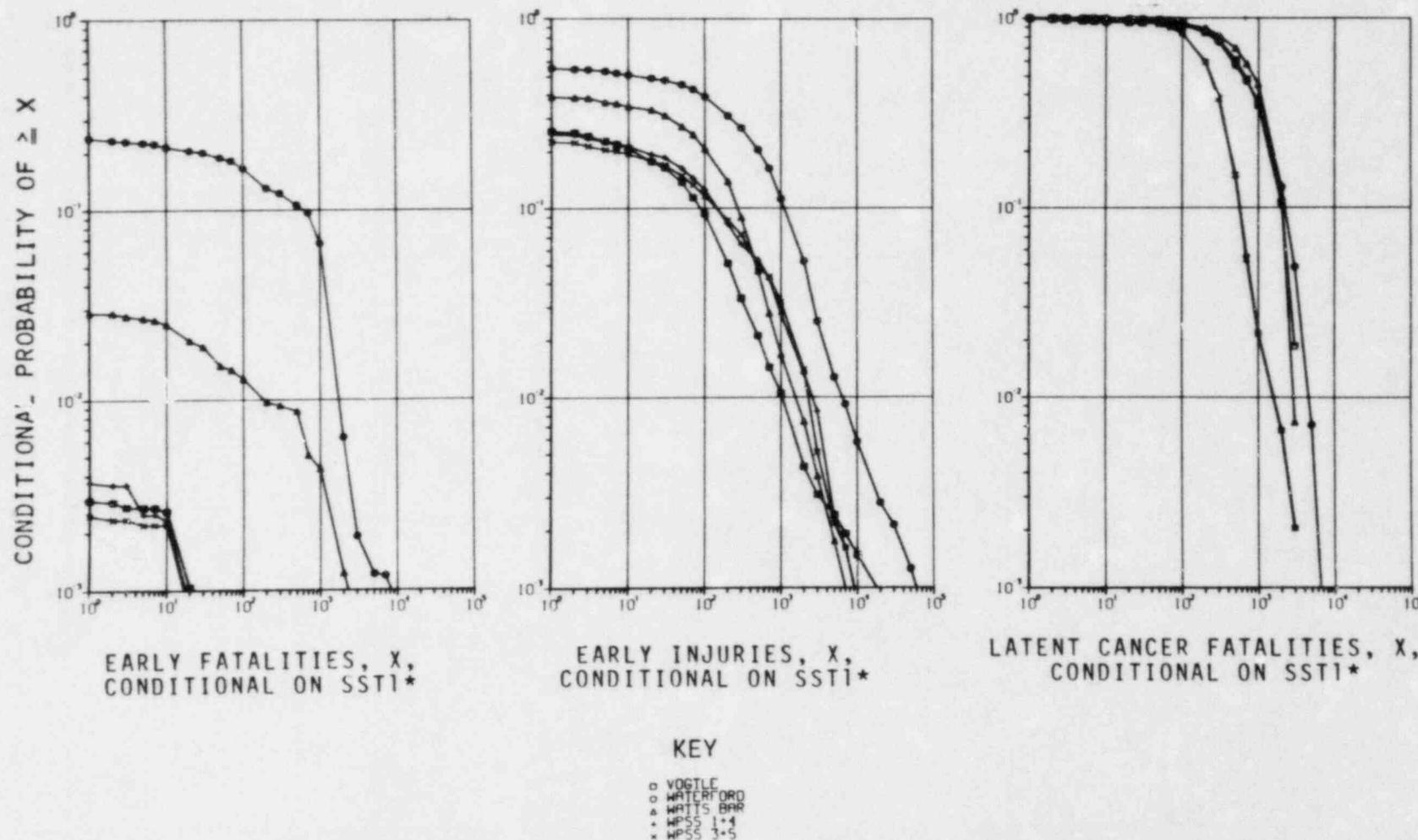


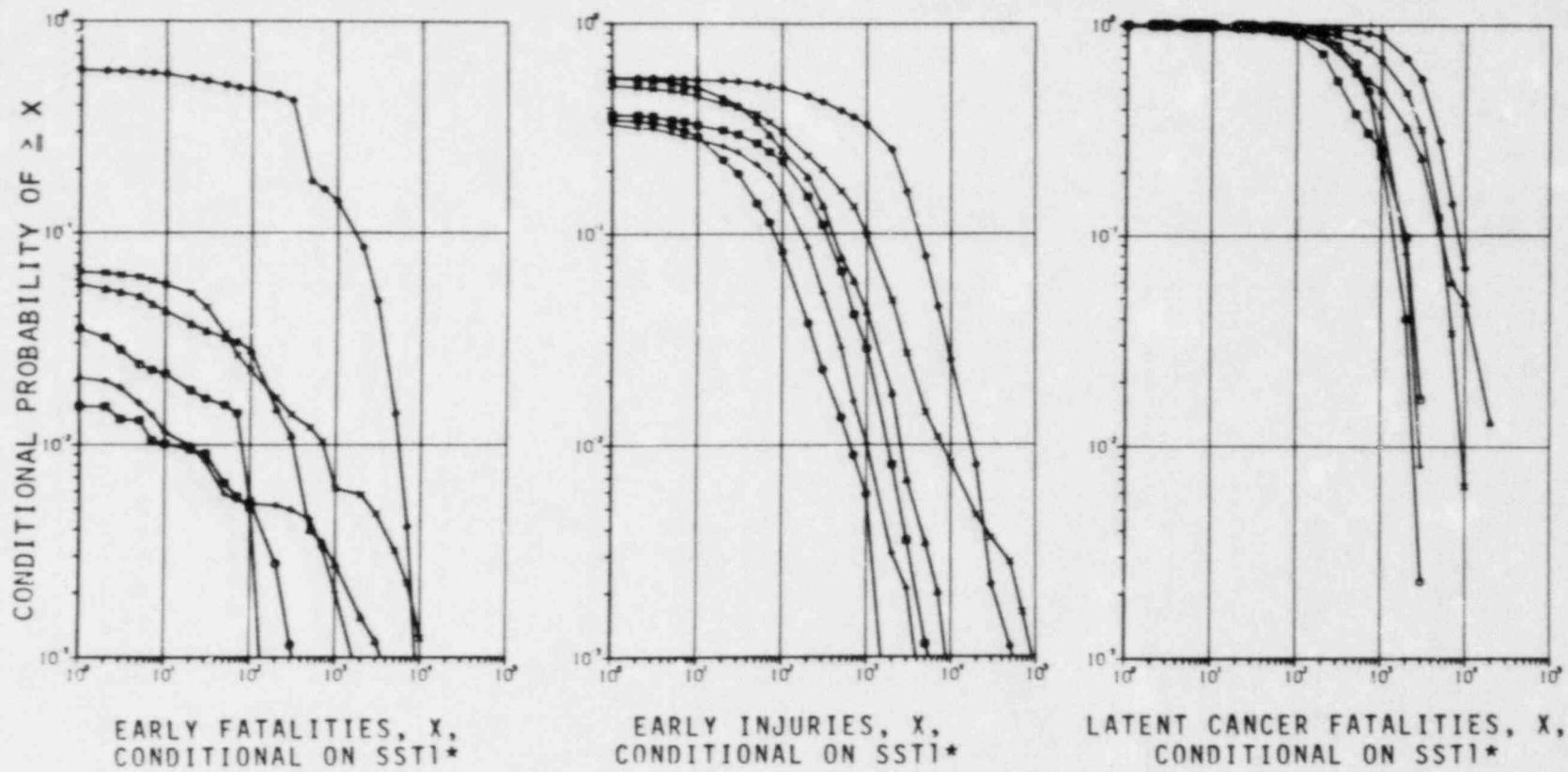
Figure C-17: Early fatality, early injury, and latent cancer fatality CCDFs at named sites, conditional on an SST1 release.

Assumptions: 1120 MWe reactor, summary evacuation, representative meteorology (see Appendix A), and actual site population and windrose.

\*See footnote, page C-2

Note: These CCDFs do not represent effects from existing reactor/site combinations, all assume an 1120 MWe reactor. In addition, these results are conditional on the occurrence of a hypothetical SST1 release. Recent evidence suggests that the source term magnitude assumed for SST1 may be overestimated by a factor of 10 or more (see section 2.3.2).

C-24



#### KEY

- WPPSS 2
- HOLLOW CREEK
- ▲ YANKEE RIDGE
- YELLOW CREEK
- × ZIMMER
- ZION

Figure C-18: Early fatality, early injury, and latent cancer fatality CCDFs at named sites, conditional on an SST1 release.

Assumptions: 1120 MWe reactor, summary evacuation, representative meteorology (see Appendix A), and actual site population and windrose.

\*See footnote, page C-2

## Appendix D: Additional Population Statistics for Current Reactor Sites

The demographic characteristics of the 91 reactor sites described in Chapter 2 and Appendix A were analyzed for this study. These data, which were summarized in Chapter 3, provide a perspective of previous siting decisions and delineate the population characteristics of current reactor sites. This appendix contains additional demographic data which complement the data presented in Chapter 3. These data are presented in the following sections.

<u>Section</u>	<u>Data Description</u>
D.1	Site Population Statistics
D.2	Exclusion Distances
D.3	Site Population Factors

### D.1 Site Population Statistics

The 91 population distributions examined in this report were all constructed on a 16 sector, circular polar grid. For any specified portion (a circle, an annulus, a sector) of that grid, 91 values of population density are available, one for each of the 91 population distributions. By cumulation of the 91 values for a given portion of the grid, a population density CCDF may be constructed.\* Six different sets of population density CCDFs have been constructed for the following areas of the population distribution grid:

Set 1 (Figures D.1-1 thru D.1-8): Eight annuli (0-2, 2-5, 5-10, 10-20, 20-30, 30-50, 50-100, and 100-200 mi).

Set 2 (Figures D.1-9 thru D.1-16): eight radial distances (0-2, 0-5, 0-10, 0-20, 0-30, 0-50, 0-100, and 0-200 mi).

\*Population density CCDFs are Log-Log plots of the fraction of sites vs population density. Any point on the distribution gives the fraction of sites (y-axis value), which have a population density within the specified portion of the grid (annulus, circle, sector), that is greater than or equal to the specified population density (x-axis value).

Set 3 (Figures D.1-17 thru D.1-22): the most populated 22.5° sector in each of six annuli (0-2, 2-5, 5-10, 10-20, 20-30, and 30-50 mi) on the 16 sector grid.

Set 4 (Figures D.1-23 thru D.1-28): the most populated 22.5° sector in each of six radial distances (0-2, 0-5, 0-10, 0-20, 0-30 and 0-50 mi) on the 16 sector grid.

Set 5 (Figures D.1-29 thru D.1-34): the most populated 45° sector (two adjacent 22.5° sectors) in each of six annuli (0-2, 2-5, 5-10, 10-20, 20-30, and 30-50 mi) on the 16 sector grid.

Set 6 (Figures D.1-35 thru D.1-40): the most populated 45° sector (two adjacent 22.5° sectors) in each of six radial distances (0-2, 0-5, 0-10, 0-20, 0-30, and 0-50 mi) on the 16 sector grid.

Each figure contains six CCDFs, one for each of the five NRC administrative regions (NE, MW, S, W, SW, see Figure 3-1) and one for all regions combined (All).

Tables D.1-1 thru D.1-4 present the data used to construct the CCDFs in Figures D.1-1 thru D.1-28. Table D.1 presents, for each of the 91 sites, population densities within eight annuli; Table D.2 presents similar data for eight radial distances; Table D.3 for the most populated 22.5° sector of six annuli; and Table D.4 for the most populated 22.5° sector of six radial distances.

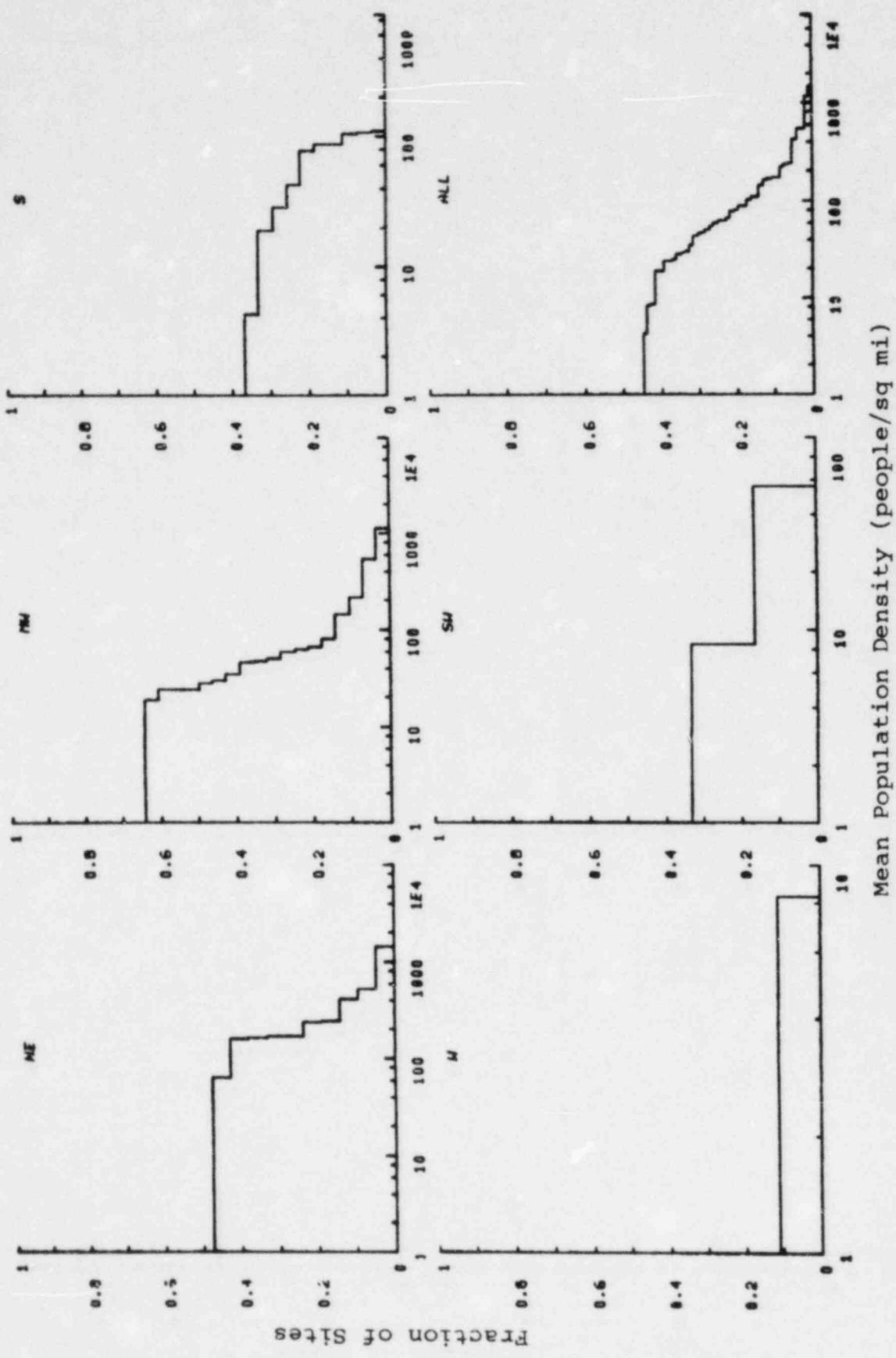


Figure D.1-1. CCDFs of Population Density (people/sq mi) at 91 Reactor Sites for the Five NRC Administrative Regions (NE, MW, S, W, SW) and for All Regions Combined (All): Population Density Within the Annulus Interval 0-2 Miles.

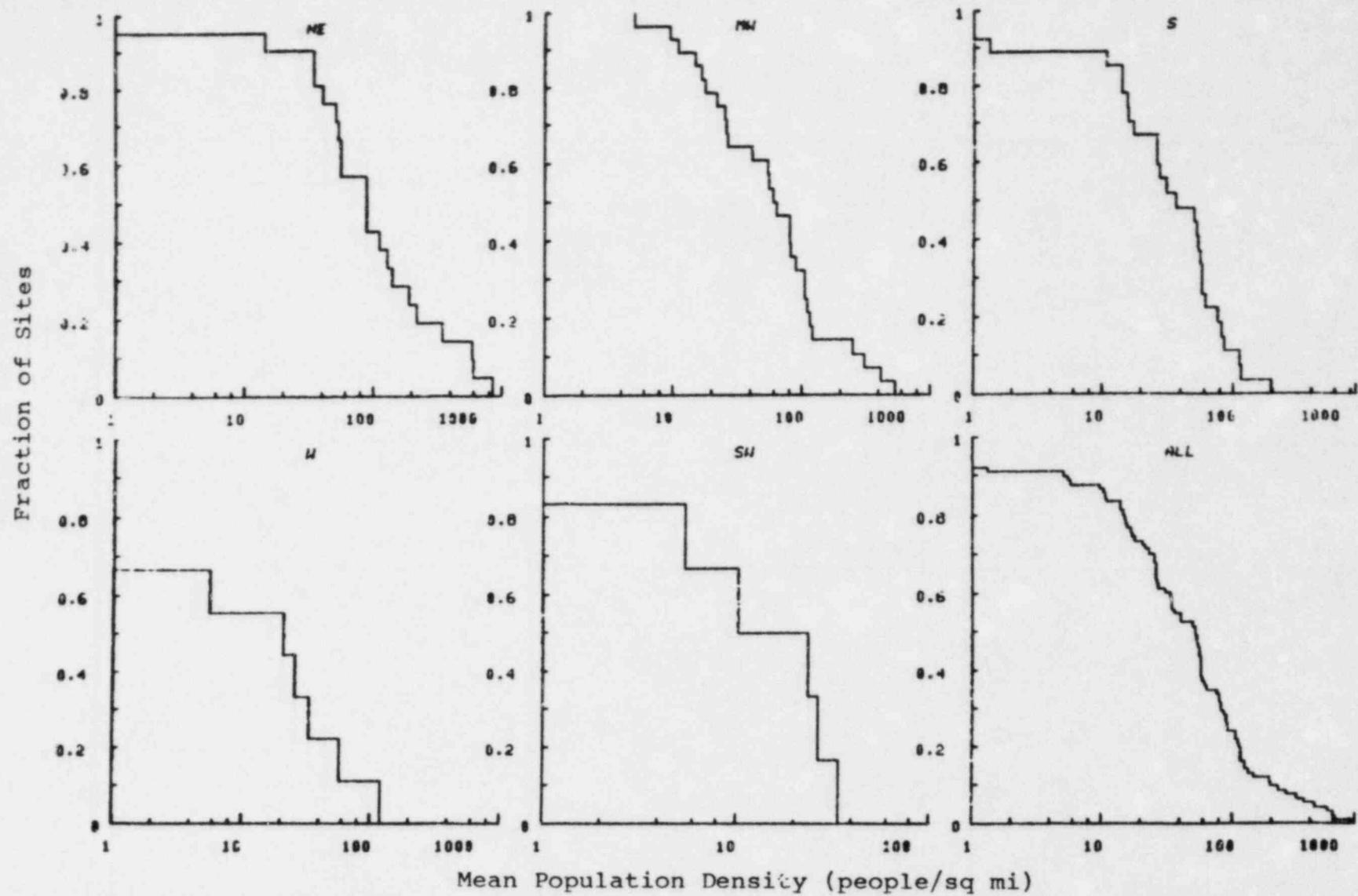


Figure D.1-2. CCDFs of Population Density (people/sq mi) at 91 Reactor Sites for the Five NRC Administrative Regions (NE, MW, S, W, SW) and for All Regions Combined (All): Population Density Within the Annulus Interval 2-5 Miles.

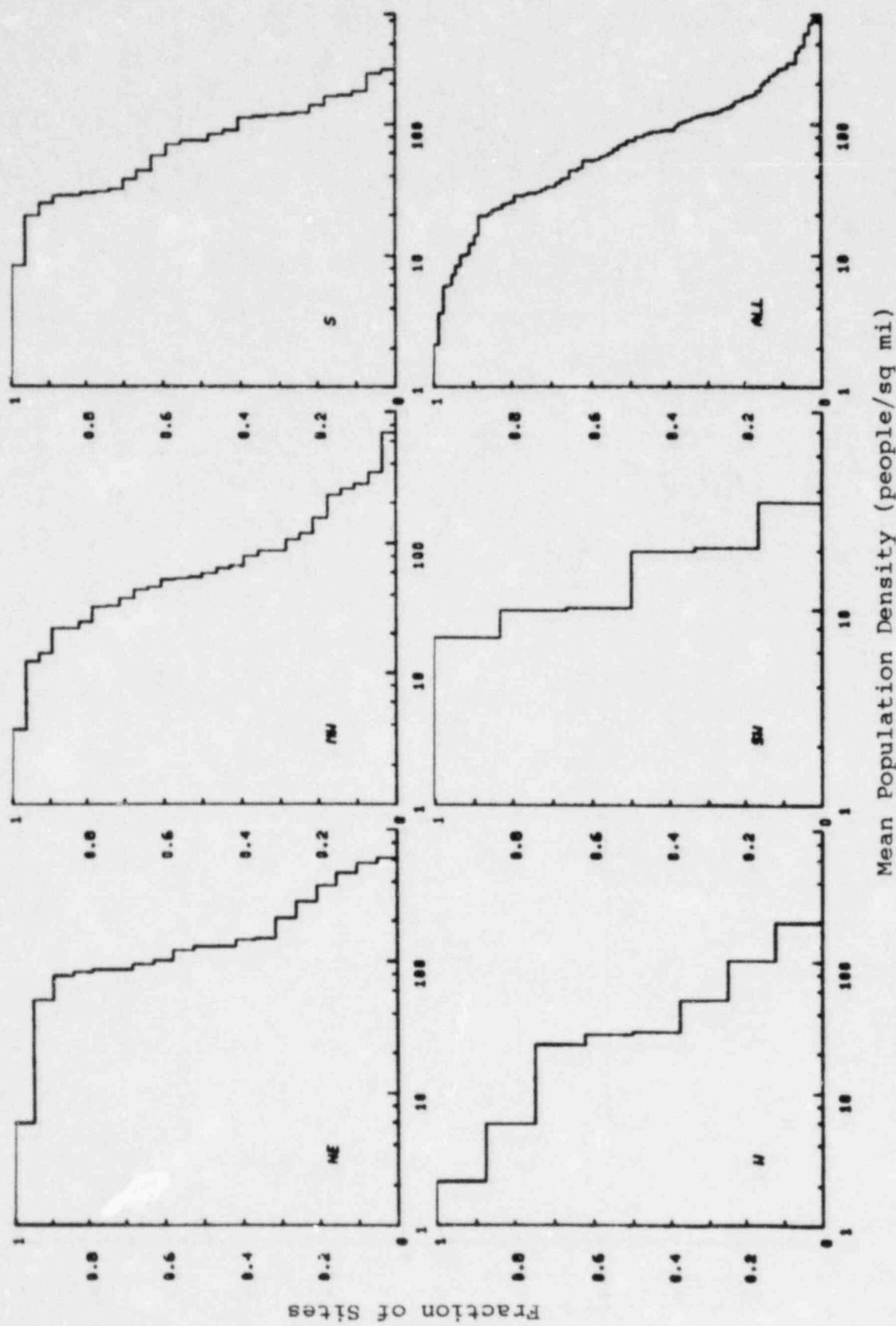


Figure D.1-3. CCDFs of Population Density (people/sq mi) at 91 Reactor Sites for the Five NRC Administrative Regions (NE, MW, S, W, SW) and for All Regions Combined (All): Population Density Within the Annulus Interval 5-10 Miles.

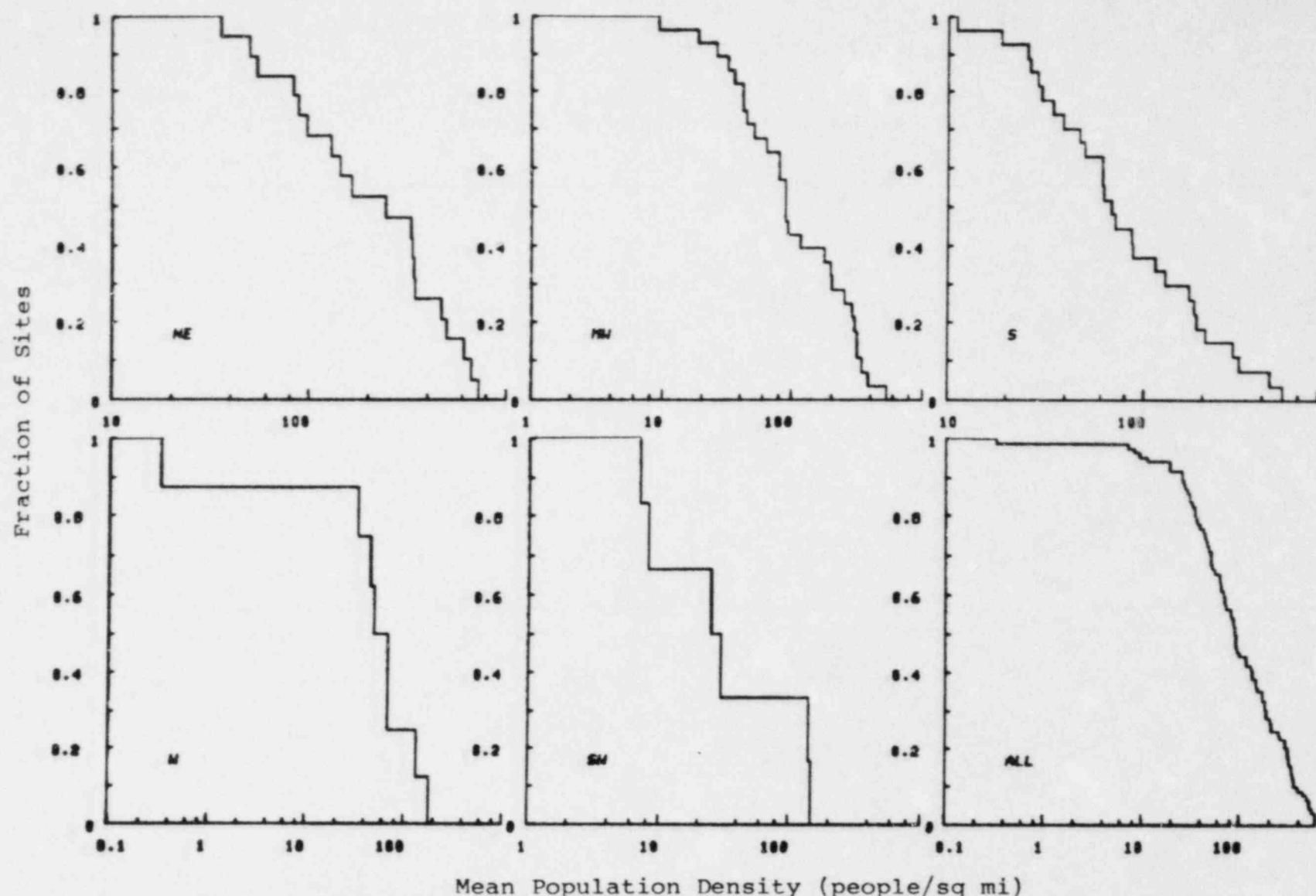


Figure D.1-4. CCDFs of Population Density (people/sq mi) at 91 Reactor Sites for the Five NRC Administrative Regions (NE, MW, S, W, SW) and for All Regions Combined (All): Population Density Within the Annulus Interval 10-20 Miles.

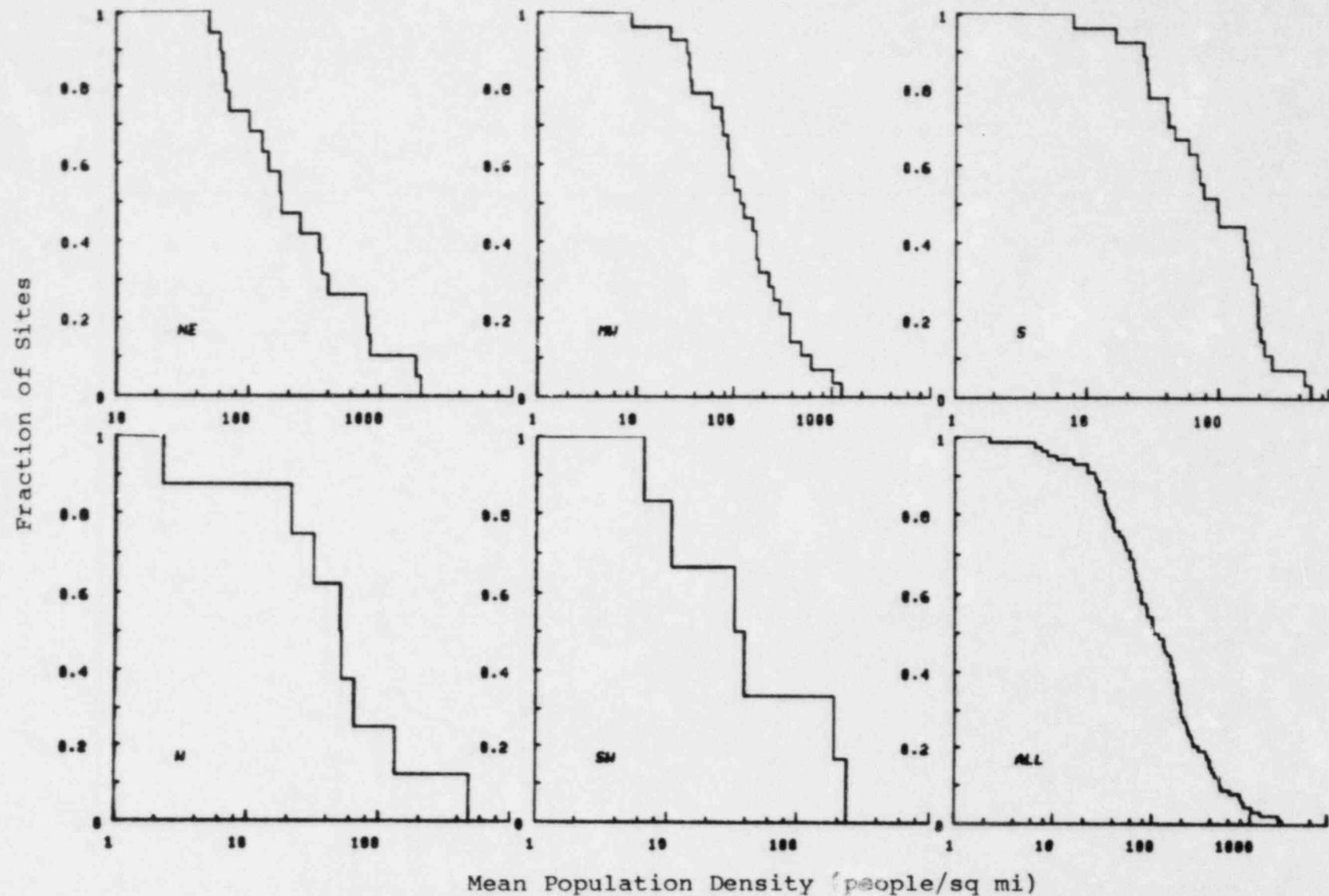


Figure D.1-5. CCDFs of Population Density (people/sq mi) at 91 Reactor Sites for the Five NRC Administrative Regions (NE, MW, S, W, SW) and for All Regions Combined (All): Population Density Within the Annulus Interval 20-30 Miles.

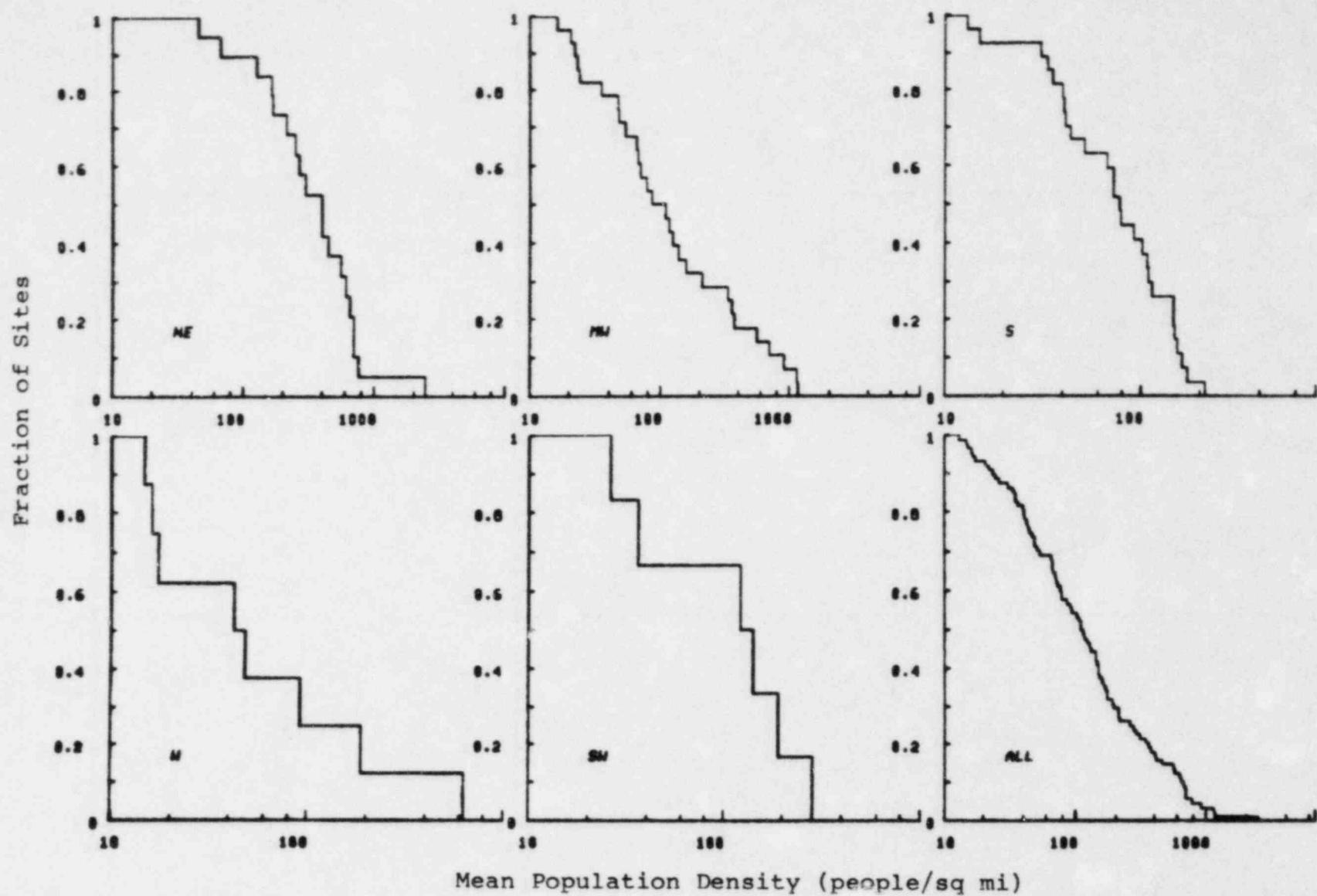


Figure D.1-6. CCDFs of Population Density (people/sq mi) at 91 Reactor Sites for the Five NRC Administrative Regions (NE, MW, S, W, SW) and for All Regions Combined (All): Population Density Within the Annulus Interval 30-50 Miles.

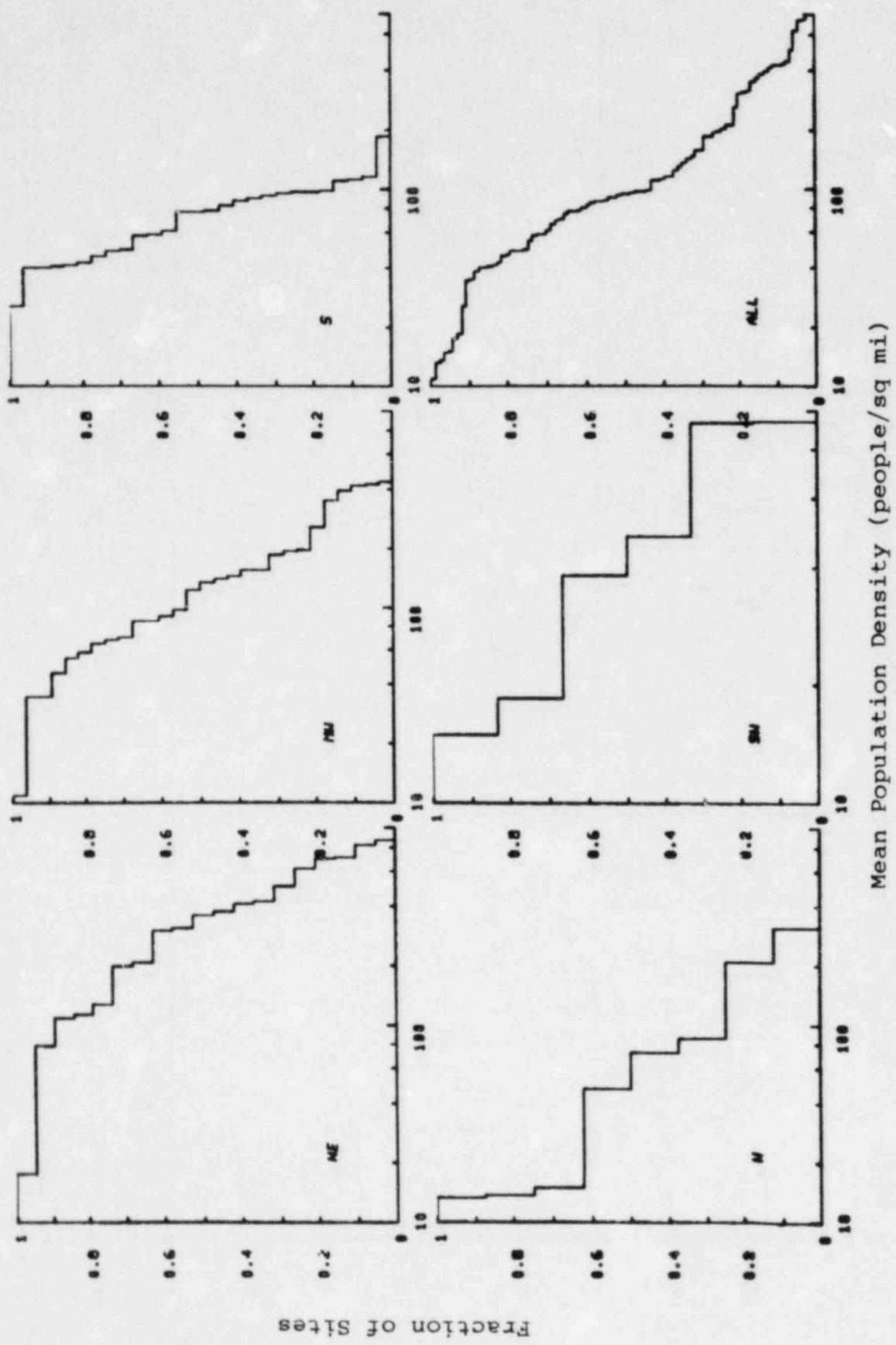


Figure D.1-7. CCDFs of Population Density (people/sq mi) at 91 Reactor Sites for the Five NRC Administrative Regions (NE, MW, S, W, SW) and for All Regions Combined (All): Population Density Within the Annulus Interval 50-100 Miles.

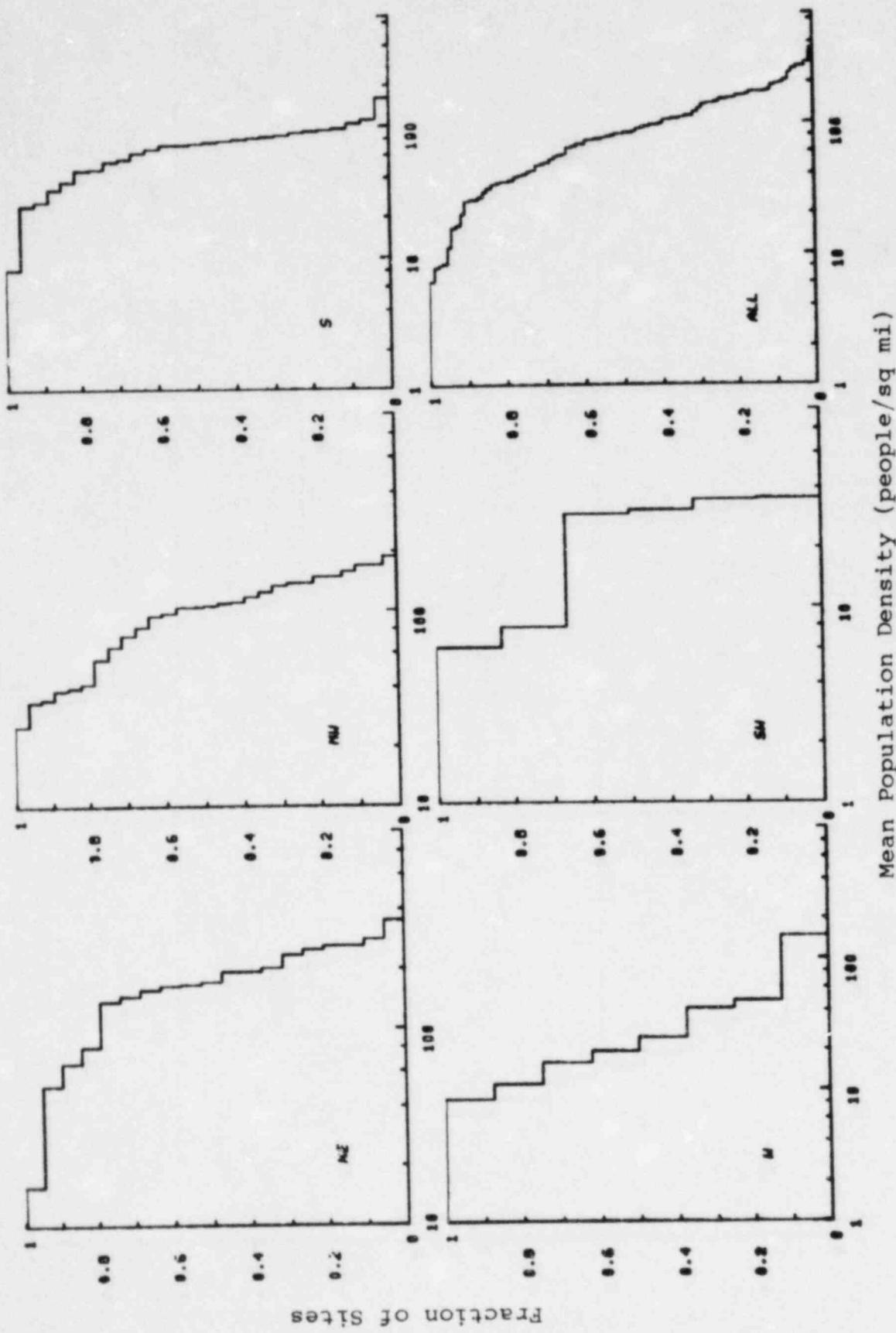


Figure D.1-8. CCDFs of Population Density (people/sq mi) at 91 Reactor Sites for the Five NRC Administrative Regions (NE, MW, S, W, SW) and for All Regions Combined (All): Population Density Within the Annulus Interval 100–200 Miles.

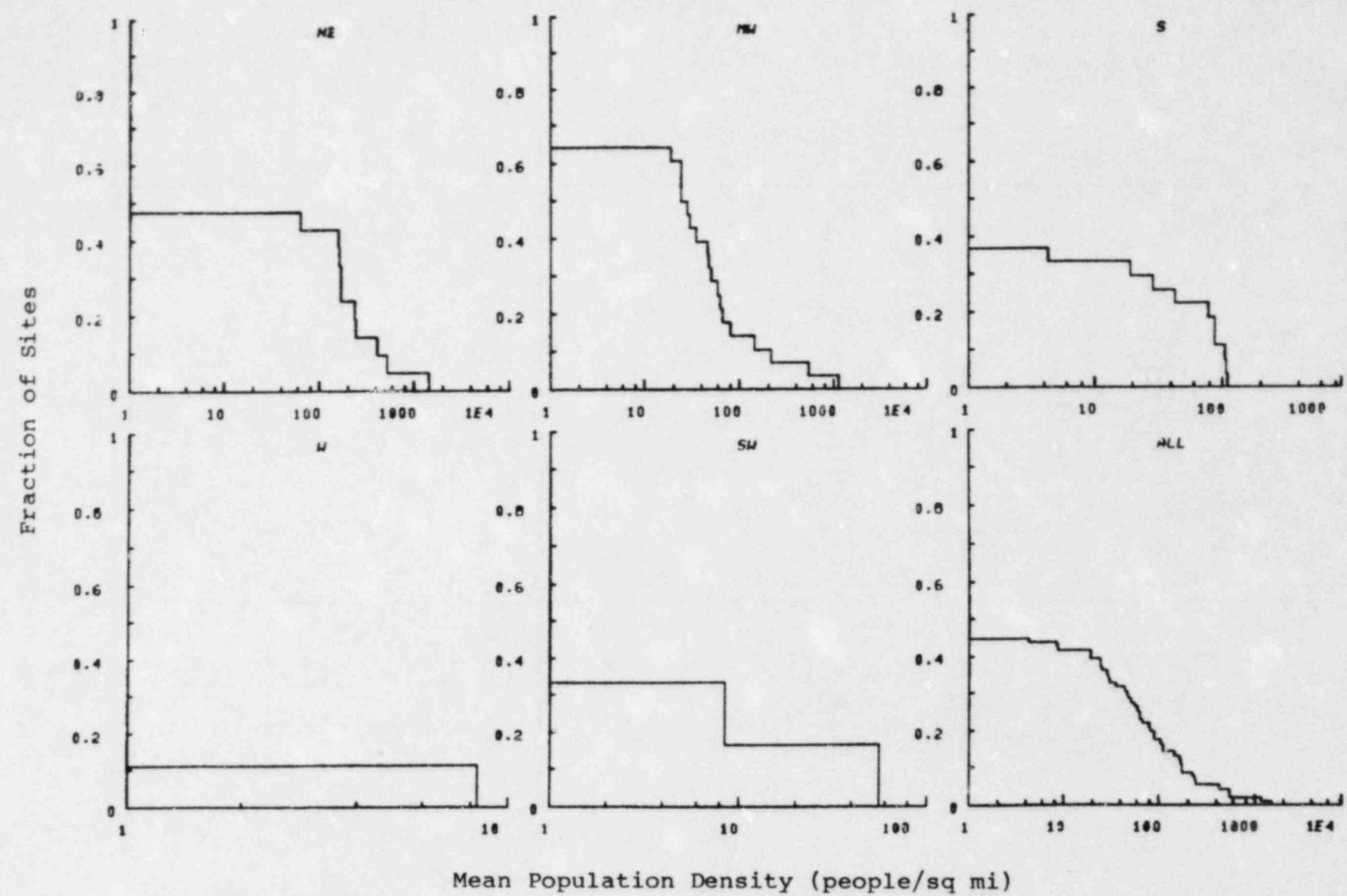


Figure D.1-9. CCDFs of Population Density (people/sq mi) at 91 Reactor Sites for the Five NRC Administrative Regions (NE, MW, S, W, SW) and for All Regions Combined (All): Population Density Within the Radial Distance 0-2 Miles.

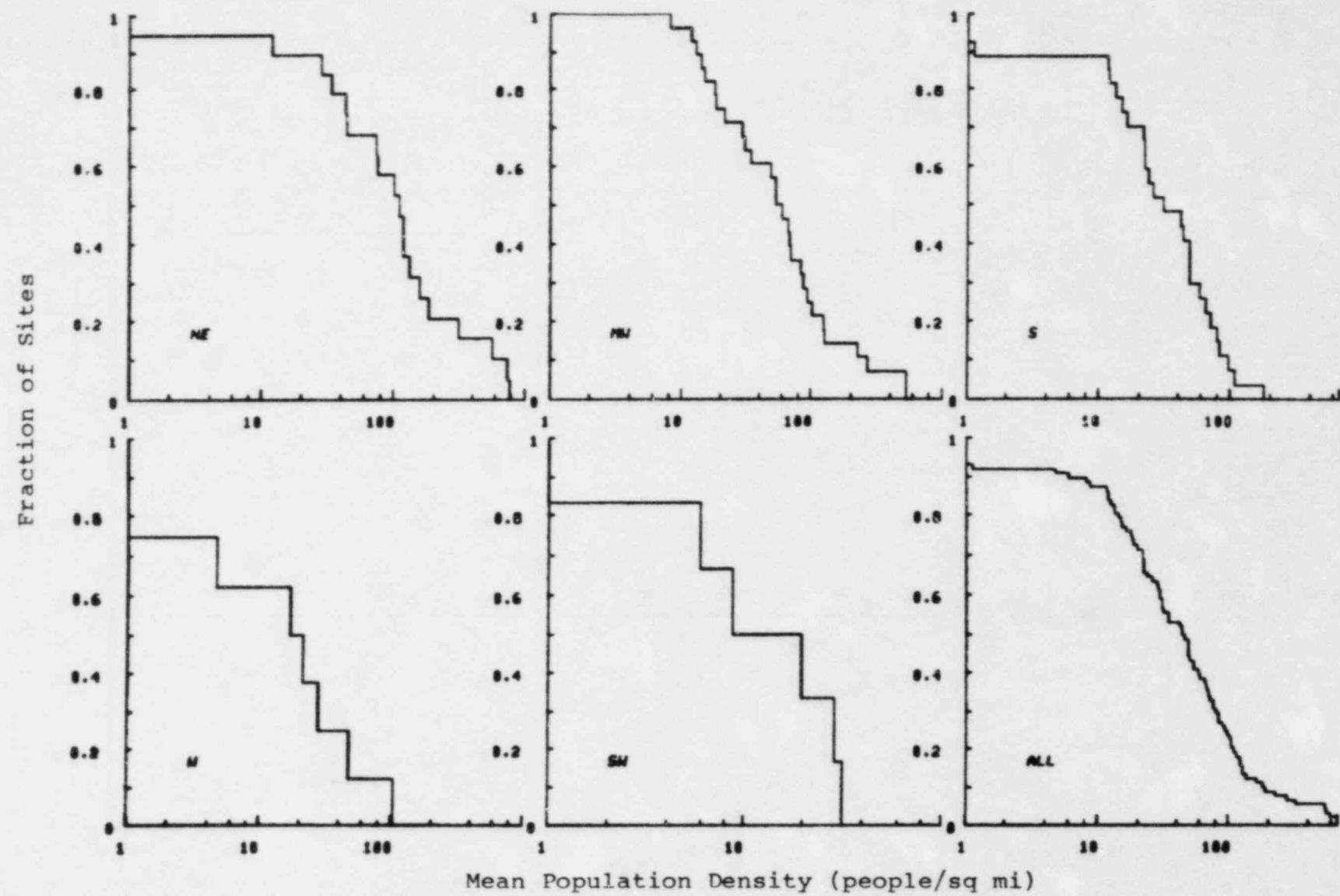


Figure D.1-10. CCDFs of Population Density (people/sq mi) at 91 Reactor Sites for the Five NRC Administrative Regions (NE, MW, S, W, SW) and for All Regions Combined (All): Population Density Within the Radial Distance 0-5 Miles.

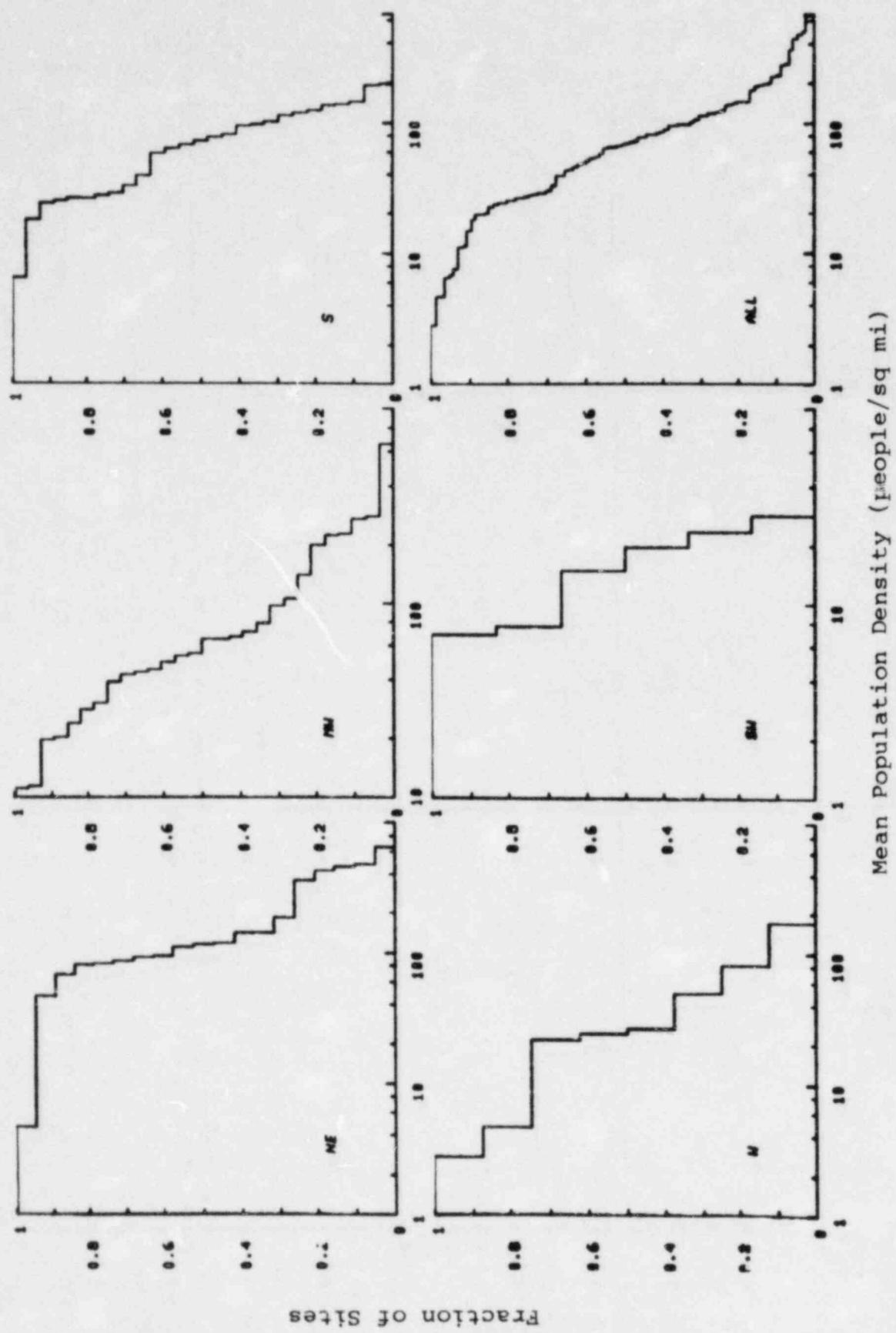


Figure D.1-11. CCDFs of Population Density (people/sq mi) at 91 Reactor Sites for the Five NRC Administrative Regions (NE, MW, S, W, SW) and for All Regions Combined (All): Population Density Within the Radial Distance 0-10 Miles.

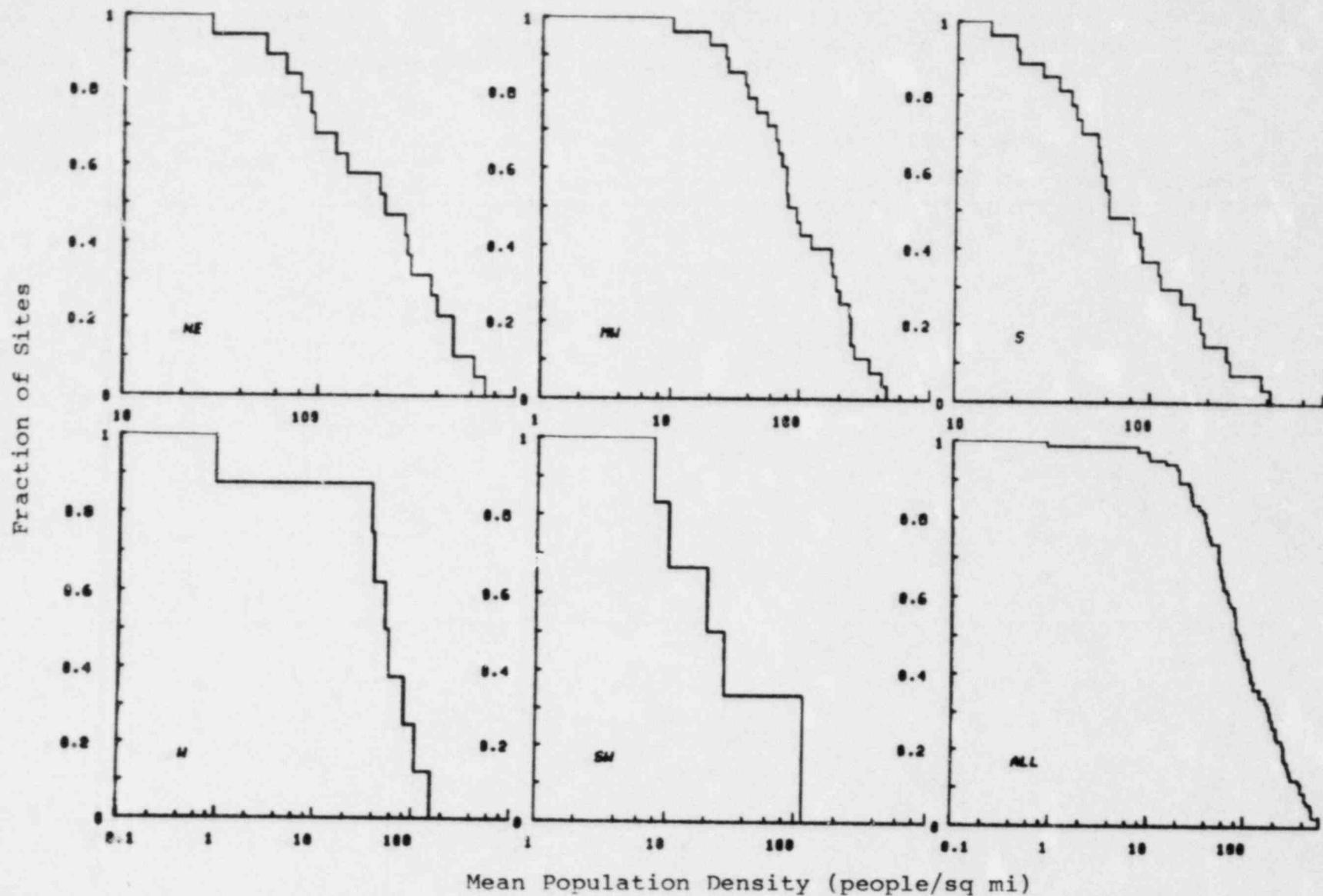


Figure D.1-12. CCDFs of Population Density (people/sq mi) at 91 Reactor Sites for the Five NRC Administrative Regions (NE, MW, S, W, SW) and for All Regions Combined (All): Population Density Within the Radial Distance 0-20 Miles.

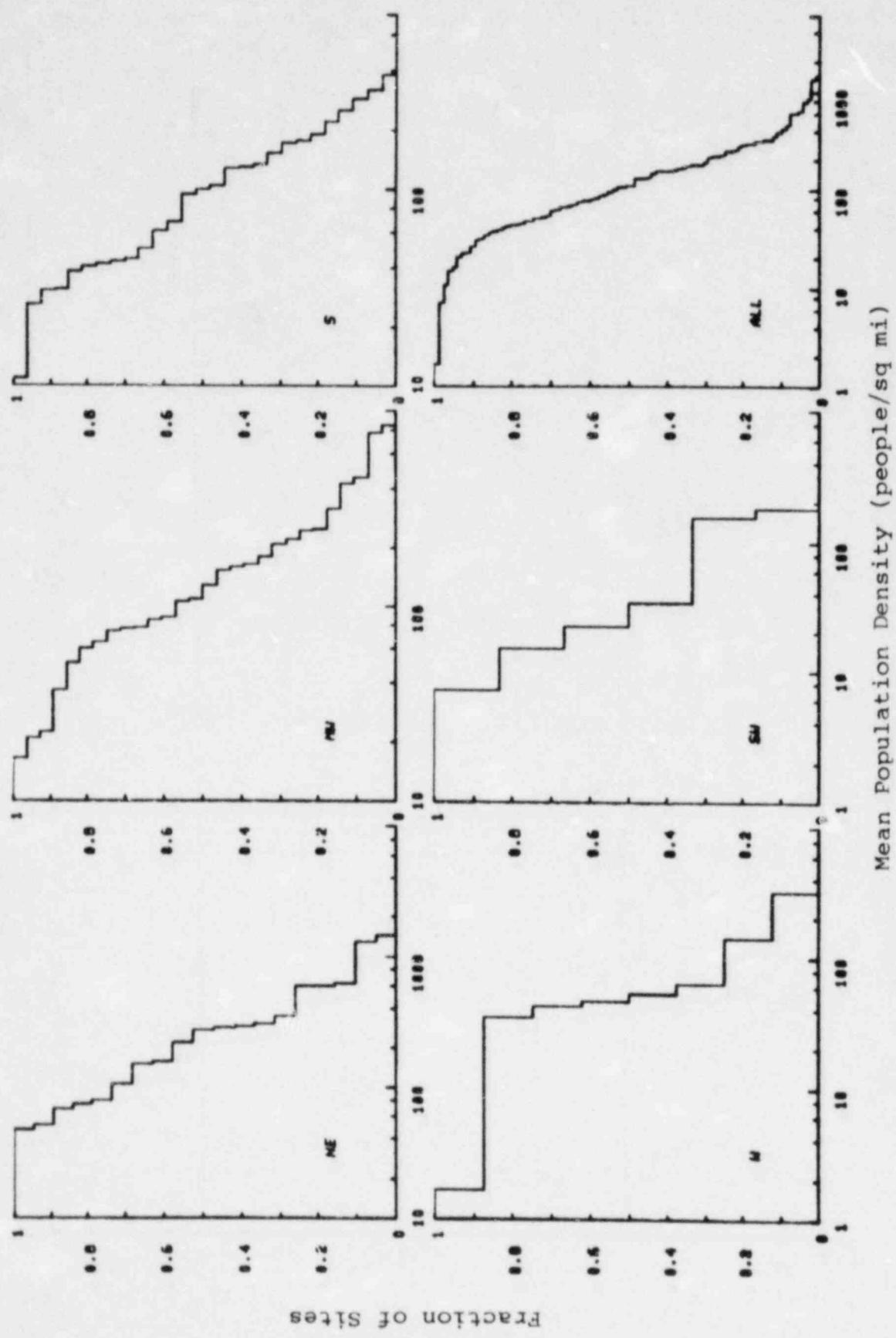


Figure D.1-13. CCDFs of population density (people/sq mi) at 91 reactor sites for the five NRC Administrative Regions (NE, MW, S, W, SW) and for All Regions Combined (All): Population Density Within the Radial Distance 0-30 Miles.

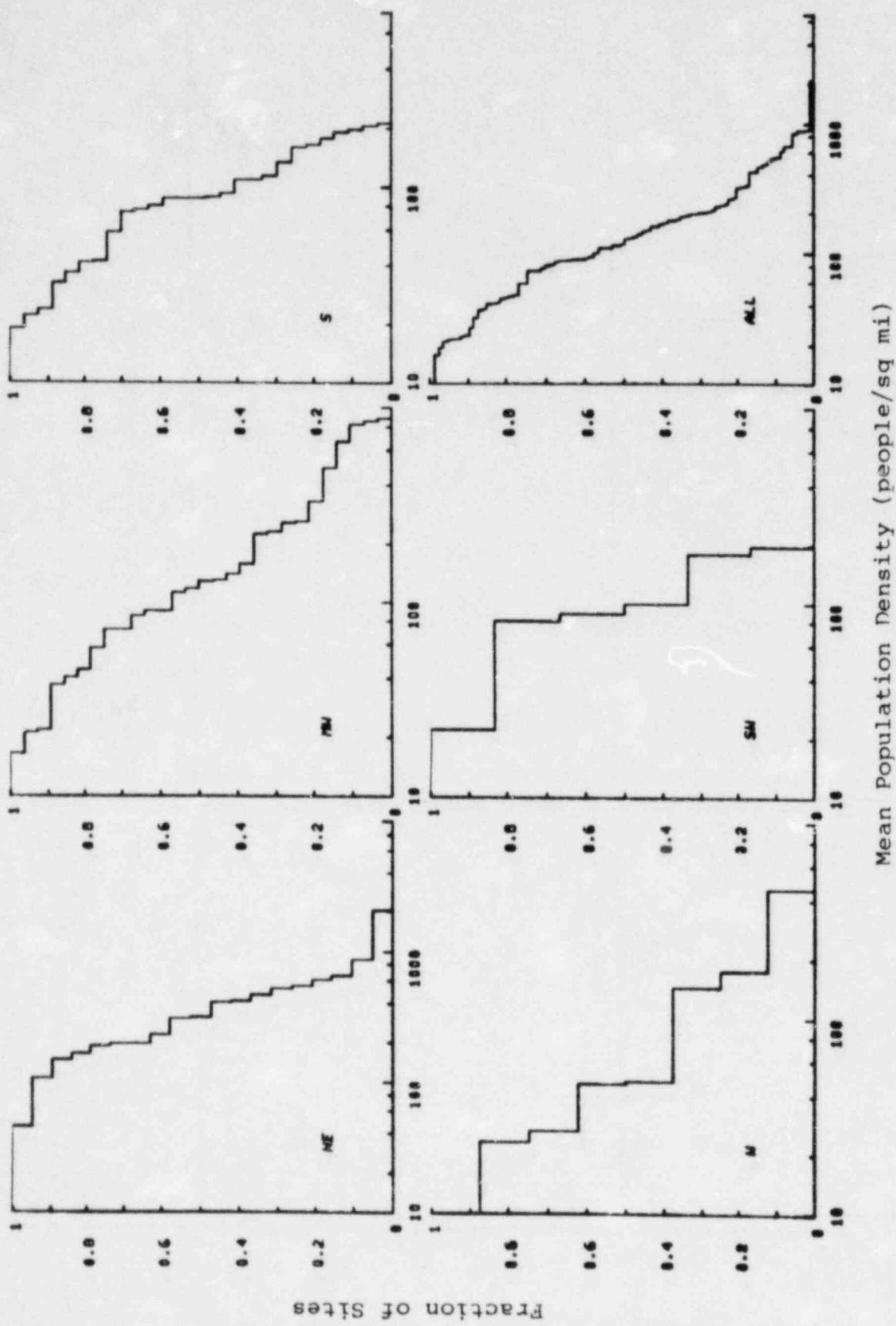


Figure D.1-14. CCDFs of Population Density (people/sq mi) at 91 Reactor Sites for the Five NRC Administrative Regions (NE, MW, S, W, SW) and for All Regions Combined (All): Population Density Within the Radial Distance 0-50 Miles.

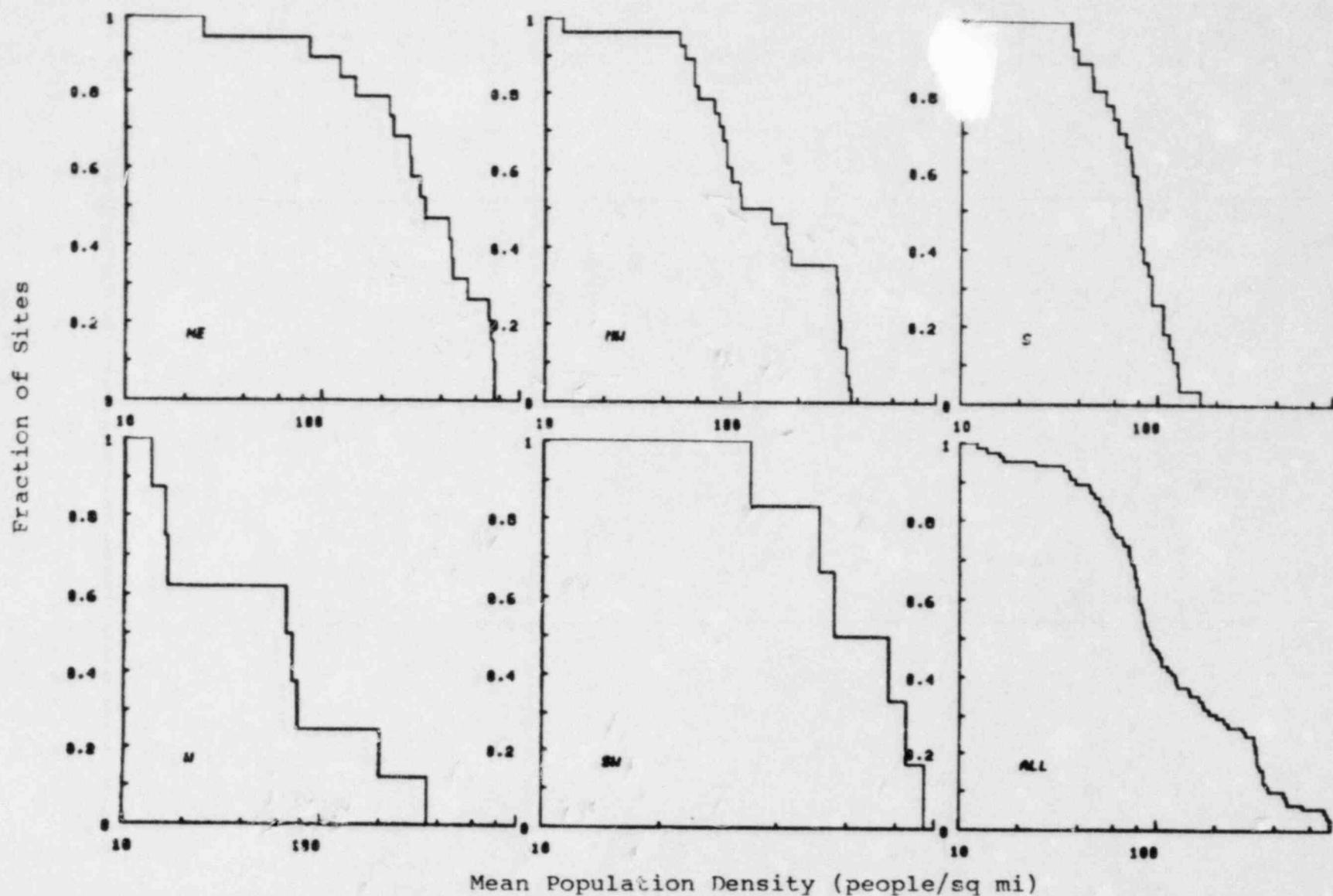


Figure D.1-15. CCDFs of Population Density (people/sq mi) at 91 Reactor Sites for the Five NRC Administrative Regions (NE, MW, S, W, SW) and for All Regions Combined (All): Population Density Within the Radial Distance 0-100 Miles.

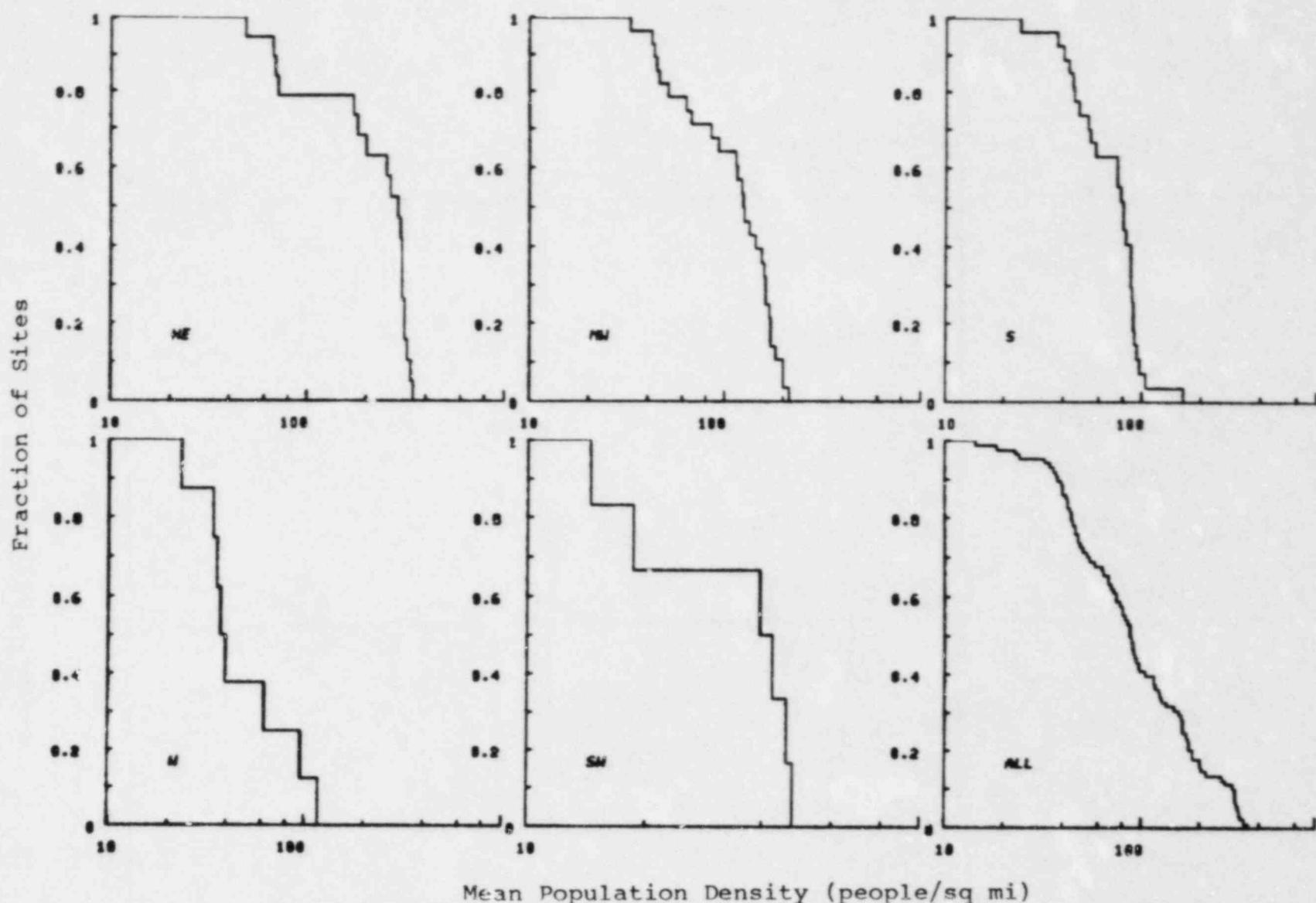


Figure D.1-16. CCDFs of Population Density (people/sq mi) at 91 Reactor Sites for the Five NRC Administrative Regions (NE, MW, S, W, SW) and for All Regions Combined (All): Population Density Within the Radial Distance 0-200 Miles.

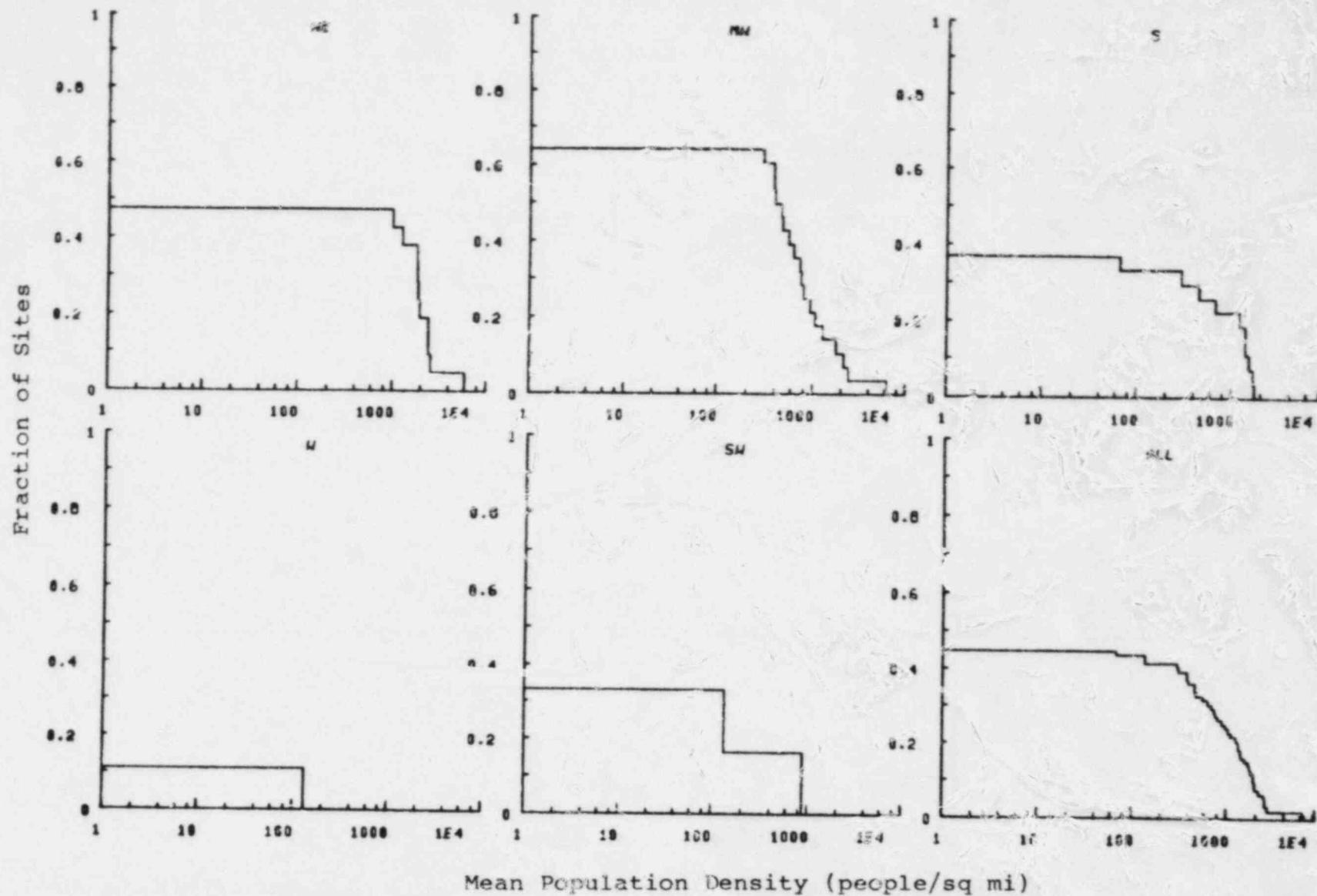


Figure D.1-17. CCDFs of Population Density (people/sq mi) at 91 Reactor Sites for the Five NRC Administrative Regions (NE, MW, S, W, SW) and for All Regions Combined (All): Population Density Within the Most Populated 22.5° Sector of the Annular Interval 0-2 Miles.

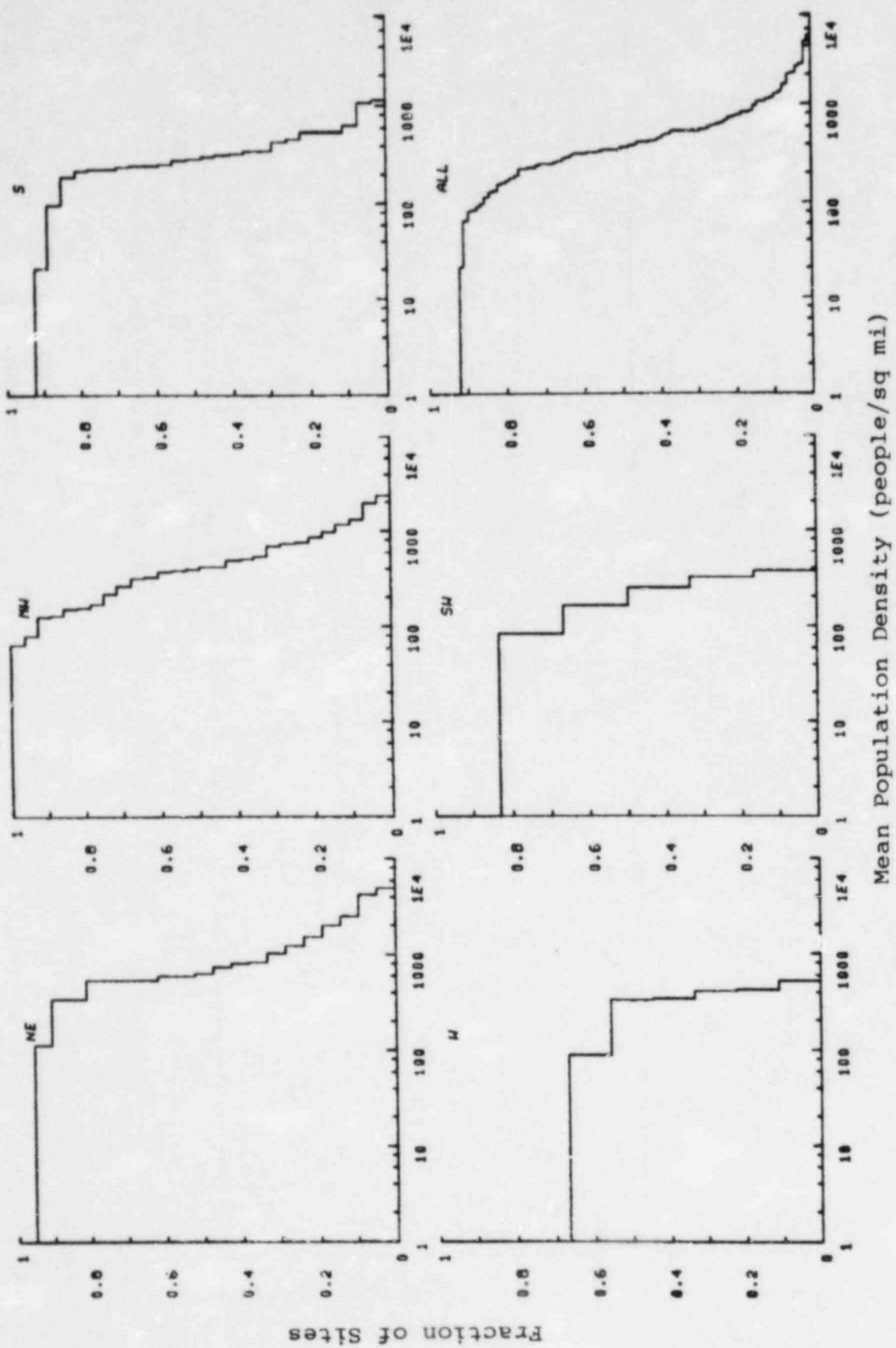


Figure D.1-18. CCDFs of Population Density (people/sq mi) at 91 Reactor Sites for the Five NRC Administrative Regions (NE, MW, S, W, SW) and for All Regions Combined (All). Population Density Within the Most Populated 22.5° Sector of the Annular Interval 2-5 Miles.

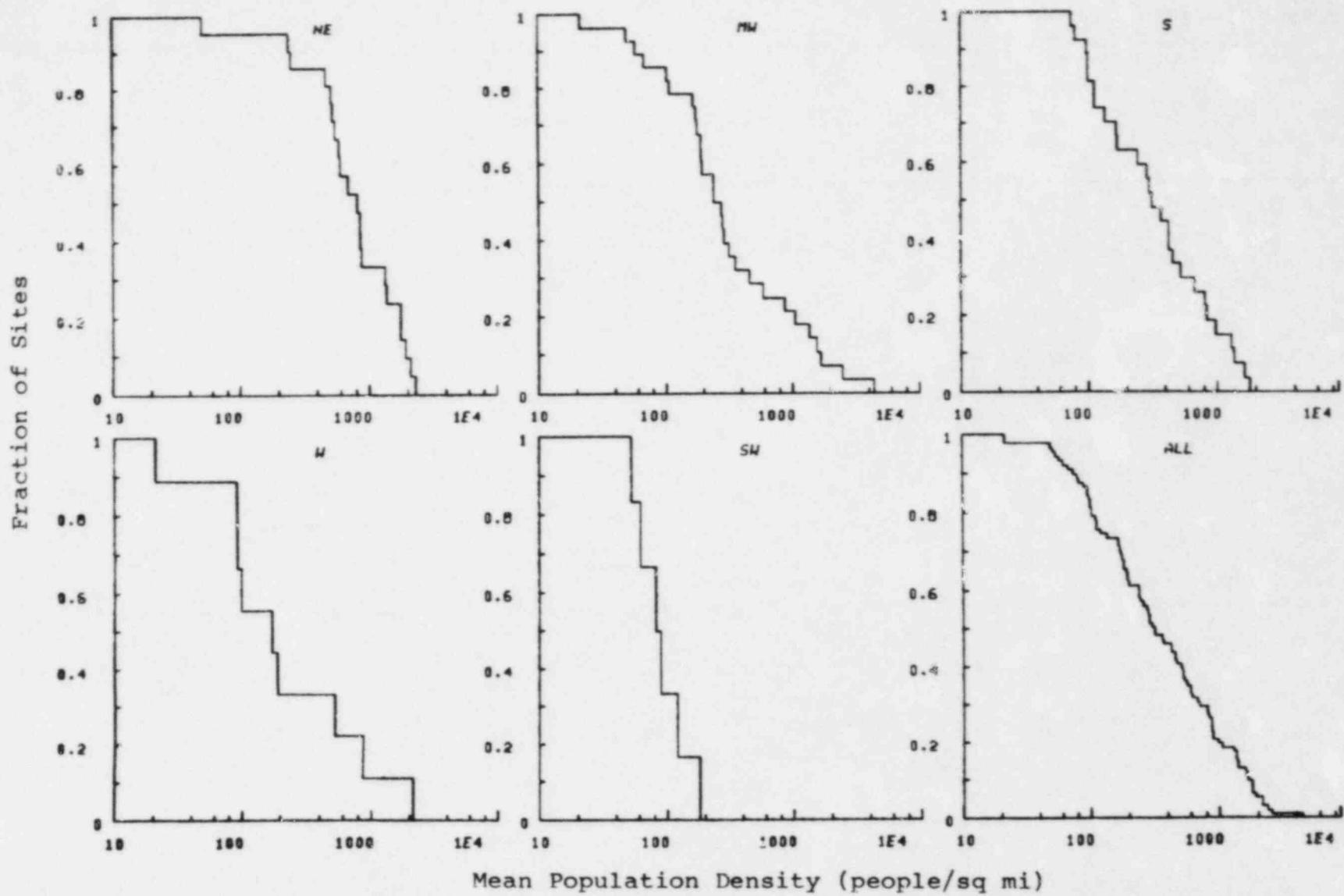


Figure D.1-19. CCDFs of Population Density (people/sq mi) at 91 Reactor Sites for the Five NRC Administrative Regions (NE, MW, S, W, SW) and for All Regions Combined (All): Population Density Within the Most Populated 22.5° Sector of the Annular Interval 5-10 Miles.

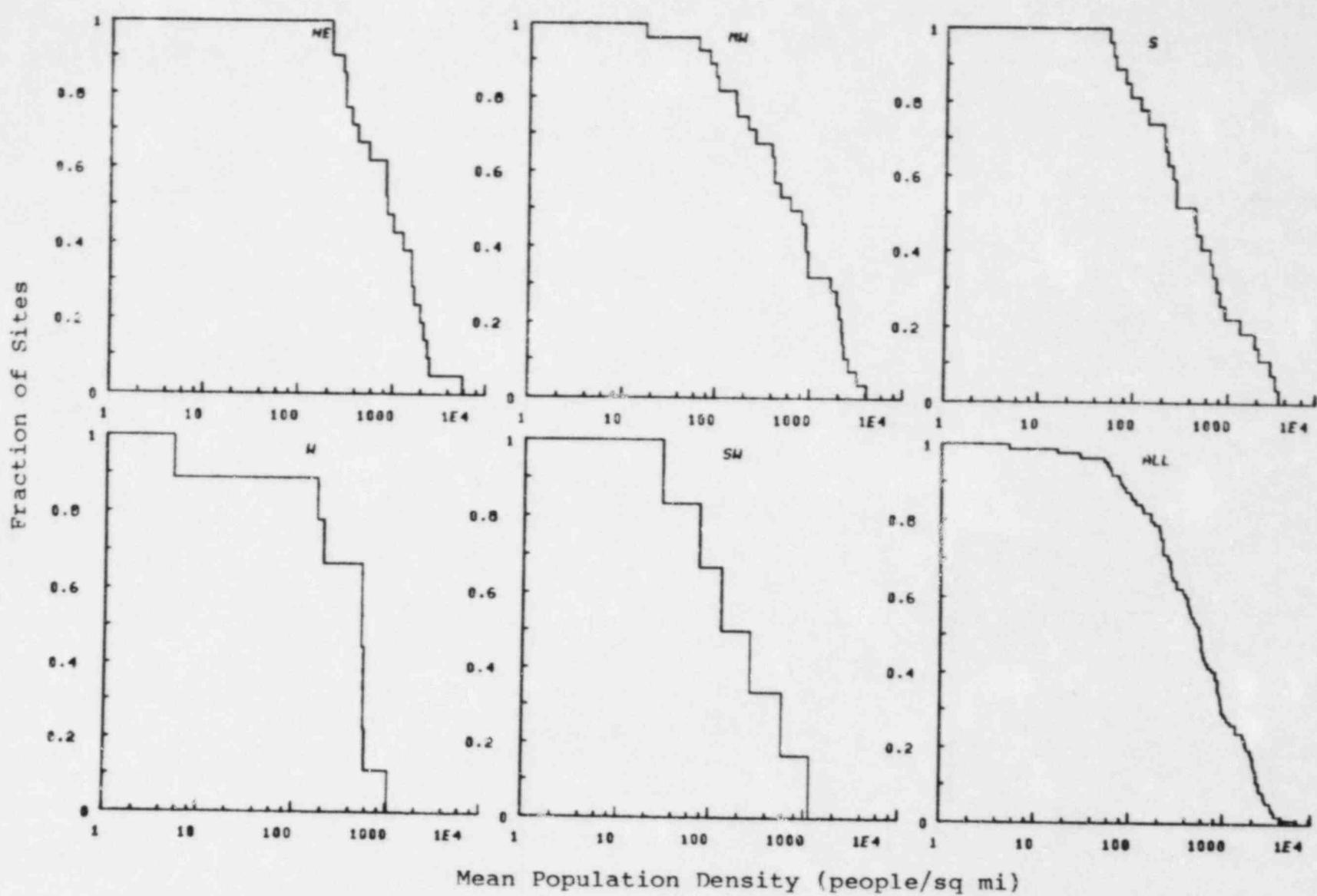


Figure D.1-20. CCDFs of Population Density (people/sq mi) at 91 Reactor Sites for the Five NRC Administrative Regions (NE, MW, S, W, SW) and for All Regions Combined (All): Population Density Within the Most Populated 22.5° Sector of the Annular Interval 10-20 Miles.

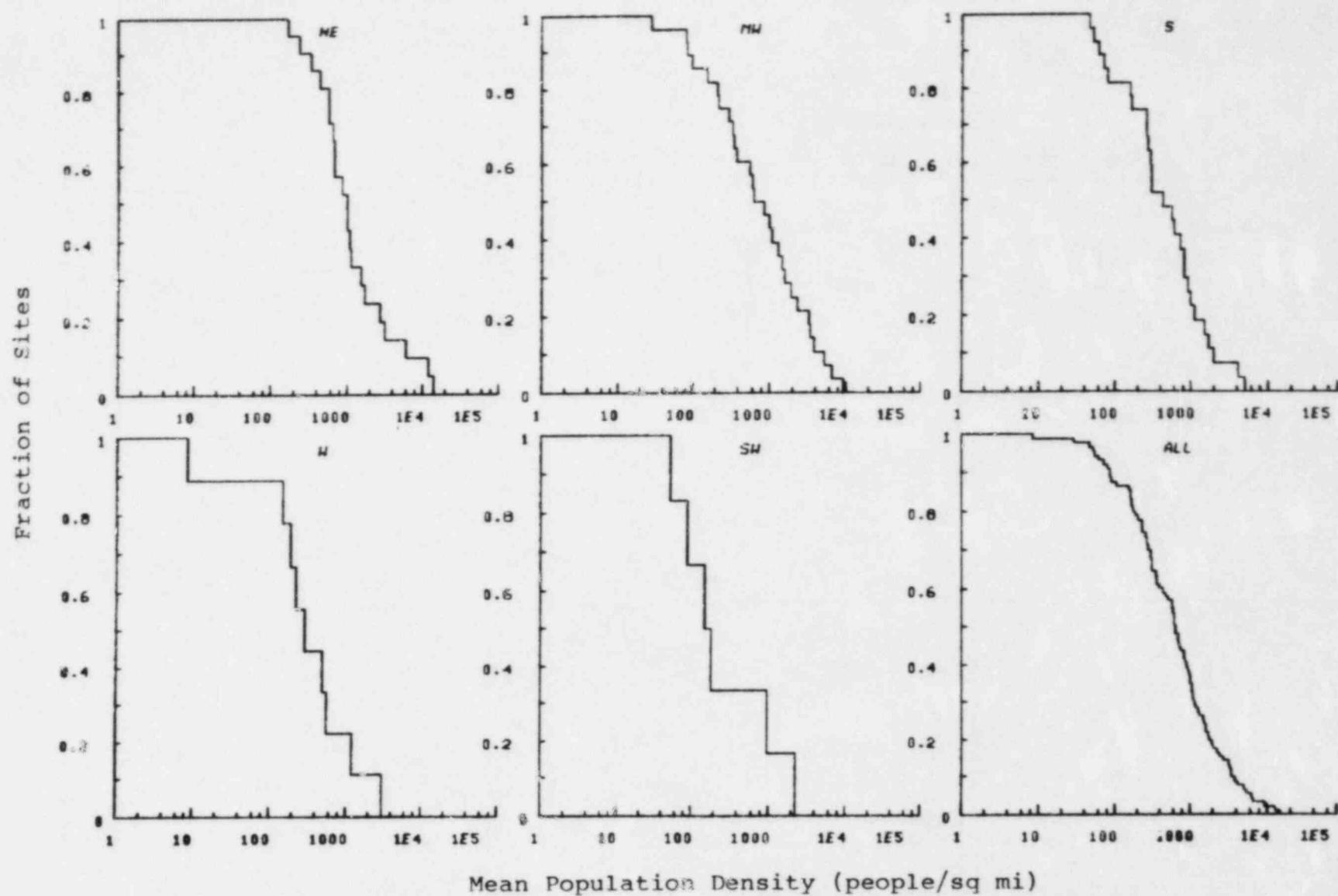


Figure D.1-21. CCDFs of Population Density (people/sq mi) at 91 Reactor Sites for the Five NRC Administrative Regions (NE, MW, S, W, SW) and for All Regions Combined (All): Population Density Within the Most Populated 22.5° Sector of the Annular Interval 20-30 Miles.

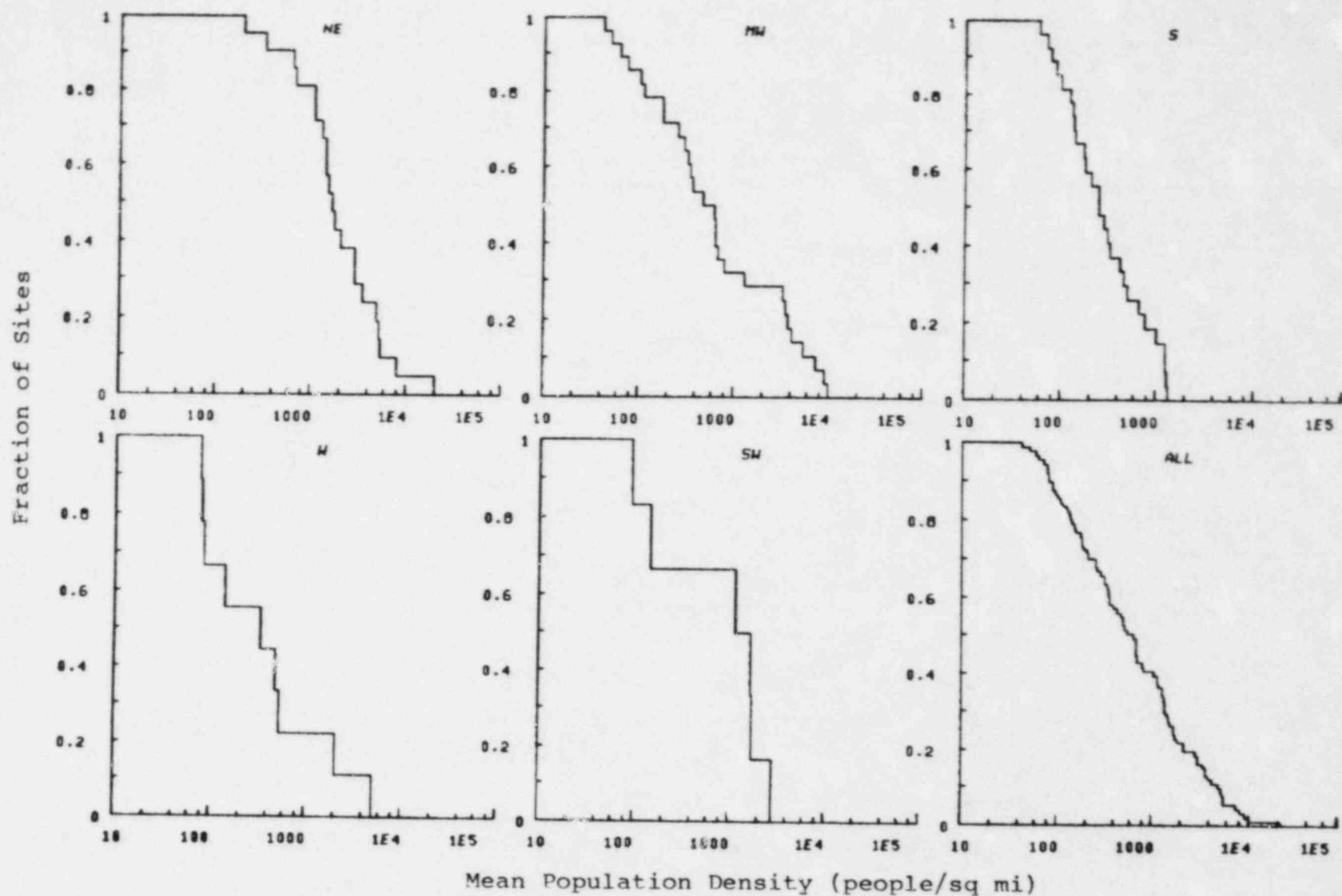


Figure D.1-22. CCDFs of Population Density (people/sq mi) at 91 Reactor Sites for the Five NRC Administrative Regions (NE, MW, S, W, SW) and for All Regions Combined (All): Population Density Within the Most Populated 22.5° Sector of the Annular Interval 30-50 Miles.

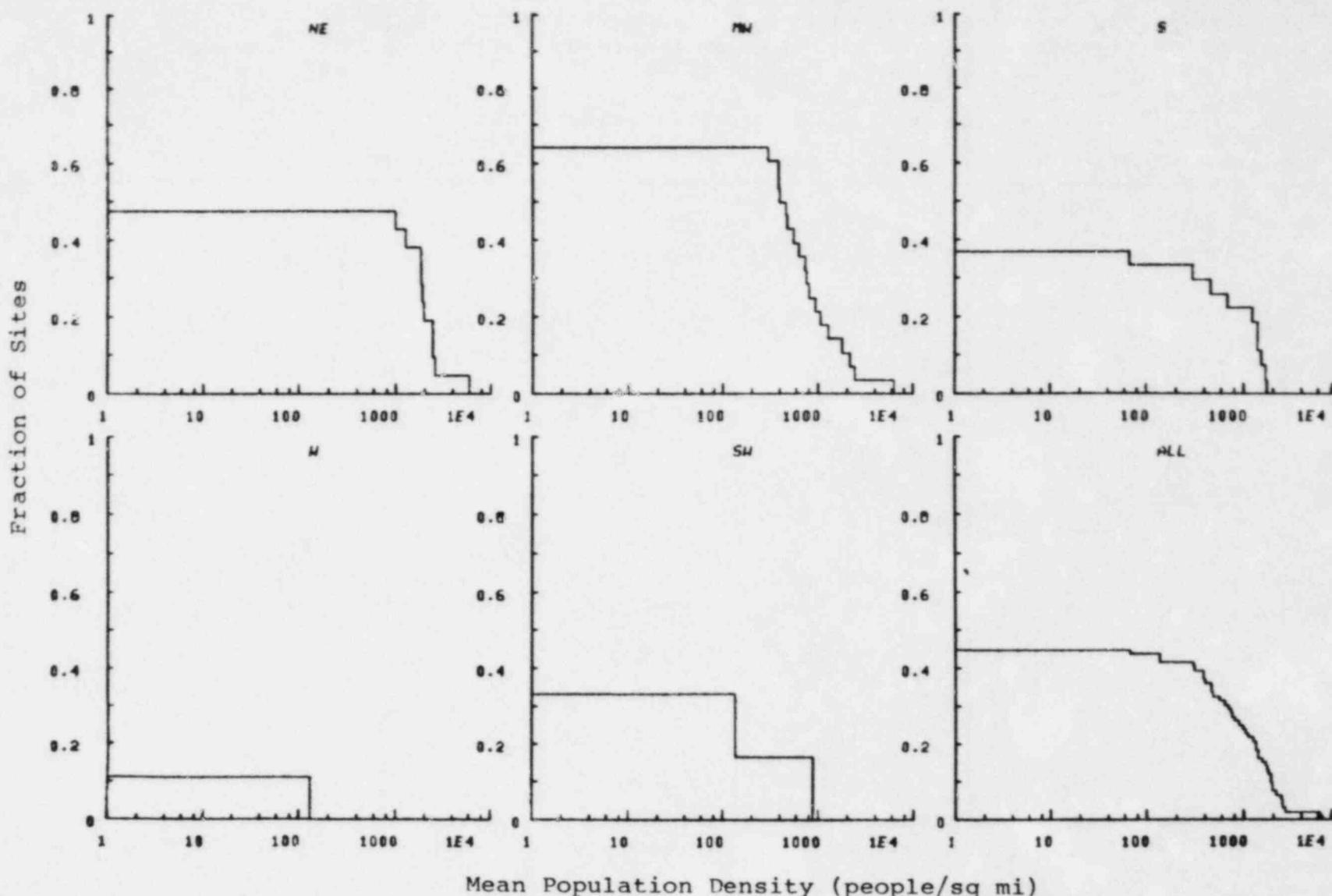


Figure D.1-23. CCDFs of Population Density (people/sq mi) at 91 Reactor Sites for the Five NRC Administrative Regions (NE, MW, S, W, SW) and for All Regions Combined (All): Population Density Within the Most Populated 22.5° Sector of the Radial Distance 0-2 Miles.

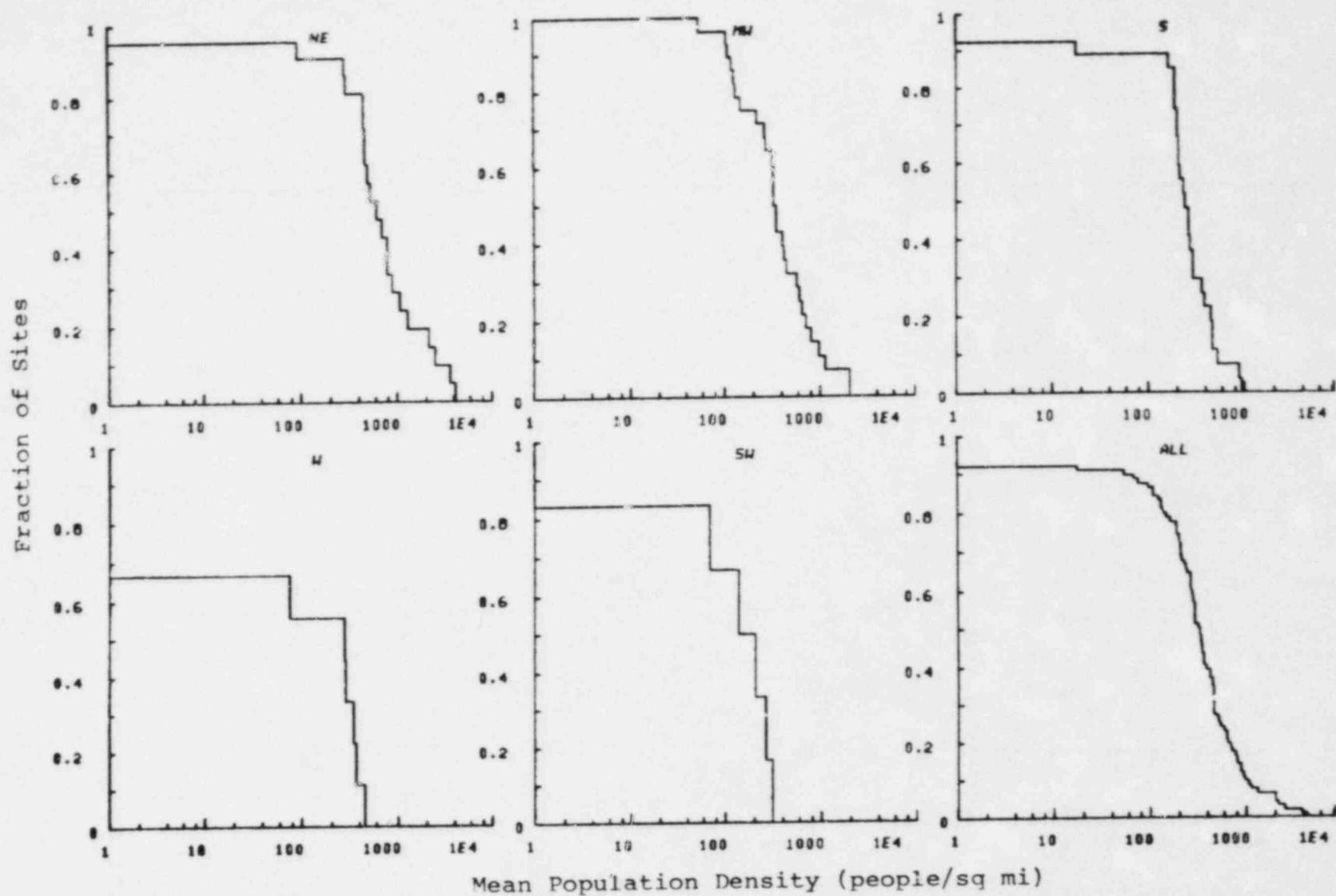


Figure D.1-24. CCDFs of Population Density (people/sq mi) at 91 Reactor Sites for the Five NRC Administrative Regions (NE, MW, S, W, SW) and for All Regions Combined (All): Population Density Within the Most Populated 22.5° Sector of the Radial Distance 0-5 Miles.

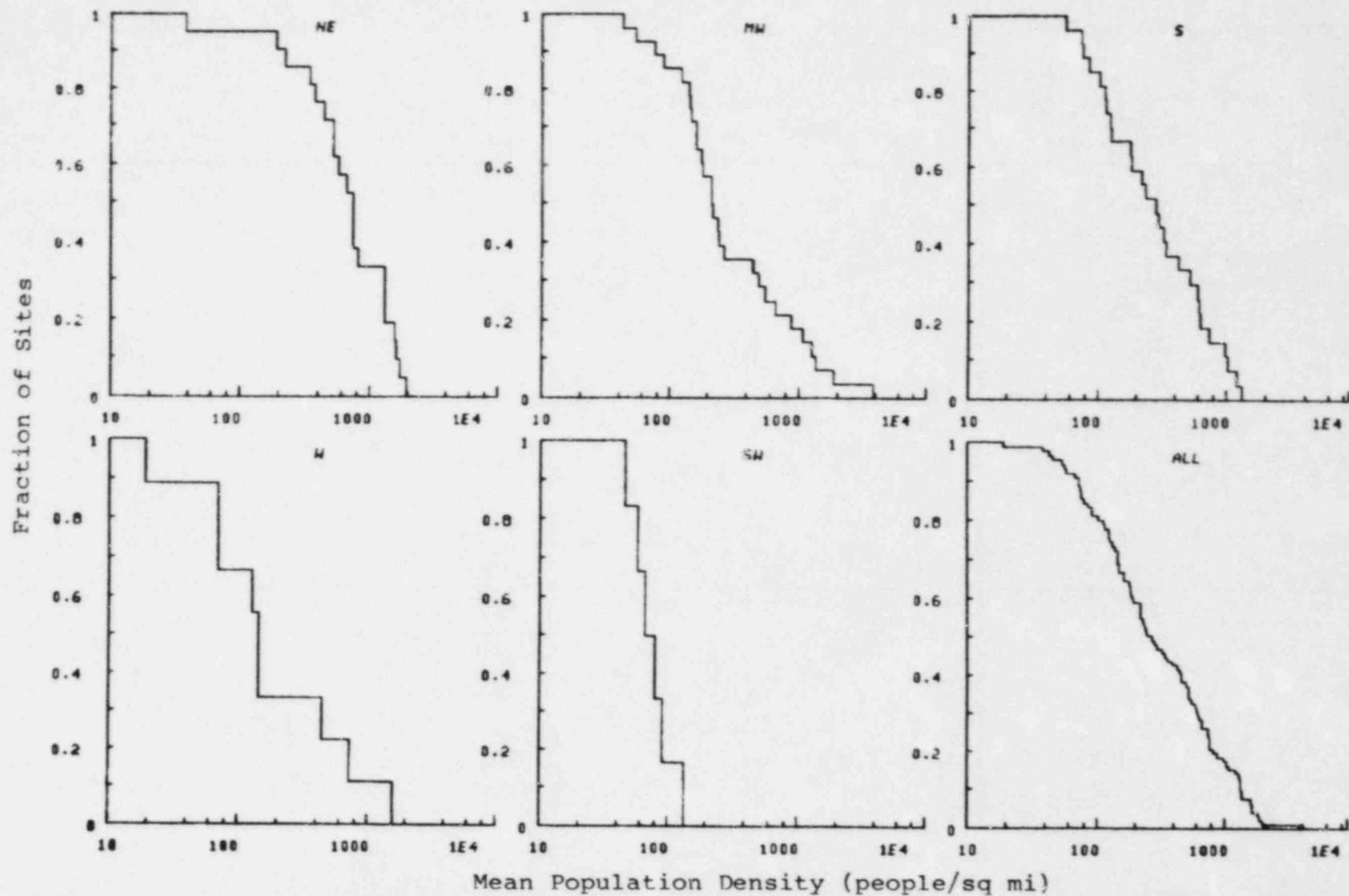


Figure D.1-25. CCDFs of Population Density (people/sq mi) at 91 Reactor Sites for the Five NRC Administrative Regions (NE, MW, S, W, SW) and for All Regions Combined (All): Population Density Within the Most Populated 22.5° Sector of the Radial Distance 0-10 Miles.

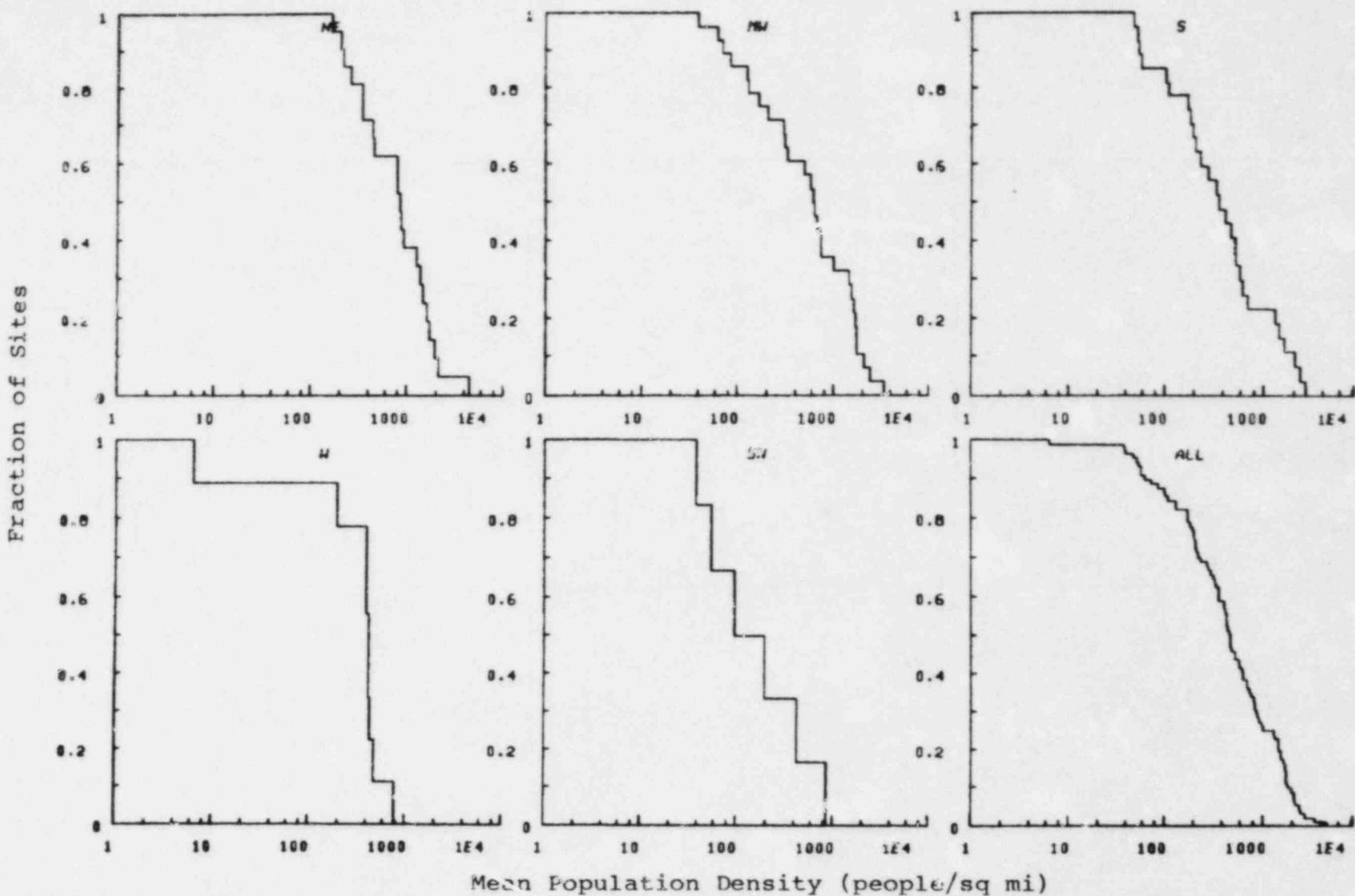


Figure D.1-26. CCDFs of Population Density (people/sq mi) at 91 Reactor Sites for the Five NRC Administrative Regions (NE, MW, S, W, SW) and for All Regions Combined (All): Population Density Within the Most Populated 22.5° Sector of the Radial Distance 0-20 Miles.

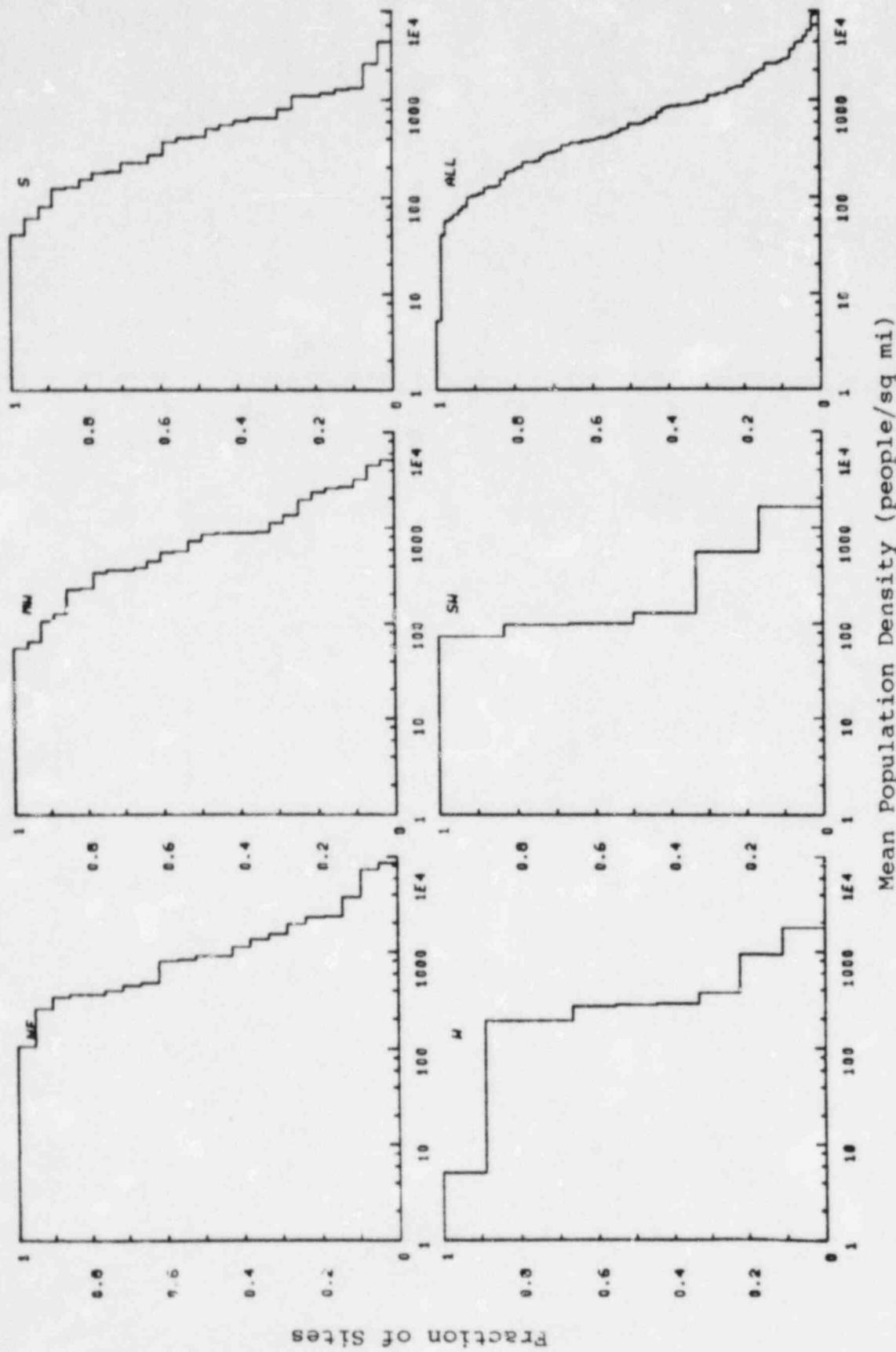


Figure D-1-27. CCDFs of Population Density (people/sq mi) at 91 Reactor Sites for the Five NRC Administrative Regions (NE, MW, S, W, SW) and for All Regions Combined (All). Population Density Within the Most Populated 22.5° Sector of the Radial Distance 0-30 Miles.

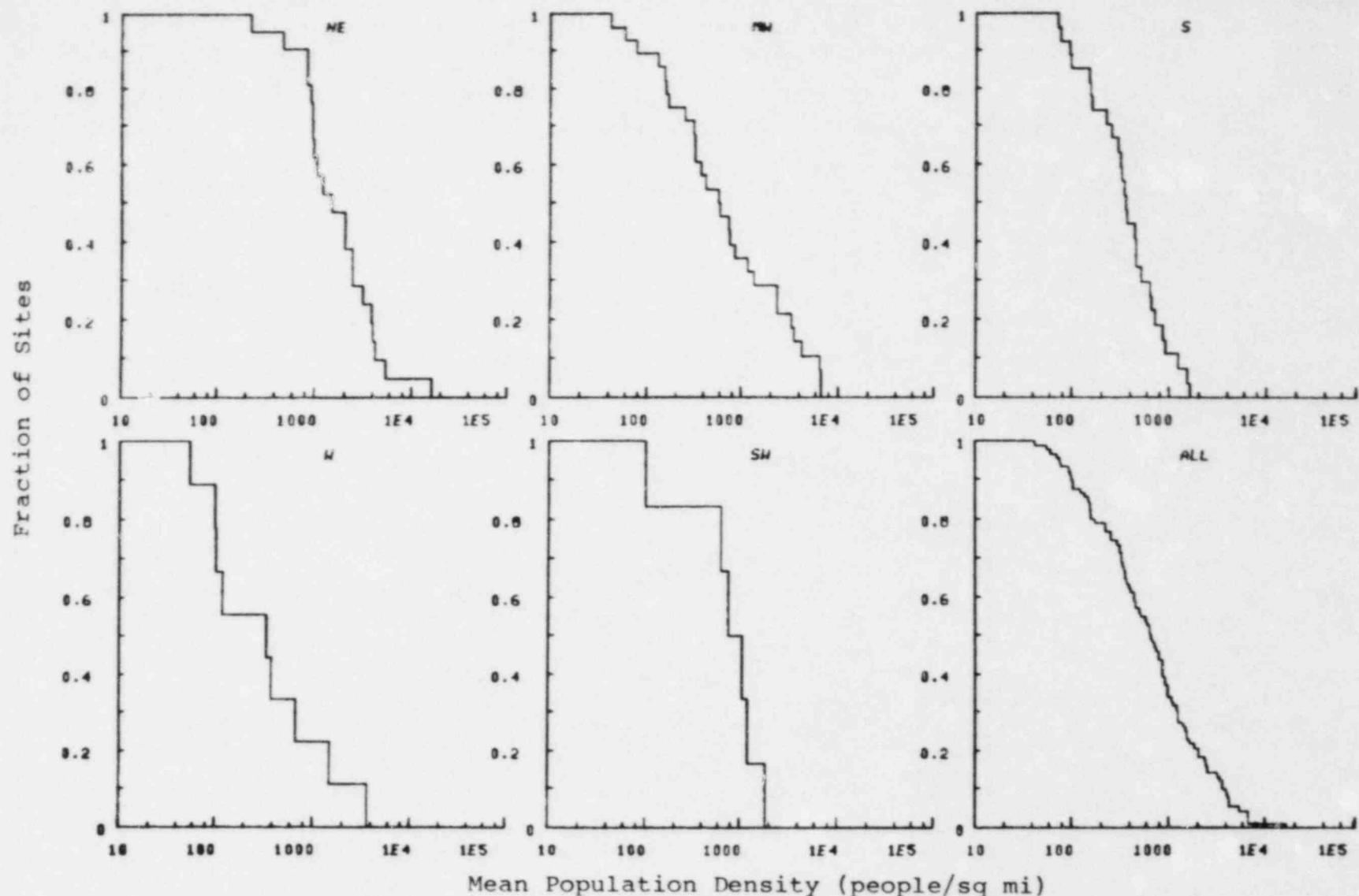


Figure D.1-28. CCDFs of Population Density (people/sq mi) at 91 Reactor Sites for the Five NRC Administrative Regions (NE, MW, S, W, SW) and for All Regions Combined (All): Population Density Within the Most Populated 22.5° Sector of the Radial Distance 0-50 Miles.

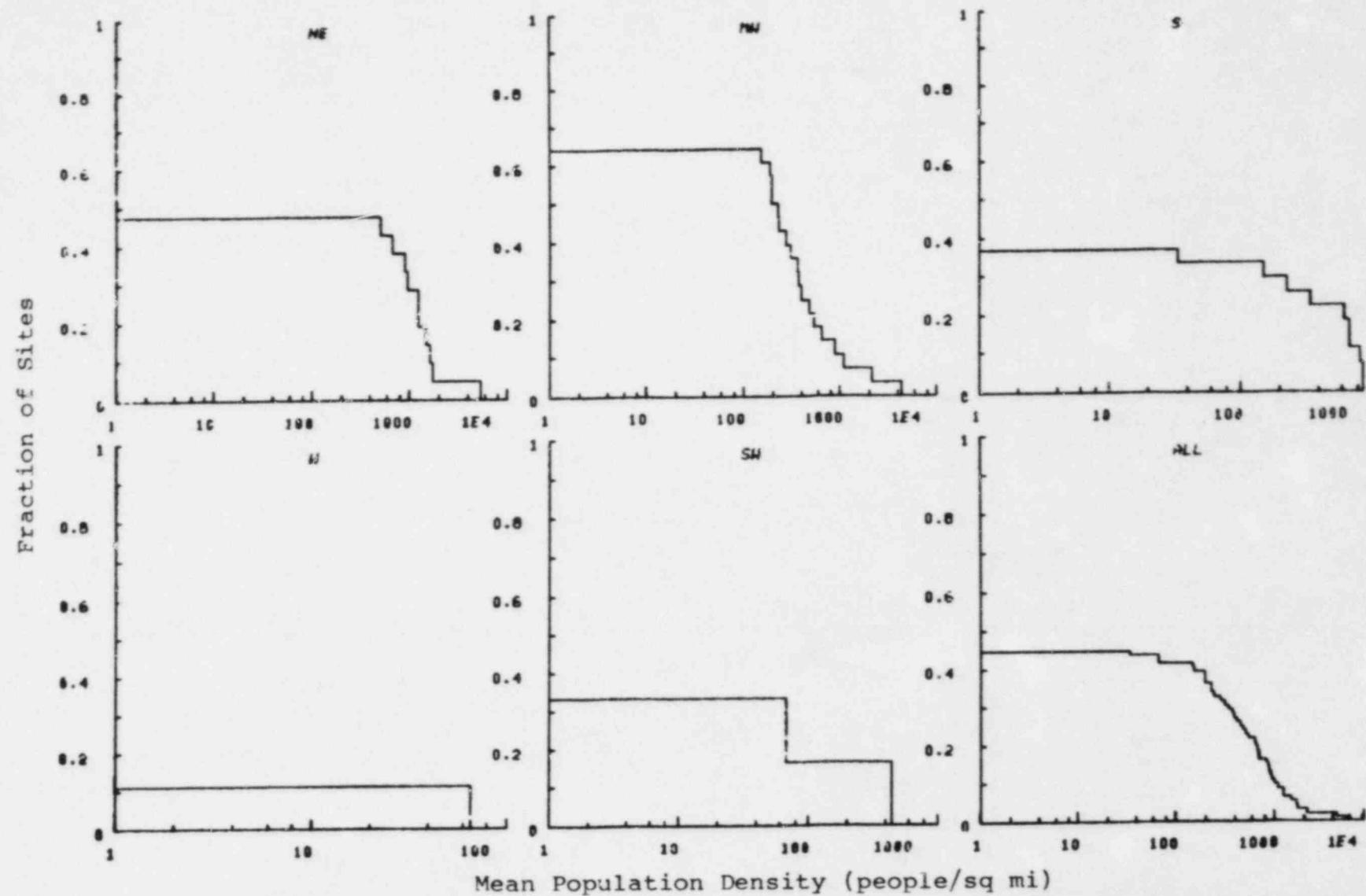


Figure D.1-29. CCDFs of Population Density (people/sq mi) at 91 Reactor Sites for the Five NRC Administrative Regions (NE, MW, S, W, SW) and for All Regions Combined (All): Population Density Within the Most Populated 45° Sector (two adjacent 22.5° sectors) of the Annular Interval 0-2 Miles.

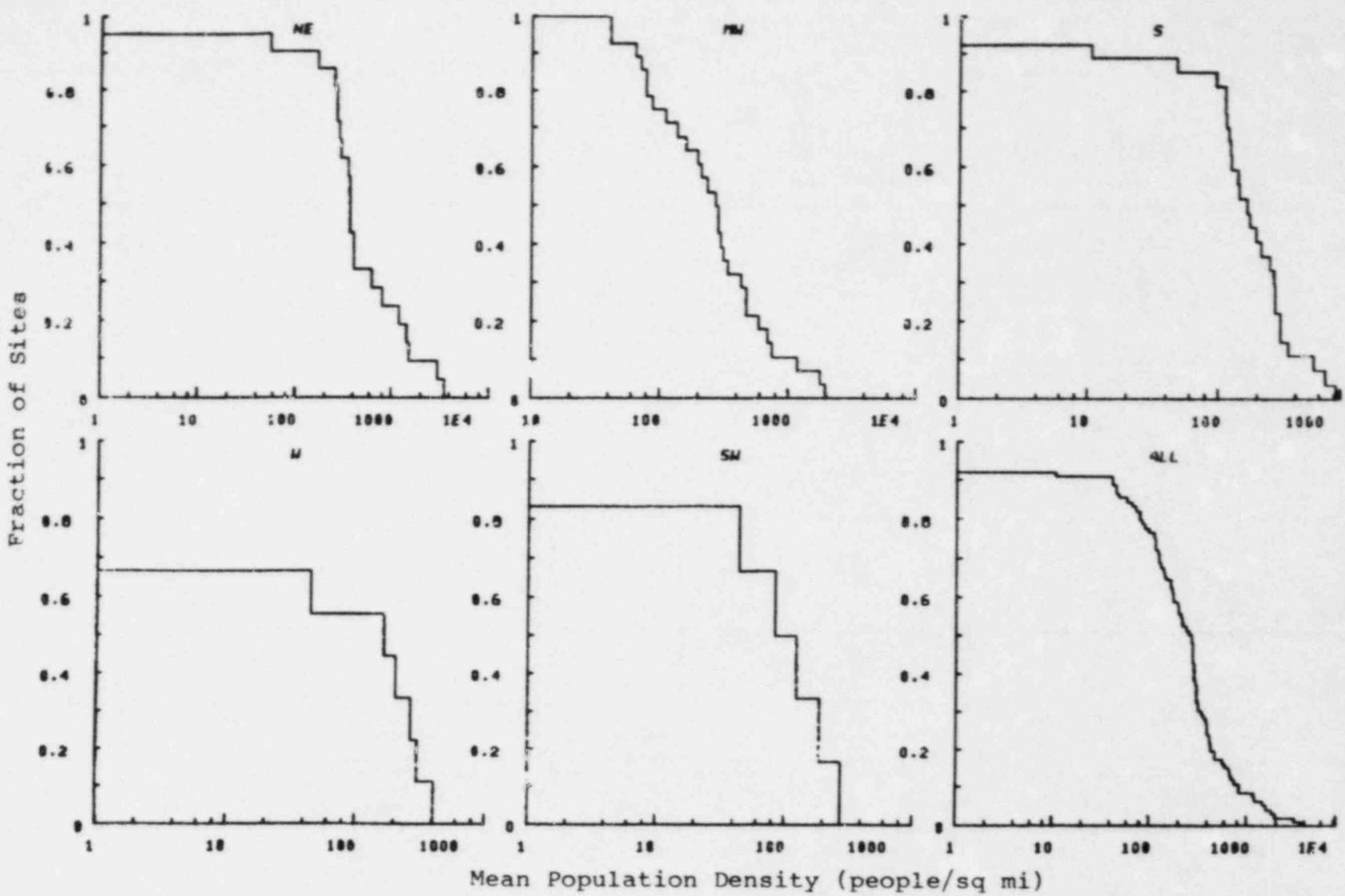


Figure D.1-30. CCDFs of Population Density (people/sq mi) at 91 Reactor Sites for the Five NRC Administrative Regions (NE, MW, S, W, SW) and for All Regions Combined (All): Population Density Within the Most Populated 45° Sector (two adjacent 22.5° sectors) of the Annular Interval 2-5 Miles.

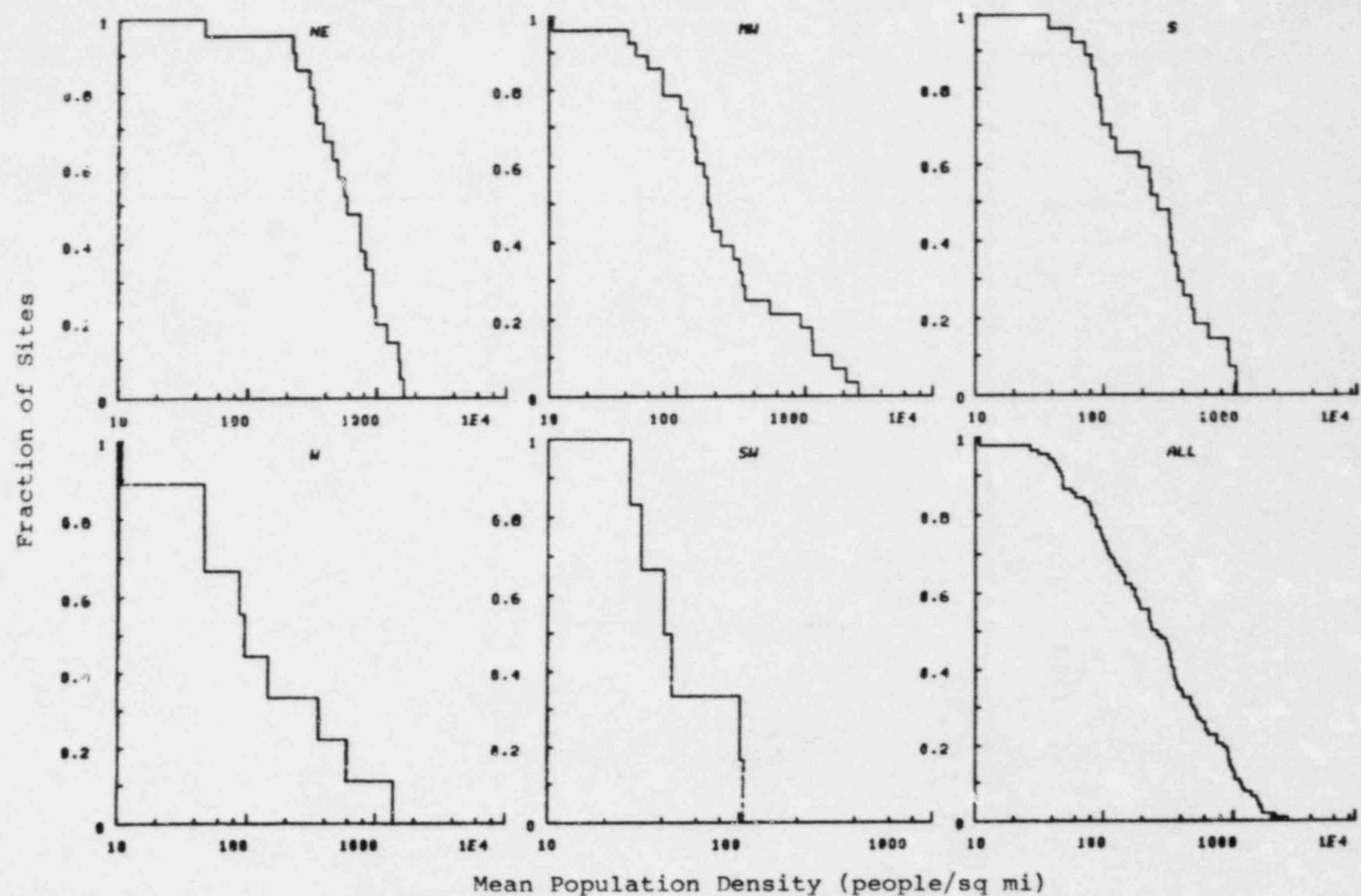


Figure D.1-31. CCDFs of Population Density (people/sq mi) at 91 Reactor Sites for the Five NRC Administrative Regions (NE, MW, S, W, SW) and for All Regions Combined (All): Population Density Within the Most Populated 45° Sector (two adjacent 22.5° sectors) of the Annular Interval 5-10 Miles.

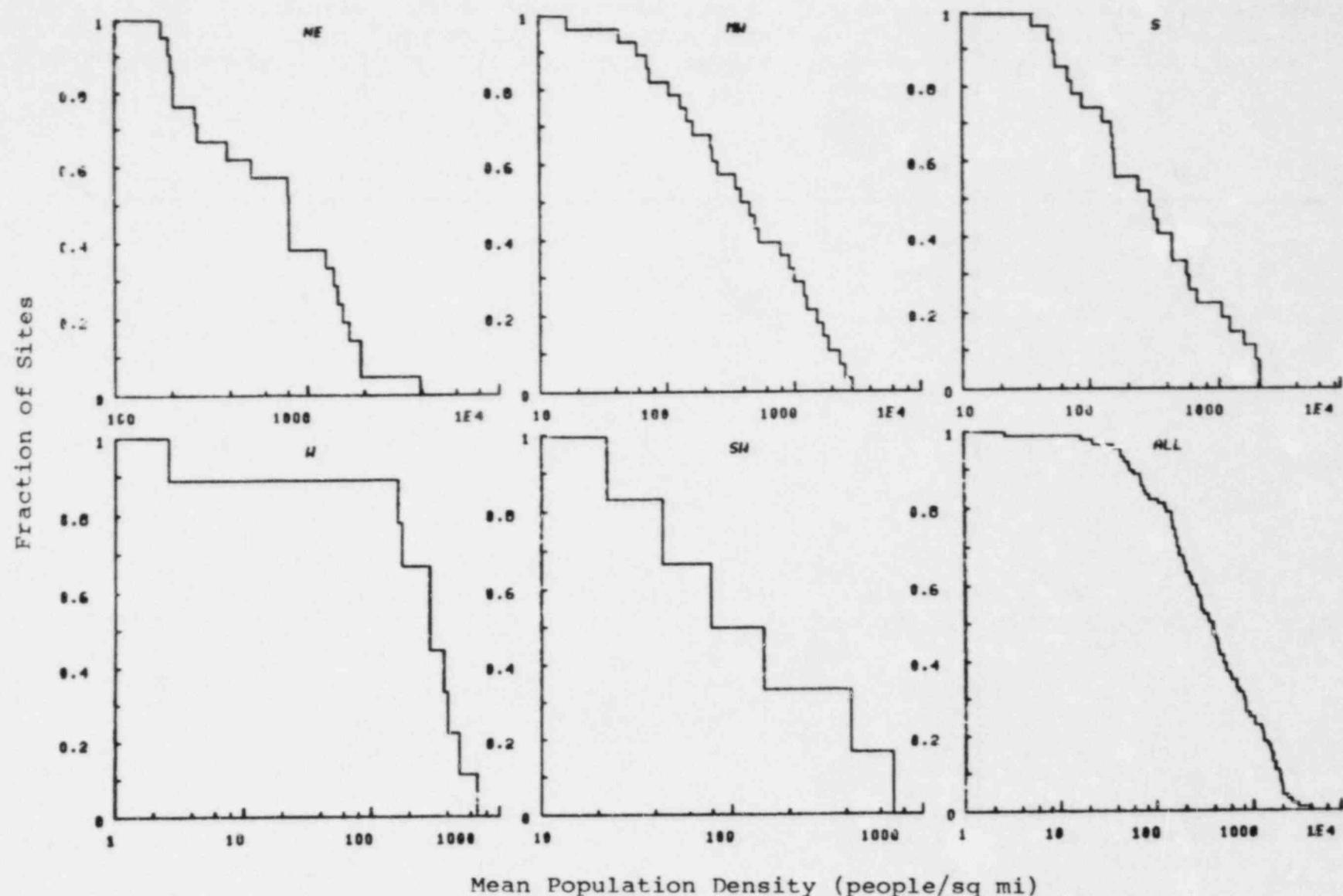


Figure D.1-32. CCDFs of Population Density (people/sq mi) at 91 Reactor Sites for the Five NRC Administrative Regions (NE, MW, S, W, SW) and for All Regions Combined (All): Population Density Within the Most Populated 45° Sector (two adjacent 22.5° sectors) of the Annular Interval 10-20 Miles.

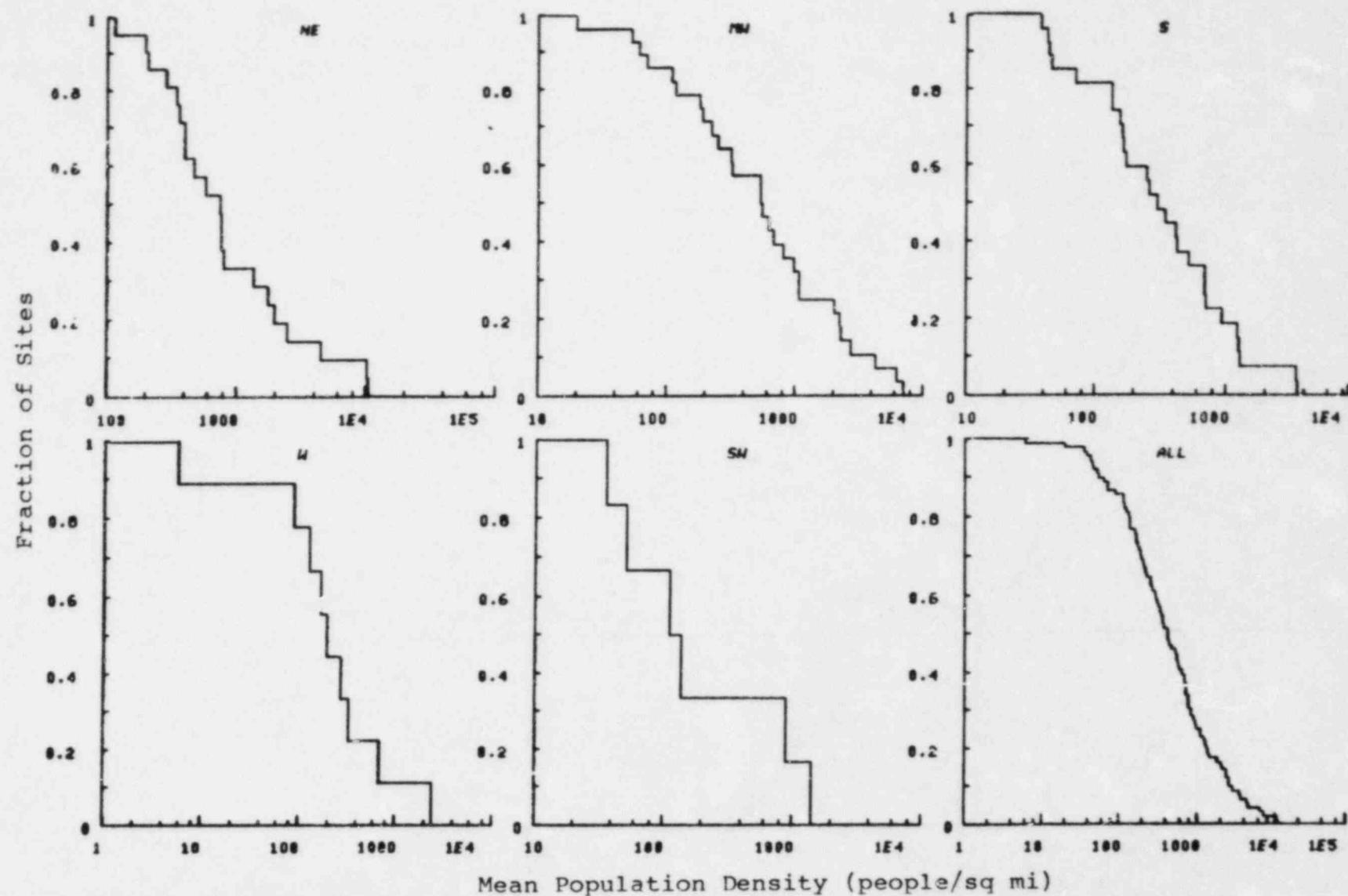


Figure D.1-33. CCDFs of Population Density (people/sq mi) at 91 Reactor Sites for the Five NRC Administrative Regions (NE, MW, S, W, SW) and for All Regions Combined (All): Population Density Within the Most Populated 45° Sector (two adjacent 22.5° sectors) of the Annular Interval 20-30 Miles.

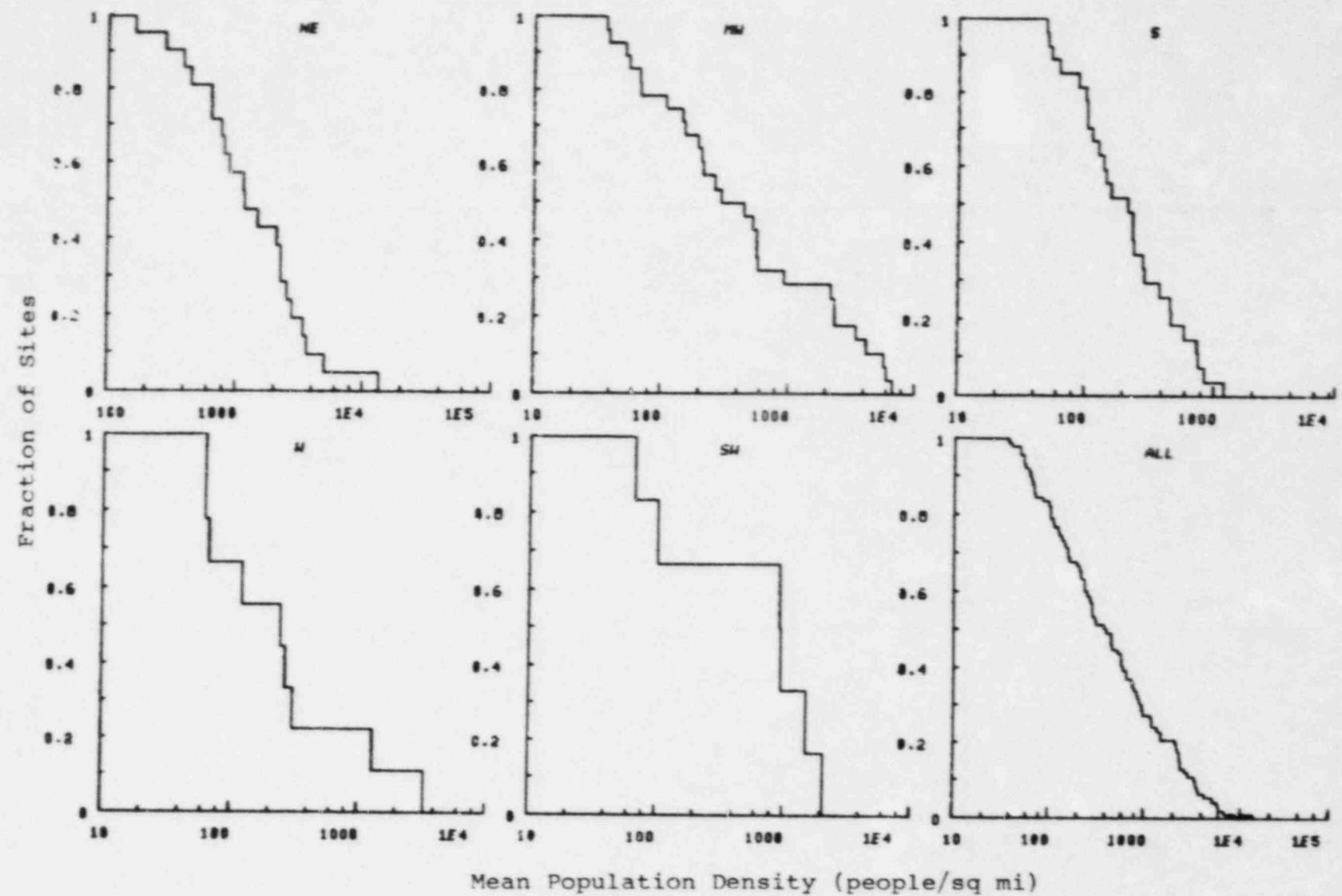


Figure D.1-34. CCDFs of Population Density (people/sq mi) at 91 Reactor Sites for the Five NRC Administrative Regions (NE, MW, S, W, SW) and for All Regions Combined (All): Population Density Within the Most Populated 45° Sector (two adjacent 22.5° sectors) of the Annular Interval 30-50 Miles.

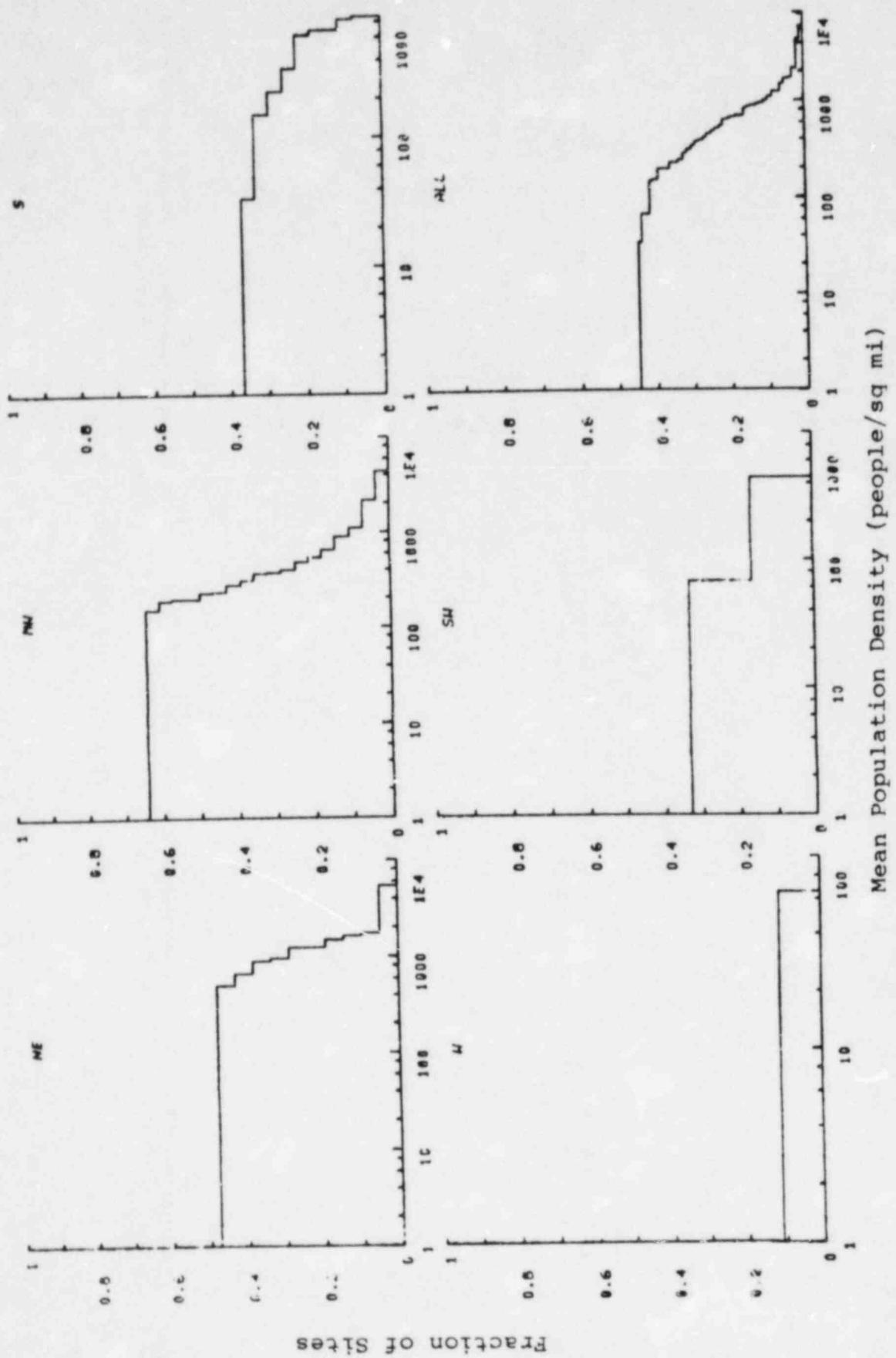


Figure D.1-35. CCDFs of population density (people/sq mi) at 91 reactor sites for the five NRC administrative regions (NE, MW, S, W, SW) and for all regions combined (All): population density within the most populated 45° sector (two adjacent 22.5° sectors) of the radial distance 0–2 miles.

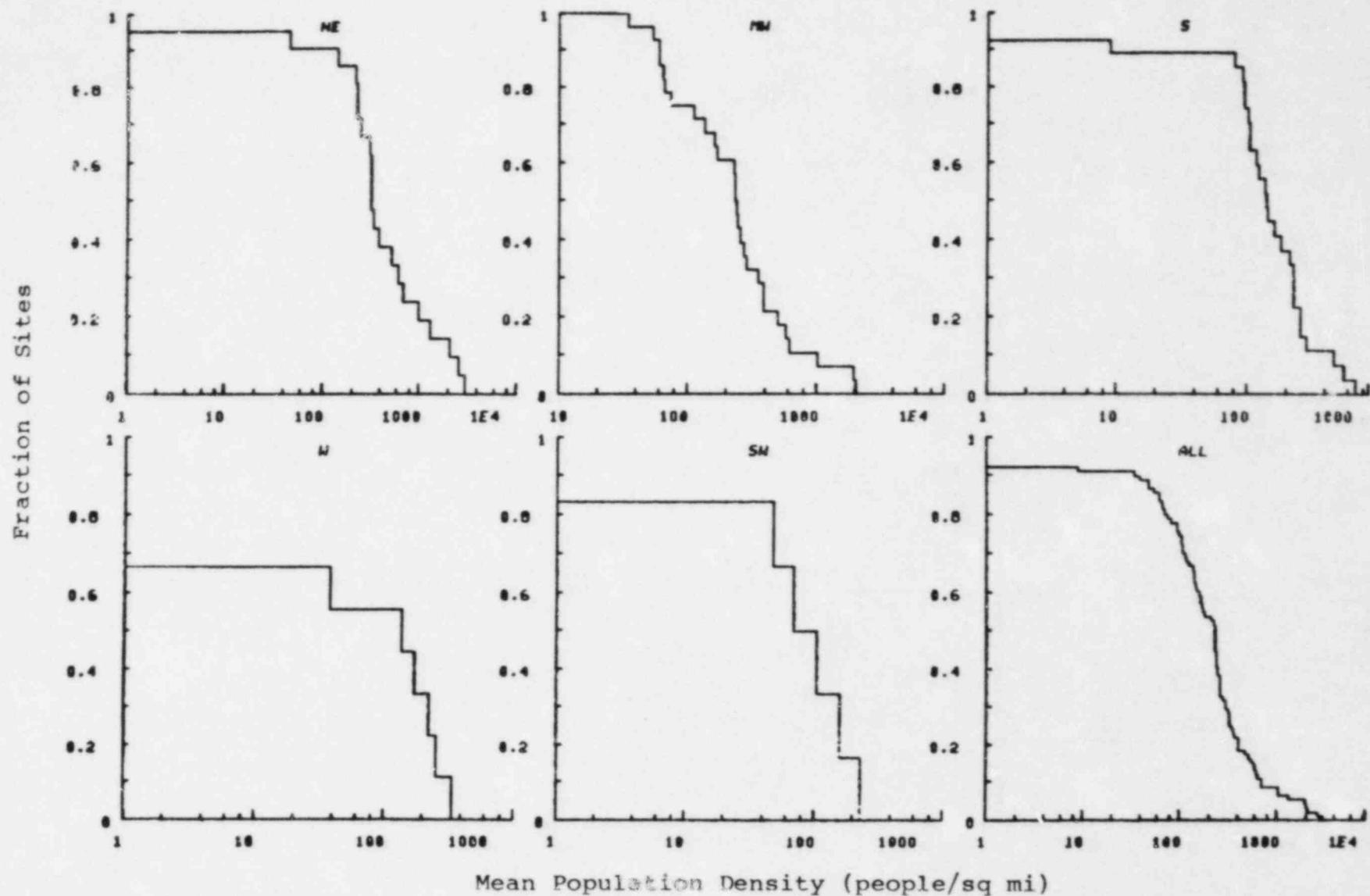


Figure D.1-36. CCDFs of Population Density (people/sq mi) at 91 Reactor Sites for the Five NRC Administrative Regions (NE, MW, S, W, SW) and for All Regions Combined (All): Population Density Within the Most Populated 45° Sector (two adjacent 22.5° sectors) of the Radial Distance 0-5 Miles.

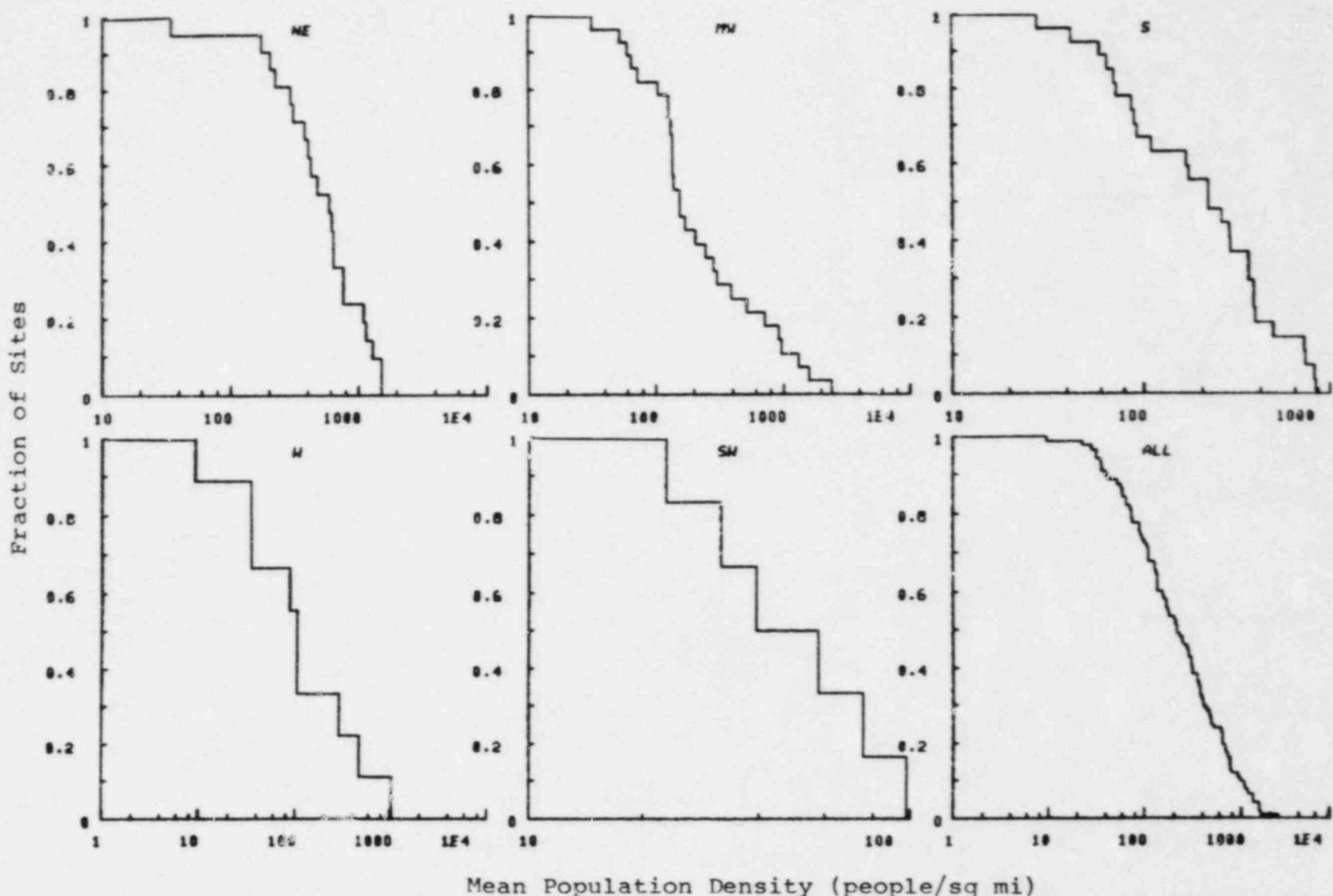


Figure D.1-37. CCDFs of Population Density (people/sq mi) at 91 Reactor Sites for the Five NRC Administrative Regions (NE, MW, S, W, SW) and for All Regions Combined (All): Population Density Within the Most Populated 45° Sector (two adjacent 22.5° sectors) of the Radial Distance 0-10 Miles.

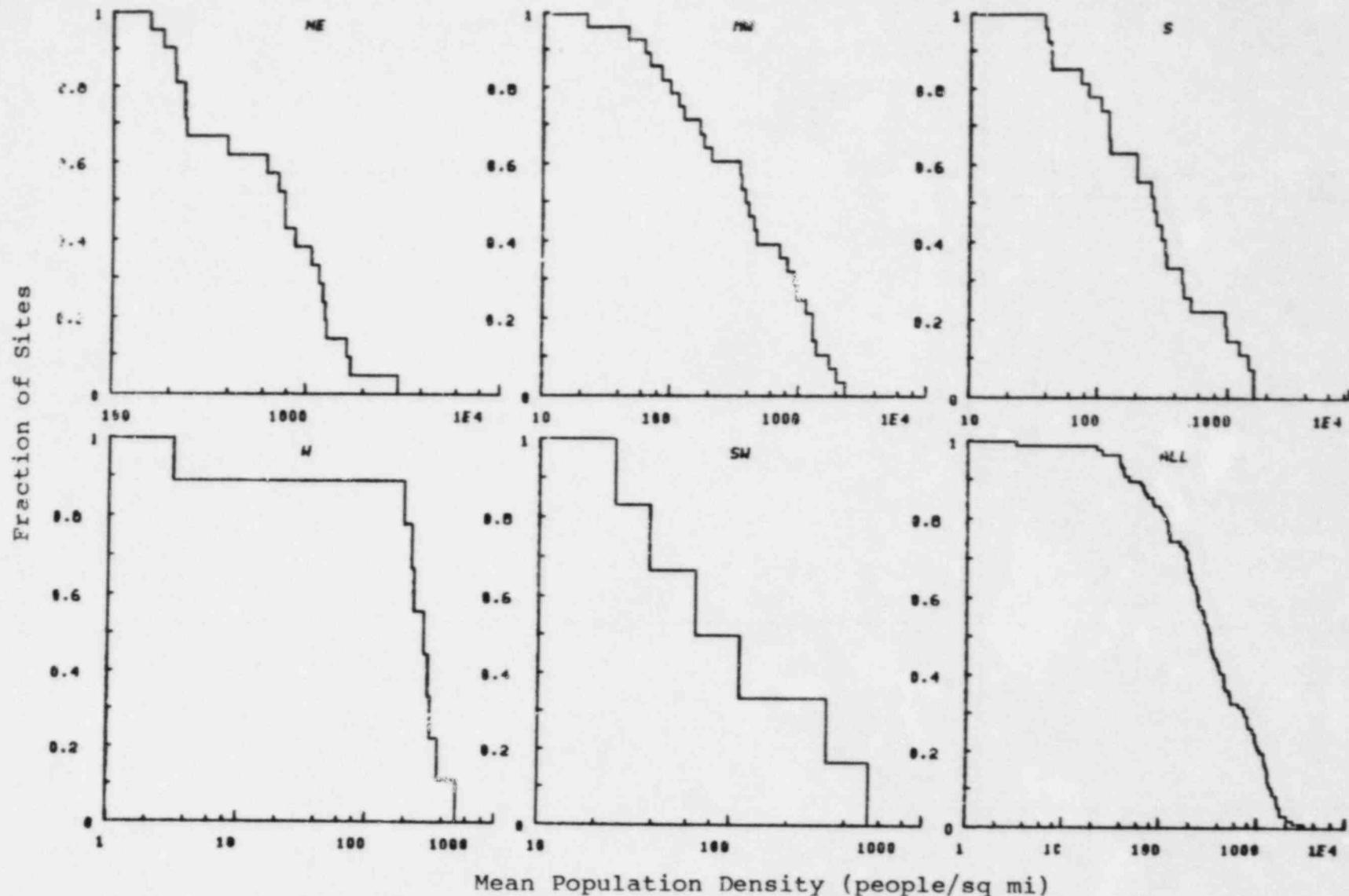


Figure D.1-38. CCDFs of Population Density (people/sq mi) at 91 Reactor Sites for the Five NRC Administrative Regions (NE, MW, S, W, SW) and for All Regions Combined (All): Population Density Within the Most Populated 45° Sector (two adjacent 22.5° sectors) of the Radial Distance 0-20 Miles.

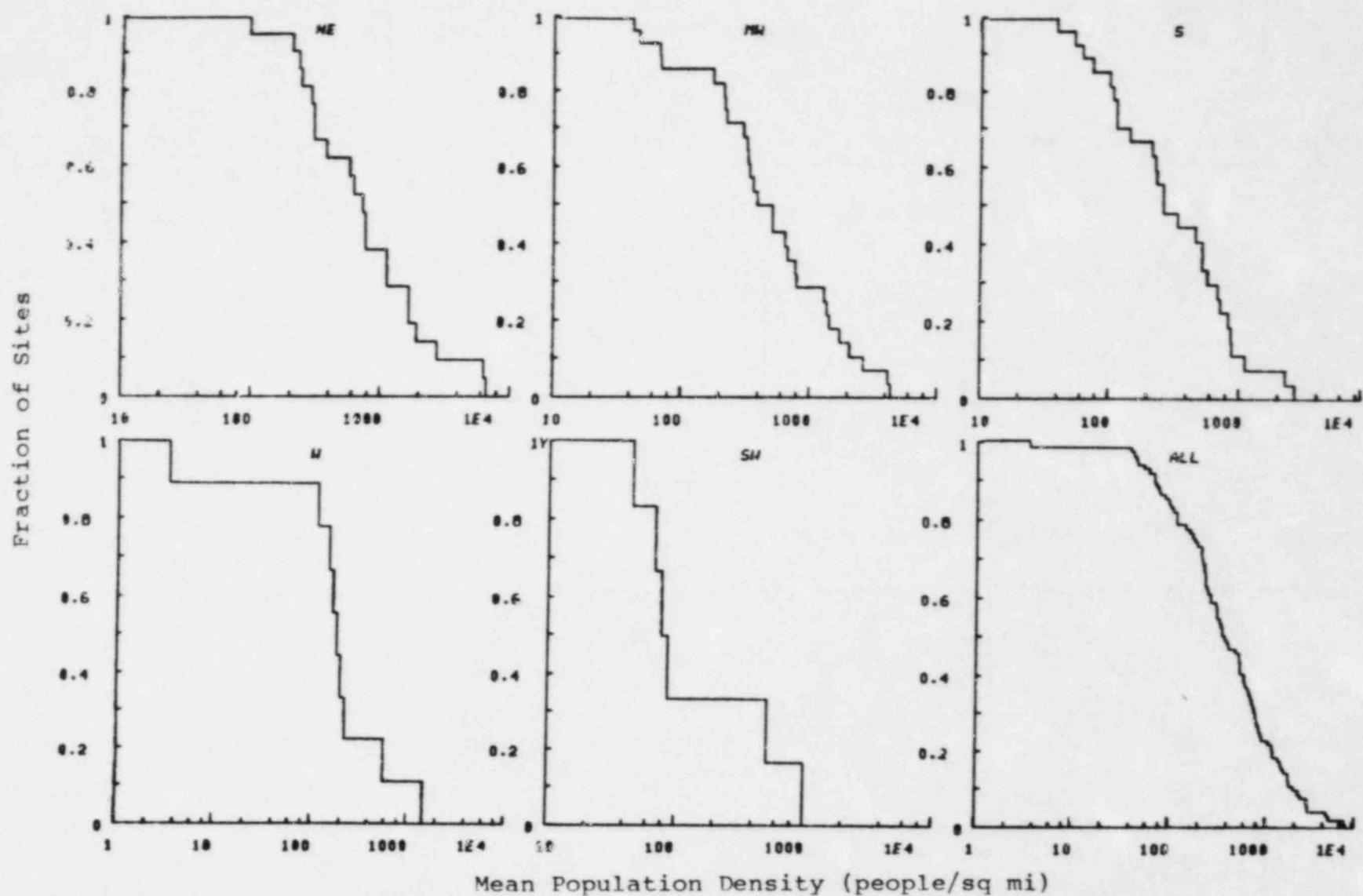


Figure D.1-39. CCDFs of Population Density (people/sq mi) at 91 Reactor Sites for the Five NRC Administrative Regions (NE, MW, S, W, SW) and for All Regions Combined (All): Population Density Within the Most Populated 45° Sector (two adjacent 22.5° sectors) of the Radial Distance 0-30 Miles.

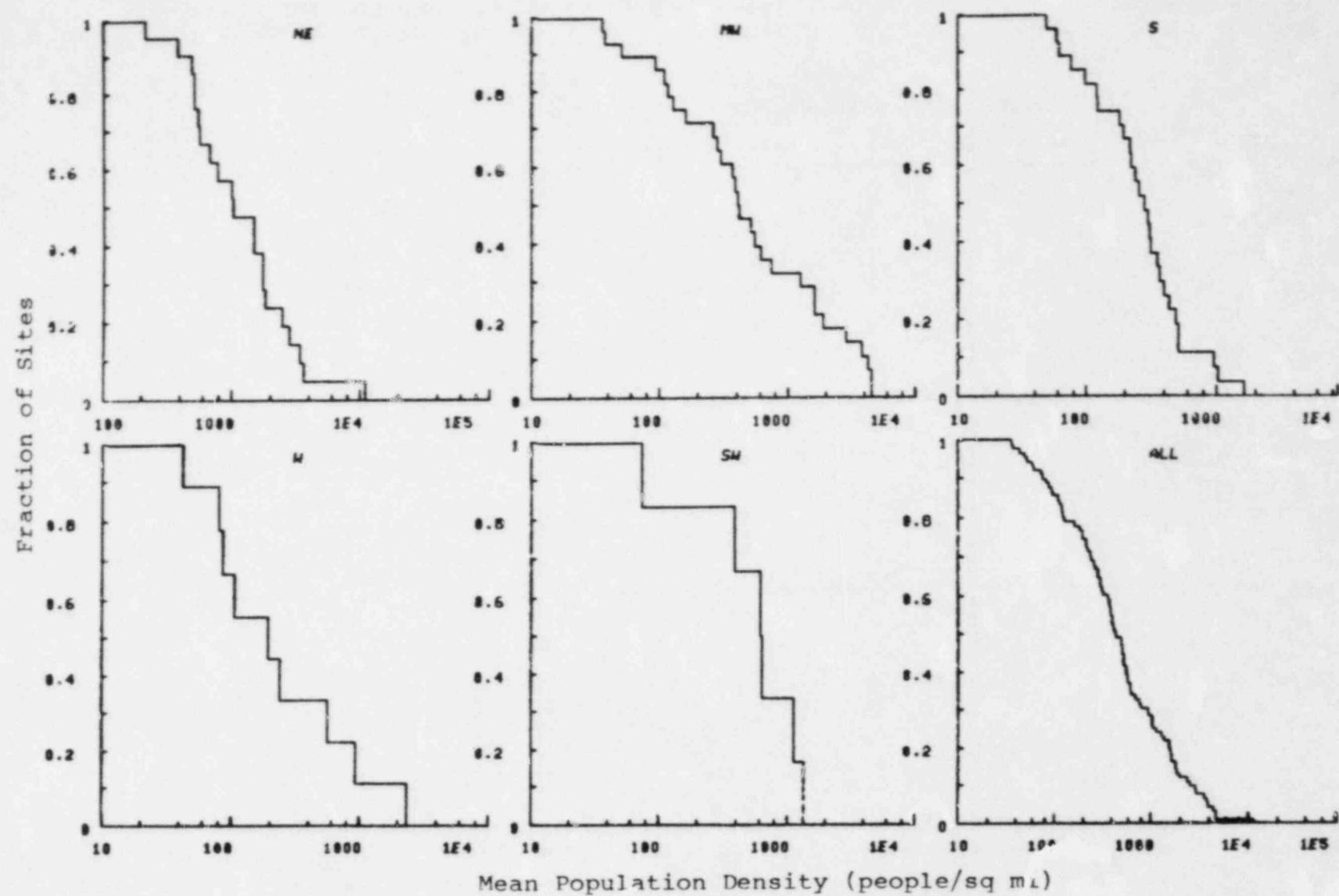


Figure D.1-40. CCDFs of Population Density (people/sq mi) at 91 Reactor Sites for the Five NRC Administrative Regions (NE, MW, S, W, SW) and for All Regions Combined (All): Population Density Within the Most Populated 45° Sector (two adjacent 22.5° sectors) of the Radial Distance 0-50 Miles.

TABLE D.1-1

POPULATION DENSITIES (PEOPLE PER SQ. MI.) FOR 91 REACTOR SITES  
 INNER AND OUTER ANNULAR RADII ARE GIVEN IN MILES

SITE	0-5	5-10	10-20	20-30	30-50	50-100	100-200
1 ALLENS CREEK	31	21	30	39	286	48	35
2 ARKANSAS 1 + 2	58	83	26	16	15	42	47
3 BAILLY S	271	283	534	1024	906	145	134
4 BEAVER VALLEY 1 + 2	160	565	342	787	403	210	139
5 BELLEFONTE 1	21	89	30	41	147	87	76
6 BIG ROCK POINT	54	14	27	9	16	11	39
7 BLACK FOX	29	10	147	234	36	38	35
8 BRAIDWOOD 1	127	53	79	168	700	258	111
9 BROWNS FERRY 1, 2, +	12	121	88	98	71	76	80
10 BRUNSWICK 1 + 2	31	25	62	26	13	40	48
11 BYRON 1	83	59	250	127	85	439	74
12 CALLAWAY	8	12	32	87	24	123	56
13 CALVERT CLIFF 1 + 2	34	52	55	51	456	201	167
14 CATAWBA 1	49	237	431	154	107	116	73
15 CHEROKEE	48	113	113	220	162	95	91
16 CLINTON	18	46	36	168	79	68	188
17 COMMANCHE PEAK	20	20	7	33	142	94	30
18 COOK DC 1 + 2	93	157	115	226	117	418	169
19 COOPER S	14	22	19	22	22	70	40
20 CRYSTAL RIVER	15	30	11	8	31	89	25
21 DAVIS-BE 1	31	55	89	380	212	350	158
22 DIABLO CANYON 1 + 2	0	30	69	32	17	13	151
23 DRESDEN 2 + 3	68	118	199	259	1157	156	108
24 DUANE ARNOLD	50	346	42	37	54	58	94
25 FARLEY 1 + 2	22	29	71	27	41	48	55
26 FERMI 2	126	259	386	1254	562	194	125
27 FITZPATRICK	29	150	50	72	129	79	67
28 FORKED RIVER 1	76	131	146	176	565	875	148
29 FORT CALHOUN	101	25	312	182	23	34	42
30 FORT ST VRAIN	9	35	143	188	192	15	6
31 R. E. GINNA	77	124	611	143	67	114	52
32 GRAND GULF 1	16	28	19	40	40	49	57
33 HADDEM NECK	113	211	473	803	305	822	158
34 HARTSVILLE	44	37	61	46	148	46	83
35 HATCH, E.I. 1 + 2	13	20	38	28	33	41	64
36 INDIAN PT 2 + 3	752	617	732	2046	2462	304	196
37 KEWAUNEE	21	33	80	99	66	84	139
38 LASALLE 1 + 2	12	53	90	75	140	391	118
39 LA CROSSE	13	22	89	34	35	55	106
40 LIMERICK 1	792	381	668	1877	619	705	169
41 MARBLE HILL	88	44	301	379	67	141	104
42 ME YANKEE	0	6	36	63	45	18	82
43 MCGUIRE 1 + 2	64	137	505	193	113	111	73
44 MIDLAND 2	535	87	289	85	109	185	97
45 MILLSTONE 1 + 2	582	284	167	102	410	624	204
46 MONTICELLO	67	38	45	155	340	35	26

TABLE D.1-1 (cont'd)

SITE	0-5	5-10	10-20	20-30	30-50	50-100	100-200
47 NINE M. PT. 1 + 2	29	150	50	72	129	79	67
48 NORTH ANNA 1, 2, + 3	12	28	29	58	146	183	161
49 OCONEE 1, 2 + 3	42	176	68	163	72	77	94
50 OYSTER CREEK	76	131	146	176	565	875	148
51 PALISADE	70	106	92	58	158	423	148
52 PALO VERDE 1	6	7	8	7	122	18	8
53 PEACH BOTTOM 2 + 3	44	96	246	362	659	428	63
54 PEBBLE SPRINGS	5	2	0	2	15	15	48
55 PERKINS	79	109	203	251	172	96	78
56 PERRY 1	224	230	178	296	374	135	170
57 PHIPPS BEND	82	57	128	98	78	78	92
58 PILGRIM 1	119	85	132	407	699	110	194
59 POINT BEACH 1 + 2	30	80	63	88	70	90	139
60 PRAIRIE 1 + 2	60	67	51	114	358	46	34
61 QUAD CITIES 1 + 2	18	64	313	77	47	85	150
62 RANCHO SECO	22	29	133	492	93	210	16
63 RIVERBEND 1	49	74	86	176	43	92	34
64 H. B. ROBINSON 2	97	75	50	75	77	98	68
65 SAINT LUCIE 1	71	160	34	29	41	58	38
66 SALEM 1 + 2	45	102	334	348	778	410	249
67 SAN ONOFRE	18	103	183	134	632	314	11
68 SEABROOK 1	120	88	89	64	272	129	16
69 SEQUOYAH 1 + 2	108	115	303	71	51	82	89
70 SHEARON HARRIS	23	69	168	205	109	97	74
71 SHOREHAM	135	146	347	847	699	714	173
72 SKAGIT	49	52	34	66	43	74	9
73 SOUTH TEXAS	0	10	25	11	26	94	31
74 VIRGIL C. SUMMER	1	43	47	194	67	110	84
75 SURRY ST 1 + 2	26	253	185	194	212	40	111
76 SUSQUEHANNA 1	188	130	330	178	172	378	354
77 THREE MILE ISLAND	320	470	499	248	168	506	281
78 TROJAN	104	197	50	52	190	48	26
79 TURKEY POINT 1 + 2	0	164	179	437	152	26	8
80 VERMONT YANKEE 1	102	79	99	68	217	363	236
81 VOGTLE	0	8	26	162	35	58	79
82 WATERFORD 3	181	119	282	490	91	40	27
83 WATTS BAR 1 + 2	22	31	61	68	101	61	103
84 WPPSS1+4	0	6	69	22	16	14	43
85 WPPSS 3 + 5	28	24	46	53	49	86	20
86 WPPSS 2	0	6	61	27	16	14	43
87 WOLF CREEK	34	4	9	32	21	97	35
88 YANKEE ROWE	12	88	84	129	255	311	261
89 YELLOW CREEK	15	32	42	35	49	66	65
90 ZIMMER 1	53	87	203	622	126	156	105
91 ZION	538	697	347	484	1130	196	83

TABLE D.1-2

CUMMULATIVE POPULATION DENSITIES (PEOPLE PER SQ. MI.) FOR 91  
REACTOR SITES, CIRCLE RADII ARE GIVEN IN MILES

SITE	0-5	0-10	0-20	0-30	0-50	0-100	0-200
1 ALLENS CREEK	31	23	28	35	196	85	48
2 ARKANSAS 1 + 2	58	77	39	26	19	37	44
3 BAILLY S	271	280	471	778	860	324	182
4 BEAVER VALLEY 1 + 2	160	464	373	603	475	277	174
5 BELLEFONTE 1	21	72	41	41	109	92	80
6 BIG ROCK POINT	54	24	26	16	16	12	32
7 BLACK FOX	29	15	114	181	88	51	39
8 BRAIDWOOD 1	127	72	77	128	494	317	163
9 BROWNS FERRY 1, 2, +	12	94	89	94	80	77	80
10 BRUNSWICK 1 + 2	31	26	53	38	22	36	45
11 BYRON 1	83	65	204	161	112	357	145
12 CALLAWAY	8	11	27	61	37	102	67
13 CALVERT CLIFF 1 + 2	34	48	53	52	310	229	182
14 CATAWBA 1	49	190	371	250	159	126	87
15 CHEROKEE	48	97	109	171	165	113	96
16 CLINTON	18	39	37	109	90	74	159
17 COMMANCHE PEAK	20	20	10	23	99	95	46
18 COOK DC 1 + 2	93	141	122	180	139	349	214
19 COOPER S	14	20	12	21	22	58	44
20 CRYSTAL RIVER	15	26	15	11	24	73	37
21 DAVIS-BE 1	31	49	79	246	225	318	198
22 DIABLO CANYON 1 + 2	0	22	57	43	27	17	117
23 DRESDEN 2 + 3	68	105	176	222	821	322	162
24 DUANE ARNOLD	50	272	100	65	58	58	85
25 FARLEY 1 + 2	22	27	60	42	41	46	53
26 FERMI 2	126	226	346	851	666	312	172
27 FITZPATRICK	29	119	67	70	107	86	72
28 FORKED RIVER 1	76	117	139	160	419	761	301
29 FORT CALF UN	101	44	245	210	91	48	43
30 FORT ST VAIN	9	29	114	155	179	56	19
31 R. E. GINNA	77	112	486	295	149	123	70
32 GRAND GULF 1	16	25	20	31	37	46	54
33 HADDEM NECK	113	187	401	624	420	722	299
34 HARTSVILLE	44	39	55	50	113	62	78
35 HATCH, E.I. 1 + 2	13	18	33	31	32	39	58
36 INDIAN PT 2 + 3	752	651	711	1453	2099	752	335
37 KEWAUNEE	21	30	68	85	73	81	124
38 LASALLE 1 + 2	12	42	78	76	117	322	169
39 LA CROSSE	13	20	71	51	41	51	92
40 LIMERICK 1	792	483	622	1319	871	746	313
41 MARPLE HILL	88	55	240	317	157	145	115
42 ME YANKEE	0	4	28	47	46	25	68
43 MCGUIRE 1 + 2	64	119	408	289	176	128	87
44 MIDLAND 2	535	199	266	166	129	171	116
45 MILLSTONE 1 + 2	582	359	215	152	317	547	290
46 MONTICELLO	67	45	45	106	256	90	42

TABLE D.1-2 (cont'd)

SITE	0-5	0-10	0-20	0-30	0-50	0-100	0-200
47 NINE M. PT. 1 + 2	29	119	67	70	107	86	72
48 NORTH ANNA 1, 2, + 3	12	24	28	44	109	165	162
49 OCONEE 1, 2 + 3	42	142	87	129	93	81	91
50 OYSTER CREEK	76	117	139	160	419	761	301
51 PALISADE	70	97	93	74	128	349	198
52 PALO VERDE 1	6	7	8	7	81	34	14
53 PEACH BOTTOM 2 + 3	44	83	205	292	527	452	311
54 PEBBLE SPRINGS	5	3	1	2	10	14	40
55 PERKINS	79	102	178	219	189	119	88
56 PERRY 1	224	228	190	249	329	183	173
57 PHIPPS BEND	82	63	112	104	87	80	89
58 PILGRIM 1	119	94	122	280	548	220	201
59 POINT BEACH 1 + 2	30	67	64	77	73	85	126
60 PRAIRIE 1 + 2	60	65	55	88	261	100	51
61 QUAD CITIES 1 + 2	18	53	248	153	85	85	134
62 RANCHO SECO	22	27	107	321	175	201	63
63 RIVERBEND 1	49	68	81	134	76	88	47
64 H. B. ROBINSON 2	97	80	58	67	73	92	74
65 SAINT LUCIE 1	71	138	60	43	42	54	42
66 SALEM 1 + 2	45	88	272	314	611	460	302
67 SAN ONOFRE	18	82	158	144	456	350	96
68 SEABROOK 1	120	96	91	76	202	147	49
69 SEQUOYAH 1 + 2	108	113	255	153	88	83	87
70 SHEARON HARRIS	23	58	141	176	133	106	82
71 SHOREHAM	135	144	296	602	664	702	305
72 SKAGIT	49	51	38	54	47	67	23
73 SOUTH TEXAS	0	7	21	15	22	76	42
74 VIRGIL C. SUMMER	1	33	43	127	89	105	89
75 SURRY ST 1 + 2	26	196	188	191	204	81	104
76 SUSQUEHANNA 1	188	144	284	225	191	331	348
77 THREE MILE ISLAND	320	433	483	352	234	438	321
78 TROJAN	104	174	81	65	145	72	37
79 TURKEY POINT 1 + 2	0	123	165	316	211	72	24
80 VERMONT YANKEE 1	102	84	95	80	168	314	255
81 VOGTLE	0	6	21	99	58	58	73
82 WATERFORD 3	181	135	245	381	195	79	40
83 WATTS BAR 1 + 2	22	29	53	61	87	68	94
84 WPPSS1+4	0	4	53	36	23	16	36
85 WPPSS 3 + 5	28	25	41	48	48	77	34
86 WPPSS 2	0	4	47	36	23	16	36
87 WOLF CREEK	34	11	10	22	21	78	46
88 YANKEE ROWE	12	69	81	107	202	283	267
89 YELLOW CREEK	15	28	39	37	44	60	64
90 ZIMMER 1	53	78	172	422	232	175	122
91 ZION	538	657	424	457	888	369	154

TABLE D.1-3

POPULATION DENSITIES (PEOPLE PER SQ. MI.) IN  
MOST POPULATED 22.5° SECTOR OF EACH ANNULUS

SITE	0-5MI	5-10MI	10-20MI	20-30MI
1 ALLENS CREEK	209.4	182.3	130.8	153.1
2 ARKANSAS 1 + 2	364.2	676.5	112.0	69.4
3 BAILLY S	1123.1	1650.5	4113.3	9294.1
4 BEAVER VALLEY 1 + 2	1073.8	2108.9	1003.9	6199.0
5 BELLEVILLE 1	199.6	420.6	89.1	79.7
6 BIG ROCK POINT	716.9	48.9	160.5	28.7
7 BLACK FOX	267.3	81.0	1148.5	2232.1
8 BRAIDWOOD 1	619.3	283.1	409.3	1462.9
9 BROWNS FERRY 1, 2, + 3	189.1	814.7	502.6	730.4
10 BRUNSWICK 1 + 2	452.3	112.9	809.6	254.8
11 BYRON 1	356.3	173.8	2191.8	355.3
12 CALLAWAY	129.8	57.3	161.3	557.6
13 CALVERT CLIFF 1 + 2	293.4	240.3	220.0	171.7
14 CATAWBA 1	263.0	1613.2	2719.9	607.2
15 CHEROKEE	276.9	981.9	448.0	807.3
16 CLINTON	107.8	287.3	83.1	1001.1
17 COMMANCHE PEAK	316.6	88.5	29.2	183.6
18 COOK DC 1 + 2	335.3	1053.0	474.3	1930.4
19 COOPER S	54.2	108.6	63.1	83.7
20 CRYSTAL RIVER	235.3	164.5	51.7	52.6
21 DAVIS-BE 1	337.6	313.3	417.2	2358.0
22 DIABLO CANYON 1 + 2	0.0	175.2	566.8	295.7
23 DRESDEN 2 + 3	332.7	359.6	2023.6	1093.6
24 DUANE ARNOLD	269.1	2488.4	102.4	86.2
25 FARLEY 1 + 2	160.7	134.5	619.9	46.1
26 FERMI 2	586.9	1364.6	2637.4	6556.7
27 FITZPATRICK	468.3	1758.1	310.2	599.6
28 FORKED RIVER 1	458.6	858.5	847.5	1029.9
29 FORT CALHOUN	976.8	239.0	3212.8	1593.9
30 FORT ST VRAIN	139.1	120.7	574.2	965.4
31 R. E. GINNA	692.2	515.3	5883.2	700.6
32 GRAND GULF 1	207.8	168.7	60.7	301.1
33 HADDEM NECK	789.6	881.2	1725.3	2730.1
34 HARTSVILLE	456.9	79.6	274.1	160.2
35 HATCH, E.I. 1 + 2	210.4	112.9	136.1	61.5
36 INDIAN PT 2 + 3	2513.7	1916.9	2363.0	14617.9
37 KEWAUNEE	225.1	197.0	814.8	1292.6
38 LASALLE 1 + 2	122.2	192.5	383.3	337.7
39 LA CROSSE	148.3	68.0	891.6	160.7
40 LIMERICK 1	4232.5	1340.1	2167.5	12296.5
41 MARBLE HILL	649.0	166.2	2318.0	3443.4
42 ME YANKEE	0.0	50.9	218.8	683.2
43 MCGUIRE 1 + 2	388.5	425.8	3096.1	433.5
44 MIDLAND 2	2006.6	276.6	2221.0	304.1
45 MILLSTONE 1 + 2	3739.0	1369.8	865.4	251.1
46 MONTICELLO	456.3	190.9	98.2	621.0

TABLE D.1-3 (cont'd)

SITE	0-5MI	5-10MI	10-20MI	20-30MI
47 NINE M. PT. 1 + 2	468.3	1758.1	310.2	599.6
48 NORTH ANNA 1, 2, + 3	187.2	98.5	57.2	294.6
49 OCONEE 1, 2 + 3	215.1	821.7	277.7	920.6
50 OYSTER CREEK	458.6	858.5	847.5	1029.9
51 PALISADE	415.8	460.0	944.2	220.5
52 PALO VERDE 1	69.7	53.2	75.4	88.1
53 PEACH BOTTOM 2 + 3	290.1	255.2	1292.9	1092.9
54 PEBBLE SPRINGS	76.4	21.2	5.3	8.6
55 PERKINS	458.8	314.9	675.8	810.2
56 PERRY 1	811.4	1561.6	899.0	3837.3
57 PHIPPS BEND	265.9	287.9	915.8	557.4
58 PILGRIM 1	886.6	611.8	413.4	1773.1
59 POINT BEACH 1 + 2	355.1	876.7	617.3	625.4
60 PRAIRIE 1 + 2	280.3	596.8	219.0	866.5
61 QUAD CITIES 1 + 2	109.8	240.1	1937.6	383.8
62 RANCHO SECO	348.6	101.5	573.9	3087.3
63 RIVERBEND 1	295.8	298.9	440.0	1673.5
64 H. B. ROBINSON 2	525.0	523.0	198.9	262.9
65 SAINT LUCIE 1	947.7	1350.3	221.0	303.3
66 SALEM 1 + 2	626.6	601.1	2014.0	1568.1
67 SAN ONOFRE	280.9	887.1	1061.9	1252.7
68 SEABROOK 1	540.7	469.8	548.7	453.3
69 SEQUOYAH 1 + 2	294.2	372.0	1900.2	274.7
70 SHEARON HARRIS	190.5	242.8	721.1	1106.3
71 SHOREHAM	805.7	816.3	1589.7	3219.4
72 SKAGIT	288.3	525.8	207.1	502.3
73 SOUTH TEXAS	0.0	61.4	265.7	53.3
74 VIRGIL C. SUMMER	17.7	99.8	206.9	1956.7
75 SURRY ST 1 + 2	244.5	1751.9	1320.4	1521.0
76 SUSQUEHANNA 1	1309.7	561.9	2560.7	869.8
77 THREE MILE ISLAND	2157.0	2319.5	1622.8	1158.4
78 TROJAN	365.9	2151.1	176.8	582.6
79 TURKEY POINT 1 + 2	0.0	1689.1	2107.5	4119.7
80 VERMONT YANKEE 1	507.7	532.1	361.2	350.6
81 VOGTLE	0.0	74.4	76.9	991.7
82 WATERFORD 3	880.3	452.7	3399.3	5068.1
83 WATTS BAR 1 + 2	203.1	98.3	248.0	163.3
84 WPPSS1+4	0.0	95.1	581.8	158.1
85 WPPSS 3 + 5	453.7	193.3	540.7	225.5
86 WPPSS 2	0.0	95.1	538.3	197.7
87 WOLF CREEK	427.6	21.5	16.8	225.3
88 YANKEE ROWE	95.5	705.1	286.1	670.6
89 YELLOW CREE..	132.2	101.6	262.6	102.3
90 ZIMMER 1	325.9	180.0	949.5	5331.2
91 ZION	2040.9	4367.4	1665.5	3344.7

TABLE D.1-4

POPULATION DENSITIES (PEOPLE PER SQ. MI.) IN  
MOST POPULATED 22.5° SECTOR OF EACH CIRCLE

SITE	0-5MI	0-10MI	0-20MI	0-30MI
1 ALLENS CREEK	209.4	136.7	98.1	128.6
2 ARKANSAS 1 + 2	364.2	598.4	194.6	125.1
3 BAILLY S	1123.1	1355.6	3423.9	5163.4
4 BEAVER VALLEY 1 + 2	1073.8	1594.2	903.2	3845.3
5 BELLEVONTE 1	199.6	335.6	107.7	80.5
6 BIG ROCK POINT	716.9	215.9	132.2	66.2
7 BLACK FOX	267.3	66.8	861.4	1622.9
8 BRAIDWOOD 1	619.3	218.8	316.7	878.7
9 BROWNS FERRY 1, 2, + 3	189.1	611.1	529.7	427.3
10 BRUNSWICK 1 + 2	452.3	113.1	607.2	411.4
11 BYRON 1	356.3	162.6	1656.5	889.0
12 CALLAWAY	129.8	43.0	129.7	341.3
13 CALVERT CLIFF 1 + 2	293.4	229.0	210.1	109.3
14 CATAWBA 1	263.0	1209.9	2075.7	1259.9
15 CHEROKEE	276.9	736.4	361.2	501.1
16 CLINTON	107.8	215.5	72.2	572.2
17 COMMANCHE PEAK	316.6	79.1	38.5	102.0
18 COOK DC 1 + 2	335.3	867.9	572.7	1141.4
19 COOPER S	54.2	90.3	63.3	56.5
20 CRYSTAL RIVER	235.3	123.4	53.5	41.4
21 DAVIS-BE 1	337.6	238.7	327.8	1367.2
22 DIABLO CANYON 1 + 2	0.0	131.4	441.6	201.4
23 DRESDEN 2 + 3	332.7	269.7	1538.2	876.8
24 DUANE ARNOLD	269.1	1922.2	505.8	241.8
25 FARLEY 1 + 2	160.7	100.8	475.8	231.4
26 FERMI 2	586.9	1073.2	2069.3	4507.6
27 FITZPATRICK	468.3	1318.6	362.0	365.6
28 FORKED RIVER 1	458.6	758.5	825.3	939.0
29 FORT CALHOUN	976.8	244.2	2417.8	1960.0
30 FORT ST VRAIN	139.1	90.6	430.7	553.9
31 R. E. GINNA	692.2	386.5	4507.8	2392.7
32 GRAND GULF 1	207.8	178.5	51.8	183.1
33 HADDEM NECK	789.6	660.9	1439.7	2009.7
34 HARTSVILLE	456.9	114.2	205.6	155.2
35 HATCH, E.I. 1 + 2	210.4	84.7	102.1	61.2
36 INDIAN PT 2 + 3	2513.7	1627.5	2161.0	8684.2
37 KEWAUNEE	225.1	147.7	618.5	735.8
38 LASALLE 1 + 2	122.2	144.4	301.9	228.0
39 LA CROSSE	148.3	53.7	682.1	392.5
40 LIMERICK 1	4232.5	1343.5	1758.1	7511.8
41 MARBLE HILL	649.0	184.6	1753.1	2692.1
42 ME YANKEE	0.0	38.1	173.6	404.3
43 MCGUIRE 1 + 2	388.5	319.4	2386.1	1301.3
44 MIDLAND 2	2006.6	549.1	1718.5	911.8
45 MILLSTONE 1 + 2	3739.0	1962.1	877.7	485.5
46 MONTICELLO	456.3	143.2	86.2	368.5

TABLE D.1-4 (cont'd)

SITE	0-5MI	0-10MI	0-20MI	0-30MI
47 NINE M. PT. 1 + 2	468.3	1318.6	362.0	365.6
48 NORTH ANNA 1, 2, + 3	187.2	73.9	47.0	178.5
49 OCONEE 1, 2 + 3	215.1	629.3	235.0	611.8
50 OYSTER CREEK	458.6	758.5	825.3	939.0
51 PALISADE	415.8	448.9	741.4	452.0
52 PALO VERDE 1	69.7	57.3	56.5	74.1
53 PEACH BOTTOM 2 + 3	290.1	191.4	969.7	841.6
54 PEBBLE SPRINGS	76.4	19.1	6.4	5.2
55 PERKINS	458.8	291.7	529.3	651.1
56 PERRY 1	811.4	1276.5	993.4	2573.3
57 PHIPPS BEND	265.9	215.9	688.3	374.3
58 PILGRIM 1	886.6	584.3	456.1	1155.4
59 POINT BEACH 1 + 2	355.1	657.6	627.4	362.0
60 PRAIRIE 1 + 2	280.3	496.1	171.2	557.5
61 QUAD CITIES 1 + 2	109.8	180.1	1456.3	860.5
62 RANCHO SECO	348.6	146.1	430.4	1814.3
63 RIVERBEND 1	295.8	231.9	335.1	1078.7
64 H. B. ROBINSON 2	525.0	523.5	280.0	270.5
65 SAINT LUCIE 1	947.7	1012.7	419.0	230.7
66 SALEM 1 + 2	626.6	450.8	1511.5	1543.0
67 SAN ONOFRE	280.9	735.6	796.4	951.5
68 SEABROOK 1	540.7	352.3	475.5	344.2
69 SEQUOYAH 1 + 2	294.2	283.0	1456.0	799.7
70 SHEARON HARRIS	190.5	182.1	580.4	647.5
71 SHOREHAM	805.7	813.7	1289.1	2361.5
72 SKAGIT	288.3	451.5	201.2	301.2
73 SOUTH TEXAS	0.0	46.0	199.3	98.3
74 VIRGIL C. SUMMER	17.7	74.9	173.9	1091.1
75 SURRY ST 1 + 2	244.5	1313.9	1318.8	1164.1
76 SUSQUEHANNA 1	1309.7	748.9	1979.1	1362.8
77 THREE MILE ISLAND	2157.0	1758.2	1656.6	824.4
78 TROJAN	365.9	1618.7	480.5	382.6
79 TURKEY POINT 1 + 2	0.0	966.8	1628.8	2316.4
80 VERMONT YANKEE 1	507.7	526.0	270.9	261.4
81 VOGTLE	0.0	55.8	57.7	559.2
82 WATERFORD 3	880.3	426.9	2618.1	3979.2
83 WATTS BAR 1 + 2	203.1	124.5	186.0	127.9
84 WPPSS1+4	0.0	71.4	436.3	281.7
85 WPPSS 3 + 5	453.7	145.0	405.5	196.8
86 WPPSS 2	0.0	71.4	403.7	289.3
87 WOLF CREEK	427.6	123.0	39.7	129.5
88 YANKEE ROWE	95.5	528.8	223.7	464.0
89 YELLOW CREEK	132.2	76.2	213.0	107.0
90 ZIMMER 1	325.9	162.0	747.0	3264.5
91 ZION	2040.9	3779.5	1724.0	2349.3

## D.2 Exclusion Distances

Table D.2-1 presents the distance to the closest boundary of the exclusion zone surrounding each of the 91 reactor sites, discussed in Chapter 2 and Appendix A. The variability of these distances is displayed in Figure 3-2 in Chapter 3.

TABLE D.2-1  
EXCLUSION DISTANCES (MILES) FOR 91 REACTOR SITES

SITE	EX. DIST.
*****	
1 ALLENS CREEK	0.82
2 ARKANSAS 1 + 2	0.65
3 BAILLY S	0.12
4 BEAVER VALLEY 1 + 2	0.38
5 BELLEVILLE 1	0.57
6 BIG ROCK POINT	0.51
7 BLACK FOX	0.53
8 BRAIDWOOD 1	0.28
9 BROWNS FERRY 1, 2, + 3	0.76
10 BRUNSWICK 1 + 2	0.57
11 BYRON 1	0.29
12 CALLAWAY	0.68
13 CALVERT CLIFF 1 + 2	0.71
14 CATAWBA 1	0.47
15 CHEROKEE	0.37
16 CLINTON	0.61
17 COMMANCHE PEAK	0.87
18 COOK DC 1 + 2	0.38
19 COOPER S	0.46
20 CRYSTAL RIVER	0.83
21 DAVIS-BE 1	0.39
22 DIABLO CANYON 1 + 2	0.50
23 DRESDEN 2 + 3	0.42
24 DUANE ARNOLD	0.27
25 FARLEY 1 + 2	0.78
26 FERMI 2	0.57
27 FITZPATRICK	0.61
28 FORKED RIVER 1	0.38
29 FORT CALHOUN	0.23
30 FORT ST VRAIN	0.37
31 R. E. GINNA	0.28
32 GRAND GULF 1	0.47
33 HADDEMEYER	0.33
34 HARTSVILLE	0.76
35 HATCH, E.I. 1 + 2	0.78
36 INDIAN PT 2 + 3	0.21
37 KEWAUNEE	0.75
38 LASALLE 1 + 2	0.32
39 LA CROSSE	0.21
40 LIMERICK 1	0.47
41 MARBLE HILL	0.42
42 ME YANKEE	0.38
43 MCGUIRE 1 + 2	0.47
44 MIDLAND 2	0.31
45 MILLSTONE 1 + 2	0.31

TABLE D.2-1 (cont'd)

SITE	EX. DIST.
*****	
46 MONTICELLO	0.30
47 NINE M. PT. 1 + 2	0.97
48 NORTH ANNA 1, 2, + 3	0.84
49 OCONEE 1, 2 + 3	1.00
50 OYSTER CREEK	0.25
51 PALISADE	0.42
52 PALO VERDE 1	0.56
53 PEACH BOTTOM 2 + 3	0.51
54 PEBBLE SPRINGS	0.49
55 PERKINS	0.37
56 PERRY 1	0.57
57 PHIPPS BEND	0.47
58 PILGRIM 1	0.27
59 POINT BEACH 1 + 2	0.75
60 PRAIRIE 1 + 2	0.44
61 QUAD CITIES 1 + 2	0.24
62 RANCHO SECO	0.40
63 RIVERBEND 1	0.57
64 H. B. ROBINSON 2	0.26
65 SAINT LUCIE 1	0.97
66 SALEM 1 + 2	0.72
67 SAN ONOFRE	0.50
68 SEABROOK 1	0.57
69 SEQUOYAH 1 + 2	0.36
70 SHEARON HARRIS	1.33
71 SHOREHAM	0.19
72 SKAGIT	0.38
73 SOUTH TEXAS	0.89
74 VIRGIL C. SUMMER	1.01
75 SURRY ST 1 + 2	0.35
76 SUSQUEHANNA 1	0.35
77 THREE MILE ISLAND	0.38
78 TROJAN	0.41
79 TURKEY POINT 1 + 2	0.79
80 VERMONT YANKEE 1	0.17
81 VOGTLE	0.68
82 WATERFORD 3	0.57
83 WATTS BAR 1 + 2	0.75
84 WPPSS1+4	1.21
85 WPPSS 3 + 5	0.81
86 WPPSS 2	1.21
87 WOLF CREEK	0.75
88 YANKEE ROWE	0.59
89 YELLOW CREEK	0.43
90 ZIMMER 1	0.24
91 ZION	0.57

### D.3 Site Population Factors

Table D.3-1 presents the Site Population Factor ( $SPF_n$ ) and the Wind Rose Weighted Site Population Factor ( $WRSPF_n$ ) for each of the 91 reactor sites discussed in Chapter 2 and Appendix A. For every site, the factors have been calculated for each of the following four distances: 5, 10, 20, and 30 miles. The equations used in these calculations are presented in Section 3.2 of Chapter 3.

Table D.3-1. SITE POPULATION FACTORS (SPF) AND WIND ROSE  
WEIGHTED SITE POPULATION FACTORS (WRSPF)  
FOR 91 REACTOR SITES

SITE NAME	REGION	SPF5	SPF10	SPF20	SPF30	WRSPF5	WRSPF10	WRSPF20	WRSPF30
ALLENS CREEK	SW	.31084E-01	.26170E-01	.27085E-01	.29669E-01	.29167E-01	.28190E-01	.29807E-01	.33529E-01
ARKANSAS 1 + 2	S	.34737E-01	.60184E-01	.48306E-01	.41624E-01	.26405E-01	.60023E-01	.48555E-01	.42815E-01
BAILLY S	NN	.17129E+00	.21447E+00	.33316E+00	.46225E+00	.15890E+00	.24294E+00	.40154E+00	.51750E+00
BEAVER VALLEY 1 + 2	NE	.90963E-01	.25042E+00	.24870E+00	.38618E+00	.76206E-01	.22261E+00	.24205E+00	.34474E+00
BELLEVILLE 1	S	.60386E-01	.72908E-01	.58133E-01	.54642E-01	.68453E-01	.84200E-01	.65040E-01	.59719E-01
BIG ROCK POINT	NN	.32287E-01	.29840E-01	.27861E-01	.23975E-01	.33586E-01	.27221E-01	.27625E-01	.23875E-01
BLACK FOX	SW	.17274E-01	.14603E-01	.55139E-01	.93730E-01	.14052E-01	.13323E-01	.41818E-01	.73356E-01
BRAIDWOOD 1	NN	.13580E+00	.10993E+00	.96233E-01	.11376E+00	.12594E+00	.10149E+00	.88696E-01	.99541E-01
BROWNS FERRY 1, 2, +	S	.79286E-02	.44405E-01	.64548E-01	.70623E-01	.83789E-02	.52503E-01	.78023E-01	.77970E-01
BRUNSWICK 1 + 2	S	.20188E-01	.22260E-01	.32303E-01	.31477E-01	.17567E-01	.22345E-01	.32863E-01	.31882E-01
BYRON 1	NN	.71963E-01	.67722E-01	.11826E+00	.12009E+00	.78011E-01	.73451E-01	.10725E+00	.11010E+00
CALLAWAY	NN	.90153E-02	.10237E-01	.19330E-01	.32205E-01	.51928E-02	.8736E-02	.17588E-01	.28591E-01
CALVERT CLIFF 1 + 2	NE	.19608E-01	.30431E-01	.40544E-01	.42677E-01	.25289E-01	.41027E-01	.55162E-01	.53537E-01
CATAWBA 1	S	.28386E-01	.97801E-01	.20199E+00	.19320E+00	.15367E-01	.58678E-01	.24996E+00	.24078E+00
CHEROKEE	S	.32364E-01	.60843E-01	.74998E-01	.10486E+00	.38406E-01	.82775E-01	.90473E-01	.11931E+00
CLINTON	NN	.19499E-01	.31270E-01	.33278E-01	.62732E-01	.15294E-01	.23542E-01	.29829E-01	.65338E-01
COMMANCHE PEAK	SW	.84912E-02	.12700E-01	.10855E-01	.15435E-01	.20515E-01	.20185E-01	.15354E-01	.11798E-01
COOK DC 1 + 2	NN	.84697E-01	.11303E+00	.11942E+00	.14056E+00	.88946E-01	.10599E+00	.10839E+00	.13656E+00
COOPER S	NN	.1007ME-01	.14811E-01	.16822E-01	.17901E-01	.10219E-01	.14122E-01	.15045E-01	.17808E-01
CRYSTAL RIVER	S	.16346E-01	.22168E-01	.18219E-01	.16187E-01	.29057E-01	.30043E-01	.24442E-01	.21577E-01
DAVIS-BE 1	NN	.32672E-01	.40451E-01	.55738E-01	.12140E+00	.55239E-01	.59H27E-01	.70913E-01	.12767E+00
DIABLO CANYON 1 + 2	N	.0	.98598E-02	.35215E-01	.34517E-01	.0	.57107E-02	.14153E-01	.18607E-01
DRESDEN 2 + 3	NN	.44720E-01	.67169E-01	.11713E+00	.14378E+00	.42523E-01	.70489E-01	.94596E-01	.11579E+00
DUANE ARNOLD	NN	.39515E-01	.12939E+00	.12819E+00	.10952E+00	.31349E-01	.13590E+00	.13519E+00	.11556E+00
FARLEY 1 + 2	S	.11499E-01	.17446E-01	.33556E-01	.32332E-01	.88854E-02	.15052E-01	.28956E-01	.28465E-01
FERMI 2	NN	.15821E+00	.19137E+00	.24531E+00	.44021E+00	.12502E+00	.17859E+00	.18463E+00	.31792E+00
FITZPATRICK	NE	.19642E-01	.68462E-01	.62453E-01	.63600E-01	.18665E-01	.98174E-01	.83389E-01	.82734E-01
FORKED RIVER 1	NE	.80588E-01	.94443E-01	.11249E+00	.12548E+00	.59297E-01	.72795E-01	.88249E-01	.10273E+00
FORT CALHOUN	NN	.73958E-01	.55546E-01	.12552E+00	.14071E+00	.10434E+00	.73081E-01	.20558E+00	.23105E+00
FORT ST VRAIN	N	.73534E-02	.20285E-01	.63296E-01	.86448E-01	.57551E-02	.21997E-01	.60302E-01	.97050E-01
GINNA H.E.	NE	.47184E-01	.72451E-01	.23521E+00	.21771E+00	.46365E-01	.81548E-01	.33809E+00	.31177E+00
GRAND GULF 1	S	.12290E-01	.19601E-01	.19342E-01	.23405E-01	.13523E-01	.22849E-01	.21192E-01	.24940E-01
HADDON NECK	NE	.12231E+00	.14928E+00	.24418E+00	.36523E+00	.95413E-01	.19216E+00	.39105E+00	.52525E+00
HARTSVILLE	S	.21557E-01	.26881E-01	.37927E-01	.39203E-01	.20832E-01	.26524E-01	.36980E-01	.37850E-01
HATCH, E.I. 1 + 2	S	.11122E-01	.14720E-01	.22731E-01	.23489E-01	.12330E-01	.13566E-01	.21401E-01	.22487E-01
INDIAN PT 2 + 3	NE	.81326E+00	.74045E+00	.73557E+00	.98620E+00	.11763E+01	.95346E+00	.87477E+00	.11167E+01
KEWAMIEE	NN	.93780E-02	.17390E-01	.36599E-01	.47946E-01	.16358E-01	.24622E-01	.49717E-01	.72662E-01
LASALLE 1 + 2	NN	.13544E-01	.29233E-01	.55404E-01	.59426E-01	.90269E-02	.24474E-01	.60214E-01	.64040E-01
LA CROSSE	NN	.17126E-01	.19149E-01	.39487E-01	.38589E-01	.18270E-01	.20467E-01	.50259E-01	.48158E-01
LIMERICK 1	NE	.69580E+00	.58125E+00	.59208E+00	.53770E+00	.82582E+00	.65562E+00	.64060E+00	.71140E+00
MARBLE HILL	NN	.52590E-01	.48820E-01	.12305E+00	.18073E+00	.42417E-01	.45750E-01	.14729E+00	.21629E+00
ME YANKEE	NE	.0	.17540E-02	.14157E-01	.23032E-01	.0	.11468E-02	.11221E-01	.19443E-01
MCGUIRE 1 + 2	S	.64527E-01	.89197E-01	.22294E+00	.21853E+00	.55320E-01	.86377E-01	.20439E+00	.1979E+00
MIDLAND 2	NN	.51550E+00	.36272E+00	.32814E+00	.27855E+00	.47273E+00	.33162E+00	.27510E+00	.3171E+00
MILLESTONE 1 + 2	NE	.44527E+00	.39795E+00	.31930E+00	.27479E+00	.38361E+00	.33459E+00	.27492E+00	.24015E+00
MONTELELO	NN	.30455E-01	.36498E-01	.40104E-01	.63248E-01	.35726E-01	.34907E-01	.38814E-01	.66259E-01
NINE M. 1, 2 + 3	NE	.19542E-01	.68452E-01	.62453E-01	.83590E-01	.18934E-01	.95586E-01	.83033E-01	.82225E-01
NORTH ANNA 1, 2, + 3	S	.74517E-02	.15242E-01	.20497E-01	.23285E-01	.17726E-01	.22356E-01	.24040E-01	.29313E-01
OCONEE 1, 2 + 3	S	.20946E-01	.71083E-01	.71072E-01	.87774E-01	.20375E-01	.564069E-01	.56339E-01	.70361E-01

Table D.3-1. (continued)

SITE NAME	REGION	SPF5	SPF10	SPF20	SPF30	WRSPF5	WRSPF10	WRSPF20	WRSPF30
OYSTER CREEK	NE	.8058E-01	.94443E-01	.11249E+00	.12548E+00	.59297E-01	.72795E-01	.88249E-01	.10273E+00
PALISADE	NN	.54980E-01	.74781E-01	.78747E-01	.74770E-01	.64503E-01	.90103E-01	.10466E+00	.10047E+00
PALO VERDE 1	SM	.59341E-02	.57060E-02	.64845E-02	.63884E-02	.72781E-02	.65526E-02	.68225E-02	.67669E-02
PEACH BOTTOM 2 + 3	NE	.21262E-01	.46280E-01	.10392E+00	.15471E+00	.16166E-01	.43229E-01	.10408E+00	.16286E+00
PEBBLE SPRINGS	A	.32039E-02	.26601E-02	.18549E-02	.19379E-02	.10765E-02	.18643E-02	.12494E-02	.16177E-02
PERKINS	S	.56485E-01	.73595E-01	.11950E+00	.14722E+00	.69392E-01	.77501E-01	.12927E+00	.16576E+00
PERRY 1	NN	.18134E+00	.19633E+00	.14713E+00	.20736E+00	.19364E+00	.22503E+00	.21854E+00	.25700E+00
PHIPPS BEND	S	.10524E+00	.84880E-01	.97704E-01	.97858E-01	.14545E+00	.11126E+00	.97714E-01	.90997E-01
PILGRIM 1	NE	.11534E+00	.10936E+00	.11597E+00	.17272E+00	.10559E+00	.10456E+00	.12056E+00	.18916E+00
POINT BEACH 1 + 2	NN	.27477E-01	.42796E-01	.59163E-01	.56737E-01	.30607E-01	.61374E-01	.78181E-01	.84759E-01
PRairie 1 + 2	NN	.52533E-01	.60495E-01	.58239E-01	.68463E-01	.68078E-01	.89779E-01	.81843E-01	.98066E-01
QUAD CITIES 1 + 2	NN	.92684E-02	.28898E-01	.11576E+00	.10374E+00	.74518E-02	.26202E-01	.15572E+00	.14606E+00
RANCHO SECO	A	.11965E-01	.16493E-01	.49437E-01	.14276E+00	.21786E-01	.24511E-01	.62468E-01	.18807E+00
RIVERBEND 1	S	.30502E-01	.43167E-01	.55359E-01	.81505E-01	.26084E-01	.38615E-01	.53917E-01	.83519E-01
H. B. ROBINSON 2	S	.44152E-01	.60479E-01	.56364E-01	.59736E-01	.31658E-01	.40944E-01	.40993E-01	.46573E-01
SAINT LUCIE 1	S	.54634E-01	.83901E-01	.69659E-01	.61506E-01	.26940E-01	.57388E-01	.50326E-01	.43984E-01
SALEM 1 + 2	NE	.20414E-01	.44992E-01	.12554E+00	.17034E+00	.10118E-01	.37371E-01	.13494E+00	.17374E+00
SAN ONOFRE	A	.69002E-02	.48712E-01	.84849E-01	.96513E-01	.66242E-02	.33136E-01	.69350E-01	.73546E-01
SEABROOK 1	NE	.67564E-01	.70954E-01	.75767E-01	.74094E-01	.51712E-01	.53509E-01	.61434E-01	.60380E-01
SEDNOYAH 1 + 2	S	.74540E-01	.92644E-01	.15438E+00	.14585E+00	.10185E+00	.99429E-01	.24659E+00	.22043E+00
SHEARON HARRIS	S	.19205E-01	.32954E-01	.71277E-01	.10028E+00	.18313E-01	.27659E-01	.62005E-01	.89401E-01
SKAGIT	A	.34859E-01	.43992E-01	.42098E-01	.47567E-01	.55447E-01	.72359E-01	.61431E-01	.61842E-01
SHOREHAM	NE	.16493E+00	.15862E+00	.22164E+00	.35388E+00	.14089E+00	.14828E+00	.23875E+00	.39059E+00
SOUTH TEXAS	SM	0	.52669E-02	.11954E-01	.11540E-01	0	.31599E-02	.11627E-01	.10927E-01
VIRGIL C. SUMMER	S	.50986E-03	.16901E-01	.25477E-01	.59106E-01	.55440E-03	.16344E-01	.27235E-01	.54535E-01
SURRY ST 1 + 2	S	.11499E-01	.10123E+00	.12692E+00	.14067E+00	.14834E-01	.98112E-01	.11710E+00	.12781E+00
SUSQUEHANNA 1	NE	.88449E-01	.10759E+00	.17990E+00	.17990E+00	.13817E+00	.14840E+00	.14670E+00	.18442E+00
THREE AILE ISLAND	NE	.22949E+00	.31201E+00	.39992E+00	.37154E+00	.19179E+00	.30919E+00	.42313E+00	.39465E+00
TROJAN	A	.60039E-01	.10794E+00	.88903E-01	.80567E-01	.68927E-01	.18613E+00	.15456E+00	.14013E+00
TURKEY POINT 1 + 2	S	0	.53684E-01	.94549E-01	.16790E+00	0	.44722E-01	.72504E-01	.10295E+00
VERMONT YANKEE 1	NE	.95964E-01	.94227E-01	.95035E-01	.88480E-01	.10817E+00	.14159E+00	.11733E+00	.10599E+00
VOGTL	S	0	.36768E-02	.10877E-01	.39824E-01	0	.34954E-02	.10971E-01	.29954E-01
WATERFORD 3	S	.16326E+00	.14243E+00	.13643E+00	.25577E+00	.14376E+00	.14389E+00	.15424E+00	.20717E+00
WATTS BAR 1 + 2	S	.15094E-01	.22252E-01	.34829E-01	.41281E-01	.10158E-01	.17324E-01	.24514E-01	.37329E-01
WPPSS1+4	A	0	.33914E-02	.25414E-01	.24944E-01	0	.14139E-02	.33159E-01	.31451E-01
WPPSS 2	A	0	.26920E-02	.22559E-01	.23771E-01	0	.11239E-02	.29499E-01	.30428E-01
WPPSS 3 + 5	A	.11904E-01	.18371E-01	.27064E-01	.31975E-01	.14094E-01	.15227E-01	.19507E-01	.25535E-01
WOLF CREEK	NN	.16991E-01	.12359E-01	.11440E-01	.15201E-01	.85718E-02	.66541E-02	.80477E-02	.13311E-01
YANKEE ROWE	A	.12403E-01	.35226E-01	.51955E-01	.67440E-01	.15425E-01	.30277E-01	.44781E-01	.50389E-01
YELLOW CREEK	S	.65005E-02	.14111E-01	.22903E-01	.25371E-01	.66200E-02	.16426E-01	.25034E-01	.26547E-01
ZIMMER 1	NN	.27940E-01	.46397E-01	.93655E-01	.20515E+00	.20134E-01	.37355E-01	.19700E-01	.17520E+00
ZION	NN	.71363E+00	.70561E+00	.59157E+00	.55685E+00	.87472E+00	.84040E+00	.68575E+00	.65741E+00

## Appendix E: CRAC 2: A Brief Description

The accident consequence calculations presented in Chapter 2 were performed using CRAC2 [1,2], an improved version of the WASH-1400 consequence model CRAC. A number of modifications were made in the upgrade from CRAC to CRAC2. These include changes in the treatments of atmospheric dispersion parameters, plume rise, precipitation scavenging (wet deposition), mixing heights, weather sequence sampling, emergency response (evacuation and sheltering), and latent cancer risk factors. These changes are briefly described below. In addition, several errors found in CRAC were corrected in the CRAC2 version.

### E.1 Atmospheric Dispersion Parameters

The values of the horizontal dispersion coefficients,  $\sigma_y$ , obtained from the Pasquill-Gifford curves (and parameterized by Tadmore and Gur [3]) correspond to a release duration of three minutes. To correct the standard dispersion coefficients for releases of longer duration, the summary report of the National Commission on Air Quality's Atmospheric dispersion Modeling Panel [4] endorses the method suggested by Gifford [5]. An adjustment for releases of duration  $t_2$  (minutes) is made by means of the formula

$$\frac{\sigma_{y_2}}{\sigma_{y_{PG}}} = \left( \frac{t_2}{3 \text{ min}} \right)^Q$$

where Q is within the range 0.25-0.3 for  $1 \text{ hr} < t_2 < 100 \text{ hr}$  and equals  $\sim 0.2$  for  $3 \text{ min} < t_2 < 1 \text{ hr}$ . In CRAC2, Q is equal to 0.2 for release durations between 3 minutes and one hour and 0.25 for release durations greater than one hour. The lower value of 0.25, rather than 0.3, was selected for long-duration releases because it results in higher concentrations.

The vertical dispersion coefficients,  $\sigma_z$ , obtained from the Pasquill-Gifford curves (parameterized by Martin and Tikvart [6]) are based on data from releases over

terrain with very low surface roughness (grasslands with roughness length of approximately 3 cm). In CRAC2 a more typical roughness length of 10 cm (crops, bushes) is assumed. The vertical dispersion coefficients are adjusted using the following recommended equation [7,8]:

$$\sigma_{z2}/\sigma_{z1} = (r_2/r_1)^{0.2},$$

where  $\sigma_{z1}$  is the unadjusted parameter,  $\sigma_{z2}$  is the adjusted parameter,  $r_1 = 3$  cm, and  $r_2 = 10$  cm. Impacts of these changes in the treatment of dispersion parameters were examined in [9].

#### E.2 Plume Rise

The WASH-1400 consequence model used plume rise equations recommended in Briggs (1969) [10]. The plume rise model used in CRAC2 is based on a more recent paper by Briggs (1975) [11].

#### E.3 Precipitation Scavenging (Wet Deposition)

The WASH-1400 consequence model (CRAC) used weather data which reported rainfall in terms of the incidence or nonincidence of rain within any clock hour. To calculate precipitation scavenging, the model assumed that rain reported for a clock hour fell at a rate of 1 mm/hr for half the hour. The CRAC2 code contains a more sophisticated wet deposition model which requires as input the amount of rain falling in an hour. Rain is assumed to occur during the entire hour with a constant rate. The hourly rainfall rate is multiplied by a rainout coefficient to determine precipitation scavenging. A coefficient of  $1.0 \times 10^{-4} (\text{sec})^{-1}(\text{mm/hr})^{-1}$  is used for stable conditions and  $1.0 \times 10^{-3} (\text{sec})^{-1}(\text{mm/hr})^{-1}$  for neutral and unstable conditions.

#### E.4 Mixing Heights

The WASH-1400 consequence model used Holzworth [12] morning and afternoon mixing heights for all stability conditions. In CRAC2, the treatment is somewhat simplified. For stable conditions (E and F stability), the inversion layer is ground based and no mixing depth

is assumed. For neutral and unstable conditions, the Holzworth afternoon mixing height is assumed. This change has minimal impact on resulting predicted consequences.

#### E.5 Improved Weather Sequence Sampling Technique

WASH-1400's consequence model (CRAC) used a stratified sampling technique by which sequences are selected every four days ± thirteen hours to provide coverage of diurnal, seasonal and four-day weather cycles [13]. In this manner, a total of 91 weather sequences were chosen to represent one year of data (8760 hours). Sensitivity studies have shown that considerable variation in predicted consequences result from sampling by this method. Consequences can vary significantly for calculations performed using different sets of weather sequences (see Figure E5-1A). Differences in peak predicted consequences of an order of magnitude or more are not uncommon.

There are several reasons for the large variation in consequences due to the WASH-1400 sampling technique. Given an accident, large consequences are normally associated with relatively low probability weather conditions such as rainfall within a few 10's of kilometers of the site [14], wind-speed slowdowns, or stable weather conditions with moderate wind speeds. Not only is the occurrence of rainfall or a slowdown important, but where it occurs as well. Rain beginning over a densely populated area could result in extremely high consequences. Because of their low probability, such weather conditions will be selected infrequently, if at all, by the WASH-1400 sampling technique. Furthermore, estimated probabilities for adverse weather conditions can be significantly in error. For example, a particularly adverse weather sequence with actual probability of 1/8760 would, if sampled, be assigned a probability of 1/91.

CRAC2 uses a new weather sequence sampling method [15] which produces improved estimates of accident-consequence frequency distributions. Prior to sequence selection, the entire year of weather data is sorted into 29 weather categories (termed "bins"), as defined in Table E.5-1. Each of the 8760 potential sequences is first examined to determine if rain occurs anywhere within 50 kilometers (30 miles) of the accident site.

If not, a similar examination is made for wind-speed slowdowns. If neither of these conditions occurs, the sequence is categorized by the stability and wind speed at the start of the accident. A probability for each weather bin is estimated from the number of sequences placed in the bin. Sequences are then sampled from each of the bins (with appropriate probabilities) for use in risk calculations. In the current analysis, four sequences were selected from each bin. Sampling with this method assures that low probability adverse weather conditions are adequately included.

A comparison of the variation in consequences due to sampling by the two methods is provided in Figure E.5-1. For both methods, early-fatality frequency distributions (CCDF's) for a PWR2 release [15] were calculated with CRAC, using 32 different sets of weather sequences sampled from the New York City weather data summarized in Table E.5-1. Also assumed were a uniform population density of 100 people/mile<sup>2</sup> and a relatively ineffective evacuation. The results clearly indicate that the weather bin method results in substantially less variation due to sampling than the previous WASH-1400 technique.

#### E.6 Emergency Response (Evacuation) Model

The CRAC2 evacuation model [16,17] is significantly different from the RSS evacuation model. In lieu of the small "effective" evacuation speeds assumed in the RSS model, the revised treatment incorporates a delay time before public movement, followed by evacuation radially away from the reactor. Both an assumed delay time and evacuation speed are required as input to the model. Different shielding factors and breathing rates are used while stationary or in transit. In addition, all persons within the designated evacuation area move as a group with the same delay time and evacuation speed. Therefore, the possibility that some people may not leave the evacuated area is ignored. This latter assumption results in upper bound estimates of evacuation effectiveness, given a specific delay time and speed.\* Unlike the RSS model in which persons continue

\*The evacuation effectiveness would decrease linearly with an increasing nonparticipating fraction of the population. In actual evacuations, Civil Defense personnel have observed a nonparticipating minority of approximately 5%.

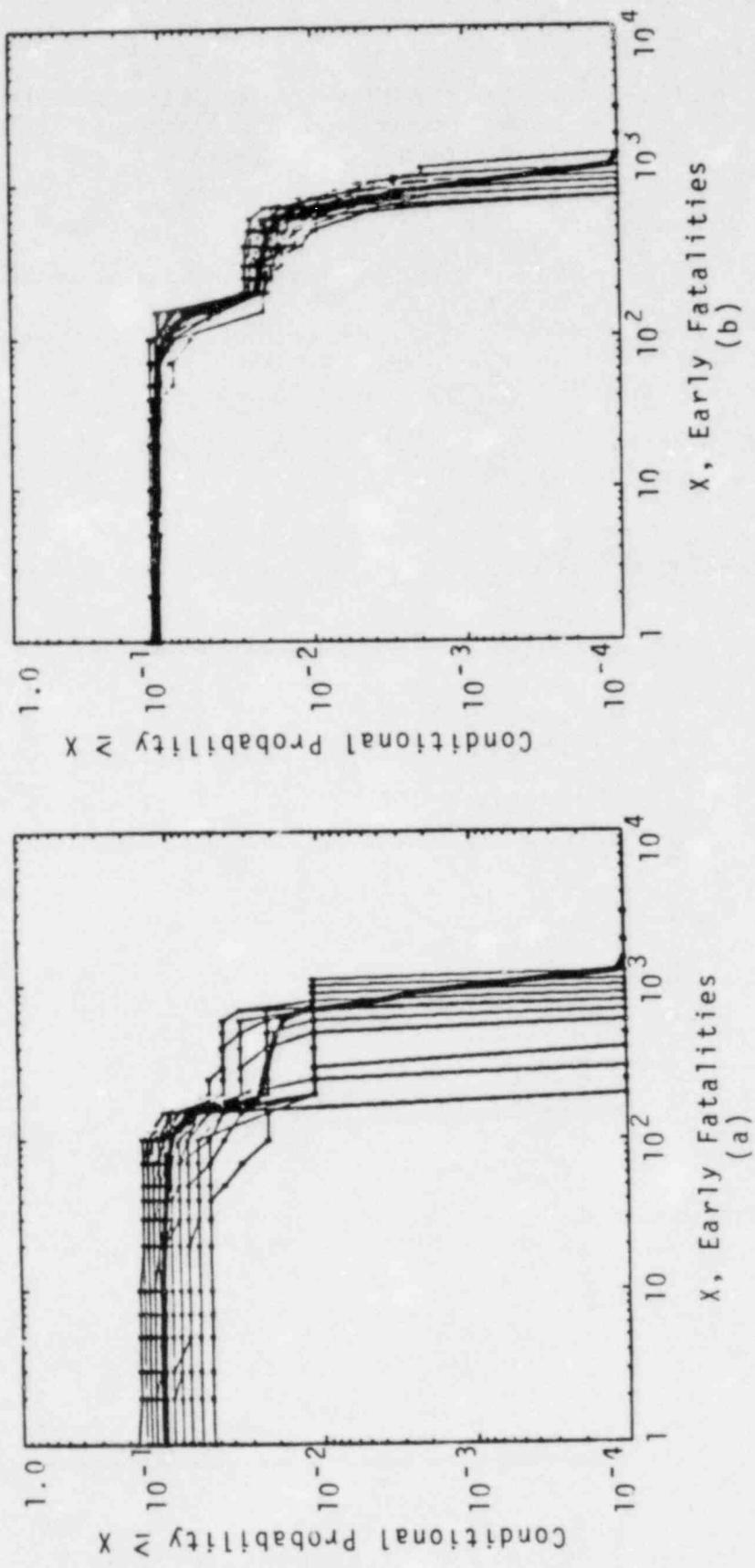


Figure E. 5-1. Comparison of Uncertainty Due to Sampling by (A) WASH-1400 and (B) Weather Bin Techniques. For each technique, 32 different sets of weather sequences are used to generate early-fatality frequency distributions for a PWR2 release. A "best estimate" using all 8760 available sequences, is shown by the darkened line.

Table E.5-1 One Year of New York City Meteorological  
Data Summarized Using Weather Bin  
Categories

Weather Bin Definitions

R - Rain starting within indicated interval  
(miles).

S - Slowdown occurring within indicated  
interval (miles).

A-C D E F - Stability categories.

1(0-1), 2(1-2), 3(2-3), 4(3-5), 5(GT 5) - Wind Speed  
intervals (m/s).

<u>Weather Bin</u>	<u>Number of Sequences</u>	<u>Percent</u>
1 R (0)	697	7.96
2 R (0-5)	12	.14
3 R (5-10)	62	.71
4 R (10-15)	102	1.16
5 R (15-20)	75	.86
6 R (20-25)	67	.76
7 R (25-30)	61	.70
8 S (0-10)	24	.27
9 S (10-15)	16	.18
10 S (15-20)	18	.21
11 S (20-25)	14	.16
12 S (25-30)	18	.21
13 A-C 1,2,3	168	1.92
14 A-C 4,5	892	10.18
15 D 1	0	0.00
16 D 2	61	.70
17 D 3	226	2.58
18 D 4	948	10.82
19 D 5	3325	37.96
20 E 1	0	0.00
21 E 2	27	.31
22 E 3	167	1.91
23 E 4	682	7.79
24 E 5	270	3.08
25 F 1	0	0.00
26 F 2	116	1.32
27 F 3	310	3.54
28 F 4	402	4.59
29 F 5	0	0.00
	8760	100.00

evacuating until they are either overtaken by the cloud or leave the model grid, all evacuating persons in the new model travel a designated distance from the evacuated area and are then removed from the problem. This treatment allows for the likelihood that after traveling outward for some distance, people may learn their position relative to the cloud and be able to avoid it.

The new model also calculates more realistic exposure durations to airborne and ground-deposited radionuclides than the RSS evacuation model. The RSS consequence model employs an exposure model for an instantaneous point source and thus all released plumes have zero effective lengths. Because of this, evacuating persons overtaken by the cloud in the RSS evacuation model are exposed to the entire cloud at the point overtaken. However, a released cloud of radioactive material would have a finite release duration and a length that depends on the wind speed during and following the release. A person overtaken by the front of the cloud might still escape before being passed by the entire cloud and thus receive only a fraction of the full cloud exposure.\* The revised evacuation model assigns the cloud a finite length which is calculated using the assumed release duration and wind speed during the release. To simplify the treatment, the length of the cloud is assumed to remain constant following the release (i.e., the front and back of the cloud travel at the same speed), and the concentration of radioactive material is assumed to be uniform over the length of the cloud. The radial position of evacuating persons, while stationary and in transit, is compared to both the front and the back of the cloud as a function of time to determine a more realistic period of exposure to airborne radionuclides.

The revised treatment calculates the time periods during which people are exposed to radionuclides on the ground while they are stationary and while they

\*It is also possible that an evacuating person may travel under the cloud for a long time and thus receive more exposure than if he had remained stationary during the passage of the cloud.

are evacuating. Because radionuclides would be deposited continually from the cloud as it passed a given location, a person while under the cloud would be exposed to ground contamination less concentrated than than if the cloud had completely passed. To account for this, the new model assumes that persons completely passed by the cloud are exposed to the total ground contamination concentration, calculated to exist after complete passage of the cloud, to one-half the calculated concentration when anywhere under the cloud, and to no concentration when in front of the cloud. A more detailed discussion of the models is provided in [16] and [17].

The CRAC2 model of public evacuation requires as input estimates of the delay time before evacuation commences and the evacuation speed. Reexamination of the EPA evacuation data used to develop the WASH-1400 model [18] show that, if a constant evacuation speed was assumed, a distribution of delay times could be estimated. For assumed evacuation speeds of 10 mph or greater, delay times were found to be satisfactorily represented by a normal distribution with 15, 50, and 85 percentile delay times of approximately 1, 3, and 5 hours respectively.

The CRAC2 evacuation model can incorporate this distribution of evacuation delay times by calculating a 30:40:30% weighted sum of consequences for 10 mph evacuations after delays of 1, 3, and 5 hours. The weighted distribution of evacuations is denoted "Summary Evacuation", and was discussed in Sections 2.2 and 2.5.

The CRAC2 model is also capable of considering population sheltering as an emergency protective action. Sheltering would involve the expedient movement of people into basements or masonry buildings, if possible, followed by relocation. Table A.1-3 of Appendix A lists sheltering factors for different regions in the U.S. A discussion of sheltering is provided in [19].

### E.7 Updated Cancer Risk Factors

The latent cancer fatality risk factors used in CRAC2 are updated versions of those reported in WASH-1400. The RSS factors assumed a latency period during which the risk of cancer was assumed to be zero, followed by a risk period where the individual is assumed to be at a constant risk (risk plateau). Depending on the type of cancer and the age of the exposed individual, the latency periods ranged from 0 to 15 years and the risk periods ranged from 10 to 30 years. Based on recommendations in BEIR III [20], the factors used in CRAC2 were updated to reflect extension of the risk period to the end of an individual's life for all cancers except leukemia and for all age groups (of exposed individuals) other than those exposed in utero. Table E.7-1 compares the updated factors to those from WASH-1400. The 0-1 year factors are used for external exposures.

Table E.7-1 Expected Total Latent Cancer (Excluding Thyroid) Deaths per  
 $10^6$  Man-Rem From Internal Radionuclides Delivered During  
 Specified Periods

WASH-1400

	Time Period (years) After Accident								
	0-1	1-10	11-20	21-30	31-40	41-50	51-60	61-70	71-80
Leukemia	28.4	27.2	18.7	13.8	9.7	6.8	4.0	1.7	0.5
Lung	22.2	22.2	22.2	14.5	8.1	4.0	1.5	0.2	0
GI Tract <sup>(a)</sup>	13.5	13.6	13.6	8.9	5.0	2.5	0.9	0.1	0
Pancreas	3.4	3.4	3.4	2.2	1.3	0.6	0.2	0	0
Breast	25.6	25.6	25.6	16.8	9.4	4.6	1.7	0.3	0
Bone	6.9	6.7	5.0	2.6	1.6	0.9	0.4	0.1	0
All Other	21.6	19.8	17.1	11.2	6.3	3.1	1.2	0.2	0

UPDATED WASH-1400 (CRAC2)

Leukemia	28.4	27.2	18.7	13.8	9.7	6.8	4.0	1.7	0.5
Lung	27.5	27.5	27.5	15.8	8.1	4.0	1.5	0.2	0.0
GI Tract <sup>(a)</sup>	16.9	16.9	16.9	9.7	5.0	2.5	0.9	0.1	0.0
Pancreas	4.2	4.2	4.2	2.4	1.3	0.6	0.2	0.0	0.0
Breast	31.7	31.7	31.7	18.3	9.4	4.6	1.7	0.3	0.0
Bone	11.1	10.6	7.0	3.0	1.7	0.9	0.4	0.1	0.0
All Other	28.0	26.3	21.1	12.2	6.3	3.0	1.2	0.2	0.0

### References for Appendix E

1. Ritchie, L. T., J. D. Johnson, and R. M. Blond, Calculations of Reactor Accident Consequences, Version 2: User's Guide, NUREG/CR-2326, SAND81-1994, Sandia National Laboratories (to be published).
2. Ritchie, et al., Calculations of Reactor Accident Consequences Version 2: CRAC2 Model Description, NUREG/CR-2552, SAND82-0342, Sandia National Laboratories (to be published).
3. Tadmor, J. and Y. Gur, "Analytical Expressions for the Vertical and Lateral Dispersion Coefficients in Atmospheric Diffusion," Atmos. Environ. 3: 688-689 (1969).
4. Summary Report of the NCAQ Atmospheric Dispersion Modeling Panel, Vol. 1, Recommendations (Final Report), M. W. Chandler, et al., Dames and Moore, Bethesda, MD, PB80-174964, pg. 37 (1980)
5. Gifford, F., Atmospheric Dispersion Models for Environmental Pollution Applications, Lectures on Air Pollution and Environmental Impact Analyses, American Meteorological Society, Boston, pg. 42 (1975).
6. Martin, D. O., and J. A. Tikvart, "A General Atmospheric Diffusion Model for Estimating the Effects on Air Quality of One or More Sources," paper presented at 61st annual meeting of the Air Pollution Control Association (1968)
7. "AMS Workshop on Stability Classification Schemes and Sigma Curves - Summary of Recommendations," Bulletin American Meteorological Society, Vol. 58, #12 (December 1977).
8. Hoffman, F. Owen (General Chairman), Proceedings of a Workshop on the Evaluation of Models Used for the Environmental Assessment of Radionuclide Releases (Gatlinburg, TN, September 6-9, 1977), CONF-770901, Oak Ridge National Laboratory (April 1978).

9. Aldrich, D. C., Impact of Dispersion Parameters on Calculated Reactor Accident Consequences, SAND 79-2081, NUREG/CR-1150, Sandia Laboratories, Albuquerque, NM March (1980).
10. Briggs, G. A., Plume Rise, USAEC Critical Review Series, TID-25075 (1969).
11. Briggs, G. A., "Plume Rise Predictions" in Lectures on Air Pollution and Environmental Impact Analysis, D. A. Haugen, ed., American Meteorological Society, Boston, MA, pp 59-105 (1975).
12. Holzworth, G. C., Mixing Heights, Wind Speeds, and Potential for Urban Air Pollution Throughout the Contiguous United States, Publ. No. AP-101, U.S. Environmental Protection Agency, Office of Air Programs, Research Triangle Park, N. C. (1972).
13. Reactor Safety Study Appendix VI: Calculations of Reactor Accident Consequences, WASH-140C, (NUREG 75/014), U. S. Nuclear Regulatory Commission (October 1975).
14. Ritchie, L. T., W. D. Brown and J. R. Wayland, Effects of Rainstorms and Runoff on Consequences of Nuclear Reactor Accidents, SAND79-0379, Sandia Laboratories, Albuquerque, NM (February 1980).
15. Ritchie, L. T., D. C. Aldrich, and R. M. Blond, "Weather Sequence Sampling for Risk Calculations", Transactions of the American Nuclear Society (June 1981).
16. Aldrich, D. C., R. M. Blond and R. B. Jones, A Model of Public Evacuation for Atmospheric Radiological Releases, SAND78-0092, Sandia Laboratories, Albuquerque, NM (1978).
17. Aldrich, D. C., L. T. Ritchie and J. L. Sprung, Effect of Revised Evacuation Model on Reactor Safety Study Accident Consequences, SAND79-0095, Sandia Laboratories, Albuquerque, NM (1979).

18. Hans, J. M., Jr. and T. C. Sell, Evacuation Risks - An Evaluation, U. S. Environmental Protection Agency, EPA-520/6-74-002 (1974).
19. Aldrich, D. C., D. M. Ericson, and J. D. Johnson, Public Protection Strategies for Potential Nuclear Reactor Accidents: Sheltering Concepts with Existing Public and Private Structures, SAND77-1725, Sandia Laboratories, Albuquerque, NM (1978).
20. BEIR (Committee on the Biological Effects of Ionizing Radiation), The Effects on Populations of Exposures to Low Levels of Ionizing Radiation, National Academy of Sciences, Washington, D.C. (1980).

## Appendix F: Site Availability Maps and Tables

This appendix contains the site availability data that was discussed in Chapter 4.0. Figure F1 shows legally protected and wetland areas in the U. S. where reactor siting would be restricted. Seismic acceleration contours are shown in Figure F2. Figure F4 shows the topographic character of the U. S. in terms of percent land that is gently sloping (gently sloping was defined as less than 8% slope). Figures F3, F5, F6, and F7 show seismic hardening costs, surface, water availability costs, groundwater availability costs, and combined water availability costs (the lesser of surface water and groundwater costs) for the 48 contiguous United States. Associated with these costs are the utility values discussed in Section 4.4.1 of Chapter 4.0. Tables F1.1-F1.5 show the fractions of land, by state, that fall within each of the environmental suitability categories shown in Figures F3-F7.

Figures F8.1-F8.13 show land that would be restricted from reactor siting by standoff distances to cities. The cities and standoff distances considered in each figure are tabulated below.

Figure	Standoff Distance (mile)	Cities (Population $\geq$ )
F8.1	5	25,000
F8.2	10	25,000
F8.3	10	100,000
F8.4	15	100,000
F8.5	25	100,000
F8.6	25	200,000
F8.7	30	200,000
F8.8	40	200,000
F8.9	50	200,000
F8.10	100	200,000
F8.11	125	250,000
F8.12	18	500,000
F8.13	25	1,000,000

Figures F8.11, F8.12, and F8.13 show the restricted areas for the Northeastern U. S. only.

Figures F9.1-F9.26 show areas that would be restricted from reactor siting by population density criteria. These criteria restrict the number of people that can reside in an annulus surrounding a reactor site. The population density restrictions and the annuli considered in each figure are tabulated below. The population restrictions are shown in terms of average population density (people within the annulus/annulus area).

Figure	Radii of the Annulus (mile)	Average Population Density (people/mile <sup>2</sup> )
F9.1	0-2	100
F9.2	0-2	250
F9.3	0-2	500
F9.4	0-2	750
F9.5	0-5	100
F9.6	0-5	200
F9.7	0-5	350
F9.8	0-5	500
F9.9	0-10	100
F9.10	0-10	200
F9.11	0-10	350
F9.12	0-10	500
F9.13	0-20	200
F9.14	0-30	500
F9.15	0-30	1000
F9.16	5-10	150
F9.17	5-10	350
F9.18	5-10	500
F9.19	5-20	800
F9.20	10-20	400
F9.21	10-20	500
F9.22	10-20	1000
F9.23	20-30	500
F9.24	20-30	1000
F9.25	30-50	500
F9.26	30-50	1000

Figures 9.3 and 9.4 show restricted areas for the Northeastern U. S. only.

Figures F10.1-F10.4 show areas in the NE U. S. that would be restricted from siting by composite density criteria between 2 and 30 miles of a prospective site. Each criterion would simultaneously restrict the mean

population densities within six annuli: 2-3 miles, 2-4 miles, 2-5 miles, 2-10 miles, 2-20 miles, and 2-30 miles. The mean population densities in each of the six annuli can not exceed the prescribed density limits for the site to be acceptable. Figures F10.1, F10.2, F10.3 and F10.4 consider density restrictions of 500, 750, 1000, and 1500 people/mile<sup>2</sup>, respectively for the Northeastern U. S.

Figures F11 and F12 show areas in the 48 contiguous United States that would be restricted from reactor siting by the combination of a population density restriction within two miles and a composite population density restriction between 2 and 30 miles of the site. Figure F11 considers a population density restriction of 100 people/mile<sup>2</sup> within 2 miles and a composite population density of 500 people/mile<sup>2</sup>. Figure F12 is based on a 250 people/mile<sup>2</sup> density restriction within 2 miles and a composite population density restriction (2-30 miles) of 500 people/mile<sup>2</sup>. The 2-30 mile composite restriction is as defined for Figures F10.1-F10.4.

Tables F2.1-F2.24 show the fractions of land available for reactor siting in each state if sector population restrictions are added to a composite population density criterion. These restrictions would limit the number of people that could reside within any sector in each of the composite annuli (see Section 4.5.4 of Chapter 4.0). For these tables, five annuli were considered: 0-2 miles, 0-5 miles, 0-10 miles, 0-20 miles, and 0-30 miles. The allowable populations in each annuli were calculated assuming 250 people/mile<sup>2</sup> between zero and two miles and from 250 to 1500 people/mile<sup>2</sup> in the two to thirty mile region. An acceptable site must satisfy the sector population restriction for each of the composite annuli. The sector population restrictions (fraction of annulus population allowed within the sector), sector widths, and the 2-30 mile average population densities (people within an annulus/annulus area) considered in each table are given below. Tables F2.1-F2.12 show the land areas that are uniquely restricted by the specified criterion. Tables F2.13-F2.24 show the fraction of land available for reactor siting based on the specified criterion.

Table	Width	Sector Population Restrictions							Population Density (2-30 miles) (people/mile <sup>2</sup> )
		1/16	1/8	1/6	1/4	1/3	1/2	1	
F2.1 & F2.13	22.5°	1/16	1/8	1/6	1/4	1/3	1/2	1	250
F2.2 & F2.14	22.5°	1/16	1/8	1/6	1/4	1/3	1/2	1	500
F2.3 & F2.15	22.5°	1/16	1/8	1/6	1/4	1/3	1/2	1	750
F2.4 & F2.16	22.5°	1/16	1/8	1/6	1/4	1/3	1/2	1	1500
F2.5 & F2.17	45°	1/8	1/6	1/4	1/3	1/2	1	1	250
F2.6 & F2.18	45°	1/8	1/6	1/4	1/3	1/2	1	1	500
F2.7 & F2.19	45°	1/8	1/6	1/4	1/3	1/2	1	1	750
F2.8 & F2.20	45°	1/8	1/6	1/4	1/3	1/2	1	1	1500
F2.9 & F2.21	90°	1/4	1/3	1/2	1				250
F2.10 & F2.22	90°	1/4	1/3	1/2	1				500
F2.11 & F2.23	90°	1/4	1/3	1/2	1				750
F2.12 & F2.24	90°	1/4	1/3	1/2	1				1500

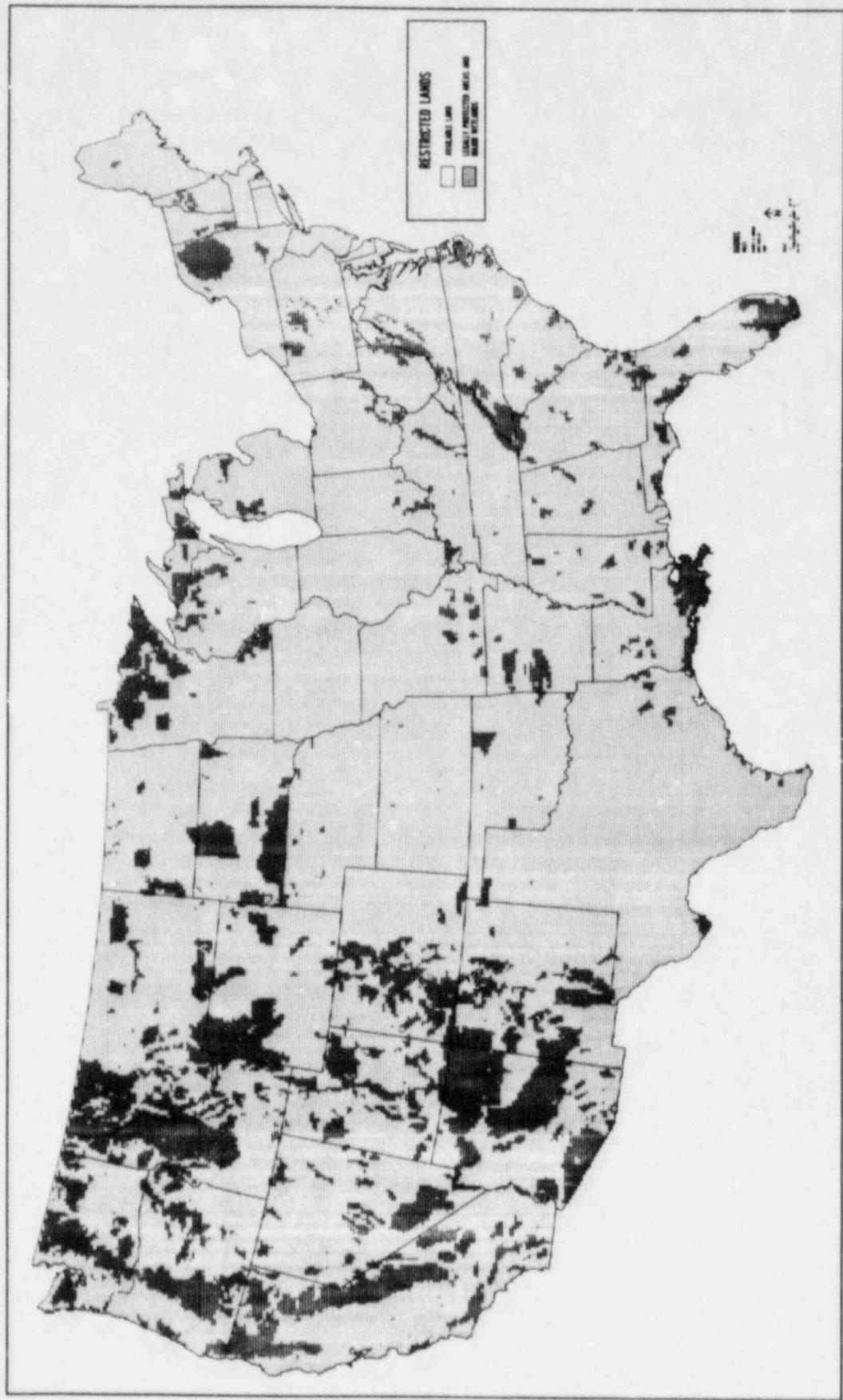
Tables F3.1-F3.5 show the environmental suitability of land not restricted by each of 5 population siting criteria. (The environmental suitability classifications were discussed in Section 4.4 of Chapter 4.0). These tables show the fraction of land, by state, that 1) lies within each of the five suitability categories and 2) satisfies the population criteria. The population criteria consist of a population restriction within two miles and a composite population restriction within the 2 to 30 mile region. (The annuli considered by the 2 to 30 mile composite population restriction include 2-3 miles, 2-4 miles, 2-5 miles, 2-10 miles, 2-20 miles, and 2-30 miles.) The population criterion considered by each table are tabulated below.

<u>Table</u>	<u>Population Case</u>	<u>0-2 miles (people/mile<sup>2</sup>)</u>	<u>2-30 miles (composite) (people/mile<sup>2</sup>)</u>
F3.1	1	100	250
F3.2	2	250	500
F3.3	3	500	750
F3.4	4	500	750
F3.5	5	500	1500

Tables F3.6-F3.10 show the effect of applying different population criteria (the five cases considered in Tables F3.1-F3.5) on land available within each of the suitability categories. The suitability category considered in each table is tabulated below.

<u>Table</u>	<u>Environmental Suitability Category</u>
F3.6	low
F3.7	medium-low
F3.8	medium
F3.9	medium-high
F3.10	high

FIGURE F1



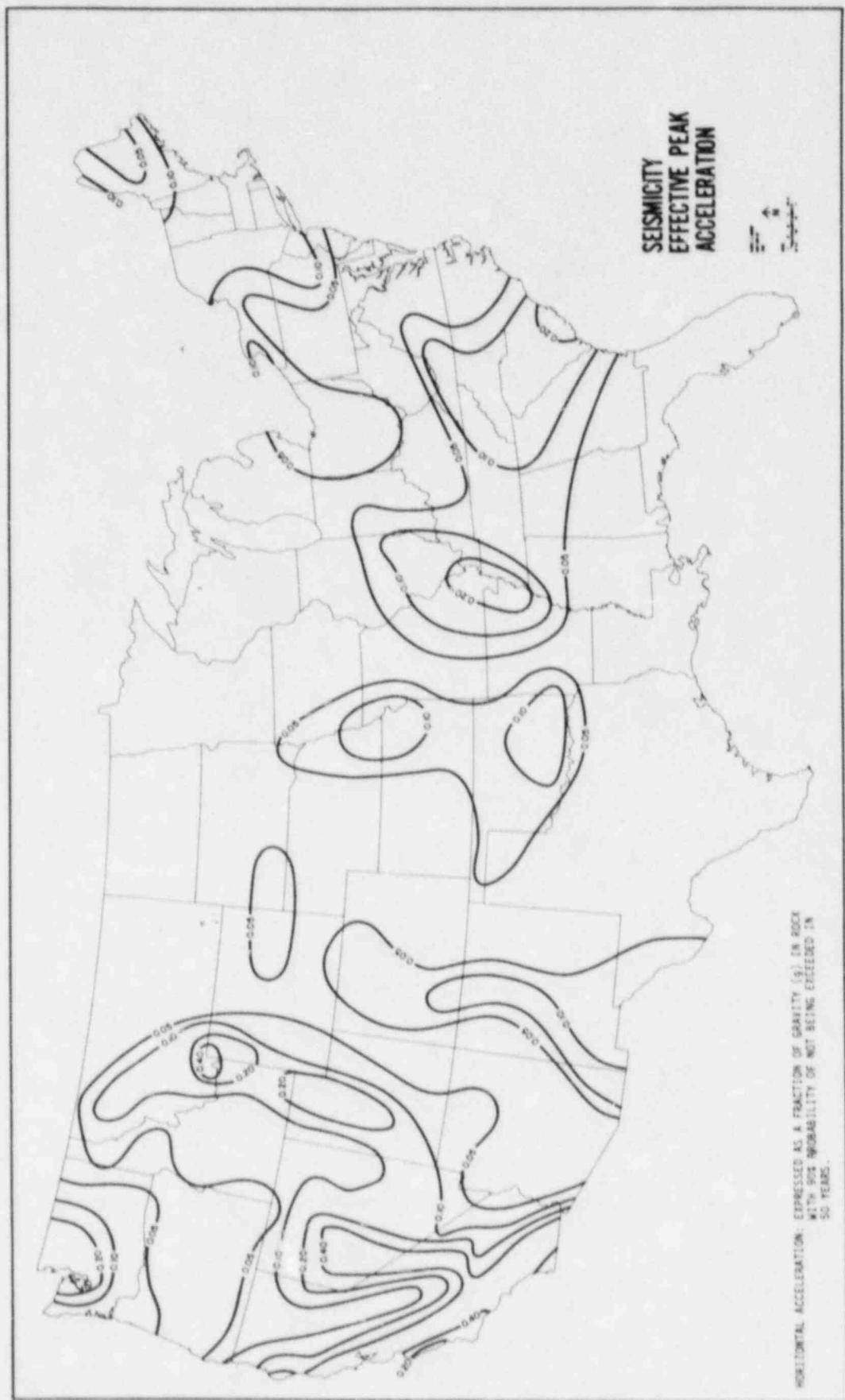


FIGURE F3

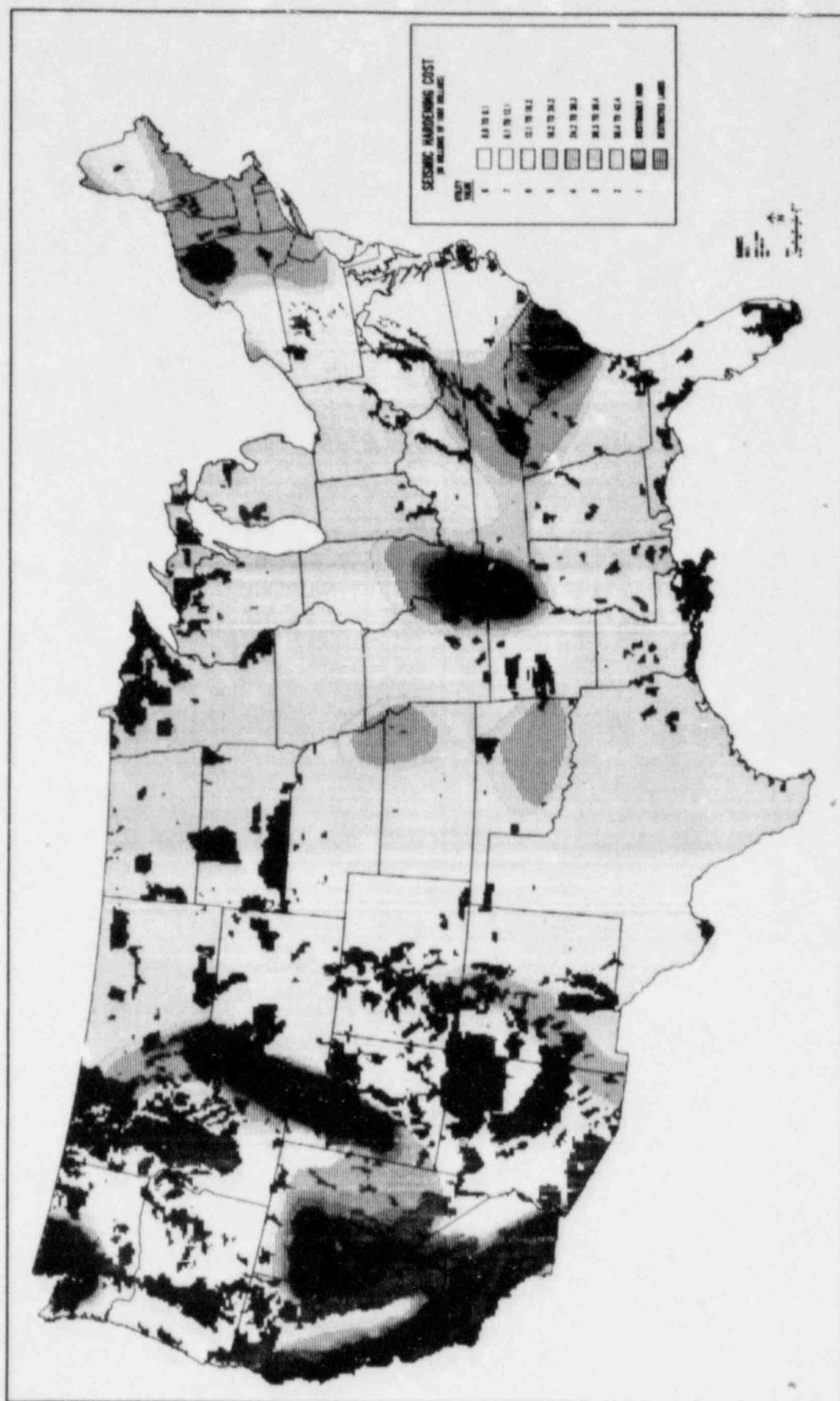


FIGURE F4



FIGURE F5

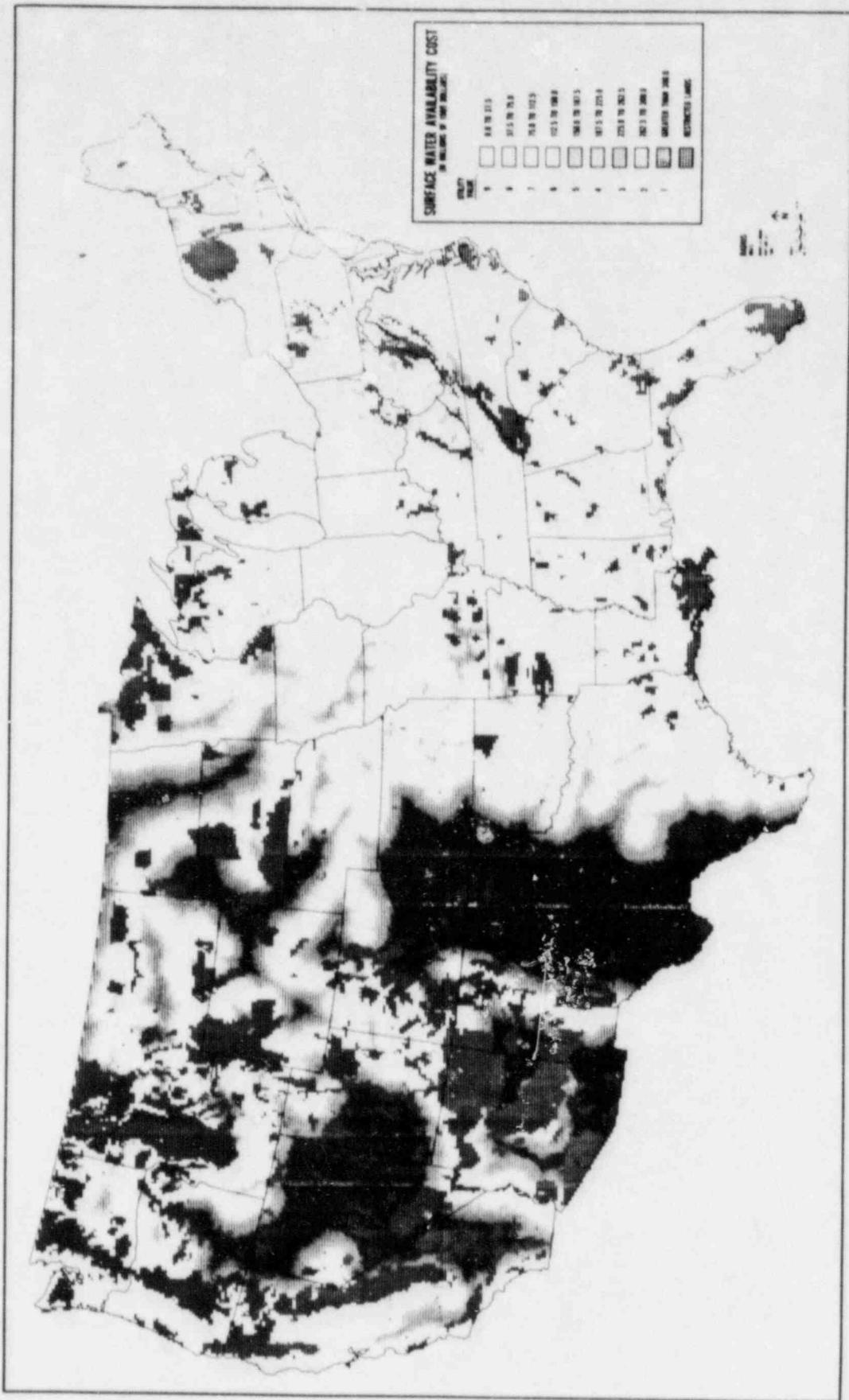


FIGURE F6

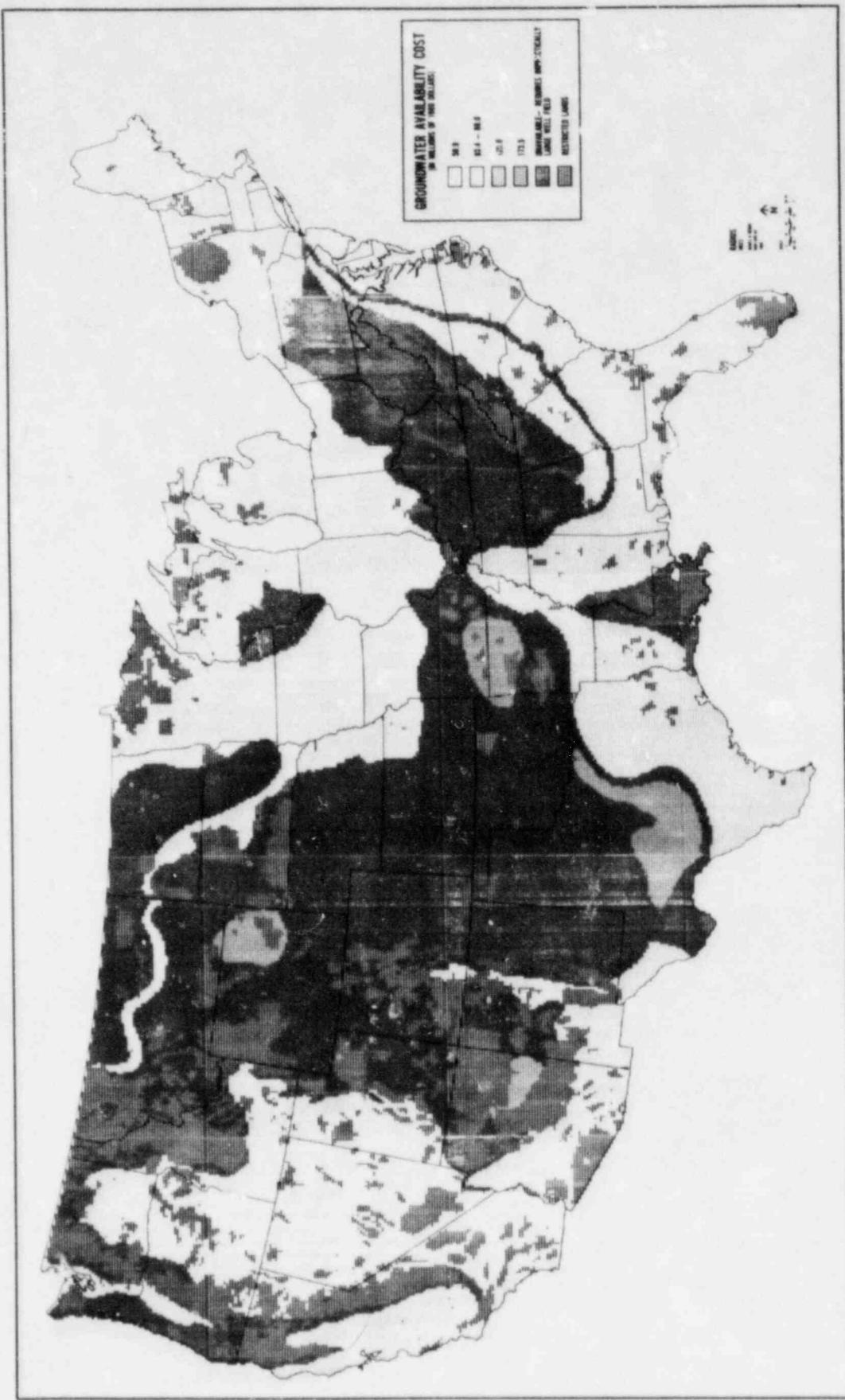


FIGURE F7

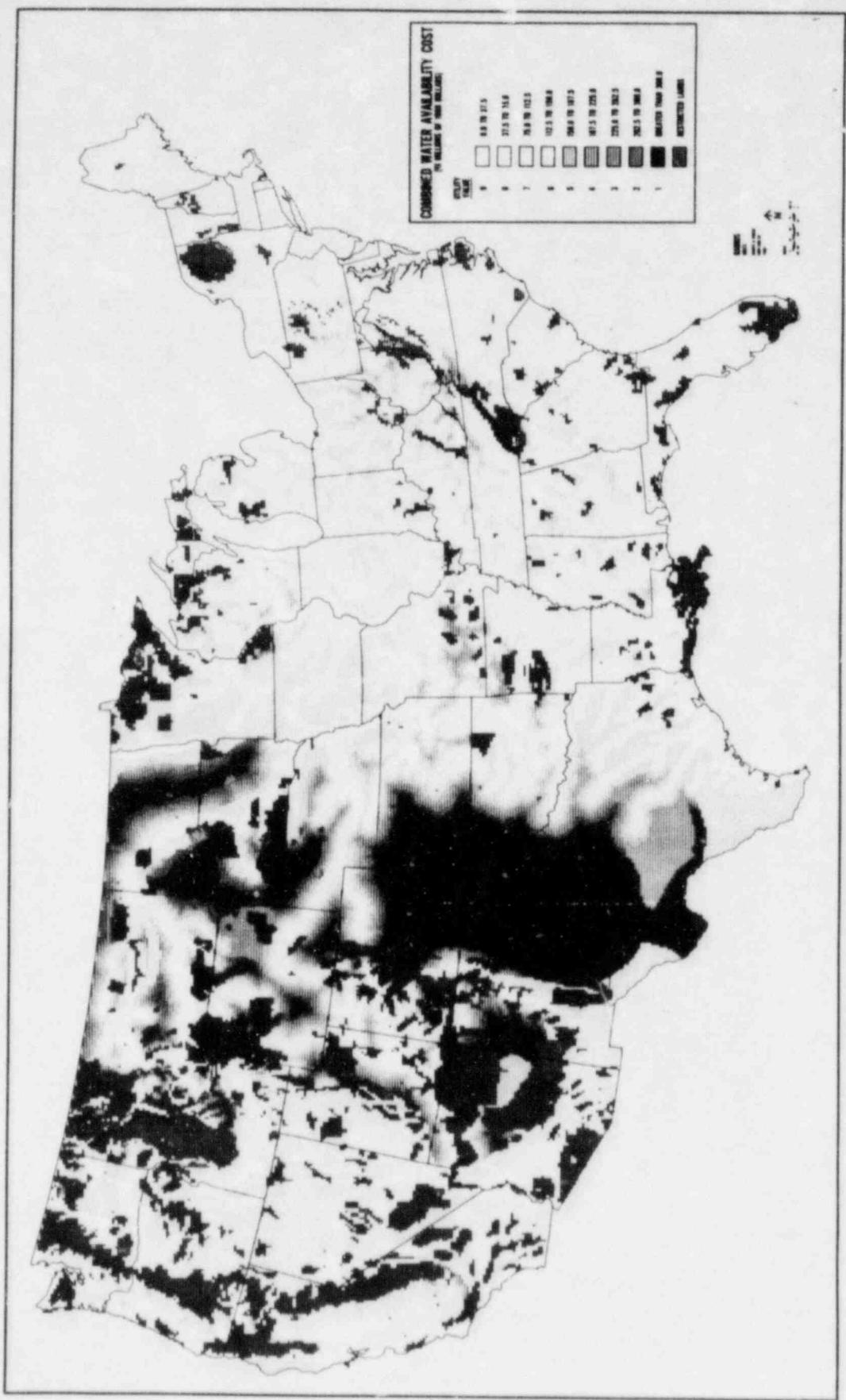
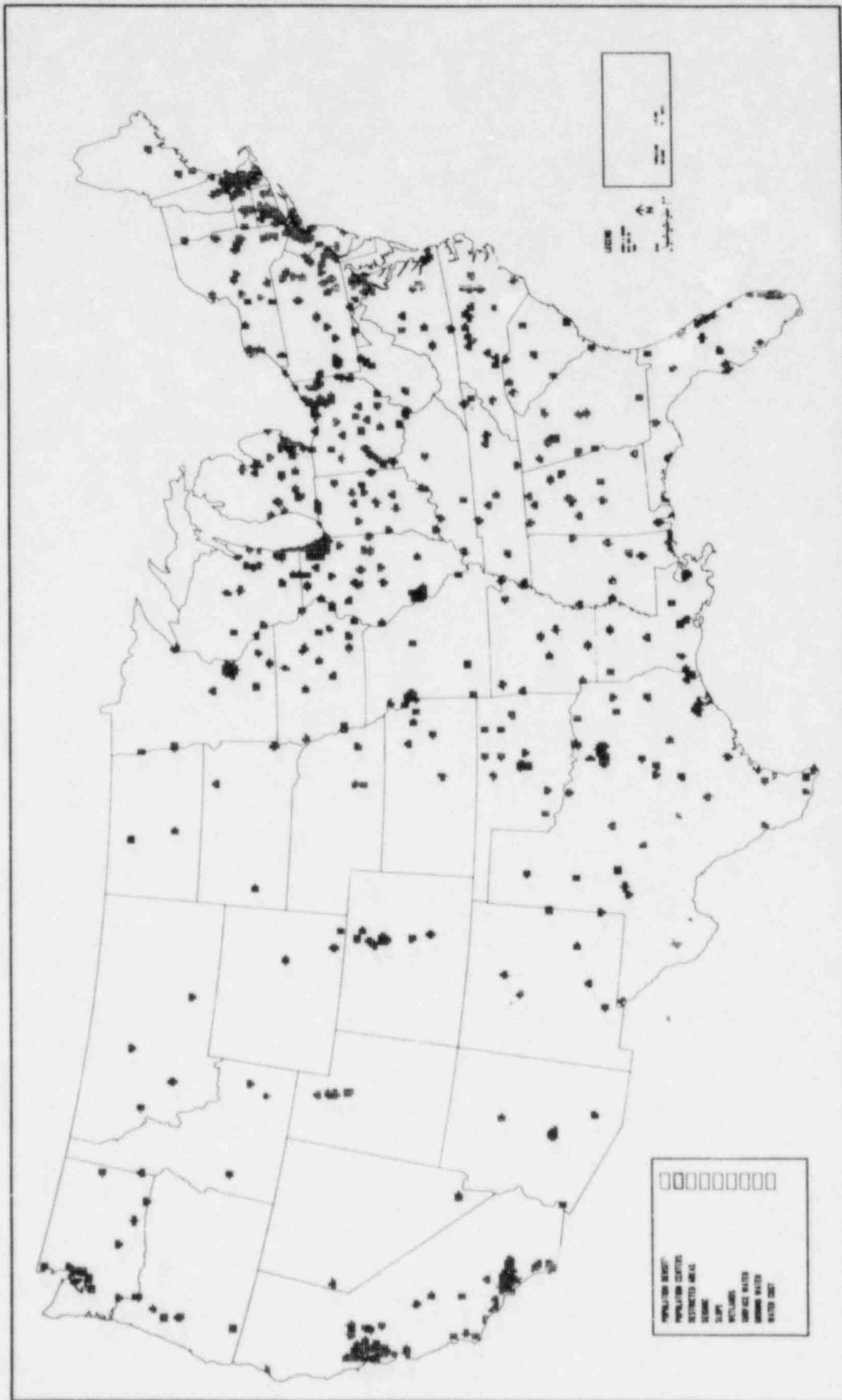


FIGURE F8.1





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FIGURE F8.2

FIGURE F8.3

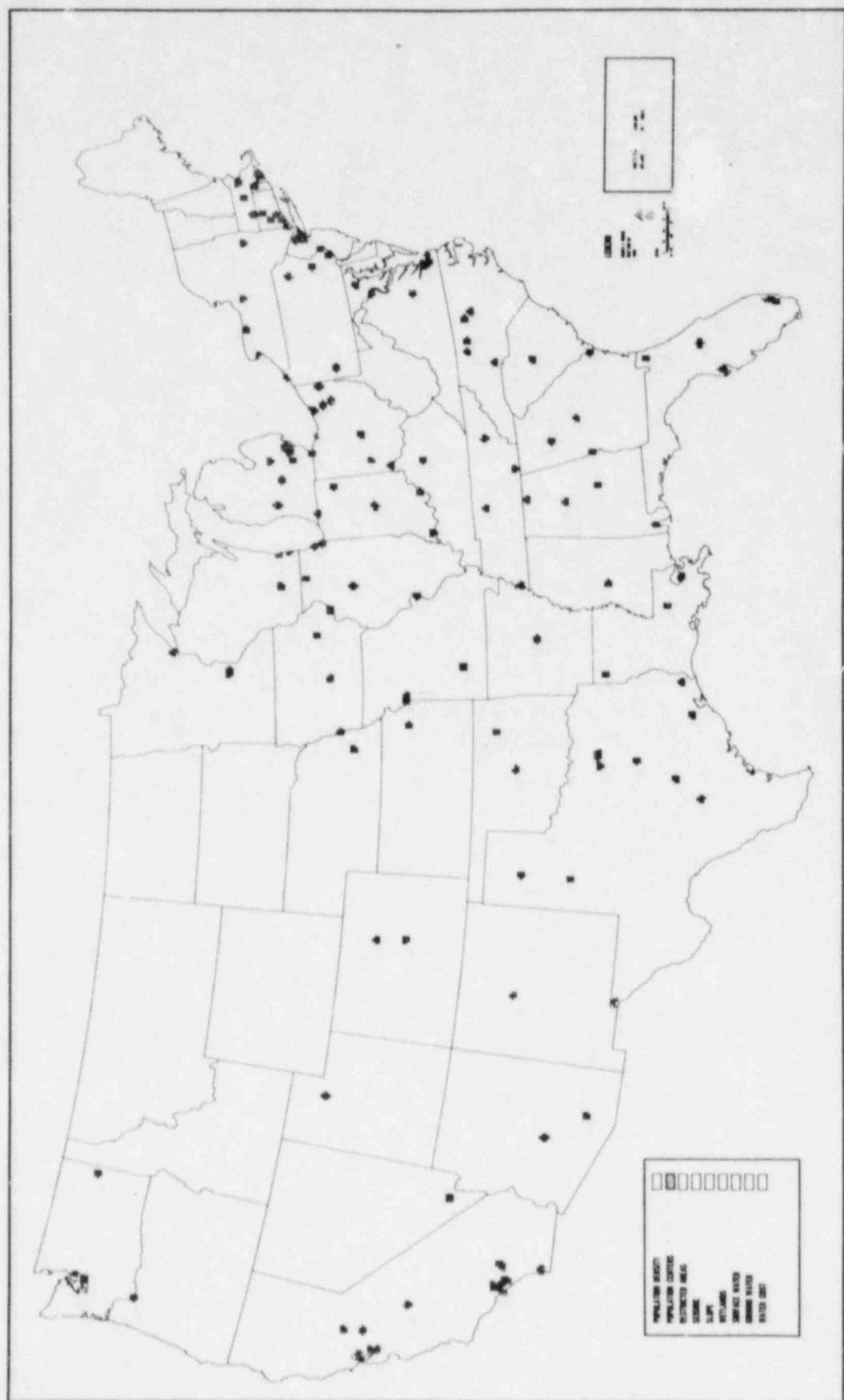


FIGURE F8.4

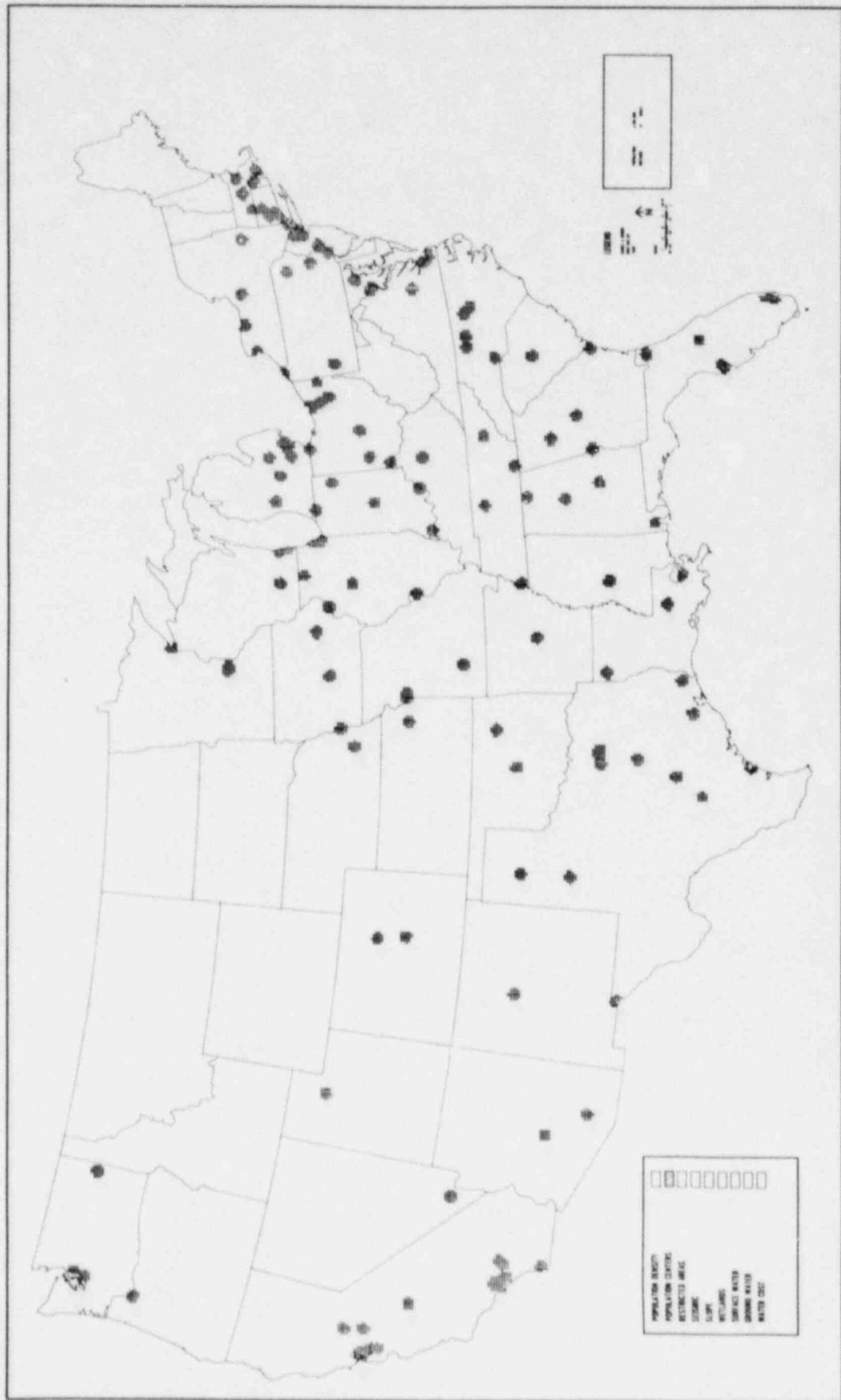


FIGURE F8.5

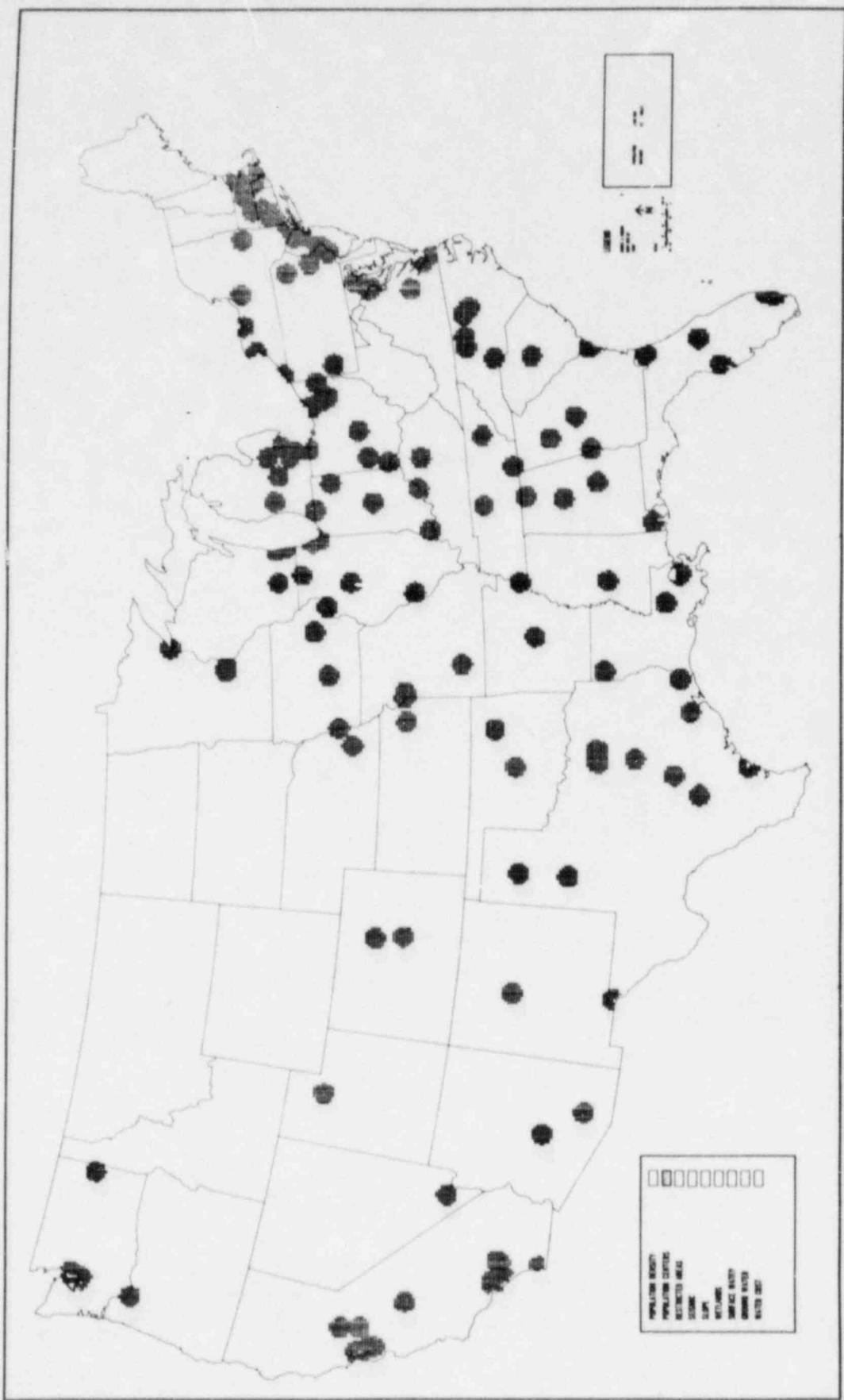


FIGURE F8.6

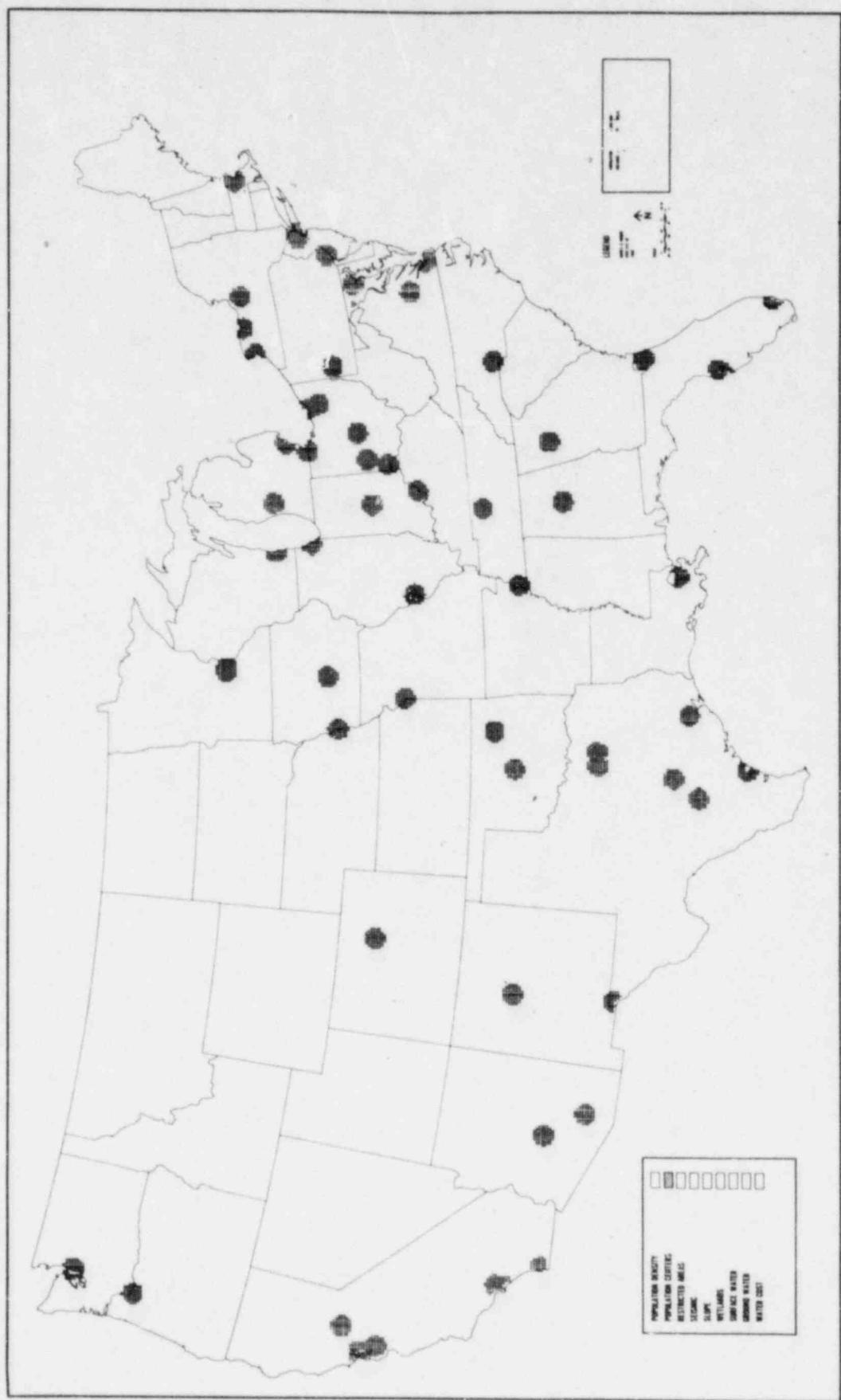


FIGURE F8.7

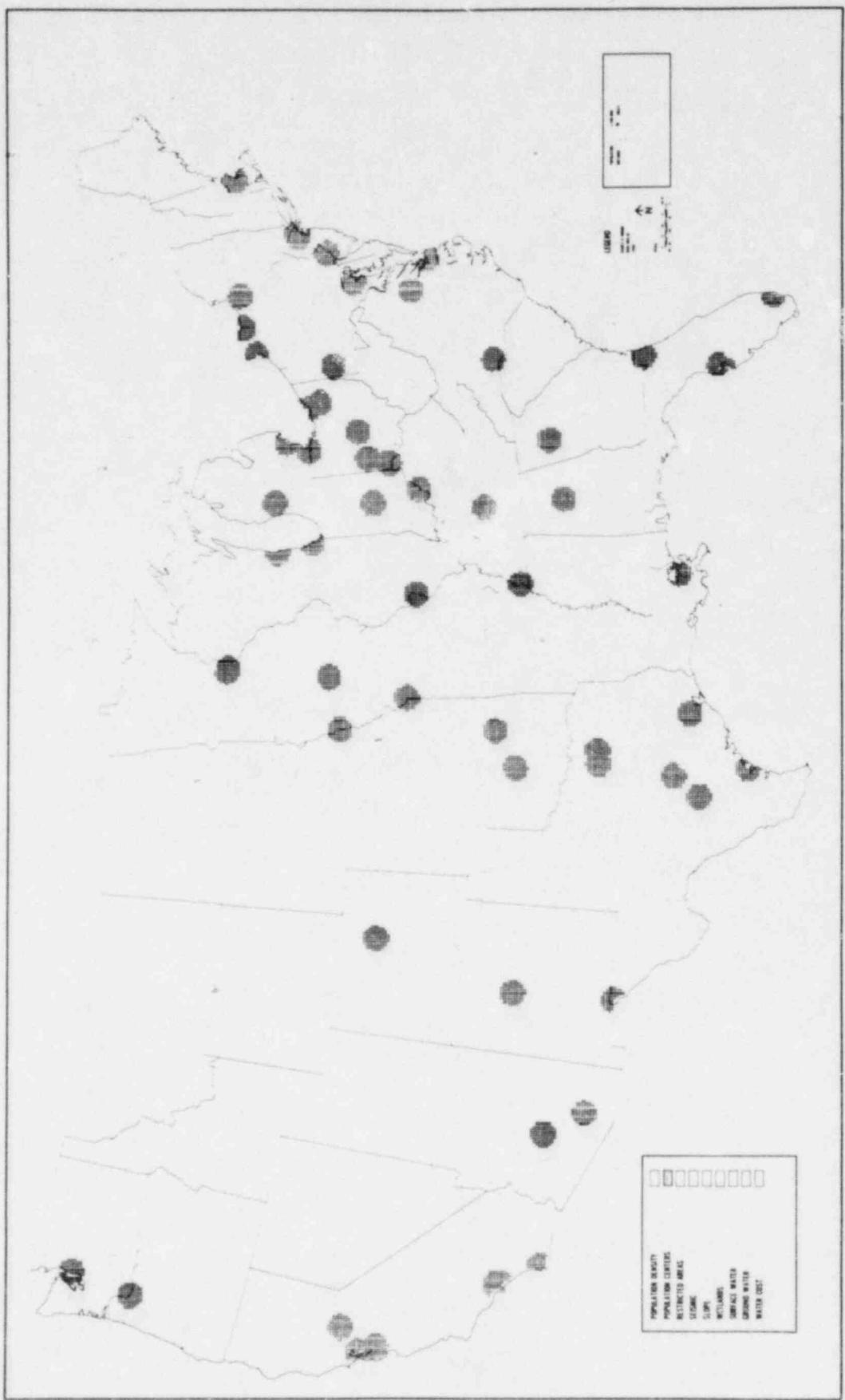


FIGURE F8.8

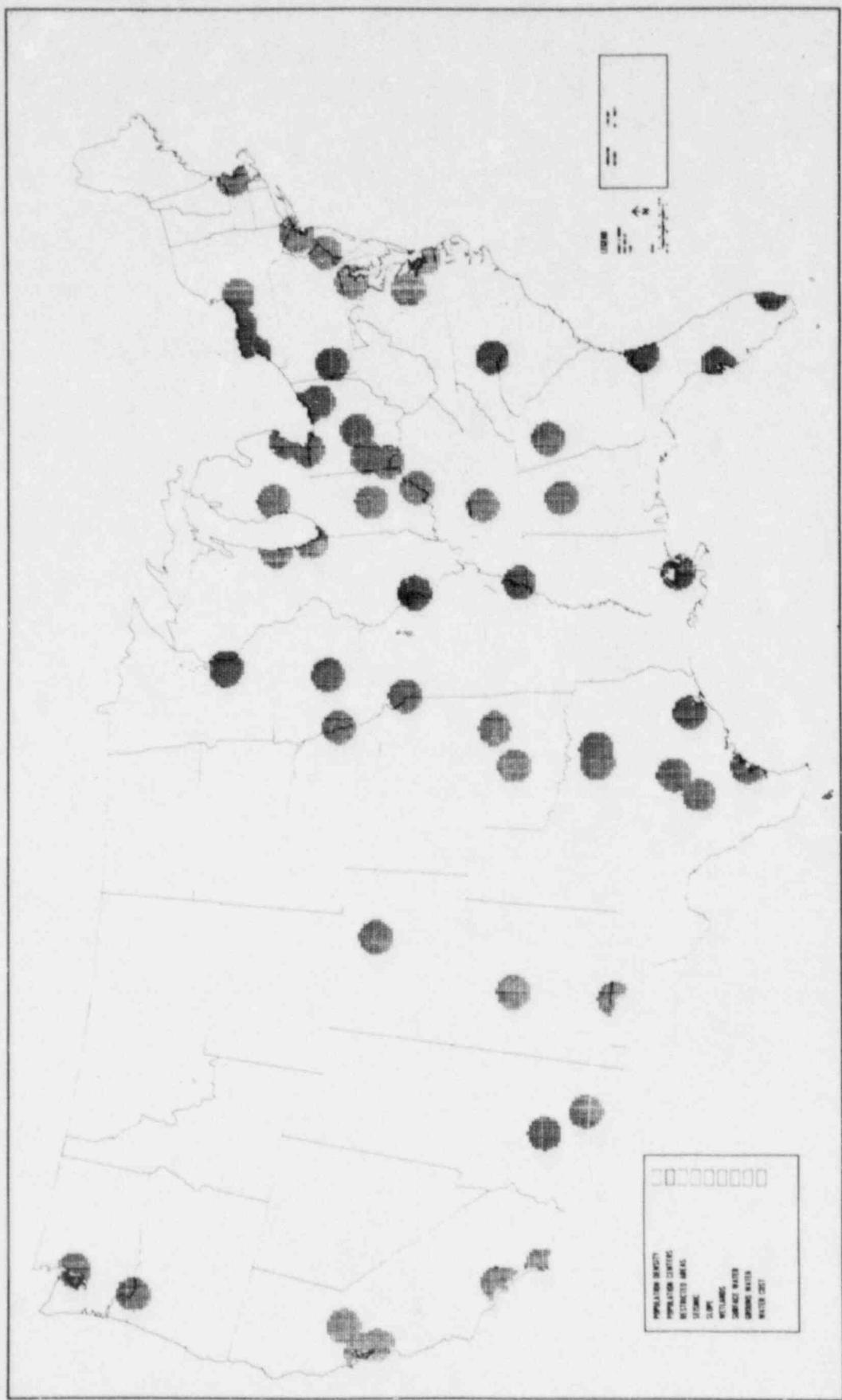


FIGURE F8.9



FIGURE F8.10

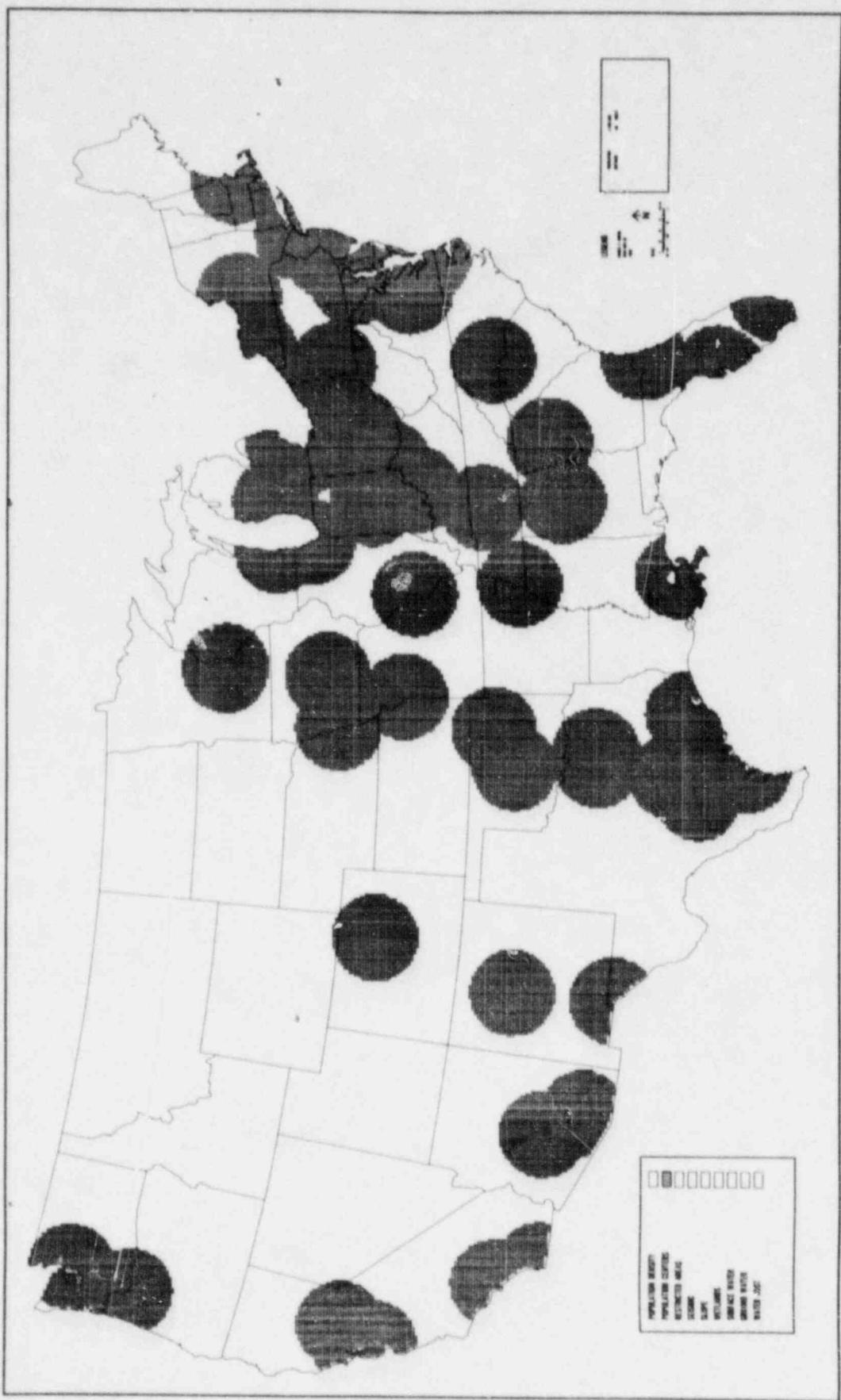


FIGURE F8.11

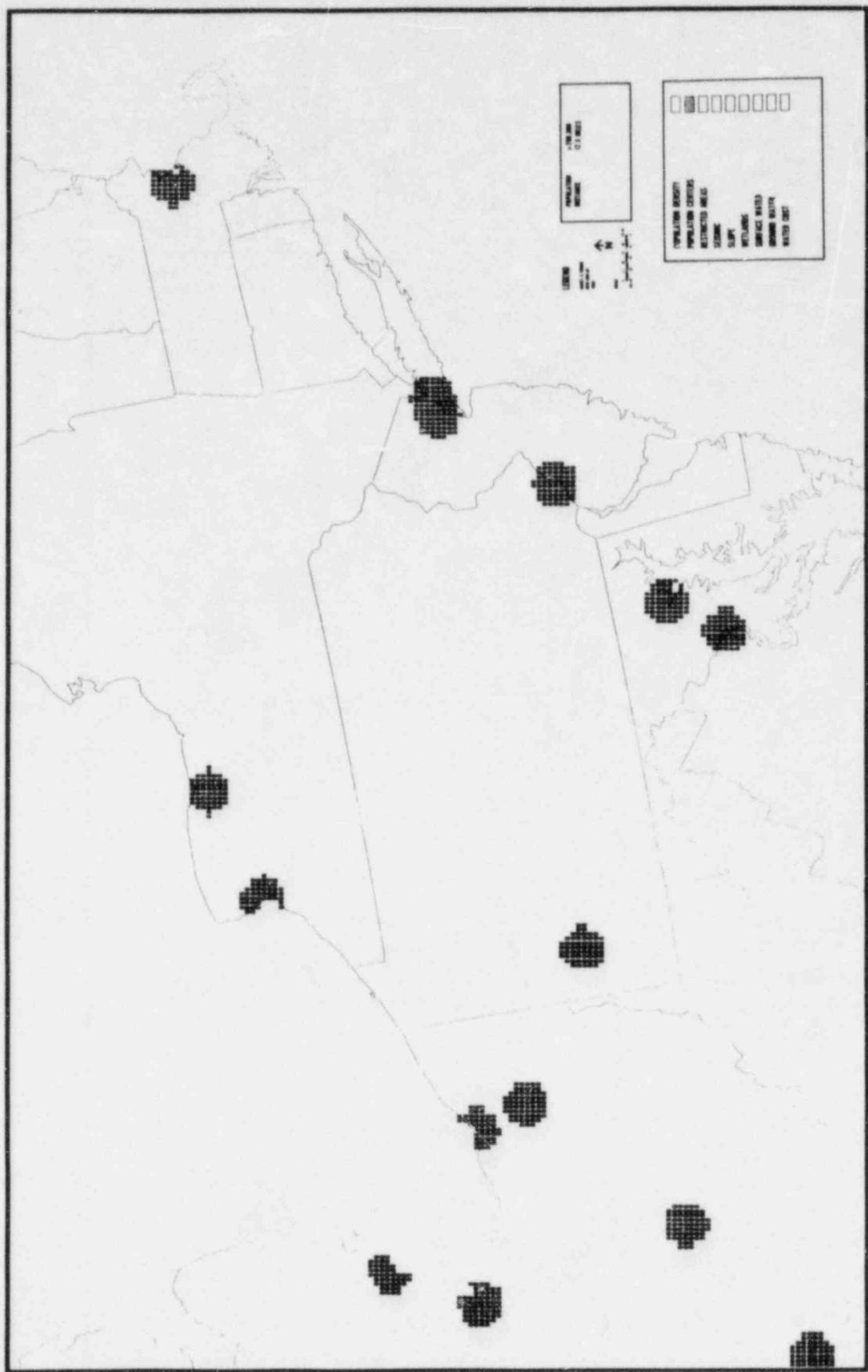


FIGURE F8.12

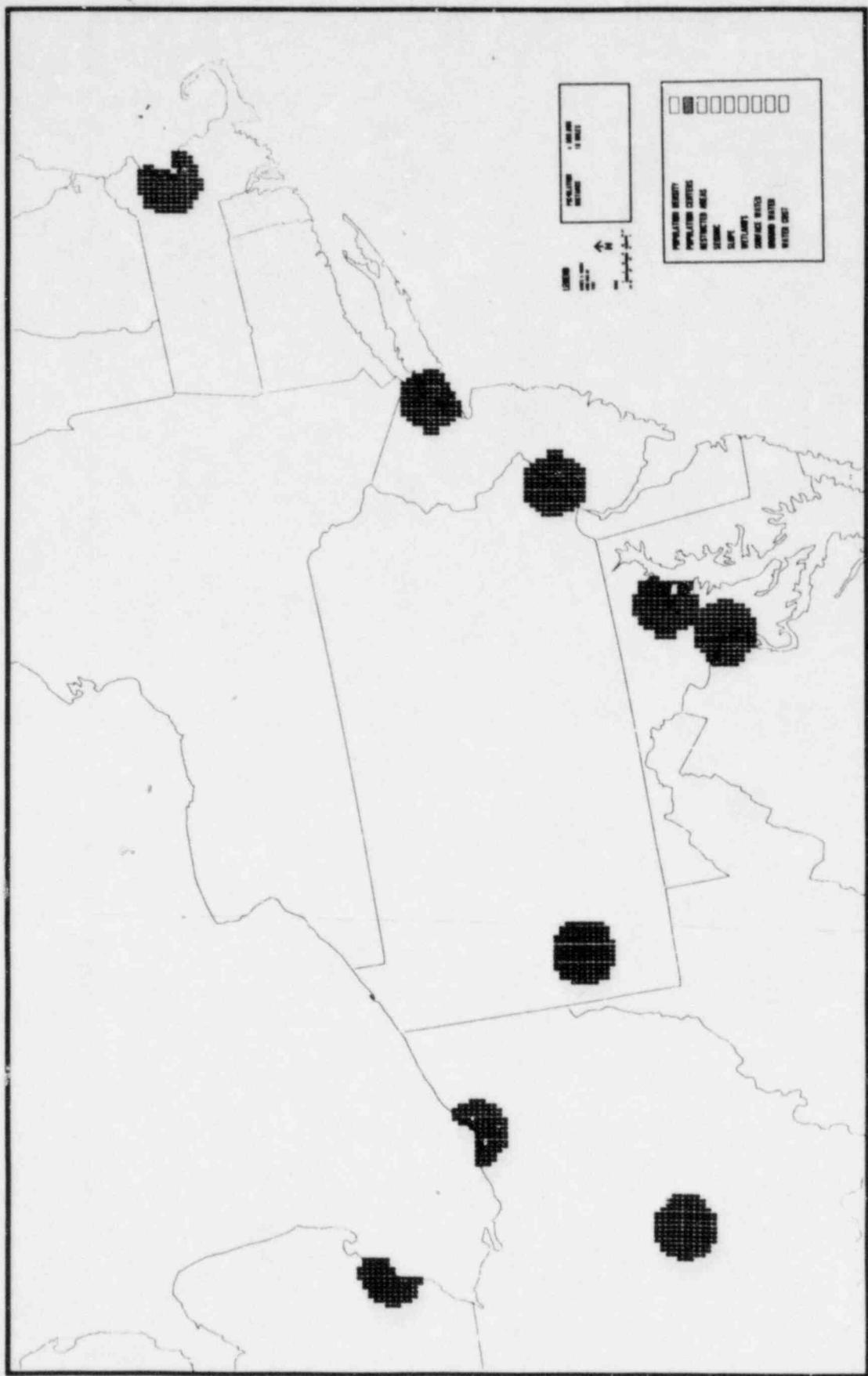




FIGURE F8.13

FIGURE F9.1

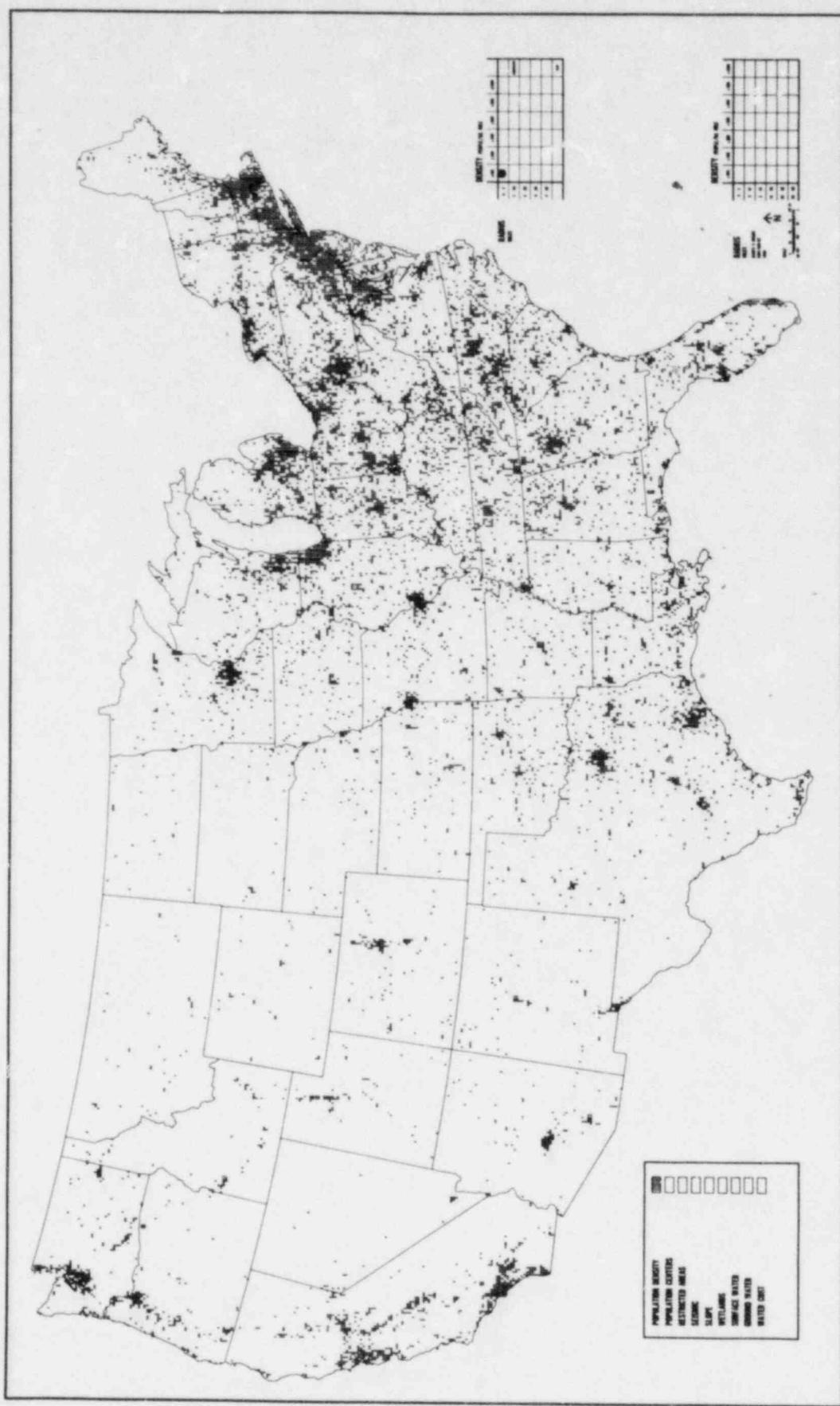


FIGURE F9.2

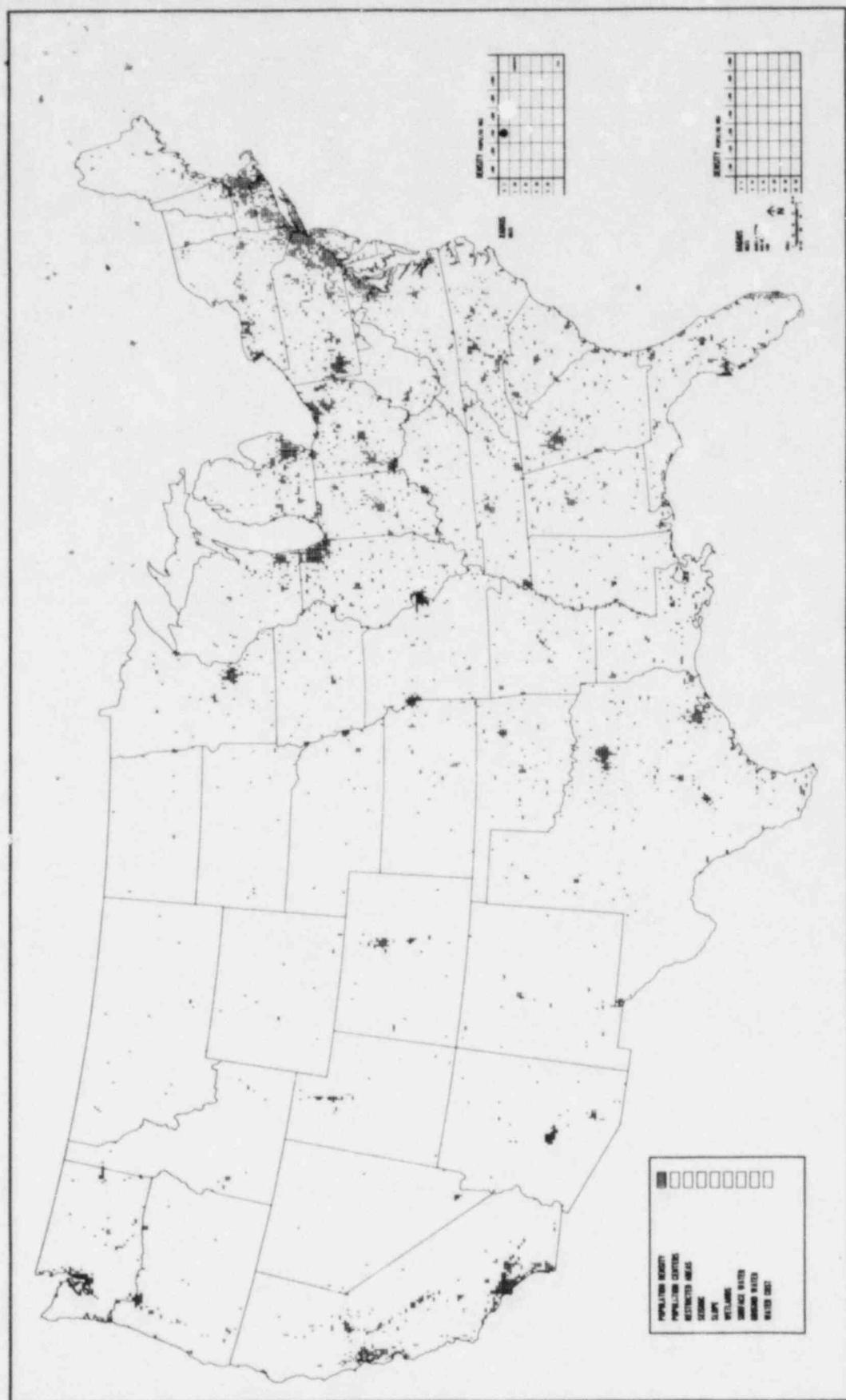
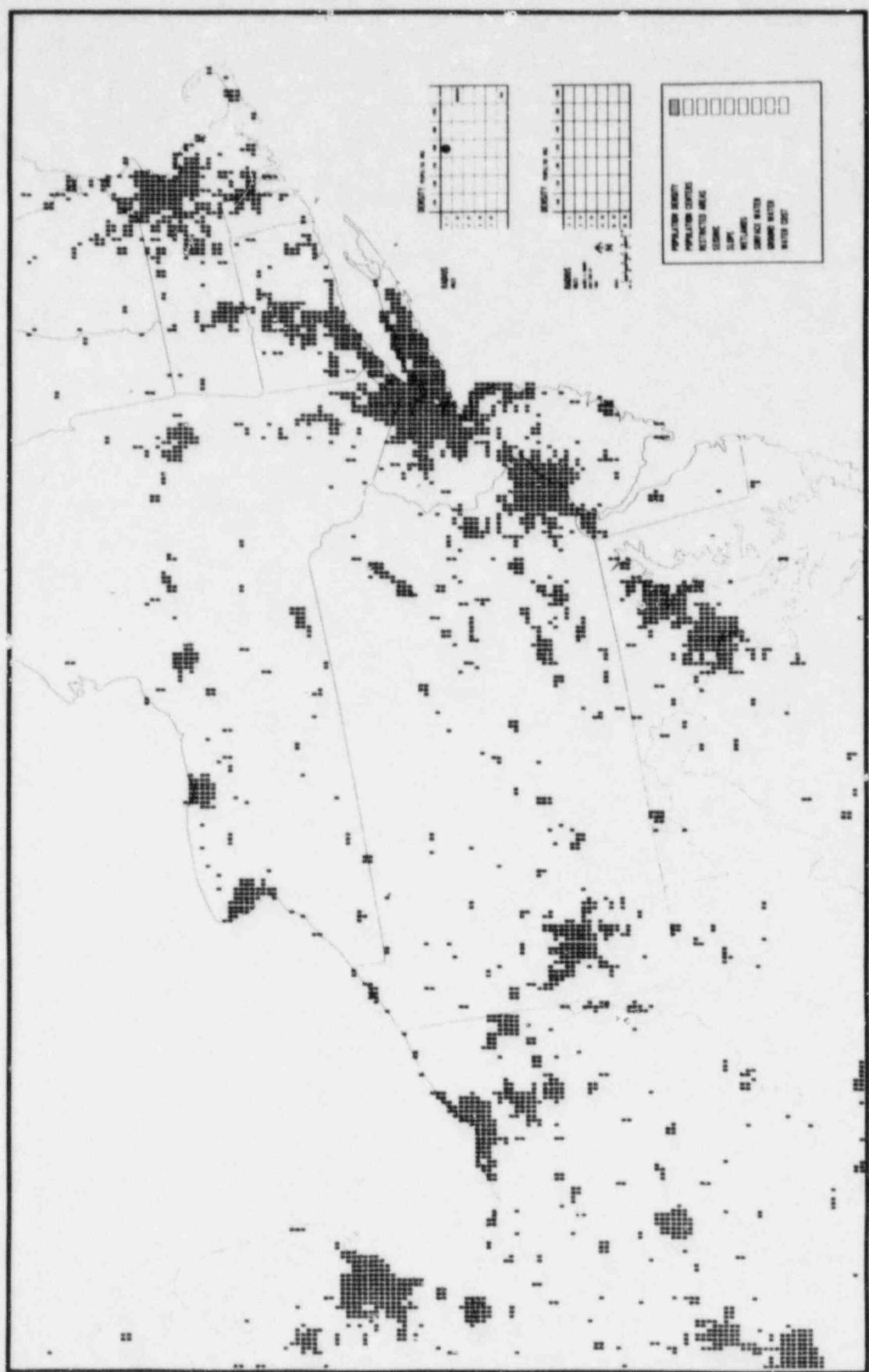


FIGURE F9.3



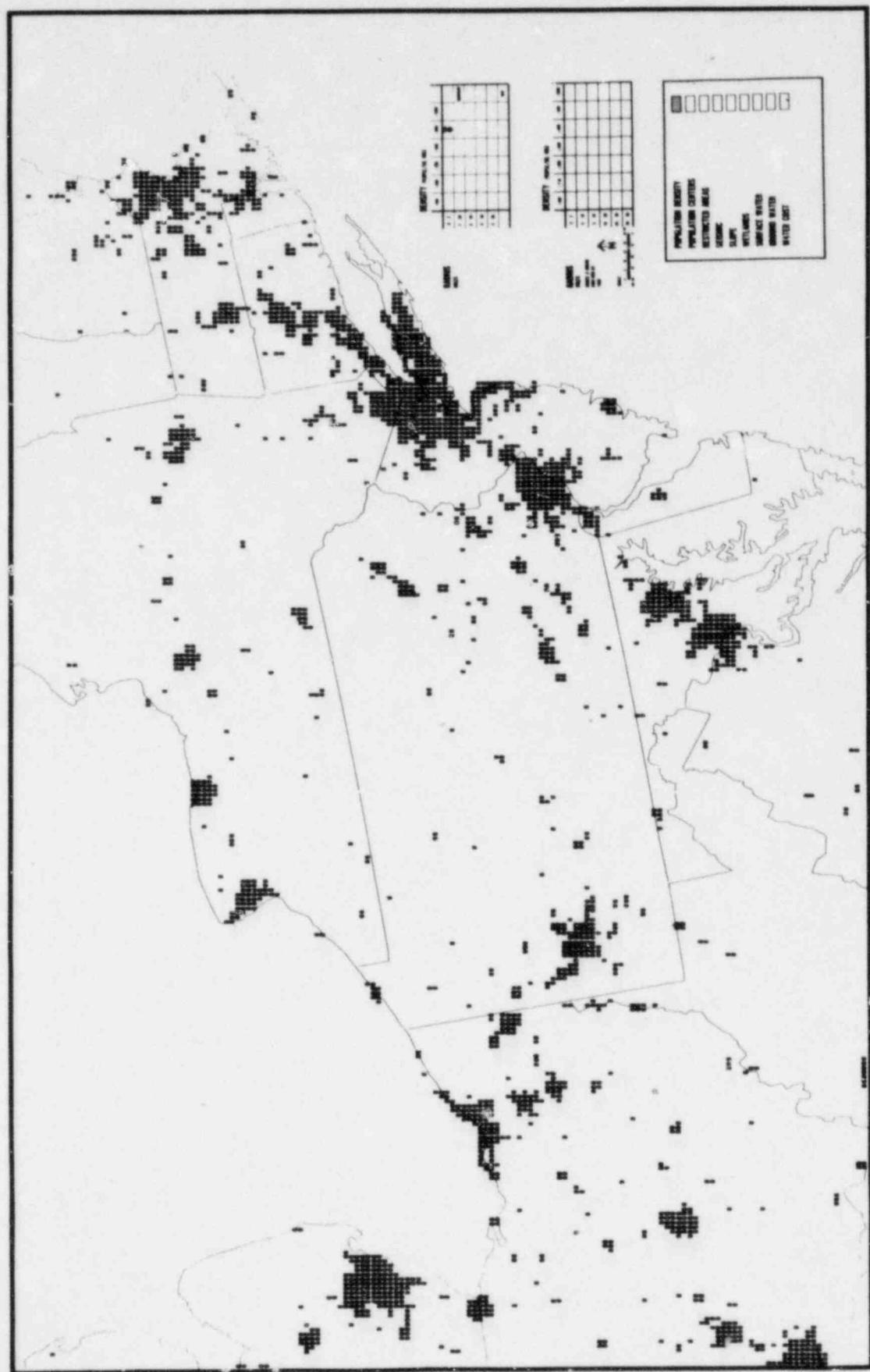


FIGURE F9.4

FIGURE F9.5

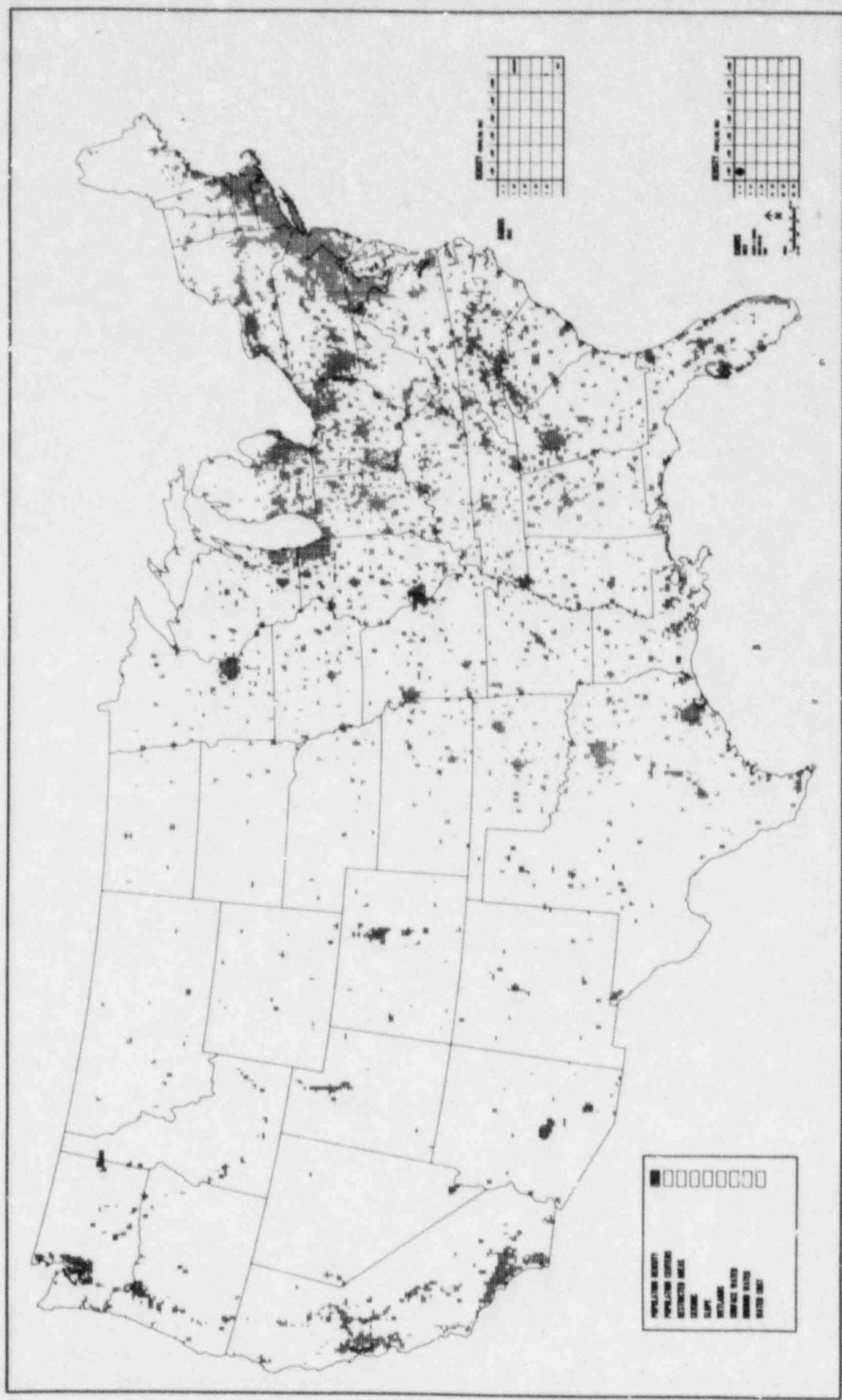


FIGURE F9.6

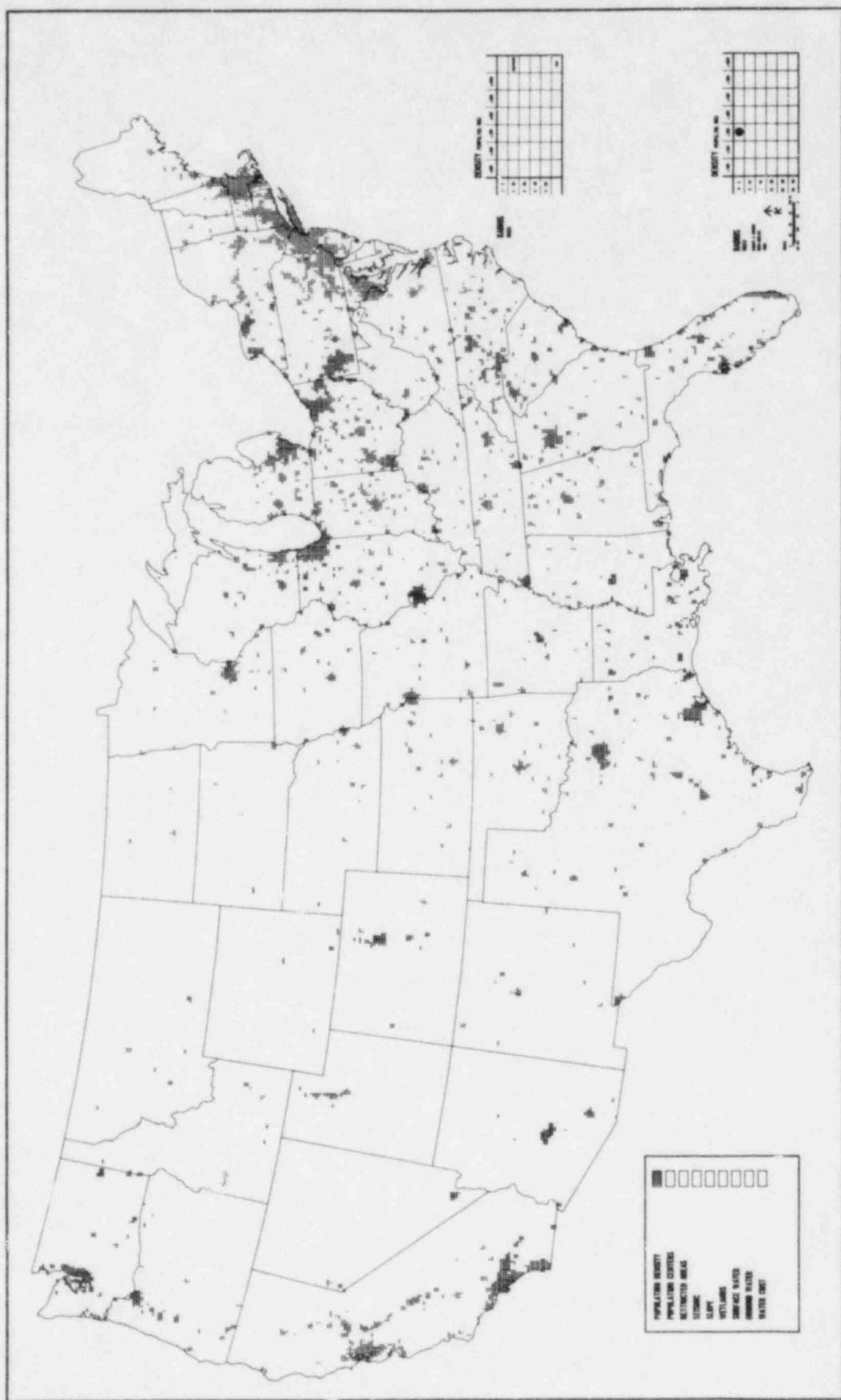


FIGURE F9.7

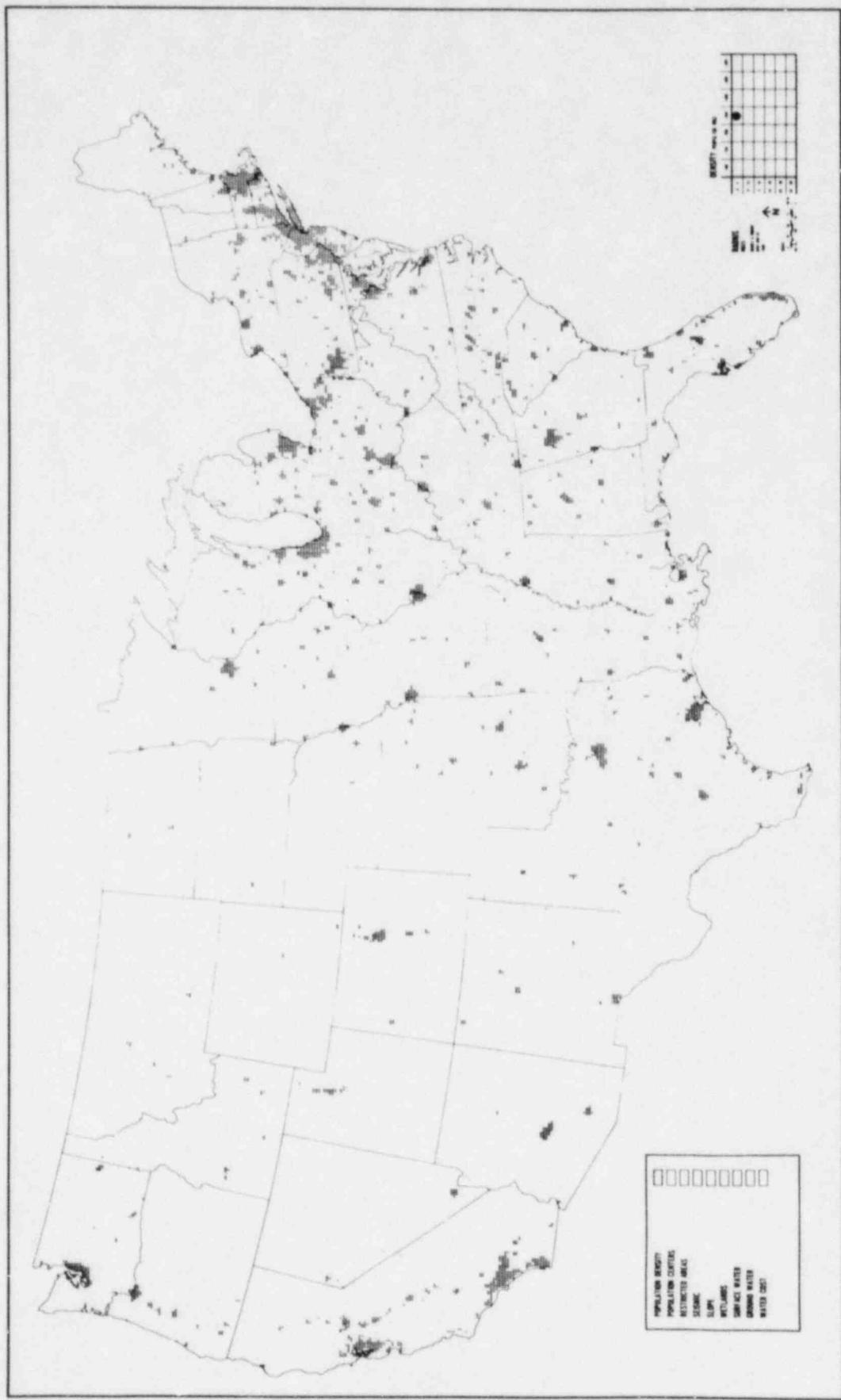


FIGURE F9.8

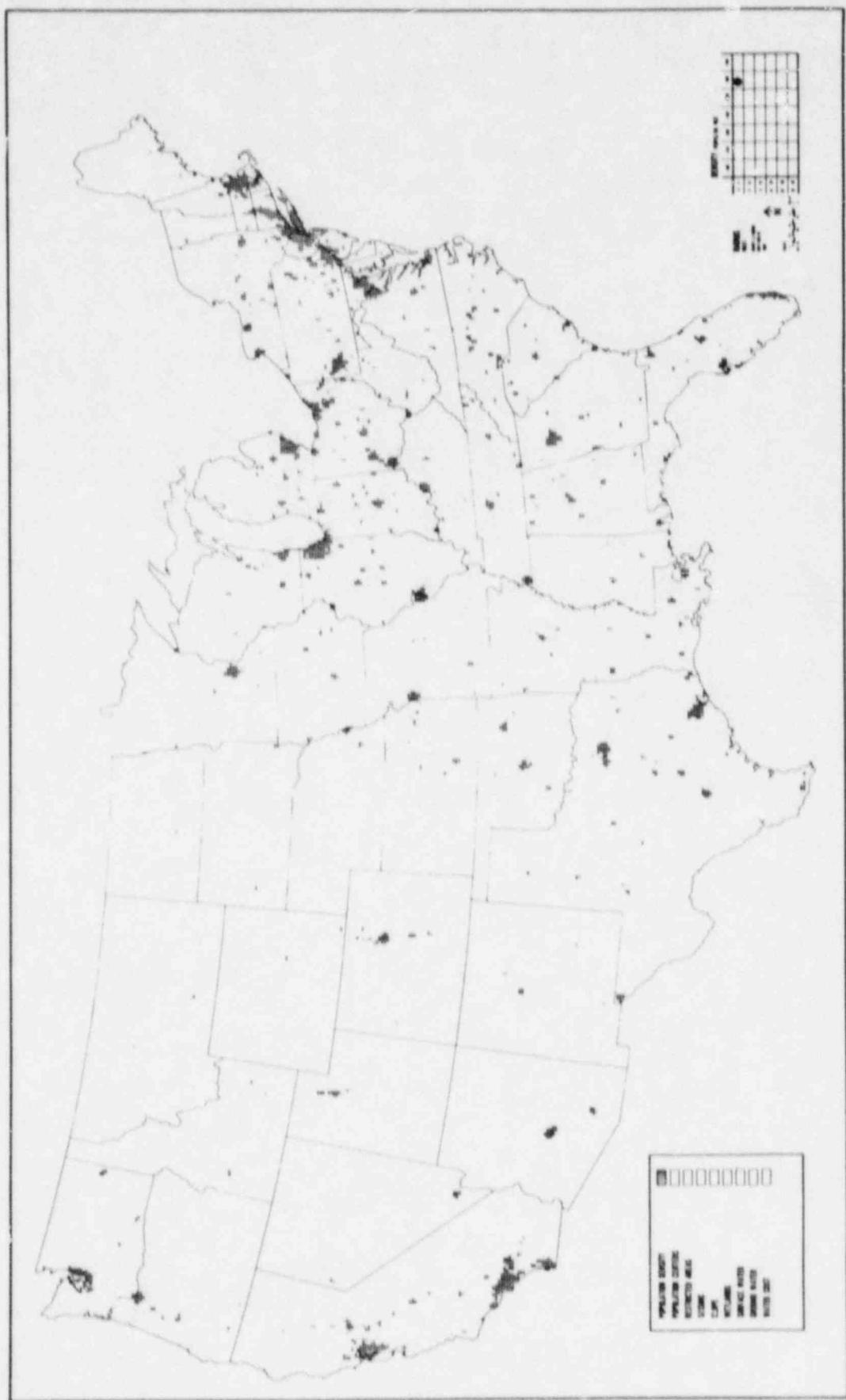
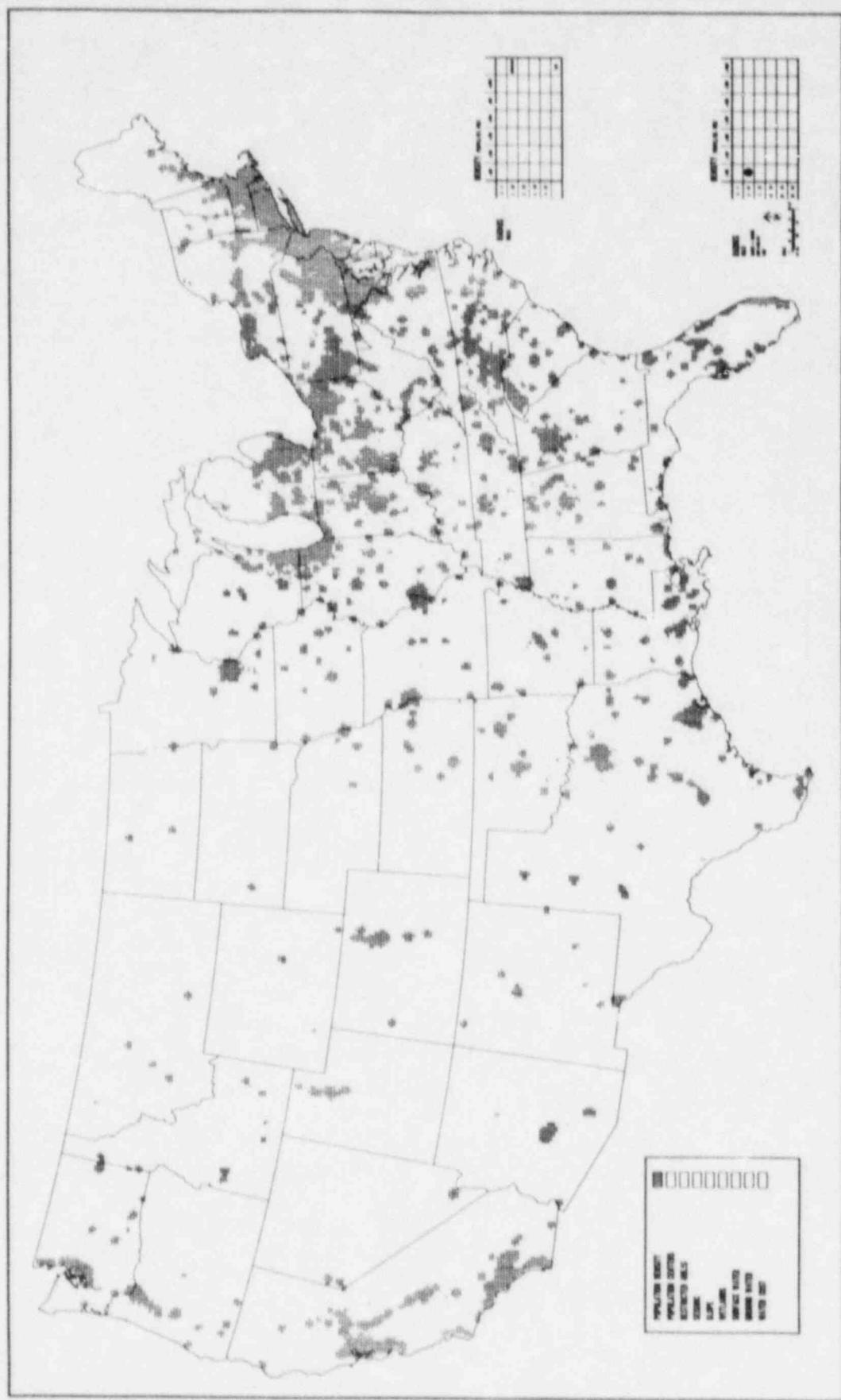


FIGURE F9.9



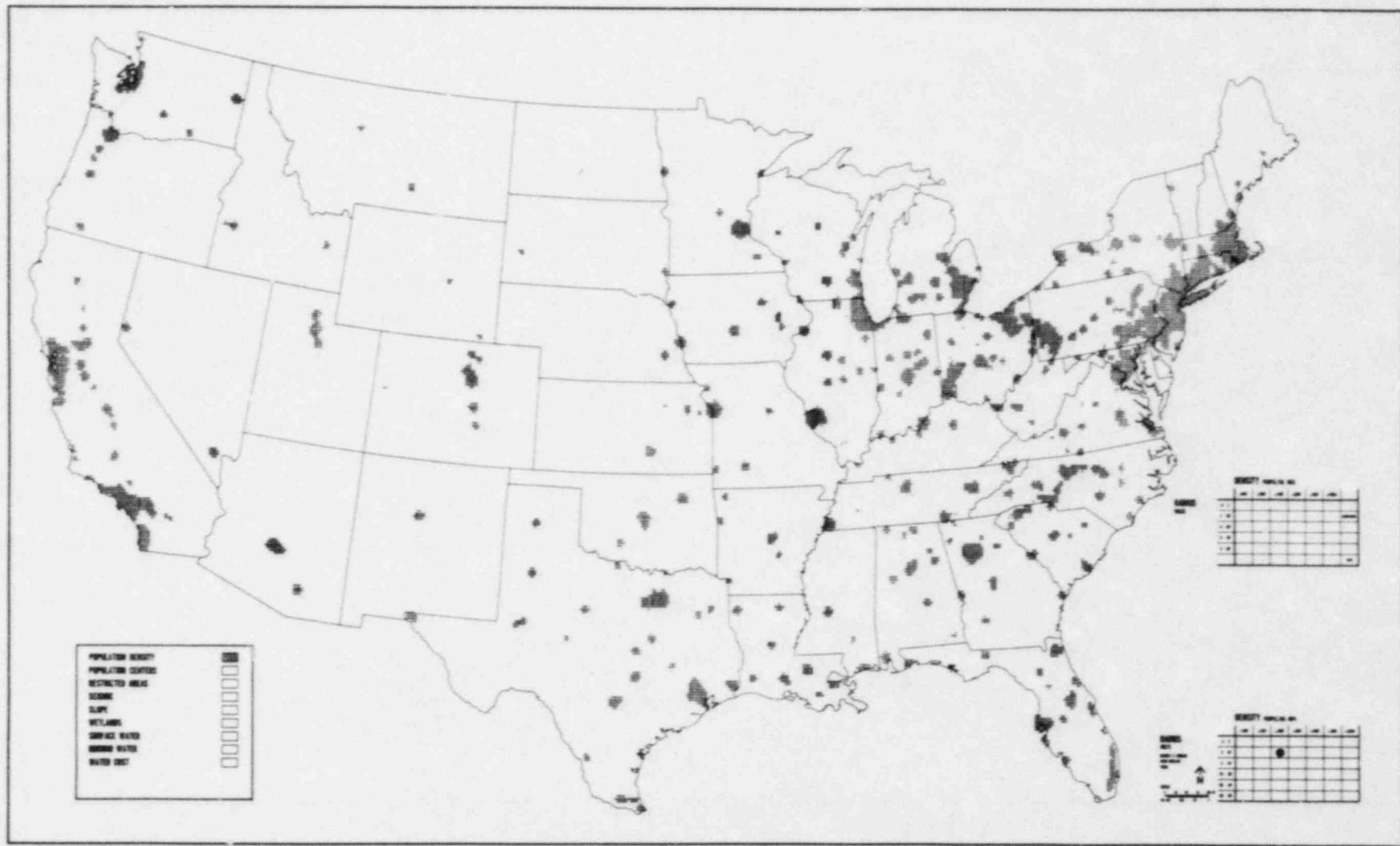


FIGURE F9.10

FIGURE F9.11

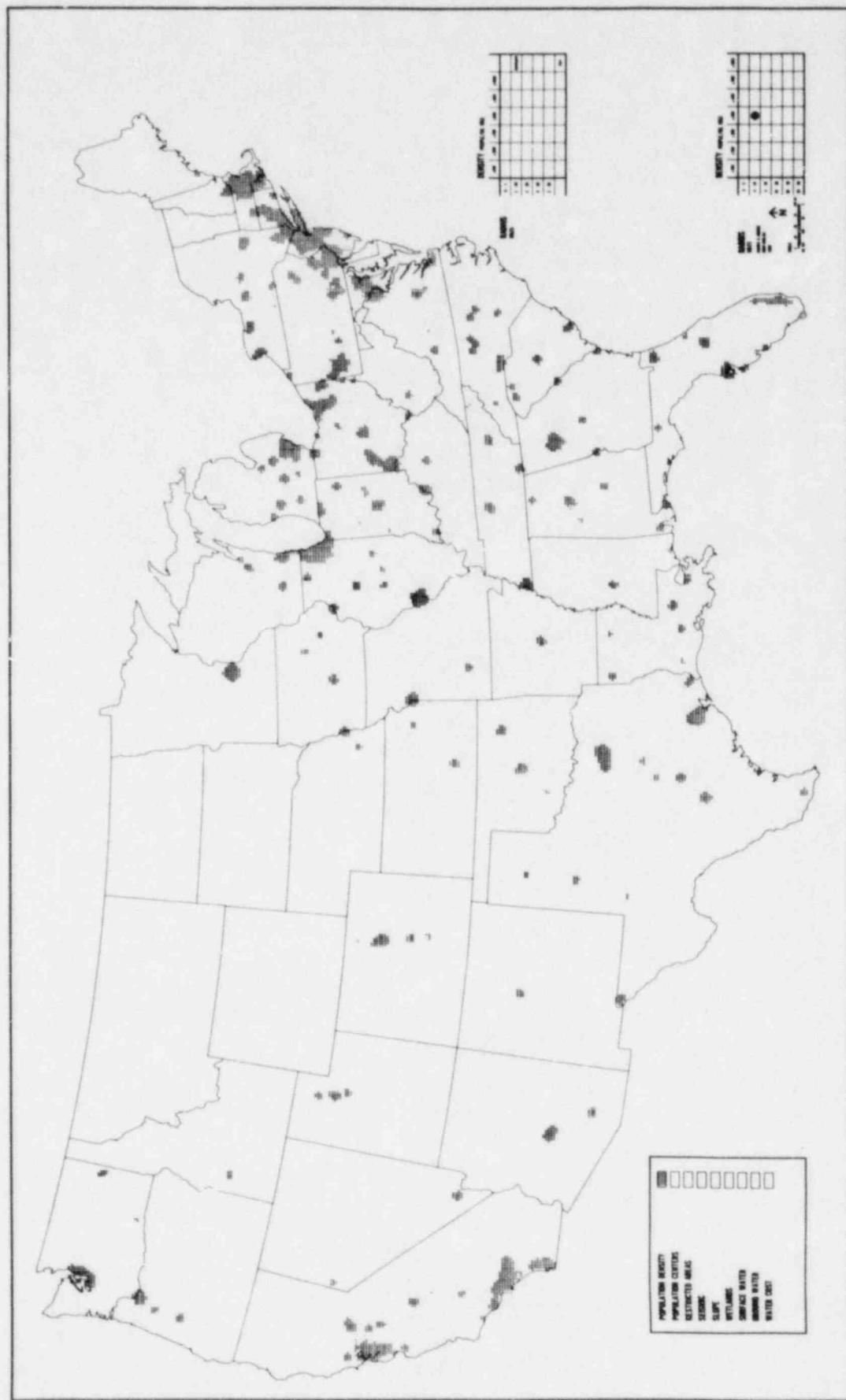
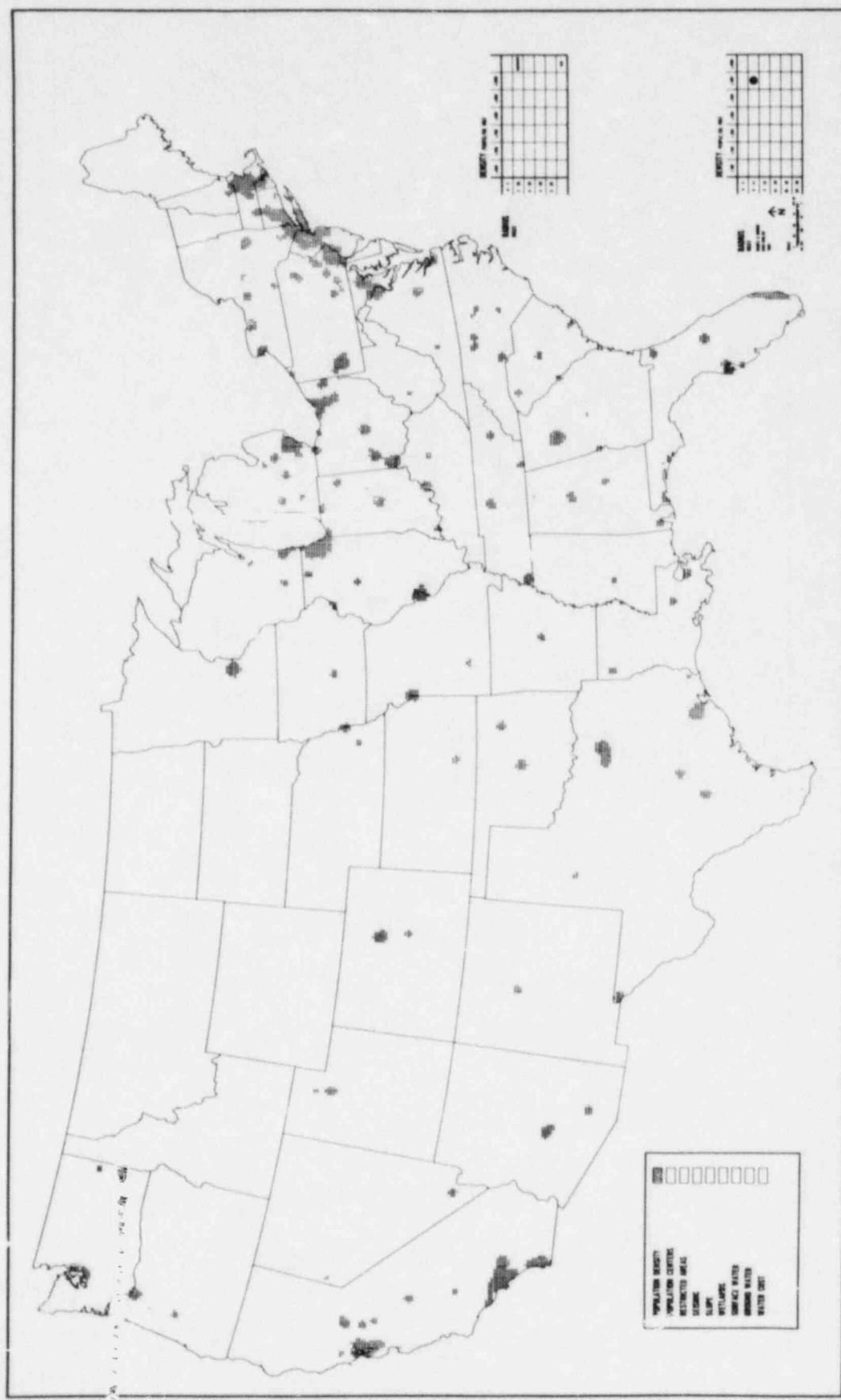


FIGURE F9.12



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FIGURE F9.13

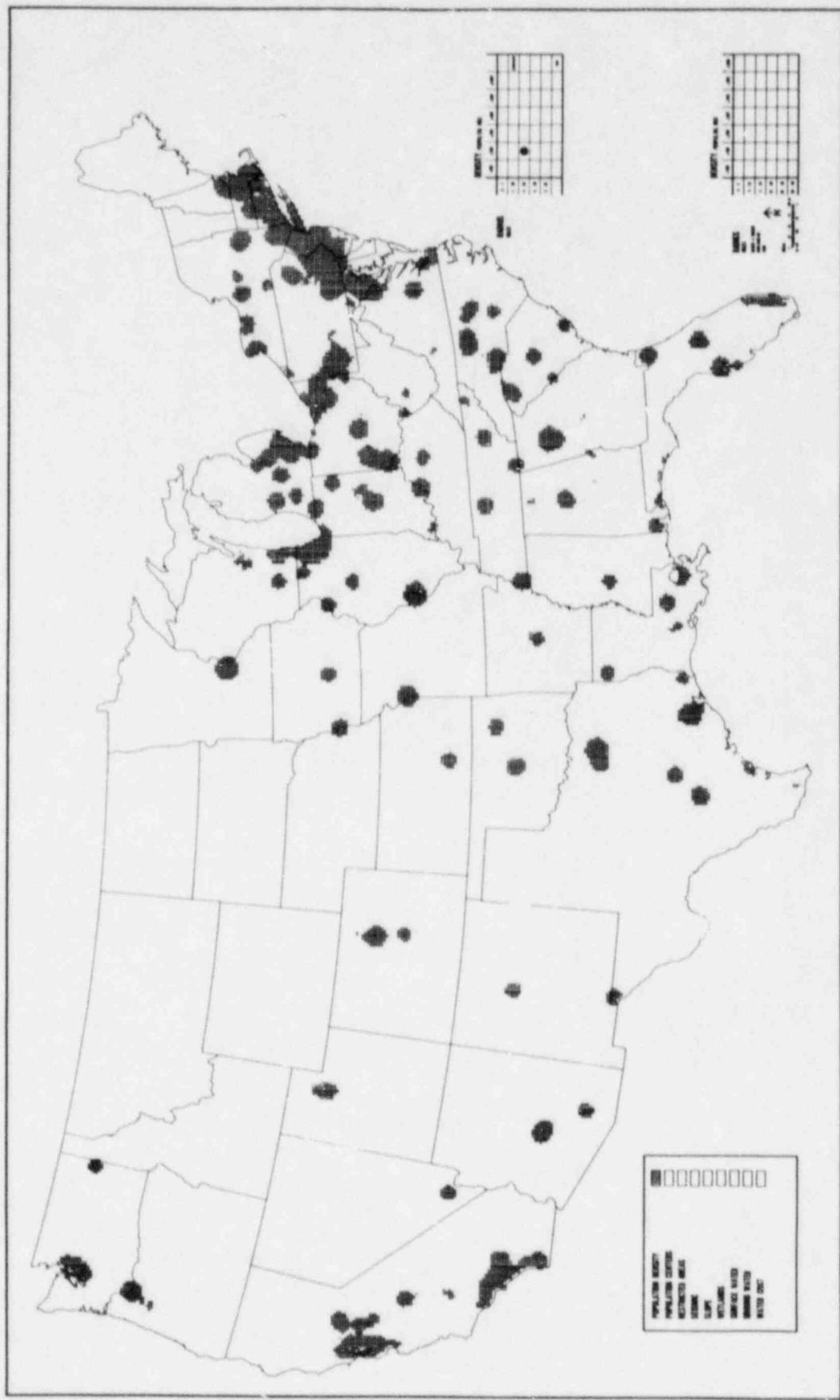


FIGURE F9.14

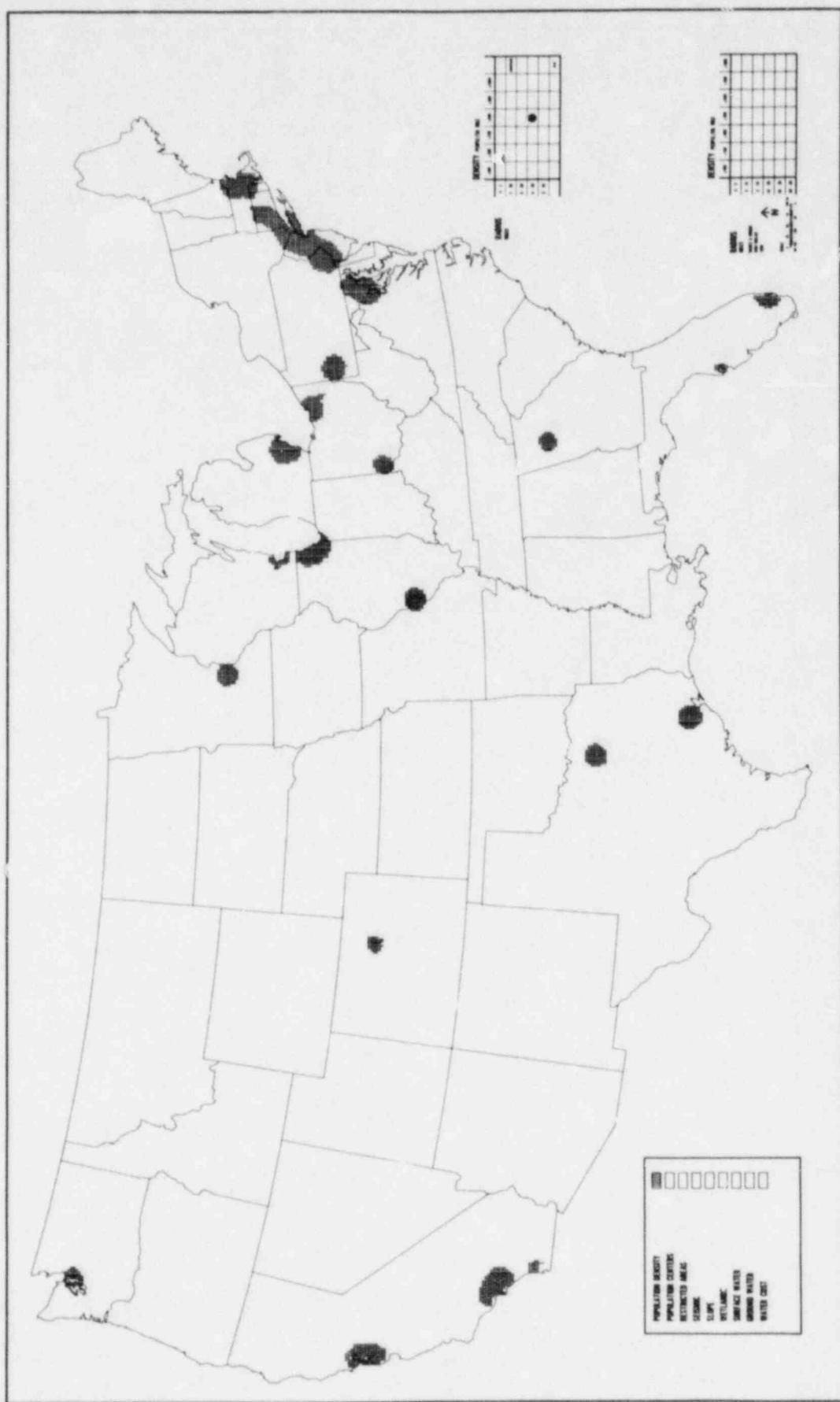


FIGURE F9.15

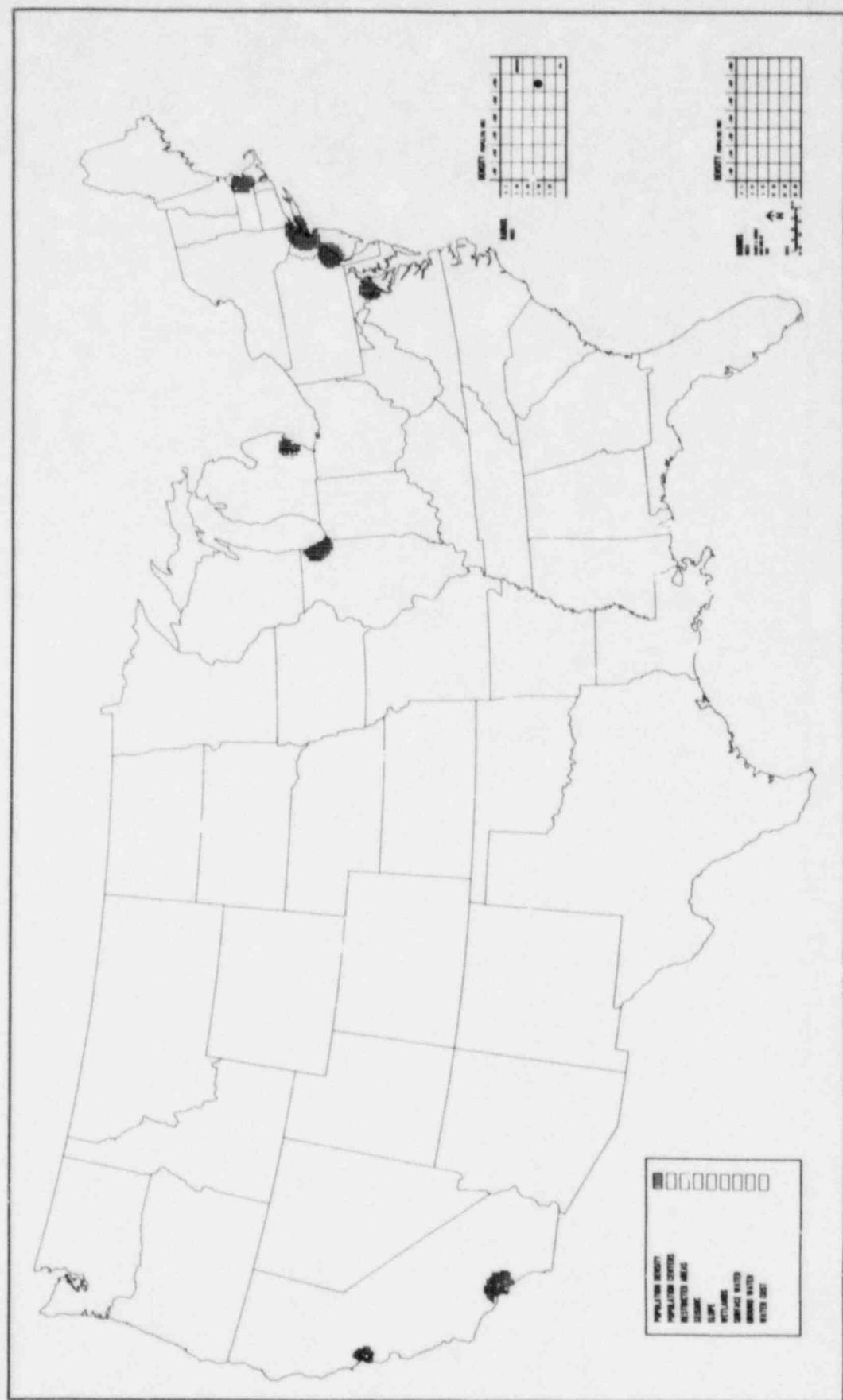


FIGURE F9.16

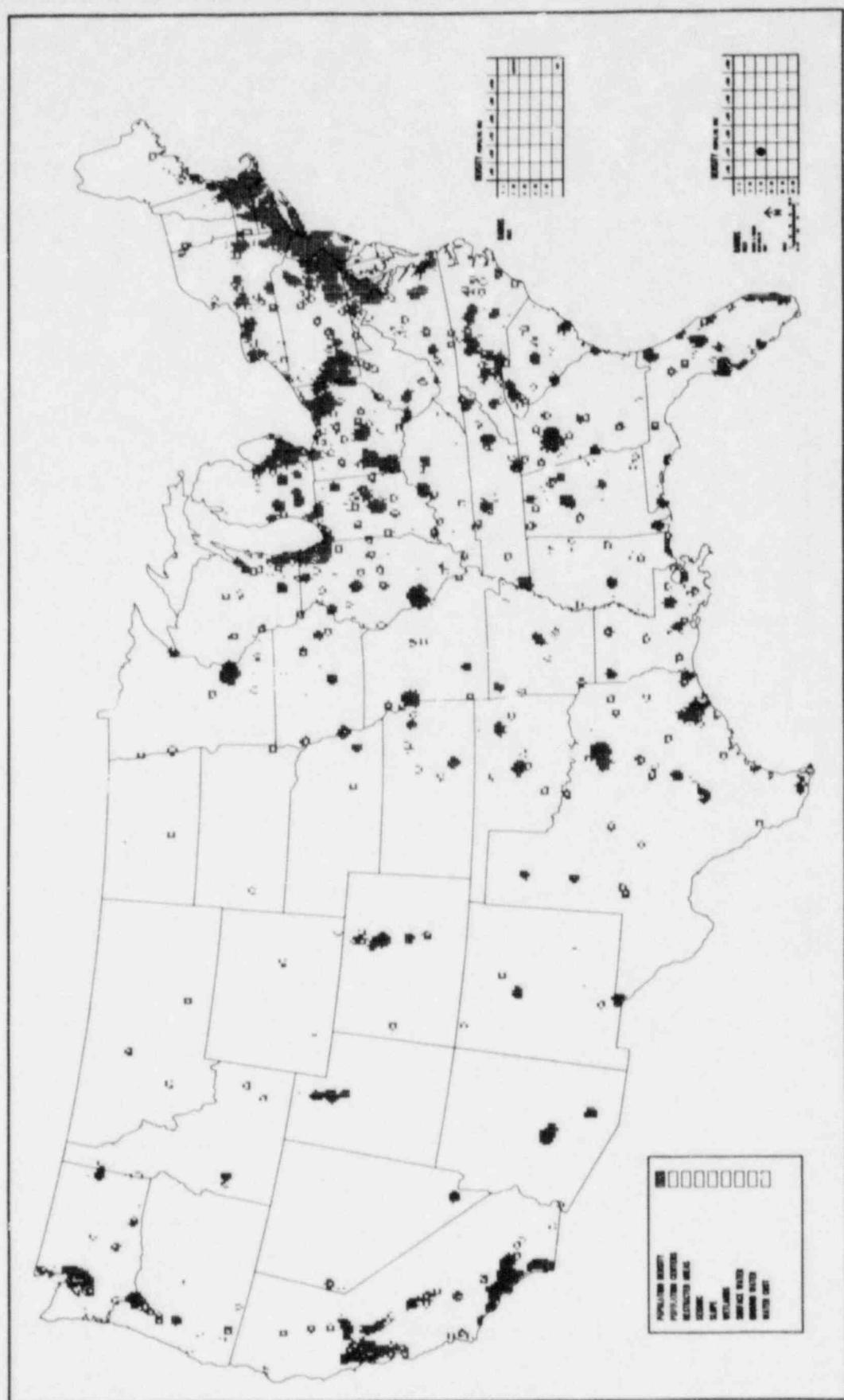


FIGURE F9.17

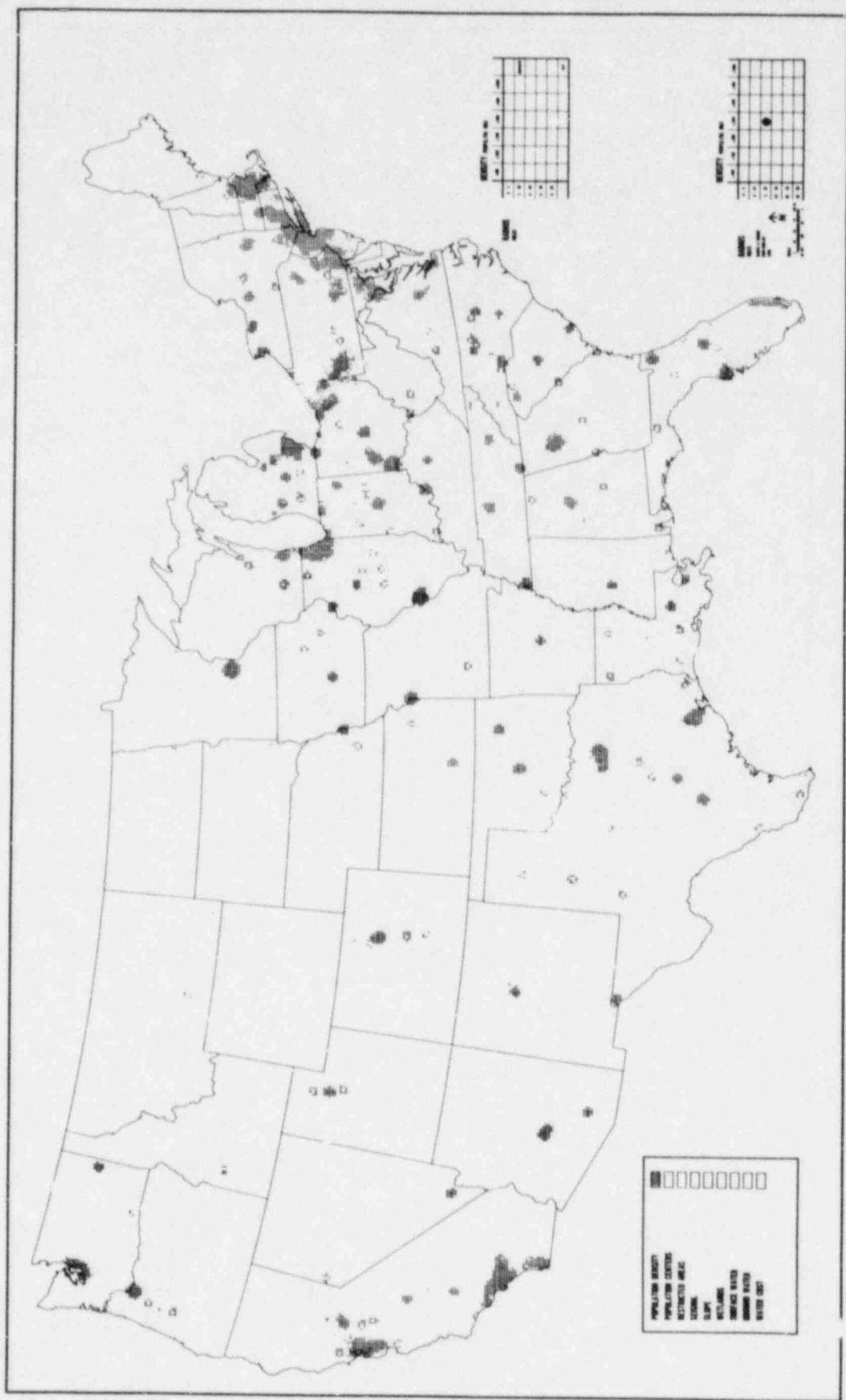


FIGURE F9.18

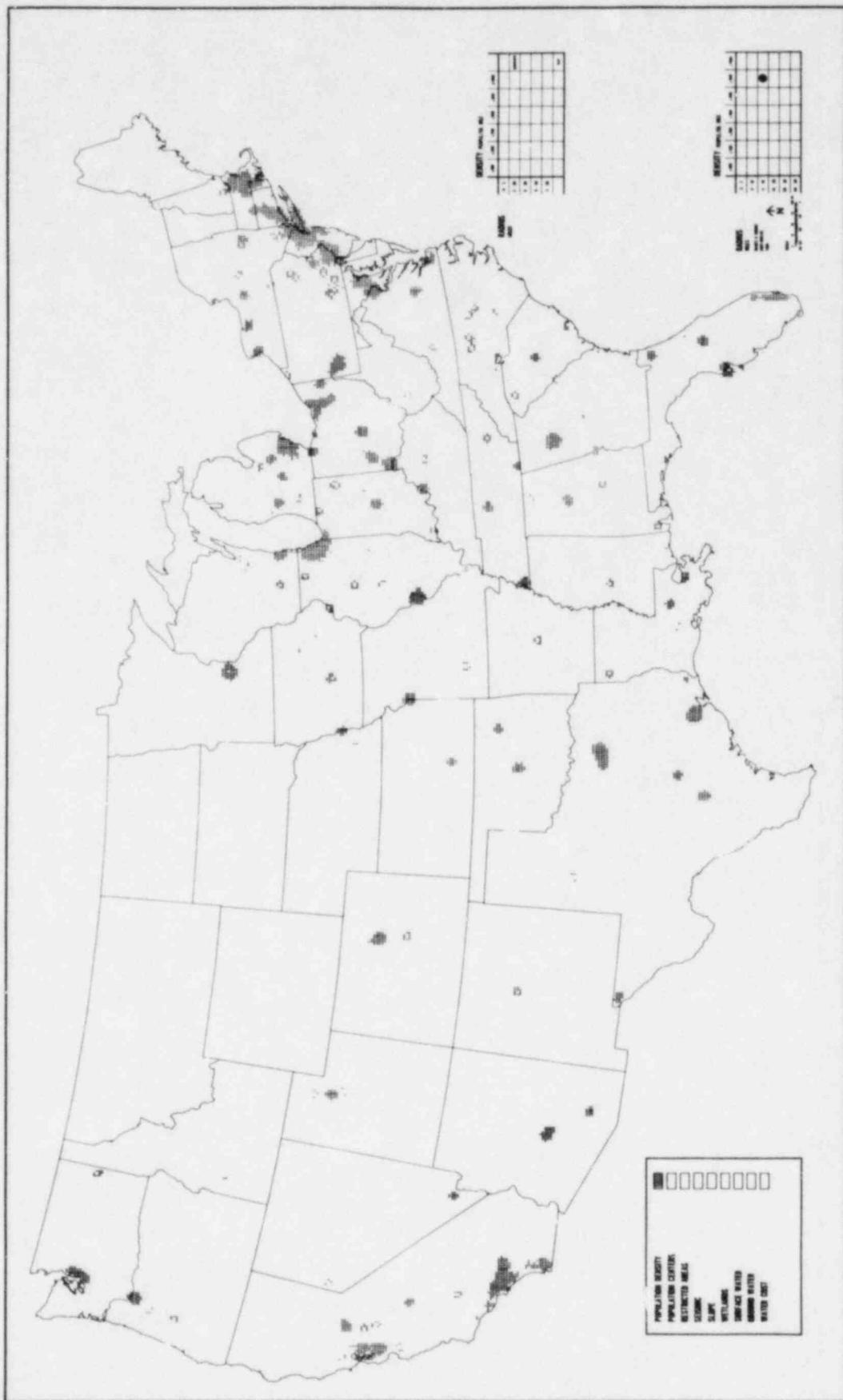


FIGURE F9.19

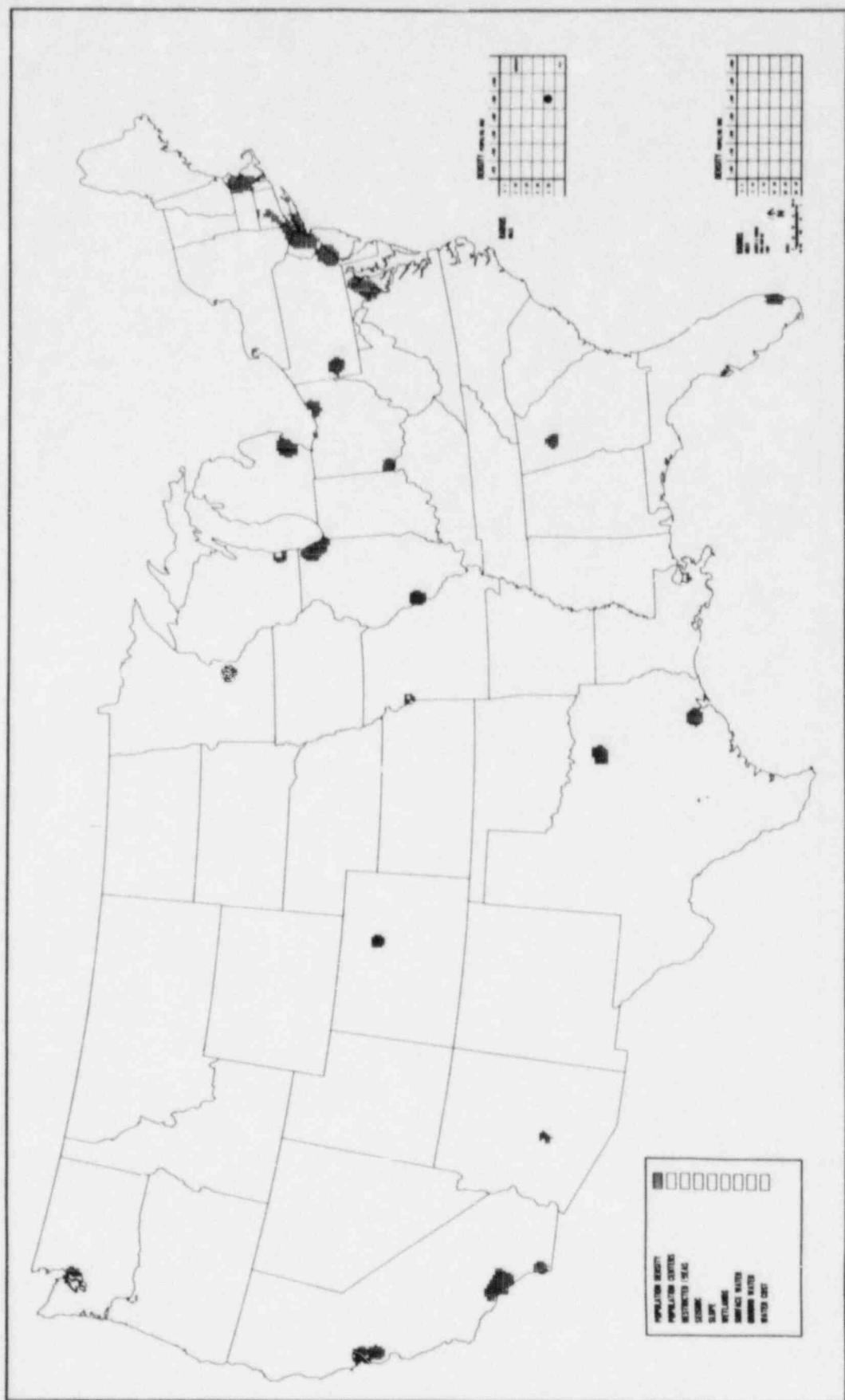


FIGURE F9.20

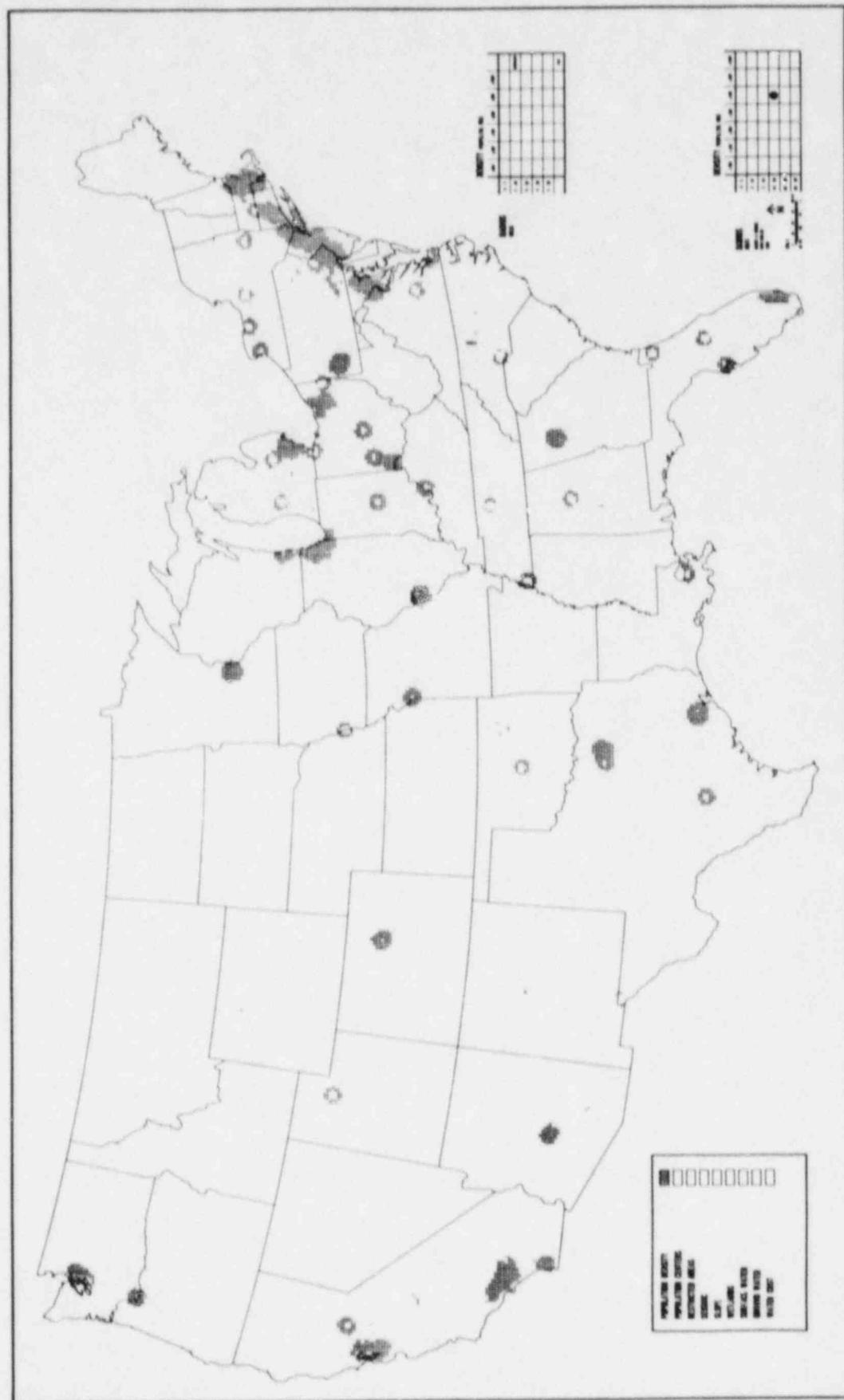


FIGURE F9.21

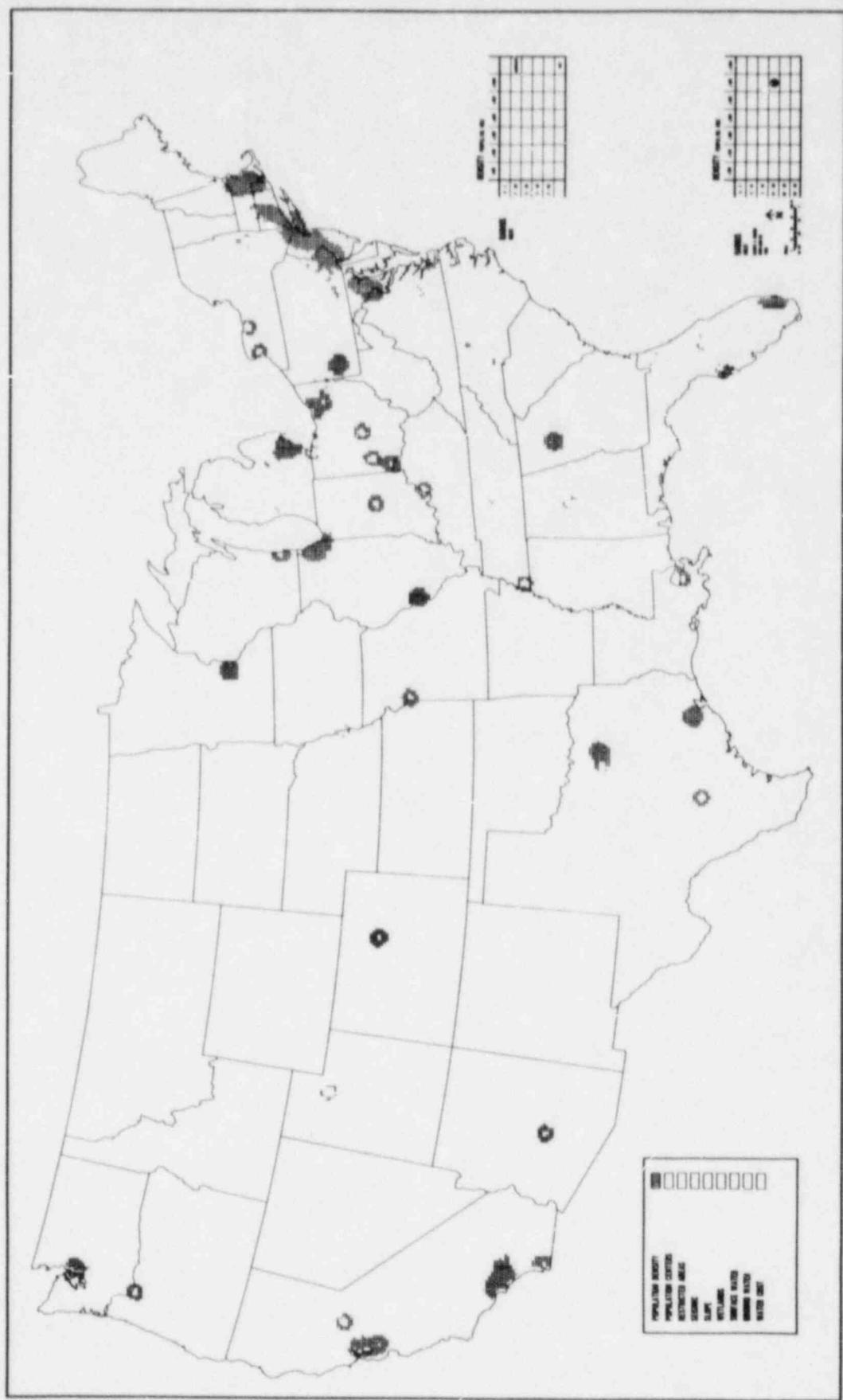


FIGURE F9.22

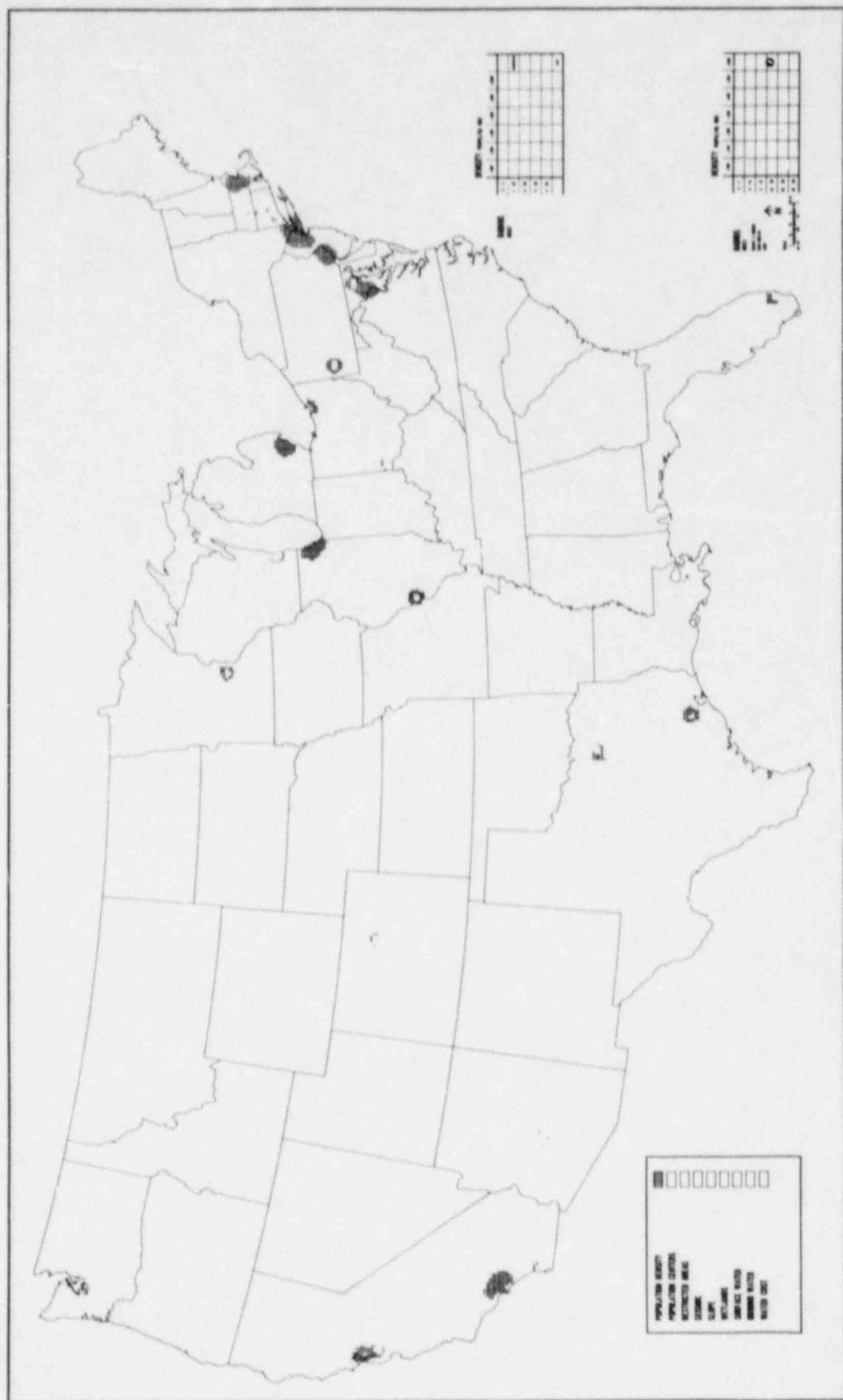


FIGURE F9.23

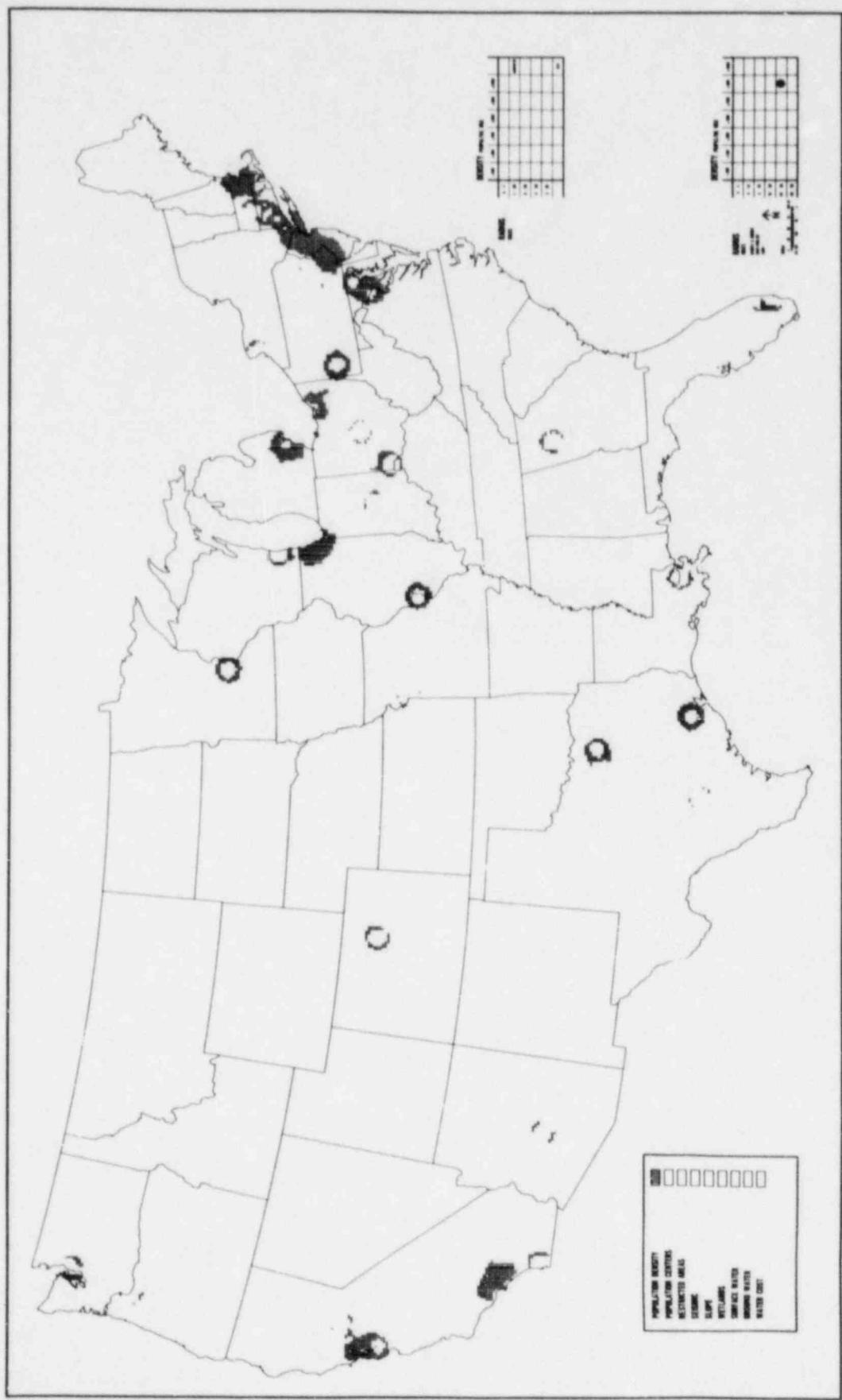
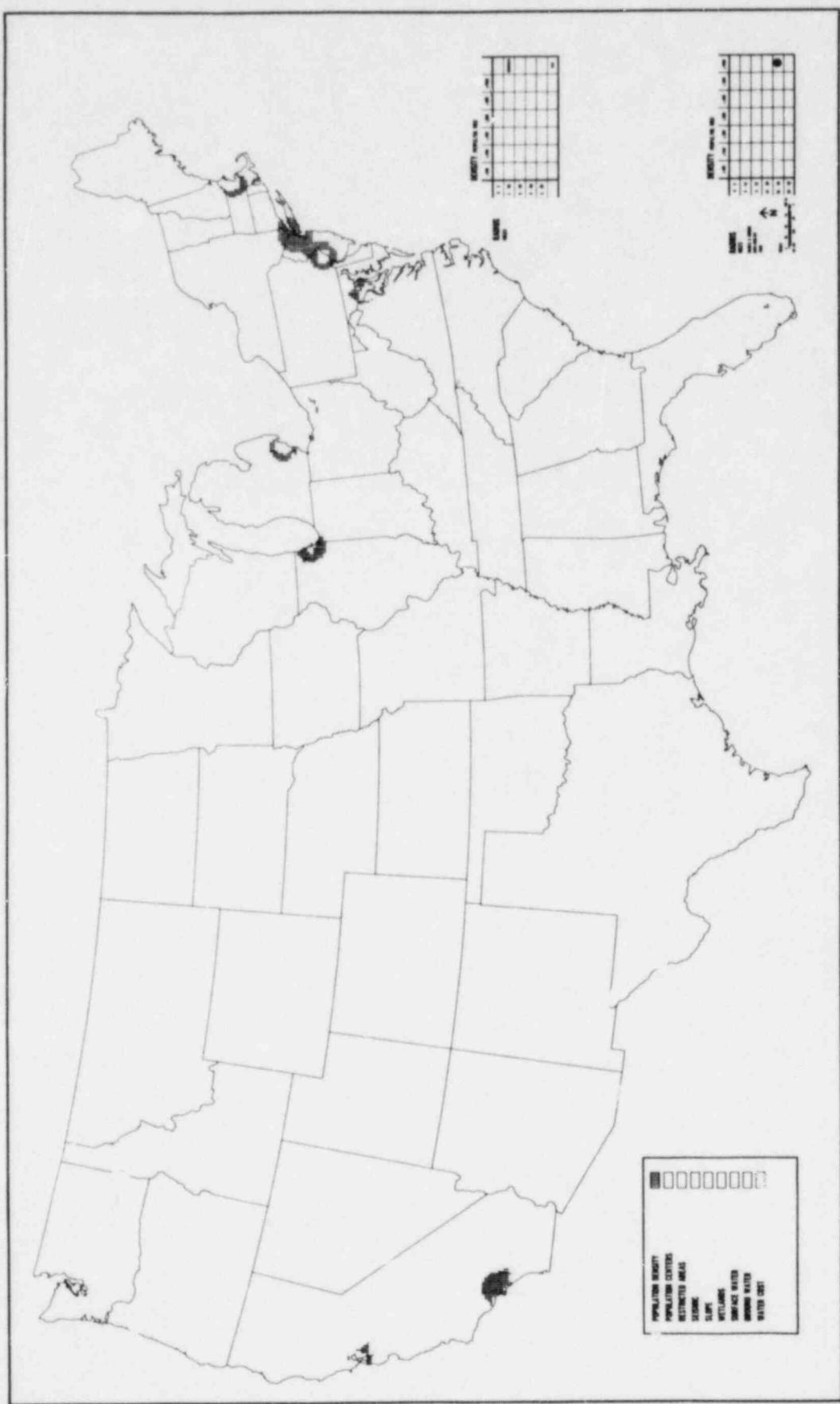


FIGURE F9.24



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FIGURE F9.25

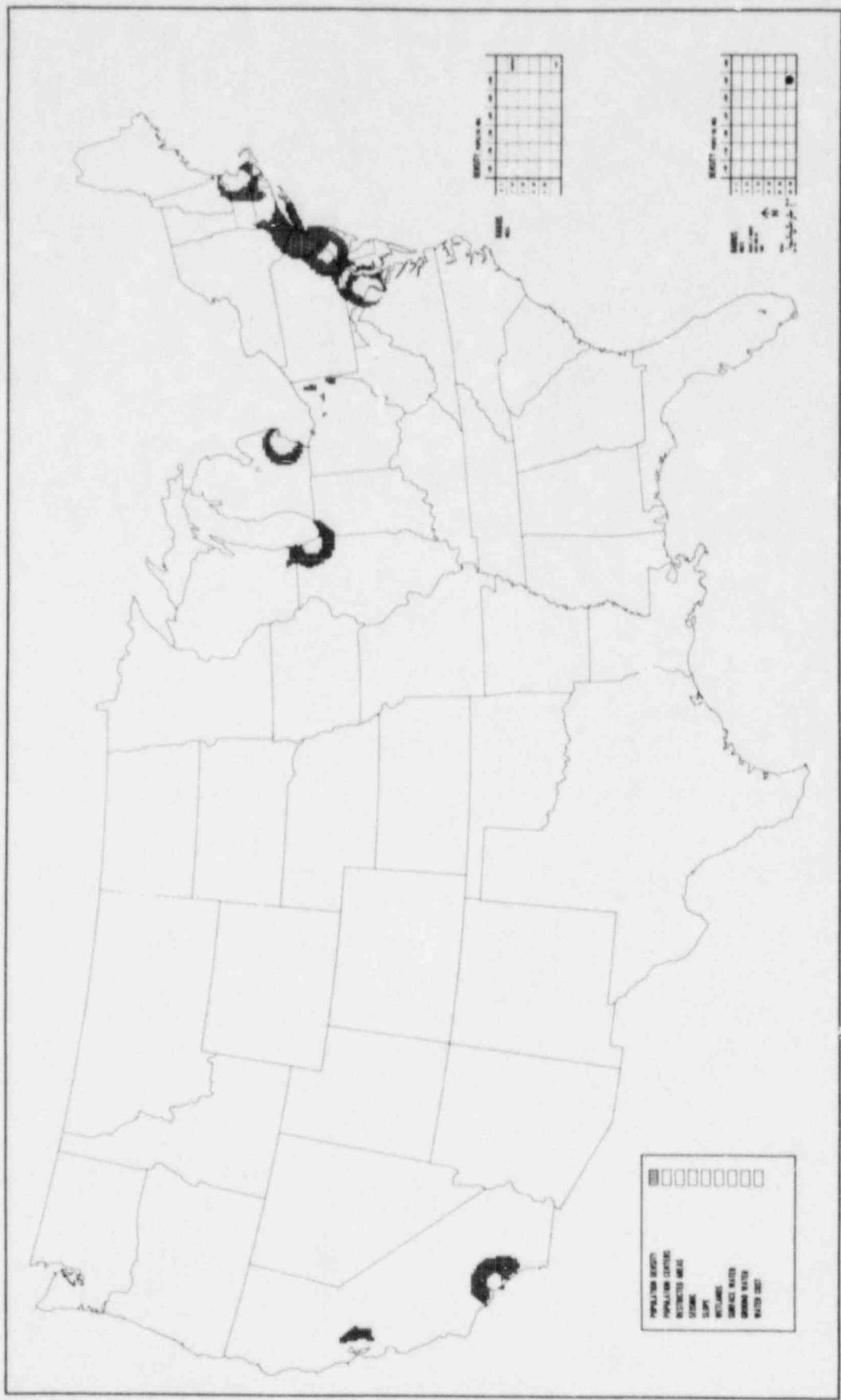


FIGURE F9.26



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FIGURE F10.1

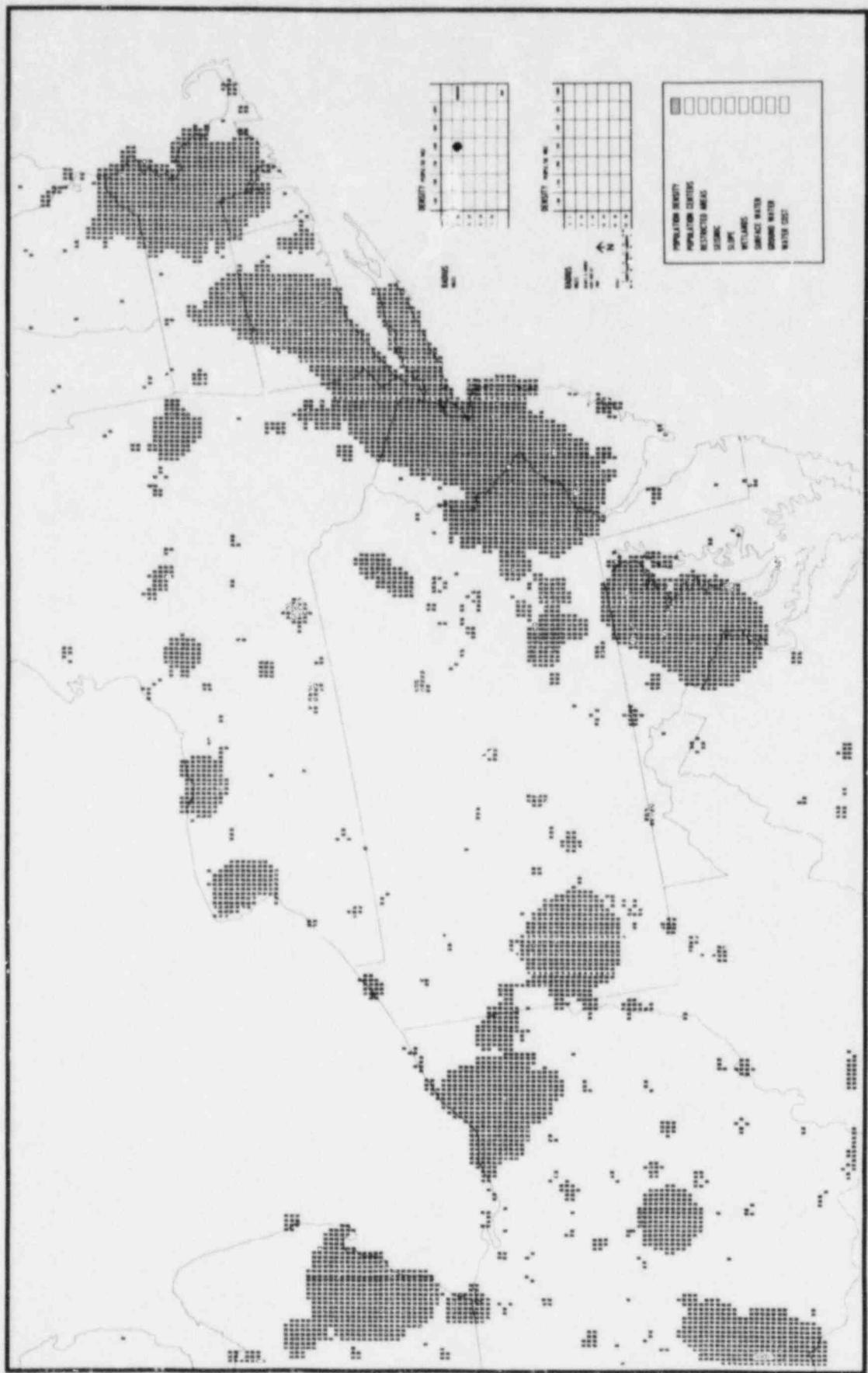


FIGURE F10.2

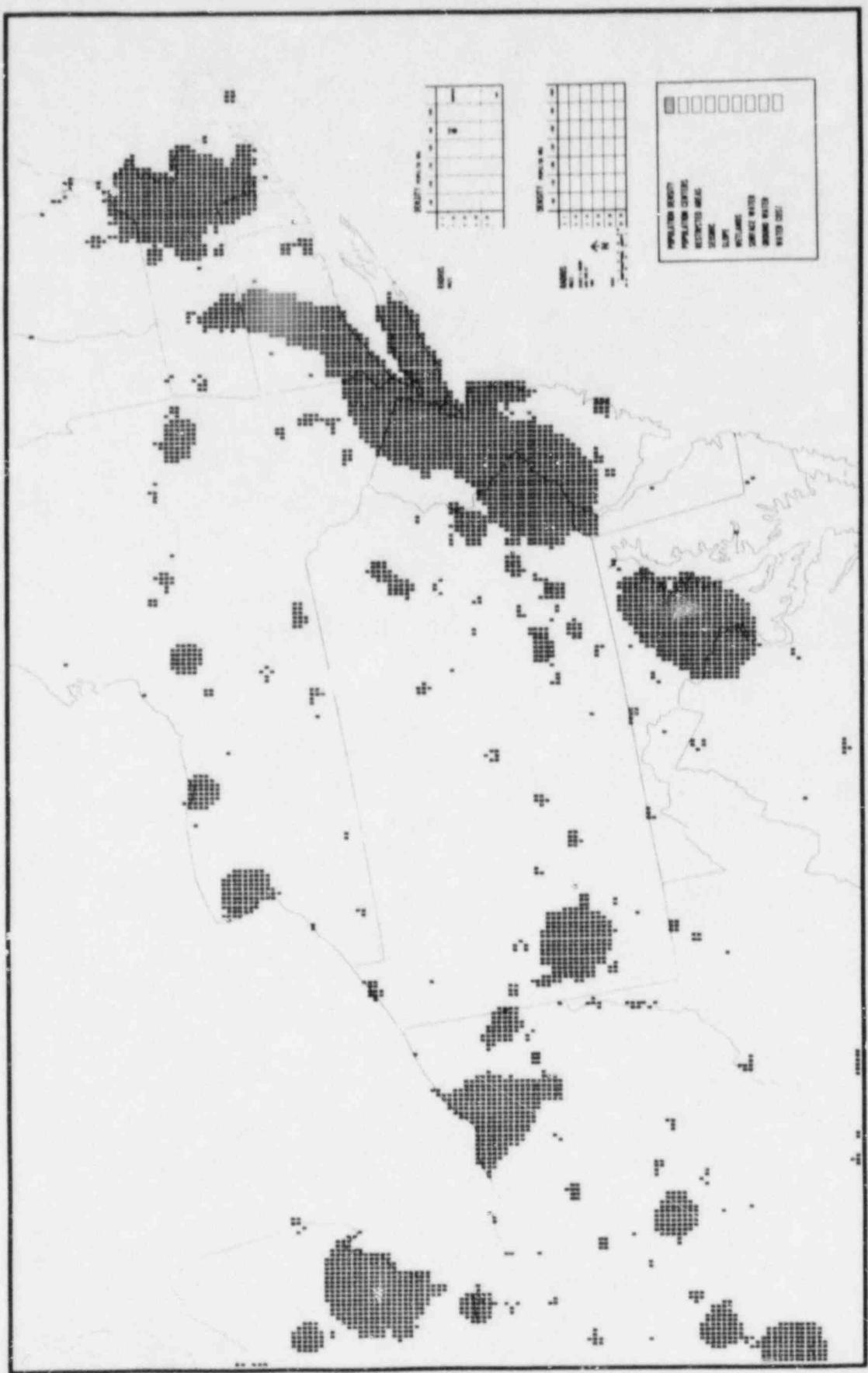
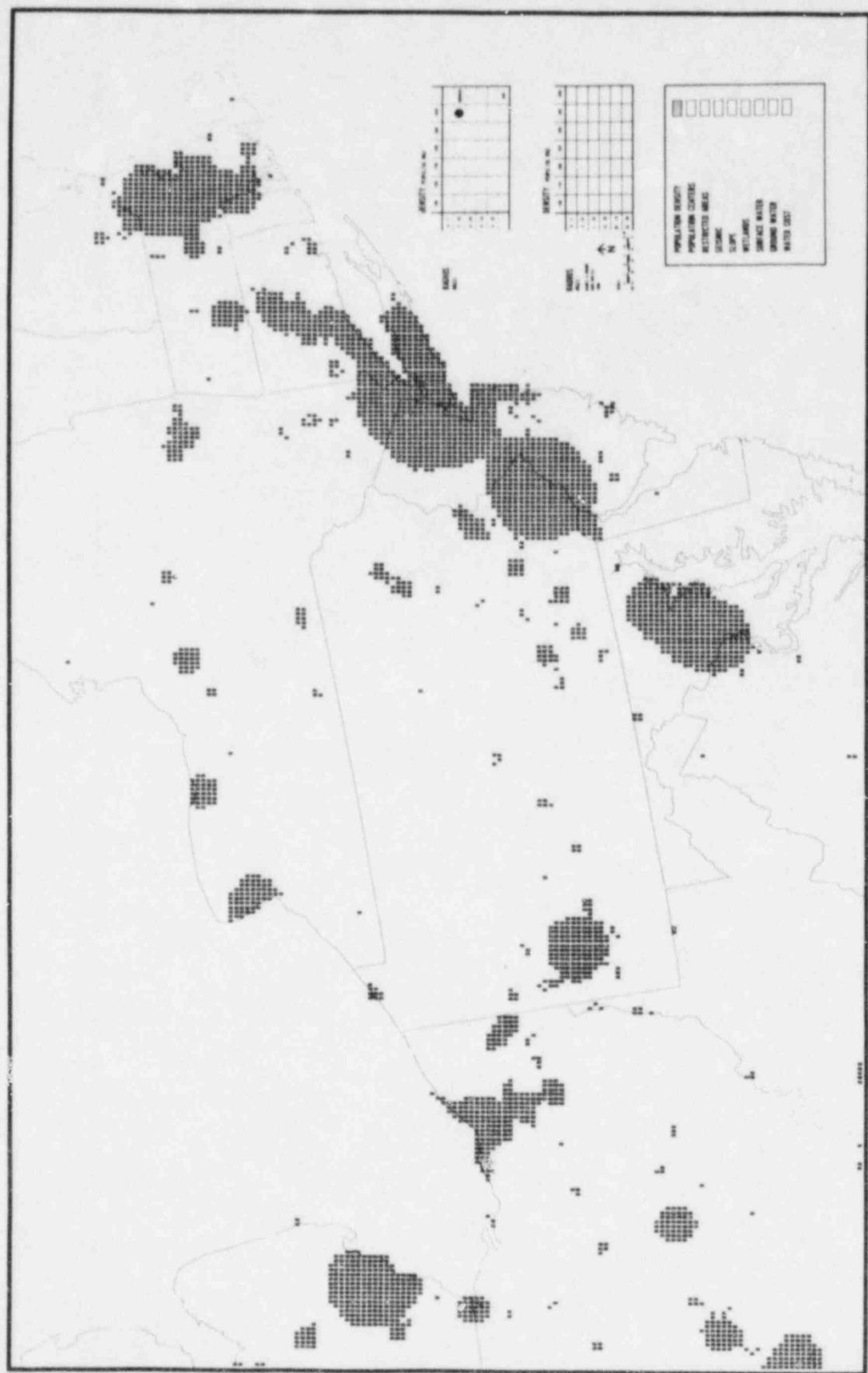


FIGURE F10.3



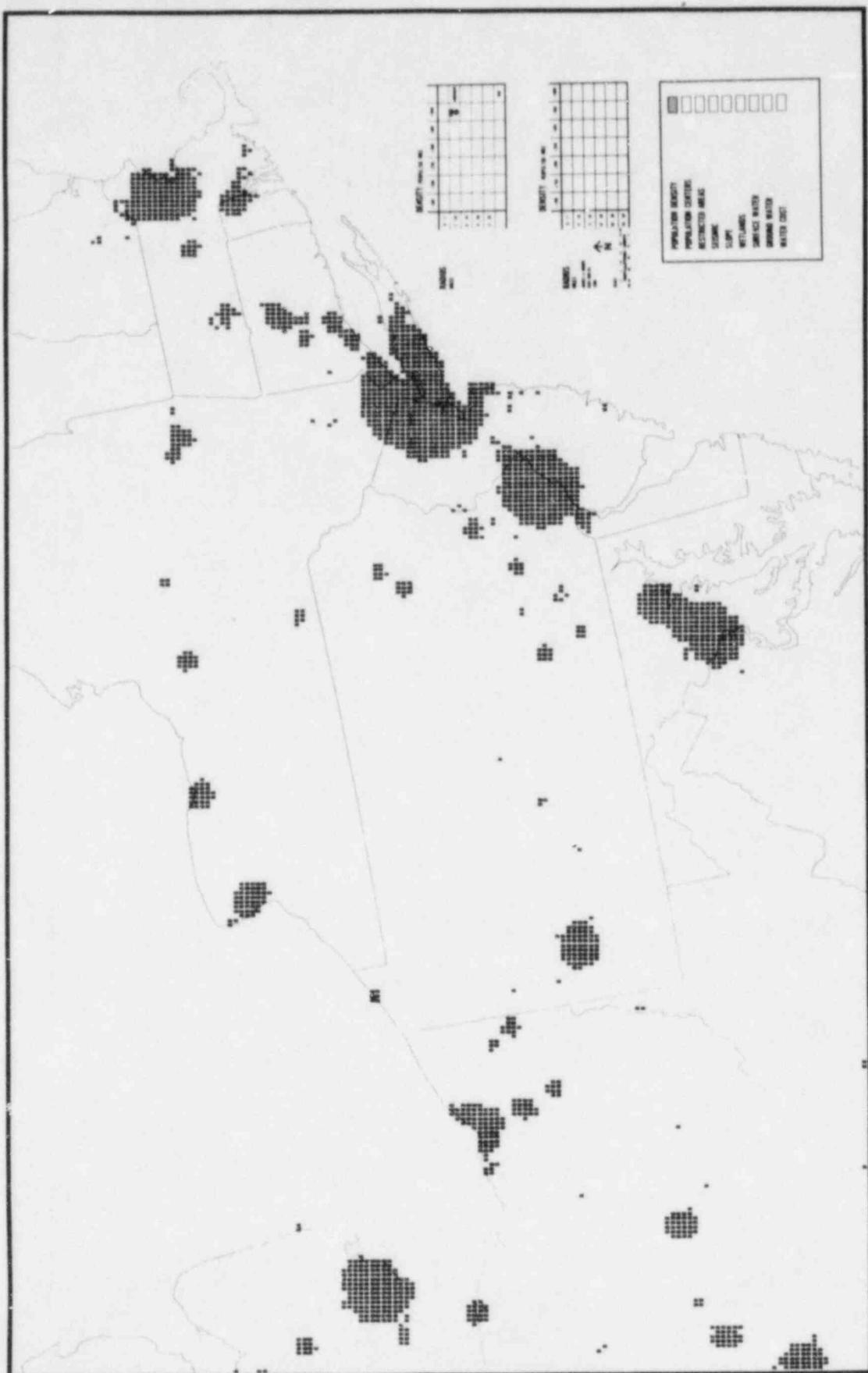


FIGURE F10.4

FIGURE F1'

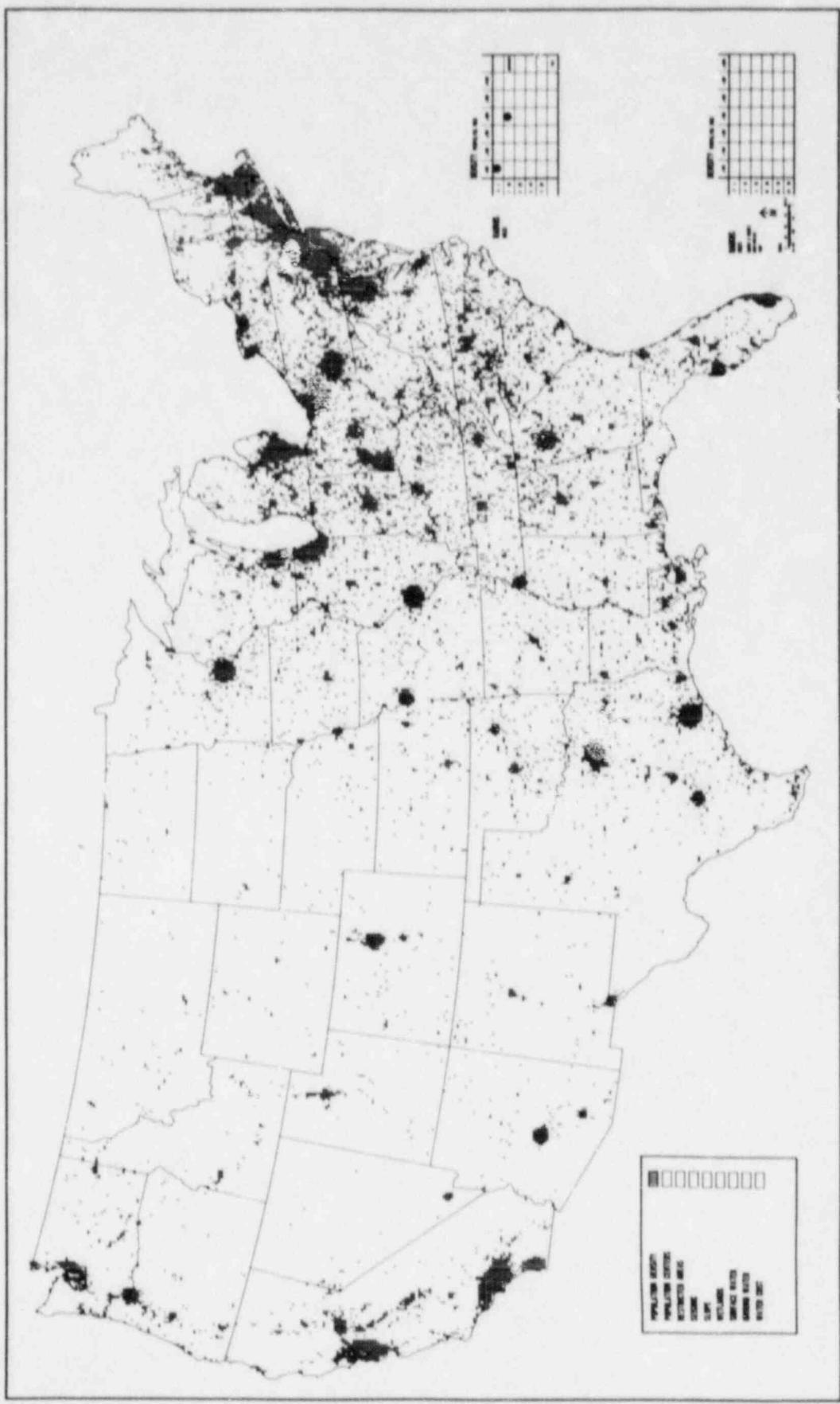


FIGURE F12

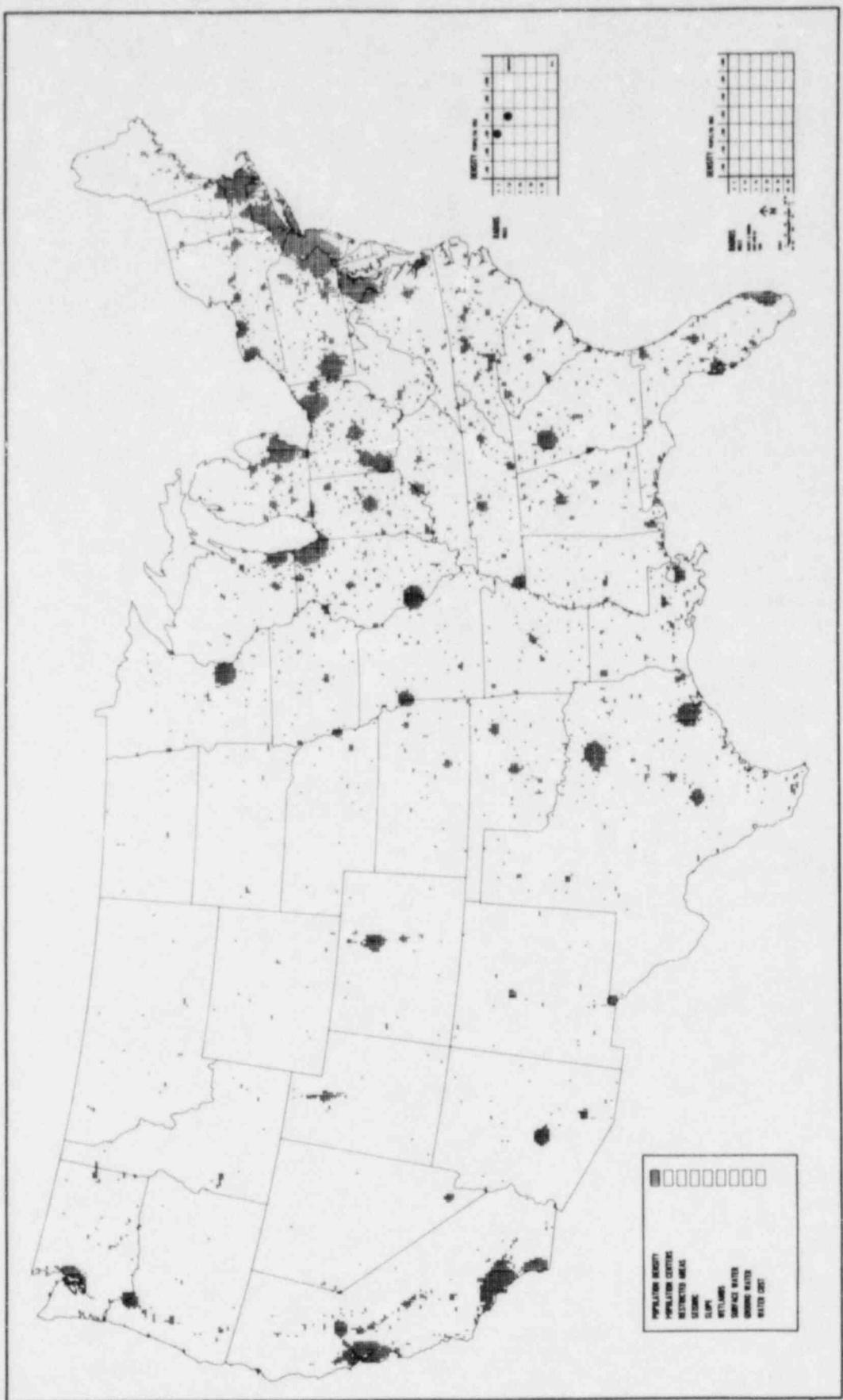


TABLE F1.1

SEISMIC HARDENING UTILITY FUNCTION \*\*\*  
COSTS IN MILLIONS OF DOLLARS 1980  
STATE AREAS IN SQUARE MILES AND % OF STATE

UTILIZATION	INESTIMABLY HIGH								FESTIVELY HIGH	
	3x 4 TO 4x 4		3x 3 TO 3x 4		2x 2 TO 3x 3		1x 2 TO 2x 2			
	12	1	12	1	12	1	12	1		
UTILITY VALUE	1	2	3	4	5	6	7	8		
ALASKA	0.0	0.0	0.0	0.0	0.0	164	5701	22453	2775 81417	
ARIZONA	0.21	0.10	0.01	0.01	0.01	11.01	46.71	147.71	11.021	
AR-KANSAS	0.21	0.01	0.15	0.21	0.21	5.11	2.31	4.21	12.21	
ARIZONA	0.345	47.3	521	768	724	1968	3742	34552	42.31	
CALIFORNIA	0.161	0.43	1.01	1.21	1.45	2.71	3.01	3.71	6.01	
CALIFORNIA	0.426	2200	2162	2071	3416	8238	14147	11957	14.421	
CALIFORNIA	0.39	1.43	1.21	1.01	0.13	5.21	6.01	7.31	27.31	
COLORADO	0.0	0.0	0.0	0.0	0.0	0.0	4211	2152	7212 21421	
CONNECTICUT	0.0	0.0	0.0	0.0	0.0	160.01	60.01	6.01	0.01	
DELAWARE	0.0	0.01	0.01	0.01	0.01	0.01	0.01	0.01	0.01	
FLORIDA	0.0	0.01	0.01	0.01	0.01	0.01	0.01	0.01	0.01	
GEORGIA	0.0	0.71	1.31	2.41	5.81	23.91	11.81	44.31	1.71	
GEORGIA	0.736	849	946	1201	965	2342	6771	26557	3751 62541	
IDAHO	0.13	1.01	1.11	1.41	1.25	2.81	6.11	21.41	43.41	
ILLINOIS	2364	975	1341	1768	2626	8882	3848	31391	1361 26541	
ILLINOIS	4.21	1.71	2.41	3.11	4.61	15.71	10.31	55.71	2.41	
INDIANA	0.0	0.0	0.0	0.0	0.0	0.0	608	2351	2351 2221	
IDAHO	0.0	0.01	0.01	0.01	0.01	0.01	0.01	0.01	0.01	
KANSAS	0.0	0.01	0.01	0.01	0.01	0.01	0.01	0.01	0.01	
KENTUCKY	1287	232	174	212	415	2682	4201	28593	2471 40261	
KENTUCKY	3.21	0.81	0.41	0.51	1.01	6.71	10.41	71.01	0.11	
LOUISIANA	0.0	0.0	0.0	0.0	0.0	0.0	0	3373	1441 48151	
MAINE	0.0	0.0	0.0	0.0	0.0	0.0	10721	7521	15015 357 34074	
MARYLAND	0.0	0.0	0.0	0.0	0.0	0.0	0	0	11111 145 11151	
MASSACHUSETTS	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	
MISSOURI	0.0	0.0	0.0	0.0	0.0	0.0	100.01	0.01	0.01 0.01 0.01	
MISSOURI	0.0	0.0	0.0	0.0	0.0	0.0	0	52156	5374 61831	
MICHIGAN	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	
MICHIGAN	0.0	0.0	0.0	0.0	0.0	0.0	0	60966	24926 85914	
MINNESOTA	0.0	0.01	0.01	0.01	0.01	0.01	0.01	0.01	0.01	
MISSISSIPPI	0.11	0.31	0.41	0.51	0.71	2.61	6.21	81.21	8.01	
MISSISSIPPI	7981	589	618	685	926	4941	7324	42754	4516 69934	
MISSOURI	11.43	0.83	0.43	1.03	1.33	7.13	10.31	60.81	6.51	
MISSOURI	1351	251	376	521	2036	10895	6272	79593	47161 148451	
MONTANA	0.9	0.23	0.31	0.41	1.41	7.31	4.21	53.61	31.81	
NEBRASKA	0.0	0	0	0	0	0	5694	3628	66665 1534 77721	
NEVADA	0.0	0.01	0.01	0.01	0.01	0.01	7.31	4.71	30.01	
NEW HAMPSHIRE	0.0	0.0	0.0	0.0	0.0	0.0	7595	676	0 119 9465	
NEW JERSEY	0.0	0.0	0.0	0.0	0.0	0.0	3049	93	4024 0 8031	
NEW MEXICO	0.0	0.0	0.0	0.0	0.0	0.0	18210	11792	60206 31531 121744	
NEW YORK	0.0	0.0	0.0	0.0	0.0	0.0	21037	7151	12101 9930 50211	
NORTH CAROLINA	0.0	0.0	0.0	0.0	0.0	0.0	154	10905	4970 26113 8627 50761	
NORTH DAKOTA	0.0	0.0	0.0	0.0	0.0	0.0	0	64430	6572 71005	
OHIO	0.0	0.01	0.01	0.01	0.01	0.01	0.01	90.71	0.31	
OKLAHOMA	0.0	0.0	0.0	0.0	0.0	0.0	24617	15932	25601 3464 46614	
OREGON	0.0	0.0	0.0	0.0	0.0	0.0	183	454	69946 30346 97926	
PENNSYLVANIA	0.0	0.0	0.0	0.0	0.0	0.0	6745	4449	30333 355 45276	
RHODE ISLAND	0.01	0.05	0.05	0.05	0.03	100.01	0.01	0.01	0.01	
SOUTH CAROLINA	12603	1842	2548	2584	5037	3968	0	0	2663 31186	
SOUTH DAKOTA	40.41	5.41	8.21	8.31	14.21	12.51	0.01	0.01	8.51	
TENNESSEE	0.0	0	0	0	0	0	0	0	0	
TEXAS	0.0	0.0	0.0	0.0	0.01	0.01	0.41	97.91	2.01	
UTAH	20.61	1.51	1.41	2.11	2.61	5.71	6.11	30.01	30.01	
VERMONT	0.0	0.0	0.0	0.0	0.0	0.0	89.61	0.01	0.01 10.41	
VIRGINIA	0.0	0.0	0.0	0.0	0.0	0	5404	1963	28456 563 41165	
WASHINGTON	7044	394	454	511	647	2528	5037	27927	24762 46214	
WEST VIRGINIA	0.0	0.0	0.0	0.0	0	0	1146	1139	19387 2721 24101	
WISCONSIN	0.0	0.0	0.0	0.0	0.01	4.81	4.71	79.21	0.21	
WYOMING	0.01	0.31	0.31	0.21	0.31	1.31	3.31	6.71	21.71	
TOTAL	1e3286	12710	15141	20207	38349	257403	183612	1791570	551571	
	5.41	0.41	0.51	0.71	1.31	8.51	6.01	56.71	16.71	

\*\*\* UTILITY VALUES ARE DERIVED FROM MAP OF EFFECTIVE PEAK ACCELERATION EXPRESSED AS LG (GRAVITY) AND ASSOCIATED COSTS OF SEISMIC HARDENING. COSTS ARE RELEVANT TO 1100 MM PLANT FOR SAFE SHUTDOWN EARTHQUAKE. THE LG HAS A PROBABILITY OF LESS THAN 0.5% OF BEING EXCEEDED IN 50 YEARS. "UNESTIMABLY HIGH" REFERS TO AREAS WITH GREATER THAN 80% COSTS FOR AREAS WITH 20% TO 80% WERE DIVIDED INTO EQUAL INTERVALS AND ASSIGNED UTILITY VALUES.

TABLE F1.2

SITE PREPARATION UTILITY FUNCTION \*\*\*  
 PER CENT OF AREA LESS THAN 8% SLOPE (GENTLY SLOPING)  
 STATE AREAS IN SQUARE MILES AND % OF STATE

## TABULATION

UTILITY VALUE	UNDER 20% OF AREA		20% TO 50% OF AREA		50% TO BOV% OF AREA		MORE THAN BOV% AREA		RESTRICTED LANDS	
	1	2	3	4	5	6	7	8		
	0	17360	30069	2402	2075	51507				
ALABAMA	0	331	587	3%	4%					
ARIZONA	125	2972	51348	492	59407	114342				
ARKANSAS	0	31	45%	0%	52%					
CALIFORNIA	4246%	7131	14581	18914	6262	53258				
COLORADO	125	133	271	16%	22%					
CONNECTICUT	0	2673	2530	0	0	5211				
DELAWARE	0	313	491	0%	0%					
FLORIDA	0	0	4941	41312	13105	59758				
GEORGIA	530	5742	30755	15691	5867	58605				
IDAHO	4362	13609	37860	0	3719	83550				
ILLINOIS	0	1012	29461	24700	1361	56539				
INDIANA	0	2557	13017	19146	1322	36342				
LOUISIANA	0	73	373	301	4%					
MAINE	0	13890	37249	4922	0	56067				
KANSAS	0	8502	46540	25032	143	62267				
KENTUCKY	0	101	597	303	0%					
Louisiana	24%	361	523	443	2%					
MISSOURI	0	0	9973	24164	14417	48154				
MONTANA	0	0	203	501	30%					
NEBRASKA	618	11223	21877	0	357	34075				
NEVADA	0	1351	5809	3830	145	11155				
MASSACHUSETTS	0	2741	5887	0	0	3429				
MICHIGAN	0	0	30166	21992	6679	61837				
MINNESOTA	0	0	471	363	161					
MISSISSIPPI	0	4507	29770	9746	3841	47884				
MISSOURI	0	91	623	203	83					
NEVADA	7662	26904	18846	12005	4518	69933				
NEW HAMPSHIRE	0	113	261	173	63					
NEW JERSEY	0	1013	1554	5443	0	8010				
NEW MEXICO	5250	8849	42628	13471	31936	121744				
NEW YORK	2125	11252	24926	975	9930	50219				
NORTH CAROLINA	5461	2586	19454	17640	8427	50768				
NORTH DAKOTA	0	53	381	351	173					
OHIO	6447	154	14649	15257	2326	41833				
OKLAHOMA	1090	4680	49447	10932	3464	69614				
OREGON	16495	18789	32076	0	30349	479920				
PENNSYLVANIA	7054	24743	9891	39	3551	45278				
RHODE ISLAND	0	163	551	223	0%					
SOUTH CAROLINA	336	376	11495	19826	2663	31186				
SOUTH DAKOTA	1274	416%	31874	16897	22793	77007				
TENNESSEE	1312	19522	17727	965	2996	42122				
TEXAS	5294	8106	174906	74942	5491	248839				
UTAH	1708	8086	43483	6031	25553	85181				
VERMONT	1640	3616	1573	0	1020	9852				
VIRGINIA	173	573	161	0%	10%					
WISCONSIN	0	10567	33283	8145	5028	57023				
WYOMING	6253	13105	53403	0	25225	97986				
TOTAL	0	141193	415887	1383039	522176	557673				
	5%	143	451	173	183					

\*\*\* SITE PREPARATION UTILITY IS DERIVED FROM A CONSIDERATION  
 OF AN AREA'S TOPOGRAPHIC CHARACTER. SOURCE DATA IS A MAP  
 INDICATING % OF AREA THAT IS GENTLY SLOPING (LESS THAN 8%  
 SLOPE) AND CONTAINS 4 CATEGORIES. UTILITY VALUES WERE  
 ASSIGNED ON THE BASIS OF RELATIVE DEGREE OF DIFFICULTY  
 FOR ACCESS AND CONSTRUCTION.

TABLE 1.3

AGGREGATE WATER (SURFACE & GROUND) UTILITY FUNCTION \*\*\*  
 COSTS IN MILLIONS OF DOLLARS (1960)  
 STATE AREAS IN SQUARE MILES AND % OF STATE

TABULATION	MORE THAN \$300 MILLION										PERMITTED LAND						
	262.5 TO 300.0		225.0 TO 262.5		187.5 TO 225.0		150.0 TO 187.5		112.5 TO 150.0		75.0 TO 112.5		37.5 TO 75.0		0.0 TO 37.5		
UTILITY VALUE	1	2	3	4	5	6	7	8	9	10	11	12	13	14	15	16	17
ALABAMA	0	0	0	0	0	87	1370	10364	17686	18325	2072	5111					
ARIZONA	897	374	447	1119	1563	8175	36303	3634	2497	5940	11830						
ARKANSAS	17	95	115	135	155	75	325	3	21	52							
CALIFORNIA	0	0	0	0	1014	1918	6567	6745	18917	8295	83244						
COLORADO	0	0	25	65	35	45	125	2	31	51	11	11					
CONNECTICUT	297	244	204	304	449	562	1140	1745	414	2095	104321						
DELAWARE	0	0	0	0	43	53	113	113	45	87							
FLORIDA	0	0	0	0	0	0	0	0	637	165	31	232					
GEORGIA	0	0	0	0	0	0	0	0	10	2632	25611	13101	58312				
IDAHO	0	0	0	0	0	0	0	0	355	43	22						
ILLINOIS	77	309	405	720	860	1361	17426	11908	12941	37515	8155						
INDIANA	0	0	0	0	13	15	21	21	145	15	45						
IOWA	0	0	0	0	0	0	10	19480	26460	16239	361	5658					
KANSAS	0	0	0	0	0	0	0	0	480	18721	11415	132	36362				
KENTUCKY	321	244	41	41	95	41	163	20	75	0	4024						
Louisiana	0	0	0	0	0	360	217	932	1353	10391	2470	40245					
Maine	0	0	0	0	0	0	0	0	26	26	8						
MARYLAND	0	0	0	0	0	0	0	0	11	375	341	18	0				
Massachusetts	0	0	0	0	0	0	0	0	169	2779	4150	0	862				
MICHIGAN	0	0	0	0	0	0	0	0	19557	21184	88417	9674	81831				
MINNESOTA	0	0	0	0	0	0	0	0	173	313	365	163					
MISSISSIPPI	0	0	0	0	0	0	0	0	35792	17447	8687	84926	85913				
MISSOURI	0	0	0	0	0	0	0	0	77	2126	2230	7459	145	1115			
Montana	251	924	3862	6410	9344	13327	25013	23362	18422	47160	148450						
NEBRASKA	8211	2133	2837	4424	48600	9100	17660	18287	7650	1334	77721						
NEVADA	75	35	41	61	75	125	235	245	125	25							
NEW HAMPSHIRE	0	0	0	0	0	0	0	0	86339	3154	899	20255	110619				
NEW JERSEY	0	0	0	0	0	0	0	0	227	365	135	800					
NEW MEXICO	91540	3620	2277	2024	1891	1860	19486	4523	1554	31324	121743						
NEW YORK	0	0	0	0	0	0	0	0	11242	15178	2007	205	30216				
NORTH CAROLINA	0	0	0	10	154	309	447	4447	19300	17254	8627	50769					
NORTH DAKOTA	11754	4562	7121	8919	8790	8236	11011	6620	4410	4572	71005						
OHIO	0	0	0	0	0	367	1679	12445	1872	1521	4326	41833					
OKLAHOMA	7671	1264	1525	1998	2020	9336	12765	24366	8164	3464	67615						
OREGON	0	0	0	0	0	0	0	0	34952	15643	14559	30345	97926				
PENNSYLVANIA	0	0	0	0	0	173	2432	11307	15662	12053	3551	45226					
RHODE ISLAND	0	0	0	0	0	0	0	0	19	415	772	0	1206				
SOUTH CAROLINA	0	0	0	0	0	0	0	0	2760	9862	15701	2463	31189				
SOUTH DAKOTA	9139	4555	5086	5279	4893	4256	9196	7102	4709	22793	77008						
TENNESSEE	0	0	0	0	0	10	320	1940	9228	18164	11640	2594	42124				
TEXAS	80275	3474	3460	3468	2959	9071	70454	42526	18933	5491	26887						
UTAH	313	15	15	15	15	25	35	25	14	75	0						
VERMONT	0	0	0	0	0	0	0	0	4613	2837	1362	1022	9852				
VIRGINIA	0	0	0	0	0	376	1129	2142	6756	12487	12891	5647	81167				
WASHINGTON	0	0	0	0	10	183	320	9225	13074	21433	24762	87317					
WEST VIRGINIA	0	0	0	0	0	0	0	0	145	193	31	36					
WISCONSIN	0	0	0	0	0	116	1235	13394	19902	17447	3626	57112					
Wyoming	2943	3937	5317	6196	17430	9505	10607	10616	6012	25227	97481						
TOTAL	227705	32875	39778	49727	103681	110664	740567	653682	522404	55767	18						

\*\*\* AGGREGATE WATER COST DERIVATION. LEAST COST ALTERNATIVE  
 WATER COST DETERMINED FOR COMPOSITE OF GROUNDWATER COST AND SURFACE  
 WATER COST. ESTIMATED GROUNDWATER COSTS FOR MAJOR REGIONS OF  
 THE COUNTRY WERE CALCULATED FROM INFORMATION REGARDING  
 QUALITY, QUANTITY, DEPTH AND SIZE OF WELL-FIELD. PLEASE SEE  
 SURFACE WATER UTILITY TABLE FOR DESCRIPTION OF SURFACE WATER  
 COSTS. AGGREGATE COSTS LESS THAN \$300 MILLION WERE DIVIDED  
 INTO 8 EQUAL INTERVALS.

TABLE F1.4

SURFACE WATER UTILITY FUNCTION \*\*\*  
COSTS IN MILLIONS OF DOLLARS (1980)  
STATE AREAS IN SQUARE MILES AND % OF STATE

TABULATION	MORE THAN \$300 MILL.											
	262.5 TO 300.0		225.0 TO 262.5		187.5 TO 225.0		150.0 TO 187.5		112.5 TO 150.0		75.0 TO 112.5	
UTILITY VALUE	1	2	3	4	5	6	7	8	9	10	11	12
ALABAMA	0	0	0	0	125	2731	8945	19684	18325	2675	81107	
ARIZONA	01	02	03	04	05	06	07	08	09	010	011	012
ARKANSAS	02012	4777	4364	4333	4140	4813	4362	3638	2997	94405	114342	
CALIFORNIA	173	42	43	41	42	43	43	43	43	25	82	
CONNECTICUT	0	0	77	473	1264	2374	6176	17795	18977	1877	53259	
DELAWARE	01	02	03	04	05	06	07	08	09	010	011	
FLORIDA	0	0	0	0	0	0	0	0	0	0	0	
GEORGIA	0	0	0	0	0	0	0	0	0	0	0	
IDAHO	01	02	03	04	05	06	07	08	09	010	011	
ILLINOIS	77	304	405	1218	3484	4386	9303	11908	12741	37315	83550	
INDIANA	0	0	0	0	0	0	0	0	0	0	0	
KANSAS	01	02	03	04	05	06	07	08	09	010	011	
KENTUCKY	0	0	0	0	0	0	0	0	0	0	0	
Louisiana	0	0	0	0	0	0	0	0	0	0	0	
Maine	0	0	0	0	0	0	0	0	0	0	0	
MARYLAND	0	0	0	0	0	0	0	0	0	0	0	
Massachusetts	0	0	0	0	0	0	0	0	0	0	0	
MICHIGAN	0	0	0	0	0	0	0	0	0	0	0	
MINNESOTA	19	812	1842	3619	7102	10432	14224	17447	4887	8426	85913	
MISSissippi	0	0	0	0	0	0	0	0	0	0	0	
MISSOURI	0	0	0	0	0	0	0	0	0	0	0	
Montana	01	02	03	04	05	06	07	08	09	010	011	
NEBRASKA	0	0	0	0	0	0	0	0	0	0	0	
NEVADA	69671	4767	4024	3210	3611	2750	2415	3154	899	20255	110618	
NEW HAMPSHIRE	0	0	0	0	0	0	0	0	0	0	0	
NEW JERSEY	0	0	0	0	0	0	0	0	0	0	0	
NEW MEXICO	87793	4236	3976	3860	3890	3648	4800	6503	1954	31534	121744	
NEW YORK	473	31	23	35	31	31	43	43	43	12	265	
NORTH CAROLINA	0	0	0	0	0	0	0	0	0	0	0	
NORTH DAKOTA	11748	4658	7886	7334	4719	4651	6401	6420	4410	672	71005	
OHIO	0	0	0	0	0	0	0	0	0	0	0	
Oklahoma	7691	1864	1582	8074	8924	8334	12767	9426	8164	3464	89615	
Oregon	113	27	27	25	43	87	181	291	123	53		
PENNSYLVANIA	7092	8799	3107	3184	3804	3217	10482	13423	16559	3054	97426	
RHODE ISLAND	0	0	0	0	0	0	0	0	0	0	0	
SOUTH CAROLINA	0	0	0	0	0	0	0	0	0	0	0	
SOUTH DAKOTA	9148	4955	2684	3404	8421	9494	4401	7102	4709	28793	77007	
TEXAS	11351	7996	8024	9426	12024	19474	31444	42328	18923	9471	868840	
UTAH	20615	2441	3821	4304	4400	5192	4205	7375	4873	25535	85179	
VERMONT	0	0	0	0	0	0	0	0	0	0	0	
VIRGINIA	0	0	0	0	0	0	0	0	0	0	0	
WASHINGTON	0	0	0	0	0	0	0	0	0	0	0	
WEST VIRGINIA	0	0	0	0	0	0	0	0	0	0	0	
WISCONSIN	0	0	0	0	0	0	0	0	0	0	0	
Wyoming	7334	8373	6439	7430	9042	9503	10405	10818	6012	25225	47985	
TOTAL	398480	88912	49191	86117	119619	195084	384340	467940	522404	957473		
	131	23	23	31	43	43	131	211	173	181		

\*\*\* SURFACE WATER COST DERIVATION. SUITABLE SOURCES ARE OCEANS, GREAT LAKES AND INTERNATIONAL BOUNDARY STREAMS WITH 7-DAY, 10-YEAR LOW FLOW GREATER THAN 300 cfs WITH OR WITHOUT RESERVOIR STORAGE. DISTANCE FROM SOURCES HAS COMPUTED AND COST APPLIED AS A WHILE VARYING WITH TERRAIN. RIVER'S % AND PENALTY ADDED FOR RESERVOIR NECESSITY. LEAST COST ALTERNATIVE WAS DETERMINED. COSTS LESS THAN \$200 MILLION WERE DIVIDED INTO EQUAL INTERVALS.

TABLE F1.5  
ENVIRONMENTAL SUITABILITY UTILITY FUNCTION \*\*\*  
STATE AREAS IN SQUARE MILES AND % OF STATE

UTILITY VALUE	LOW					MEDIUM-LOW					MEDIUM					MEDIUM-HIGH					HIGH					RESTRICTED LANDS					
	1	2	3	4	5	1	2	3	4	5	1	2	3	4	5	1	2	3	4	5	1	2	3	4	5	1	2	3	4	5	
ALABAMA	.96	4871	11387	19537	31840	2673	51908																								
ARIZONA	.07	127	282	387	393	42																									
KANSAS	.02	11	43	143	277	25	501																								
CALIFORNIA	99999	85429	19536	13529	8386	51492	160743																								
COLORADO	.76	44	14254	11448	13254	4199	20660	104326																							
CONNECTICUT	.99	8279	782	1794	0	0	5211																								
DELAWARE	.11	403	183	343	0	0	0	2267	39	2326																					
FLORIDA	.01	0	0	0	2132	44120	13105	99350																							
GEORGIA	.07	0	0	0	45	741	22																								
IDAHO	10847	7874	12091	10441	4777	77519	83540																								
ILLINOIS	3607	5245	9064	14475	29464	1361	83540																								
INDIANA	.43	45	95	263	521	22																									
ISLAND	.10	9476	8124	80510	12514	0	9164																								
KANSAS	18884	87902	19561	19131	7382	193	82244																								
KENTUCKY	.19	243	195	193	93	0																									
LOUISIANA	.11	865	305	211	91	6																									
MARYLAND	.05	0	376	7082	84277	14417	48153																								
MAINE	18445	8443	49775	17274	1786	357	34074																								
MASSACHUSETTS	.43	293	195	513	513	13																									
MINNESOTA	.05	347	1081	8034	7344	145	11195																								
MISSISSIPPI	.10	8425	4970	20767	19672	3841	47985																								
MISSOURI	18709	14417	15745	11985	10557	4516	89933																								
MontANA	.17	813	821	17109	35049	0	61837																								
NEBRASKA	.05	0	0	305	15	55																									
NEVADA	87982	18644	87647	8375	911	20253	110616																								
NEW HAMPSHIRE	.17	4448	1090	1899	0	1197	9444																								
NEW JERSEY	.13	475	18%	13%	0	13%																									
NEW MEXICO	47430	18283	19165	4870	3386	31336	121745																								
NEW YORK	.39	153	163	45	45	0	861																								
North CAROLINA	.94	1612	4478	13548	19239	8427	90770																								
North DAKOTA	.05	95	95	271	288	173																									
OHIO	.12	175	293	801	173	91																									
OKLAHOMA	.14	12%	85	871	423	423																									
OREGON	.04	8134	14041	29408	6487	30349	97926																								
PENNSYLVANIA	.05	22%	145	251	73	311																									
RHODE ISLAND	.05	362	321	141	51	91																									
SOUTH CAROLINA	.16	7797	16144	1983	1362	8443	31186																								
SOUTH DAKOTA	11477	7817	11380	12101	11078	82793	77006																								
TEXAS	8485	1514	9978	8076	454	2546	42120																								
UTAH	18420	42965	38660	99150	38823	5491	268879																								
VERMONT	.42	3001	1457	118	0	1023	9853																								
VIRGINIA	.49	2794	2480	13811	12448	3465	41168																								
WASHINGTON	.15	75	65	303	303	143																									
WEST VIRGINIA	.14	12	16	142	11	261																									
WISCONSIN	.15	435	275	15	15	111																									
WYOMING	17884	13793	20797	19710	4767	25225	47986																								
TOTAL	425114	433053	466810	469193	906122	957173																									
	145	145	153	213	173	181																									

\*\*\* ENVIRONMENTAL SUITABILITY DERIVATION THREE FACTORS --  
RESIDENTIAL HARDENING, SITE PREPARATION AND AGGREGATE WATER  
AVAILABILITY WERE COMPUTED ADDING THEIR MAPPED UTILITY  
VALUES (EACH RANGE 1-5) RESULTING IN A NET UTILITY MAP WITH  
VALUES FROM 4-25. NET VALUES WERE DIVIDED INTO FIVE CATE-  
GORIES OF APPROXIMATELY EQUAL AREA. BEST JOI COMPOSITE UTILITY  
MAP WAS ASSIGNED "HIGH" ENVIRONMENTAL SUITABILITY WHILE  
WORST JOI NET UTILITY MAP ASSIGNED "LOW" SUITABILITY.

TABLE F.2.1

POPULATION SECTOR ANALYSIS - TOTAL U. S.  
 DENSITY = 250 #/SQ. MI. \*\*\* SINGLE SECTOR (22.5 DEGREES)  
 STATE AREAS IN SQUARE MILES AND % OF STATE

TABULATION		AVAILABLE LAND	UNIFORM DENSITY RESTRICTED LANDS									
			> 1/16 ALLOWABLE POP.		> 1/8 ALLOWABLE POP.		> 1/4 ALLOWABLE POP.		> 1/3 ALLOWABLE POP.		> 1/2 ALLOWABLE POP.	
ALABAMA	1.0	16590	26602	4072	5674	5288	6398	2355	3703	2075	51907	
ARIZONA	2.0	14032	930	1302	1978	1071	1727	484	3329	54405	114342	
ARKANSAS	3.0	23594	3464	4960	4979	3467	2043	820	3165	4,63	53257	
CALIFORNIA	4.0	52778	7180	4417	6465	5365	5501	2142	28803	51,492	160363	
COLORADO	5.0	38587	2596	2972	2538	2200	1756	1139	3947	28660	104125	
CONNECTICUT	6.0	10	0	10	97	46	19	19	4989	0	5212	
DELAWARE	7.0	309	77	261	249	286	212	135	408	39	2326	
FLORIDA	8.0	10749	4757	4101	5510	3397	4034	2297	11397	13105	59357	
GEORGIA	9.0	13437	4092	5095	7452	5703	5008	1824	7014	5867	58604	
IDAHO	10.0	35484	2123	2210	1718	1322	1322	444	1204	37519	83950	
ILLINOIS	11.0	11599	3551	8154	8502	6127	4139	1283	11792	136	54538	
INDIANA	12.0	3620	162	3134	9407	5008	2774	1486	18547	1322	36311	
IOWA	13.0	831	452	1025	1948	1387	1073	413	2903	38		
KANSAS	14.0	26238	2529	5145	6330	4796	3300	917	2612	0	54056	
KENTUCKY	15.0	55379	300	5121	4207	3918	2753	782	3715	193	82264	
LOUISIANA	16.0	11455	2123	3127	4391	4345	3900	1071	3725	14417	48154	
MAINE	17.0	23234	1436	1978	1843	1421	1602	598	1303	357	24074	
MARYLAND	18.0	994	408	456	994	907	762	357	5742	145	11195	
MASSACHUSETTS	19.0	98	197	164	425	338	396	280	6774	0	8426	
MICHIGAN	20.0	20024	782	3621	4726	4432	3522	1204	13422	9679	61827	
MINNESOTA	21.0	25280	2679	7209	4294	3107	2490	897	4632	24926	89913	
MISSISSIPPI	22.0	18239	2345	4265	5249	3836	4101	1042	2905	3841	47684	
MISSOURI	23.0	32354	3715	7309	6987	4410	3406	984	5354	4316	49923	
ONTARIO	24.0	92204	2191	1431	2374	955	753	367	820	47160	146437	
PENNSYLVANIA	25.0	6213	135	116	472	393	413	321	7851	0	3185	
NEBRASKA	26.0	43613	940	2577	3127	1882	1523	590	2171	1534	77721	
NEVADA	27.0	85292	299	753	830	791	678	241	1177	20285	110617	
NEW HAMPSHIRE	28.0	2818	270	878	887	714	990	241	1901	117	9446	
NEW JERSEY	29.0	0	29	48	394	193	241	154	6429	0	8010	
NEW MEXICO	30.0	80344	1004	1986	1708	1293	1814	579	1474	31324	121744	
NEW YORK	31.0	5703	1737	3104	4410	4719	4362	1467	14707	9930	90219	
NORTH CAROLINA	32.0	69149	2347	3446	4041	4902	4794	2345	9110	8427	50749	
NORTH DAKOTA	33.0	54088	1930	3927	1949	1005	579	473	472	4572	71005	
OHIO	34.0	7621	273	531	273	141	0	0	0	0	935	
OKLAHOMA	35.0	1872	2229	3204	4700	4403	3725	1999	17775	2324	41833	
OREGON	36.0	26633	3580	5163	4248	4178	4130	1177	4243	3464	89614	
PENNSYLVANIA	37.0	5497	515	743	613	603	593	173	595	8	803	
PENNSYLVANIA	38.0	3590	2895	3320	4893	4092	4256	1204	4139	30349	97927	
RHODE ISLAND	39.0	0	795	445	733	1083	943	373	3845	785		
SOUTH CAROLINA	40.0	3742	2123	2393	3329	4320	4403	1177	4825	2643	31188	
SOUTH DAKOTA	41.0	46918	1805	1689	1737	647	599	212	618	22773	77008	
TENNESSEE	42.0	6045	231	221	233	083	083	037	085	2842		
TEXAS	43.0	11435	2625	2818	3995	4868	5423	2171	4176	3551	45380	
UTAH	44.0	6183	503	633	723	1123	083	0	1139	0	1207	
VERMONT	45.0	3699	579	973	994	994	950	240	960	1023	9854	
VIRGINIA	46.0	4774	3001	3213	4236	4642	5452	1983	5601	5643	41187	
WASHINGTON	47.0	1651	733	781	1031	1132	1235	381	1605	1383		
WEST VIRGINIA	48.0	3076	2519	2287	2676	2741	2741	868	2277	2721	24104	
WISCONSIN	49.0	19590	2794	3520	6823	5047	3699	1245	6128	5026	57024	
WYOMING	50.0	45330	1833	1737	1563	782	437	328	390	23225	97985	
TOTAL		1405512	113194	13316	182502	148233	137771	49348	282611	557673		
		46.25	3.73	5.43	6.03	4.93	4.53	1.81	9.31	18.33		

NOTE "AVAILABLE LAND" IS THAT AVAILABLE UNDER THE MOST CONSTRAINING CRITERIA (i.e. IF > 1/16 OF THE POPULATION ALLOWED BY A UNIFORM DENSITY CRITERION IS FOUND IN A SINGLE SECTOR OF 22.5 DEGREES. NUMBERS IN THE COLUMNS REPRESENT THAT LAND UNIQUELY CONSTRAINED BY THE GIVEN FRACTIONAL CRITERION. THIS LAND IS CONSIDERED AVAILABLE IF THE CRITERION WERE RELAXED IF SECTOR CRITERION IS APPLIED. ASSUME THAT UNIFORM DENSITY CRITERION IS ALSO IN EFFECT. ++ COMPOSITE OF 3 RADII ++

TABLE F2.2

POPULATION SECTOR ANALYSIS - TOTAL U.S.  
DENSITY = 300 /SQ. MI. \*\* SINGLE SECTOR (22.5 DEGREES)  
STATE AREAS IN SQUARE MILES AND % OF STATE

TABULATION		AVAILABLE LAND		> 1/16 ALLOWABLE POP.		> 1/8 ALLOWABLE POP.		> 1/4 ALLOWABLE POP.		> 1/3 ALLOWABLE POP.		> 1/2 ALLOWABLE POP.		UNIFORM DENSITY		RESTRICTED LANDS	
ALABAMA	1.0	24036	3590	3098	4072	4323	4815	2547	3029	2075	51907						
ARIZONA	2.0	44600	926	1197	1498	1042	984	444	2026	59405	114342						
ARKANSAS	3.0	30677	4666	4661	3612	2866	1737	820	1736	6233	53256						
CALIFORNIA	4.0	66366	4410	4883	6137	4516	4053	2191	6115	51492	150363						
COLORADO	5.0	43429	1431	2490	1678	1544	1764	618	2490	23660	101326						
CONNECTICUT	6.0	10	145	129	357	270	307	231	3725	0	5912						
DELAWARE	7.0	765	39	241	367	338	164	145	425	39	2327						
FLORIDA	8.0	20149	3739	3069	4140	2741	2425	2643	7131	13105	59356						
GEORGIA	9.0	25061	3704	3619	4239	4912	4410	2142	4421	5867	58609						
IDAHO	10.0	39736	743	1326	1156	1004	985	434	743	37519	89550						
ILLINOIS	11.0	19570	4233	874	6390	4178	3175	1013	7836	1361	56540						
INDIANA	12.0	70623	2490	2860	4941	3239	4092	1998	5290	1322	36343						
IOWA	13.0	32241	2364	634	5336	3242	1862	311	2075	0	56065						
KANSAS	15.0	5755	423	15	95	583	325	0 95	375	0 01							
KENTUCKY	15.0	67396	1467	3754	3049	2246	1554	540	2046	193	82267						
LOUISIANA	16.0	10655	2499	2731	3464	3725	2298	994	2200	14417	48153						
MAINE	17.0	36229	328	1995	1274	1426	1583	427	849	357	31074						
MARYLAND	18.0	2123	970	96	1023	877	926	475	4907	145	11155						
MASSACHUSETTS	19.0	434	222	347	830	418	472	569	3115	0	8427						
MICHIGAN	20.0	22981	1341	4974	5134	4825	3436	2793	7472	9479	41837						
MINNESOTA	21.0	41871	1042	3997	3175	2983	2112	907	3229	2426	89913						
MISSISSIPPI	22.0	24125	1197	3134	4072	3230	3048	1129	1785	3641	47883						
MISSOURI	23.0	41943	1905	3022	4746	4199	3030	866	3444	4514	69934						
MONTANA	24.0	96454	803	1197	1146	783	590	336	454	47160	148457						
NEBRASKA	25.0	44527	1197	2490	2064	1430	1052	347	1052	1534	77721						
NEVADA	26.0	86494	907	791	878	454	463	222	389	2903	110618						
NEW HAMPSHIRE	27.0	3616	434	674	637	646	801	347	1090	1197	9467						
NEW JERSEY	28.0	135	306	396	270	241	290	454	3896	0	8010						
NEW MEXICO	29.0	83801	425	2196	1842	917	791	309	946	3156	121745						
NEW YORK	30.0	96449	2936	3223	4111	4835	4844	2528	8801	9930	50219						
NORTH CAROLINA	31.0	12902	2740	3107	4140	4343	4494	3256	5037	8427	50768						
NORTH DAKOTA	32.0	98315	482	2304	1476	917	425	134	357	632	71004						
OHIO	33.0	7160	1698	3136	4034	5529	4950	2909	10490	2326	41832						
OKLAHOMA	34.0	47253	1940	2783	3985	3001	2538	1197	2355	3464	69616						
OREGON	35.0	55264	2065	1992	1457	1293	1525	2413	30349	97929							
PENNSYLVANIA	36.0	9090	2461	3349	4256	4352	4574	2526	11117	3551	45278						
RHODE ISLAND	37.0	0	0	10	39	87	104	56	907	0	1207						
SOUTH CAROLINA	38.0	9756	2133	2519	2461	3329	4149	1505	2444	2463	31189						
SOUTH DAKOTA	39.0	50799	39	1197	840	521	290	164	415	22793	77008						
TENNESSEE	40.0	16674	1370	2577	3406	4304	4844	2721	3448	2596	42122						
TEXAS	41.0	194554	10605	12545	12140	9254	8319	3030	12902	5491	268839						
UTAH	42.0	53480	1129	1004	1226	723	447	338	1071	25553	85181						
VERMONT	43.0	53088	0	772	485	878	492	347	347	1023	9852						
VIRGINIA	44.0	12120	2133	2277	3631	4344	4507	1999	4072	3465	41167						
WASHINGTON	45.0	28757	1824	2509	2422	2451	1216	3744	24762	69316							
WEST VIRGINIA	46.0	9399	994	1156	1920	2673	2616	955	1467	2721	24105						
WISCONSIN	47.0	25564	2229	5153	4770	4391	3184	1592	3908	5028	57021						
WYOMING	48.0	67663	0	955	733	492	290	222	405	25225	97985						
TOTAL		1448305	79720	135711	139287	128934	116370	56769	177165	557473							
		54.23	2.63	4.51	4.63	4.21	3.83	1.93	5.81	18.33							

NOTE: "AVAILABLE LAND" IS THAT AVAILABLE UNDER THE MOST CONSTRAINING CRITERIA (I.e. IF > 1/16 OF THE POPULATION ALLOWED BY A UNIFORM DENSITY CRITERION IS FOUND IN A SINGLE SECTOR OF 22.5 DEGREES) NUMBERS IN THE COLUMNS REPRESENT THAT LAND UNIQUELY CONSTRAINED BY THE GIVEN FRACTIONAL CRITERION. THIS LAND IS CONSIDERED AVAILABLE IF THE CRITERION WERE RELAXED IF SECTOR CRITERION IS APPLIED. ASSUME THAT UNIFORM DENSITY CRITERION IS ALSO IN EFFECT. \*\* COMPOSITE OF 5 RADII \*\*

TABLE F2.3

POPULATION SECTOR ANALYSIS - TOTAL U. S.  
 DENSITY = 750 P/SQ MI \*\*\* SINGLE SECTOR (22.5 DEGREES)  
 STATE AREAS IN SQUARE MILES AND % OF STATE

TABULATION	AVAILABLE LAND										UNIFORM DENSITY RESTRICTED LANDS	
	> 1/16 ALLOWABLE POP.					> 1/8 ALLOWABLE POP.						
	> 1/16 ALLOWABLE POP.			> 1/8 ALLOWABLE POP.		> 1/4 ALLOWABLE POP.			> 1/3 ALLOWABLE POP.			
	> 1/4 ALLOWABLE POP.		> 1/3 ALLOWABLE POP.		> 1/2 ALLOWABLE POP.		UNIFORM DENSITY RESTRICTED LANDS					
ALABAMA	1.0	27435	2972	2760	2895	3667	4593	2454	2854	2075	51507	
ARIZONA	2.0	47459	1419	1255	1554	529	531	484	1679	34405	114345	
ARKANSAS	3.0	31845	1062	4217	3425	2566	1467	320	1554	6283	53299	
CALIFORNIA	4.0	71592	4333	6736	4993	4468	3913	1304	13327	31492	140363	
COLORADO	5.0	43386	1573	1479	1764	1421	1071	704	1362	38460	104324	
CONNECTICUT	6.0	87	270	270	309	366	531	415	2943	0	3211	
DELAWARE	7.0	637	104	232	309	190	154	164	396	39	2327	
FLORIDA	8.0	25013	2248	3175	2615	1498	2269	2770	6449	13105	39357	
GEORGIA	9.0	28615	2374	2895	3704	4468	4323	2343	2812	9867	36605	
IDAHO	10.0	40637	413	1081	753	945	849	444	485	37119	80550	
ILLINOIS	11.0	25167	2490	7746	5261	4198	2731	1216	6417	1361	56539	
INDIANA	12.0	7943	1814	3679	4902	4497	3580	2343	8199	1322	36341	
IOWA	13.0	35447	1854	9019	4314	2806	1390	940	1795	0	34667	
KANSAS	14.0	67957	1554	3281	2461	1727	1159	989	1747	192	82267	
KENTUCKY	15.0	18480	1197	1727	2220	3715	3913	2393	2152	2470	40349	
LOUISIANA	16.0	18943	1486	1978	3049	3145	2260	1042	1893	14417	48153	
MAINE	17.0	24972	10	1148	1233	1251	1943	447	791	397	34574	
MARYLAND	18.0	24860	232	416	878	1004	1916	984	3999	145	11154	
MASSACHUSETTS	19.0	4756	407	518	540	415	998	743	4432	0	8427	
MICHIGAN	20.0	24047	1911	2135	4516	4304	3942	2731	6533	9479	41826	
MINNESOTA	21.0	43637	261	3472	3040	2876	1999	1148	2857	24924	89915	
MISSISSIPPI	22.0	25301	1925	2615	3484	4902	3642	1158	1983	3641	47880	
MISSOURI	23.0	43415	1042	3876	4374	4121	2977	734	2854	4514	49933	
ONTARIO	24.0	97494	0	917	741	7.05	5.75	4.43	10.63	15.73		
NEBRASKA	25.0	67.95	0.05	6.45	6.45	4.45	4.45	2.25	3.85	29.92		
NEVADA	26.0	64954	801	7.25	6.35	4.85	6.75	8.63	93.73	0.05		
NEW HAMPSHIRE	27.0	41649	164	590	949	714	782	374	3.35	8.05		
NEW JERSEY	28.0	4862	386	241	347	174	405	340	9433	0	6008	
NEW MEXICO	29.0	64544	772	1544	859	752	549	318	849	1834	77721	
NEW YORK	30.0	12091	1737	3165	3783	4413	4449	2827	7423	3930	90218	
NORTH CAROLINA	31.0	15594	2268	3615	3011	4043	4736	3942	4333	8627	90749	
NORTH DAKOTA	32.0	99183	0	2229	1554	489	388	145	997	4972	71009	
OHIO	33.0	9235	1428	3194	4747	5086	4907	3184	8106	2326	41833	
OKLAHOMA	34.0	90180	1833	3097	2519	2451	2461	1226	2084	3464	49613	
OREGON	35.0	57299	1361	1484	1573	1081	1090	1402	1986	30349	97928	
PENNSYLVANIA	36.0	11366	2267	3233	2992	4584	5086	3184	9013	3551	45278	
RHODE ISLAND	37.0	25.13	5.13	7.13	6.63	10.13	11.23	7.05	19.93	7.82		
SOUTH CAROLINA	38.0	0.05	0.63	2.43	12.83	6.43	10.43	5.81	56.81	0.05		
SOUTH DAKOTA	39.0	11754	2267	1450	1602	3117	4178	1640	2297	2643	2.186	
TENNESSEE	40.0	18142	1814	2094	2702	3956	4777	2827	3213	2594	42121	
TEXAS	41.0	208462	5568	9959	10914	8495	6205	3175	10171	5491	268840	
UTAH	42.0	54429	811	1100	1052	251	562	357	926	25553	85181	
VERMONT	43.0	3481	10	714	408	849	470	338	338	1023	4953	
VIRGINIA	44.0	14552	1486	2200	3213	4101	4460	2104	3445	5465	41146	
WASHINGTON	45.0	30774	1949	1911	2287	1737	1446	1341	3008	24762	49317	
WEST VIRGINIA	46.0	10721	145	926	1843	3415	2827	984	1312	2721	24104	
WISCONSIN	47.0	29095	1554	4777	4207	6352	3184	1479	3146	5028	57022	
WYOMING	48.0	70262	0	811	492	347	251	232	367	23225	97987	
TOTAL		1745298	61812	122845	117471	114480	107473	62298	148018	557473		
		57.41	2.01	4.05	3.93	3.88	3.91	2.01	4.93	18.35		

NOTE: "AVAILABLE LAND" IS THAT AVAILABLE UNDER THE MOST CONSTRAINING CRITERIA (i.e., IF > 1/16 OF THE POPULATION ALLOWED BY A UNIFORM DENSITY CRITERION IS FOUND IN A SINGLE SECTOR OF 22.5 DEGREES) NUMBERS IN THE COLUMNS REPRESENT THAT LAND UNIQUELY CONSTRAINED BY THE GIVEN FRACTIONAL CRITERION. THIS LAND IS CONSIDERED AVAILABLE IF THE CRITERION WERE RELAXED. IF SECTOR CRITERION IS APPLIED, ASSUME THAT UNIFORM DENSITY CRITERION IS ALSO IN EFFECT. \*\* COMPOSITE OF 5 RADII \*\*

TABLE F2.4

POPULATION SECTOR ANALYSIS - TOTAL U. S.  
DENSITY = 1300 P/SQ. MI. \*\* SINGLE SECTOR (22.5 DEGREES)  
STATE AREAS IN SQUARE MILES AND % OF STATE

TABULATION	AVAILABLE LAND											
	> 1/16 ALLOWABLE POP.					> 1/8 ALLOWABLE POP.						
	> 1/4 ALLOWABLE POP.					> 1/3 ALLOWABLE POP.						
	> 1/2 ALLOWABLE POP.					> 1/2 ALLOWABLE POP.						
	UNIFORM DENSITY					RESTRICTED LANDS						
ALABAMA	1.0	32482	704	1438	1756	3864	4593	2702	2492	5075	51726	
ARIZONA	2.0	44464	1621	743	494	261	472	482	1438	39403	114343	
ARMENIA	3.0	32765	634	3754	2914	2374	1448	830	1476	823	30258	
CALIFORNIA	4.0	79359	2730	3551	3242	3136	3086	2548	11020	51492	160262	
COLORADO	5.0	68226	608	1814	1023	848	926	704	1486	28460	104325	
CONNECTICUT	6.0	368	309	116	183	405	427	511	2873	0	5210	
DELAWARE	7.0	811	48	193	291	270	183	184	347	39	2324	
FLORIDA	8.0	29384	2142	1187	1042	1341	2258	2837	6002	13102	39358	
GEORGIA	9.0	32279	791	2104	2895	4371	4294	2384	3419	5867	98604	
IDAHO	10.0	41804	10	830	989	849	830	444	476	37519	83591	
ILLINOIS	11.0	29712	714	7246	4449	3903	2566	1303	5445	1361	36339	
INDIANA	12.0	13279	1534	3145	3242	3915	2951	2413	3812	1322	36342	
IOWA	13.0	38445	472	7486	3445	2403	1361	949	1460	0	54067	
KANSAS	14.0	72500	902	2790	1915	1448	1100	389	1431	193	82267	
KENTUCKY	15.0	29429	647	1361	1431	3297	3629	2441	2065	2470	40270	
LOUISIANA	16.0	51992	37	1399	2927	2972	2210	1042	1786	14417	48154	
MARYLAND	17.0	27194	0	1033	1146	1351	1954	856	782	357	34075	
MASSACHUSETTS	18.0	5779	276	950	1236	1071	1042	1216	2750	145	11195	
MINNESOTA	19.0	13890	473	261	364	443	447	975	4063	0	8426	
MISSOURI	20.0	27436	782	3783	3541	4092	2842	3030	5742	9479	41839	
MINNESOTA	21.0	44422	473	5414	2806	3422	2007	1239	2191	2426	89914	
MISSISSIPPI	22.0	27799	318	2044	3136	4786	3213	1187	1515	3641	47882	
MISSOURI	23.0	45345	743	5742	3879	3704	2432	1013	2597	4518	49933	
MONTANA	24.0	97726	113	823	932	533	533	143	373	633		
NEBRASKA	25.0	64405	0	645	645	441	0.25	0.25	318	0	318	
NEVADA	26.0	86345	347	374	309	222	183	164	376	20255	110617	
NEW HAMPSHIRE	27.0	6420	10	540	531	476	782	405	866	1197	9449	
NEW JERSEY	28.0	1042	280	361	386	360	367	318	409	47160	148456	
NEW MEXICO	29.0	86145	704	495	463	463	7.35	1131	4713	0.05	77721	
NEW YORK	30.0	14861	1448	2557	2647	4323	4294	2963	4996	9930	50219	
NORTH CAROLINA	31.0	19232	104	1776	2454	3756	6466	3513	4217	8627	50769	
NORTH DAKOTA	32.0	99801	0	2035	1216	486	270	145	290	4572	71005	
OHIO	33.0	11812	2024	3146	3127	4381	4391	2903	7122	2326	41834	
OKLAHOMA	34.0	33200	1119	2094	1853	2374	2384	1245	1862	3444	49815	
OREGON	35.0	99849	998	907	846	840	1042	1421	1853	30349	97927	
PENNSYLVANIA	36.0	13102	261	2422	2731	4448	3124	2532	8087	3551	43278	
RHODE ISLAND	37.0	10	194	87	29	48	116	164	598	0	1204	
SOUTH CAROLINA	38.0	130623	39	1033	1274	3078	4159	1440	2220	2463	31189	
SOUTH DAKOTA	39.0	51999	0	917	579	374	212	164	347	22793	77007	
TENNESSEE	40.0	20731	549	1322	2210	3610	4719	2866	3098	2596	42123	
TEXAS	41.0	219402	3679	8601	7180	6359	5567	3455	8685	5491	246829	
UTAH	42.0	59816	1119	473	347	193	502	376	801	25553	85180	
VERMONT	43.0	9407	10	456	549	830	482	336	338	1023	9853	
VIRGINIA	44.0	18704	1148	1534	2441	3985	4371	2162	3136	5665	41166	
WASHINGTON	45.0	33920	1224	1197	1204	1486	1380	1448	2692	24762	49317	
WEST VIRGINIA	46.0	11126	10	811	1708	2615	2827	984	1303	2721	24105	
WISCONSIN	47.0	31932	367	4248	3792	4063	3040	1747	2808	5028	57023	
WYOMING	48.0	70812	0	902	328	328	193	232	367	25225	97987	
TOTAL		1955831	32521	99454	87623	104450	104528	65959	131928	557673		
		61.0%	1.1%	3.3%	2.9%	3.4%	3.4%	2.2%	4.3%	18.2%		

NOTE: "AVAILABLE LAND" IS THAT AVAILABLE UNDER THE MOST CONSTRAINING CRITERIA (I.E. IF > 1/16 OF THE POPULATION ALLOWED BY A UNIFORM DENSITY CRITERION IS FOUND IN A SINGLE SECTOR OF 22.5 DEGREES, I NUMBERS IN THE COLUMNS REPRESENT THAT LAND UNIQUELY CONSTRAINED BY THE GIVEN FRACTIONAL CRITERION. THIS LAND IS CONSIDERED AVAILABLE IF THE CRITERION WERE RELAXED IF SECTOR CRITERION IS APPLIED. ASSUME THAT UNIFORM DENSITY CRITERION IS ALSO IN EFFECT. \*\* COMPOSITE OF 3 RADII \*\*

TABLE F2.5

POPULATION SECTOR ANALYSIS - TOTAL U. S.  
DENSITY = 250 / SQ MI \*\*\* DOUBLE SECTOR (45° DEGREES)  
STATE AREAS IN SQUARE MILES AND % OF STATE

TABULATION	AVAILABLE LAND										UNIFORM DENSITY RESTRICTED LANDS		
	> 1/8 ALLOWABLE POP			> 1/6 ALLOWABLE POP			> 1/4 ALLOWABLE POP			> 1/3 ALLOWABLE POP			
	> 1/8 ALLOWABLE POP			> 1/6 ALLOWABLE POP			> 1/4 ALLOWABLE POP			> 1/3 ALLOWABLE POP			
	> 1/8 ALLOWABLE POP			> 1/6 ALLOWABLE POP			> 1/4 ALLOWABLE POP			> 1/3 ALLOWABLE POP			
ALABAMA	1.0	19647	1554	5816	5944	4813	4355	5703	2075	51907			
		37.9%	3.0%	10.8%	11.3%	13.1%	9.9%	11.0%	4.0%				
ARIZONA	3.0	44959	330	1949	1110	1361	1882	3229	59405	114343			
		39.3%	0.3%	1.7%	1.0%	1.2%	1.6%	2.9%	52.0%				
ARKANSAS	3.0	26767	762	5616	3426	3387	1872	3165	6263	50258			
		54.0%	1.4%	10.5%	6.4%	6.4%	3.5%	5.9%	11.8%				
CALIFORNIA	4.0	59945	1448	7604	5713	6.9%	5124	22803	51492	160364			
		37.2%	0.9%	4.7%	3.6%	4.0%	2.2%	14.2%	32.1%				
COLORADO	5.0	6702	1004	2425	1360	2953	2055	3947	28660	104326			
		59.1%	1.0%	2.5%	1.3%	2.8%	2.0%	3.8%	27.5%				
CONNECTICUT	4.0	10	0	29	68	97	19	4989	0	5212			
		0.2%	0.0%	2.6%	1.3%	1.9%	0.4%	95.7%	0.0%				
DELAWARE	7.0	444	0	375	290	376	203	608	29	2326			
		19.1%	0.0%	16.2%	12.0%	16.2%	8.7%	26.1%	1.7%				
FLORIDA	8.0	13790	2413	3221	4033	4738	4642	11397	13105	59359			
		23.2%	4.1%	8.8%	6.8%	8.0%	7.9%	19.2%	22.1%				
GEORGIA	9.0	20535	1299	7218	7003	5944	3542	7016	5867	56604			
		35.0%	2.4%	12.3%	12.1%	10.1%	6.0%	12.0%	10.0%				
IDAHO	10.0	36185	154	2772	975	1679	859	1204	37519	83549			
		45.7%	0.2%	3.6%	1.2%	2.0%	1.0%	1.4%	44.9%				
ILLINOIS	11.0	17136	1795	8629	6176	6749	3922	11792	1361	56538			
		30.3%	2.3%	14.2%	10.9%	11.9%	6.2%	20.9%	2.4%				
INDIANA	12.0	479	1003	4969	5240	4960	3271	10547	1322	36341			
		13.0%	3.6%	13.7%	14.4%	13.6%	8.0%	29.0%	3.4%				
IOWA	13.0	32347	1361	4620	3640	3414	2654	3812	0	56068			
		37.7%	2.4%	11.8%	6.9%	9.7%	4.7%	6.8%	0.0%				
KANSAS	14.0	42166	415	4593	2384	3200	2499	3719	193	82263			
		79.2%	0.5%	2.6%	2.9%	4.0%	3.0%	4.5%	0.2%				
KENTUCKY	15.0	14272	950	4140	4534	6496	3329	4314	2470	40269			
		35.4%	1.4%	10.3%	11.2%	10.5%	6.2%	10.7%	6.1%				
LOUISIANA	16.0	12731	1446	4593	3870	4757	2413	3725	14417	48154			
		2.0%	1.0%	3.6%	3.8%	5.5%	5.6%	7.5%	0.0%				
MICHIGAN	20.0	22562	338	4188	4362	4729	2557	13423	9679	81808			
		36.9%	0.5%	6.8%	7.1%	7.6%	4.1%	21.7%	15.7%				
MINNESOTA	21.0	42335	1023	5163	3001	2409	1930	4632	24926	85915			
		49.7%	3.2%	6.0%	3.3%	3.4%	2.2%	5.4%	29.0%				
MISSISSIPPI	22.0	21594	444	5442	6580	4613	2584	2905	3641	47885			
		45.8%	0.9%	11.4%	12.7%	9.6%	5.4%	8.1%	8.0%				
MISSOURI	23.0	41061	704	7244	4314	4169	2470	5356	4516	69934			
		38.7%	1.0%	10.5%	6.2%	6.0%	3.5%	7.7%	6.3%				
ONTARIO	24.0	94744	811	1940	1448	897	637	820	47160	148457			
		43.8%	0.3%	1.3%	1.0%	0.6%	0.4%	0.6%	31.8%				
NEBRASKA	25.0	65475	415	2577	2441	1515	1592	2171	1534	77720			
		84.2%	0.5%	3.3%	3.1%	1.9%	2.0%	2.8%	2.0%				
NEVADA	26.0	85625	135	604	405	975	762	1177	20255	110618			
		77.6%	0.1%	0.9%	0.4%	0.9%	0.7%	1.1%	18.3%				
NEW HAMPSHIRE	27.0	3194	116	1090	934	495	398	1901	1197	9467			
		33.7%	1.2%	11.5%	9.9%	7.3%	3.6%	20.1%	12.6%				
NEW JERSEY	28.0	0	19	97	290	434	241	6929	0	8010			
		0.0%	0.2%	1.2%	3.6%	5.4%	3.0%	84.5%	0.0%				
NEW MEXICO	29.0	82363	125	1862	1013	1708	1640	1476	31536	121743			
		47.7%	0.1%	1.5%	0.8%	1.4%	1.3%	1.2%	25.9%				
NEW YORK	30.0	7449	742	4244	4458	4989	3657	14707	9930	50218			
		14.9%	1.3%	8.9%	8.7%	9.9%	7.3%	29.3%	19.8%				
NORTH CAROLINA	31.0	8443	1303	5105	4661	7469	3831	9110	8627	50769			
		16.7%	2.4%	10.1%	13.3%	14.7%	7.5%	17.9%	17.0%				
NORTH DAKOTA	32.0	36209	418	2586	945	447	936	473	4572	71006			
		82.0%	0.9%	3.6%	1.4%	0.9%	1.3%	0.7%	9.3%				
OHIO	33.0	3531	1081	4555	4899	4487	3059	17775	2324	41833			
		8.5%	2.4%	10.9%	11.9%	10.7%	7.3%	42.5%	5.6%				
OKLAHOMA	34.0	44061	1727	4526	3677	4487	3609	4043	3464	69614			
		43.3%	2.5%	6.9%	5.3%	6.4%	5.2%	5.8%	8.0%				
OREGON	35.0	32149	907	3168	2596	2229	2374	4139	30349	97928			
		53.3%	0.9%	3.2%	2.7%	2.7%	2.4%	4.2%	31.0%				
PENNSYLVANIA	36.0	5790	1052	4854	4709	5288	2567	17457	3551	45278			
		12.8%	2.3%	10.7%	10.4%	11.7%	5.7%	38.6%	7.8%				
RHODE ISLAND	37.0	0	0	0	10	48	10	1139	0	1207			
		0.0%	0.0%	0.0%	0.8%	4.0%	0.8%	94.4%	0.0%				
SOUTH CAROLINA	38.0	7337	454	3426	3735	5031	2519	4825	2663	31190			
		24.2%	1.5%	11.0%	12.0%	19.3%	8.1%	15.5%	8.5%				
SOUTH DAKOTA	39.0	4855	724	2034	907	627	647	618	22793	77007			
		63.2%	0.4%	2.6%	1.2%	0.8%	0.8%	0.8%	25.6%				
TENNESSEE	40.0	14050	676	4159	4314	6379	3773	6176	2596	42123			
		33.4%	1.6%	9.9%	10.2%	15.1%	9.0%	14.7%	6.2%				
TEXAS	41.0	181256	5221	20901	13346	12777	9718	20941	3491	268841			
		67.4%	1.9%	7.5%	5.0%	4.8%	3.6%	7.8%	2.0%				
UTAH	42.0	53220	203	1071	714	1226	1110	2084	25553	85181			
		62.5%	0.2%	1.3%	0.8%	1.4%	1.3%	2.4%	30.0%				
VERMONT	43.0	4420	311	1090	1177	666	405	560	1023	9852			
		44.9%	3.2%	11.1%	11.9%	8.8%	4.1%	5.7%	10.4%				
VIRGINIA	44.0	9325	1110	4178	4999	3722	3368	8601	5665	41168			
		23.1%	2.7%	10.2%	12.1%	13.9%	8.2%	16.0%	13.8%				
WASHINGTON	45.0	24974	1592	3175	3156	2885	2470	6301	24762	69315			
		36.0%	2.3%	4.6%	4.6%	4.2%	3.6%	9.1%	35.7%				
WEST VIRGINIA	46.0	7181	695	3831	2905	3300	1275	2277	2721	24105			
		29.4%	2.9%	15.9%	12.0%	13.7%	5.1%	9.4%	11.3%				
WISCONSIN	47.0	24849	975	7527	4603	5037	2878	6128	5028	57023			
		43.4%	1.7%	13.2%	8.1%	8.8%	3.0%	10.7%	8.8%				
WYOMING	48.0	67637	0	2338	676	618	743	350	25225	97987			
		69.0%	0.0%	2.6%	0.7%	0.6%	0.8%	0.8%	25.7%				
TOTAL		1544002	39888	199101	152051	146674	107973	282611	55764				
		90.8%	1.3%	6.2%	5.0%	5.3%	3.6%	9.3%	18.3%				

NOTE "AVAILABLE LAND" IS THAT AVAILABLE UNDER THE MOST CONSTRAINING CRITERIA (I.E. IF > 1/8 OF THE POPULATION ALLOWED BY A UNIFORM DENSITY CRITERION IS FOUND IN A DOUBLE SECTOR OF 45° DEGREES) NUMBERS IN THE COLUMNS REPRESENT THAT LAND UNIQUELY CONSTRAINED BY THE GIVEN FRACTIONAL CRITERION. THIS LAND IS CONSIDERED AVAILABLE IF THE CRITERION IS RELAXED. IF SECTOR CRITERION IS APPLIED, ASSUME THAT UNIFORM DENSITY CRITERION IS ALSO IN EFFECT. \*\* COMPOSITE OF 5 RADII \*\*

TABLE F2.6

POPULATION SECTOR ANALYSIS - TOTAL U. S.  
DENSITY = 500 \* 50 MI \*\*\* DOUBLE SECTOR (45° 0 DEGREES)  
STATE AREAS IN SQUARE MILES AND % OF STATE

TABULATION	AVAILABLE LAND									
	> 1/8 ALLOWABLE POP		> 1/8 ALLOWABLE POP		> 1/4 ALLOWABLE POP		> 1/4 ALLOWABLE POP		> 1/2 ALLOWABLE POP	
									UNIFORM DENSITY	
									RESTRICTED LANDS	
ALABAMA	1.0	26653	1718	3937	5192	5858	3146	3329	2075	51900
ARIZONA	2.0	47353	454	1312	1255	1351	1187	2026	59405	114343
ARKANSAS	3.0	33910	348	4304	2799	2499	1359	1756	6263	53258
CALIFORNIA	4.0	67422	2534	5549	5877	5267	4005	16115	51492	160263
COLORADO	5.0	63224	328	2712	1290	1824	1698	2490	28660	104328
CONNECTICUT	6.0	29	48	232	376	482	309	3735	0	5211
DELAWARE	7.0	656	29	443	728	933	593	7173	0	0
FLORIDA	8.0	23170	840	3773	3725	3821	3792	7131	13105	59357
GEORGIA	9.0	26342	1814	4825	5172	5240	2914	4429	5867	58603
IDAHO	10.0	41099	48	1457	1033	840	811	743	37519	83550
ILLINOIS	11.0	27376	1392	7498	4291	4053	2673	7836	1361	56540
INDIANA	12.0	10731	868	4642	4555	5520	3406	3299	1322	36342
IOWA	13.0	39652	415	6427	2865	2982	1631	2075	0	54067
KANSAS	14.0	7397	290	3349	2133	2277	1583	2046	193	82268
KENTUCKY	15.0	18382	917	2586	2977	4369	2924	2683	2470	40269
LOUISIANA	16.0	18653	338	4343	3388	3368	1476	2200	14417	48153
MAINE	17.0	27474	0	1563	1341	1679	791	849	357	34074
MARYLAND	18.0	2519	106	946	926	1081	926	4507	145	11156
MASSACHUSETTS	19.0	589	106	454	868	793	743	5115	0	8628
MICHIGAN	20.0	25650	907	4999	4931	4864	3146	7672	9679	61838
MINNESOTA	21.0	47420	145	3242	2463	2441	1718	3329	84954	85914
MISSISSIPPI	22.0	26059	579	4815	5443	3486	1679	1785	3841	47883
MISSOURI	23.0	47119	376	9105	3783	3339	2181	3464	4516	49933
MONTANA	24.0	97803	0	1061	454	735	541	3125	903	653
NEBRASKA	25.0	48843	482	1746	218	1187	1139	1052	1534	77721
NEVADA	26.0	87072	77	407	485	515	618	589	20255	110618
NEW HAMPSHIRE	27.0	4256	154	820	724	762	463	1090	1197	9466
NEW JERSEY	28.0	270	116	482	309	434	302	5896	0	8009
NEW MEXICO	29.0	84360	203	1505	1983	926	485	946	31536	121744
NEW YORK	30.0	11976	1158	2995	5066	5105	4169	8801	9930	50220
NORTH CAROLINA	31.0	14986	172	4053	5336	7025	3831	5037	8627	50767
NORTH DAKOTA	32.0	60698	0	1505	878	818	756	941	1703	0
OHIO	33.0	9747	482	3561	5182	5426	4420	10490	2326	41837
OKLAHOMA	34.0	50380	415	3763	3570	3735	1930	2355	3464	49615
OREGON	35.0	56588	1110	1940	1640	1650	2239	2413	30349	97429
PENNSYLVANIA	36.0	11397	1206	4458	5143	5250	3156	11117	3551	45278
RHODE ISLAND	37.0	2521	271	983	1143	1163	703	2465	785	0
SOUTH CAROLINA	38.0	11030	1052	3291	3976	4622	1911	2644	2643	31189
SOUTH DAKOTA	39.0	51508	0	811	521	569	290	415	22793	77007
TEXAS	40.0	18017	550	3702	4784	5407	3127	3640	2596	42123
UTAH	41.0	207629	2856	13182	9563	9322	7694	12902	5491	2e8829
VERMONT	42.0	54307	434	1090	427	1293	724	1071	25553	85170
VIRGINIA	43.0	5754	0	133	0.75	1.75	0.75	0.43	0.53	2963
WASHINGTON	44.0	14195	782	2551	5008	5201	2692	4072	5665	41166
WEST VIRGINIA	45.0	30571	540	2663	2374	2557	2104	3744	24762	69315
WISCONSIN	46.0	10673	174	2374	2804	2963	1124	1447	2721	24107
WYOMING	47.0	31334	685	5423	4953	3648	2403	3908	5028	57022
TOTAL		1761966	28968	145927	136449	140321	91495	177165	551673	
		58.0%	1.0%	4.8%	4.5%	4.6%	3.0%	5.8%	19.3%	

NOTE: "AVAILABLE LAND" IS THAT AVAILABLE UNDER THE MOST CONSTRAINING CRITERIA (i.e., IF  $> 1/8$  OF THE POPULATION ALLOWED BY A UNIFORM DENSITY CRITERION IS FOUND IN A DOUBLE SECTOR OF 45° 0 DEGREES). NUMBERS IN THE COLUMNS REPRESENT THAT LAND UNIQUELY CONSTRAINED BY THE GIVEN FRACTIONAL CRITERION. THIS LAND IS CONSIDERED AVAILABLE IF THE CRITERION WERE RELAXED IF SECTOR CRITERION IS APPLIED. ASSUME THAT UNIFORM DENSITY CRITERION IS ALSO IN EFFECT. \*\* COMPOSITE OF 5 RADII \*\*

TABLE F2.7

POPULATION SECTOR ANALYSIS - TOTAL U. S.  
DENSITY = 750 /sq mi \*\*\* DOUBLE SECTOR (45 0 DEGREES)  
STATE AREAS IN SQUARE MILES AND % OF STATE

TABULATION	AVAILABLE LAND	UNIFORM DENSITY									
		> 1/8 ALLOWABLE POP			> 1/6 ALLOWABLE POP			> 1/4 ALLOWABLE POP			> 1/3 ALLOWABLE POP
		> 1/8 ALLOWABLE POP		> 1/6 ALLOWABLE POP		> 1/4 ALLOWABLE POP		> 1/3 ALLOWABLE POP		> 1/2 ALLOWABLE POP	
		UNRESTRICTED LANDS		UNRESTRICTED LANDS		UNRESTRICTED LANDS		UNRESTRICTED LANDS		UNRESTRICTED LANDS	
ALABAMA	1.0	29934	800	4024	4130	5143	2895	2856	2075	51907	
ARIZONA	2.0	48308	946	1905	897	1293	456	1679	59405	114341	
ARKANSAS	3.0	25531	442	1678	2712	1999	1119	154	4263	53258	
CALIFORNIA	4.0	75270	1147	5732	4526	5134	3686	13307	51492	160364	
COLORADO	5.0	67174	511	1699	1361	1843	1216	1862	28460	16325	
CONNECTICUT	6.0	134	106	376	502	485	444	2943	0	5210	
DELAWARE	7.0	712	77	376	280	212	174	396	39	2326	
FLORIDA	8.0	26422	793	3763	2526	3136	3179	4465	13105	59357	
GEORGIA	9.0	32009	397	4246	4738	4844	2731	3612	5867	58604	
IDAHO	10.0	41926	125	830	917	880	446	485	37519	83550	
ILLINOIS	11.0	32134	907	5674	3650	3841	2355	6417	1301	56529	
INDIANA	12.0	13061	540	4767	4786	4584	3117	4159	1322	36341	
IOWA	13.0	42547	1042	4574	2827	2065	1197	1795	0	56067	
KANSAS	14.0	72008	965	2548	1911	1747	1148	1747	193	82267	
KENTUCKY	15.0	20319	193	2384	3754	4186	2818	2152	2470	40270	
LOUISIANA	16.0	20564	811	3048	3570	2413	1428	1853	14117	48134	
MAINE	17.0	27985	0	1307	1322	1523	791	791	357	34074	
MARYLAND	18.0	2885	106	897	897	1148	1476	3999	145	11153	
MASSACHUSETTS	19.0	859	174	830	579	724	930	4432	0	8626	
MICHIGAN	20.0	27850	926	4622	4854	4101	3271	6333	9679	61836	
MINNESOTA	21.0	48414	114	3086	2946	8441	1785	2897	24926	69413	
MISSISSIPPI	22.0	28207	280	4082	5008	3320	1573	1563	3841	47884	
MISSOURI	23.0	48983	270	4265	4034	3252	1756	2856	4516	69932	
ONTARIO	24.0	99179	0	917	733	618	434	415	47150	148456	
NEBRASKA	25.0	70040	125	2220	1042	1245	466	849	1534	77721	
NEVADA	26.0	87719	143	733	367	530	338	462	20255	110618	
NEW HAMPSHIRE	27.0	4661	97	418	704	801	444	946	1197	9446	
NEW JERSEY	28.0	998	193	296	318	463	406	5433	0	8009	
NEW MEXICO	29.0	85354	357	1544	849	840	415	849	31536	121744	
NEW YORK	30.0	1478	482	4178	4613	5201	3913	7422	9930	50218	
NORTH CAROLINA	31.0	18403	347	4033	4294	4890	3821	433	8627	50768	
NORTH DAKOTA	32.0	41133	0	1573	733	336	357	299	6572	71005	
OHIO	33.0	11329	447	4150	5375	5484	4217	8104	2326	41834	
OKLAHOMA	34.0	52274	714	2879	2731	2799	1469	2084	3464	69614	
OREGON	35.0	58913	135	1756	1293	1476	1795	1988	30349	97927	
PENNSYLVANIA	36.0	14728	644	4178	4130	5452	3908	9013	3551	45274	
RHODE ISLAND	37.0	0	10	29	125	193	164	485	0	1204	
SOUTH CAROLINA	38.0	13549	743	2422	3387	4207	1420	2297	2643	31188	
SOUTH DAKOTA	39.0	51965	0	840	482	270	376	22793	77006		
TENNESSEE	40.0	19937	656	3300	4275	4970	3175	3213	2396	42122	
TEXAS	41.0	217173	1361	11020	8376	9440	5987	10171	5491	268829	
UTAH	42.0	53361	251	1168	856	782	463	926	25532	85180	
VERMONT	43.0	8128	0	676	791	502	396	328	1023	9854	
VIRGINIA	44.0	16164	473	3599	4362	5095	1364	3445	5665	41167	
WASHINGTON	45.0	32192	830	2268	2374	2104	1498	3088	24762	69318	
WEST VIRGINIA	46.0	11561	0	1872	2634	2808	1177	1312	2721	24105	
WISCONSIN	47.0	38016	145	4719	4130	3570	2268	3146	5028	57022	
WYOMING	48.0	70677	0	733	386	328	270	367	25225	97986	
TOTAL		1849701	19801	131151	121809	129352	82244	148018	557673		
		60.8%	0.7%	4.3%	4.0%	4.3%	2.7%	4.9%	18.3%		

NOTE: "AVAILABLE LAND" IS THAT AVAILABLE UNDER THE MOST CONSTRAINING CRITERIA IF & IF > 1/8 OF THE POPULATION ALLOWED BY A UNIFORM DENSITY CRITERION IS FOUND IN A DOUBLE SECTOR OF 45 0 DEGREES. NUMBERS IN THE COLUMNS REPRESENT THAT LAND UNIQUELY CONSTRAINED BY THE GIVEN FRACTIONAL CRITERION. THIS LAND IS CONSIDERED AVAILABLE IF THE CRITERION WERE RELAXED IF SECTOR CRITERION IS APPLIED. ASSUME THAT UNIFORM DENSITY CRITERION IS ALSO IN EFFECT. \*\* COMPOSITE OF 5 RADII \*

TABLE F2.8

POPULATION SECTOR ANALYSIS - TOTAL U.S.  
DENSITY = 1500 / 50 MI \*\*\* DOUBLE SECTOR (45.0 DEGREES)  
STATE AREAS IN SQUARE MILES AND % OF STATE

TABULATION		AVAILABLE LAND	UNIFORM DENSITY							RESTRICTED LANDS		
			> 1/8 ALLOWABLE POP		> 1/6 ALLOWABLE POP		> 1/4 ALLOWABLE POP		> 1/3 ALLOWABLE POP			
			> 1/8 ALLOWABLE POP		> 1/6 ALLOWABLE POP		> 1/4 ALLOWABLE POP		> 1/3 ALLOWABLE POP			
			> 1/8 ALLOWABLE POP		> 1/6 ALLOWABLE POP		> 1/4 ALLOWABLE POP		> 1/3 ALLOWABLE POP			
			> 1/8 ALLOWABLE POP		> 1/6 ALLOWABLE POP		> 1/4 ALLOWABLE POP		> 1/3 ALLOWABLE POP			
			> 1/8 ALLOWABLE POP		> 1/6 ALLOWABLE POP		> 1/4 ALLOWABLE POP		> 1/3 ALLOWABLE POP			
ALABAMA	1.0	32958	0	2277	3493	4555	2856	2692	2075	51406		
ARIZONA	2.0	50498	328	1119	550	492	511	1436	59405	114341		
ARKANSAS	3.0	37201	270	3136	2297	1544	1671	1476	6263	5358		
CALIFORNIA	4.0	81842	948	4092	4227	3870	2876	31020	51492	160365		
COLORADO	5.0	69229	212	1669	1129	1062	876	1486	26660	104325		
CONNECTICUT	6.0	550	203	203	398	647	540	2673	0	5212		
DELAWARE	7.0	1013	10	270	232	212	183	367	39	2326		
FLORIDA	8.0	31527	58	1805	1525	2229	3107	6002	13105	59358		
GEORGIA	9.0	34258	347	3262	4236	4285	2731	3819	5867	5867		
IDAHO	10.0	42460	0	704	820	791	579	676	37519	83549		
ILLINOIS	11.0	25666	357	5018	3599	2992	1901	5445	1361	56539		
INDIANA	12.0	16115	772	3831	3974	3754	2760	3812	1322	36242		
IOWA	13.0	45741	39	3715	2393	1496	1023	1660	0	56067		
KANSAS	14.0	74913	154	1824	1370	1177	1004	1631	193	82266		
KENTUCKY	15.0	21598	357	2007	3445	5665	2702	2065	2470	40269		
LOUISIANA	16.0	23112	10	2422	2934	2210	1289	1766	14417	48134		
MAINE	17.0	26207	0	1129	1332	1476	772	762	357	34075		
MARYLAND	18.0	3271	164	889	1235	1332	1370	2750	145	11155		
MASSACHUSETTS	19.0	1824	87	454	444	743	1013	4063	0	8628		
MICHIGAN	20.0	31305	97	3773	4101	3821	3320	5742	9479	61838		
MINNESOTA	21.0	49533	222	2854	2499	2058	1431	2191	2498	89913		
MISSISSIPPI	22.0	29474	222	3320	757	3059	1496	1515	2641	47884		
MISSOURI	23.0	50489	309	4236	2357	1448	2557	4516	69933			
MONTANA	24.0	98575	0	917	998	444	357	405	47160	148456		
NEBRASKA	25.0	71525	68	1525	1126	806	521	801	1534	77721		
NEVADA	26.0	86712	0	1.0	1.51	0.63	0.78	1.03	2.03	110617		
NEW HAMPSHIRE	27.0	44912	10	949	685	743	463	868	1197	9467		
NEW JERSEY	28.0	1187	145	357	270	646	1197	4180	0	8010		
NEW MEXICO	29.0	87217	0	4.0	3.43	8.38	14.4%	52.3%	0.0	31.6%		
NEW YORK	30.0	17515	425	3349	4246	4371	3367	6996	9930	50219		
NORTH CAROLINA	31.0	20563	19	2760	3947	6591	3744	4217	8427	50768		
NORTH DAKOTA	32.0	61741	0	1168	647	309	290	290	6572	71007		
OHIO	33.0	14552	695	4111	4719	4256	4053	7122	2326	41834		
OKLAHOMA	34.0	34.81	1.73	9.85	11.35	10.21	9.73	17.03	5.63			
OREGON	35.0	60399	261	1409	975	945	1718	1853	30349	97929		
PENNSYLVANIA	36.0	17274	97	2808	4304	5230	3928	8087	3551	45279		
RHODE ISLAND	37.0	48	97	87	115	174	598	0	1207			
SOUTH CAROLINA	38.0	15961	0	1341	3078	4043	1882	2220	2463	31188		
SOUTH DAKOTA	39.0	52438	0	569	357	232	251	367	22793	77007		
TENNESSEE	40.0	22998	193	2220	3841	4719	3059	3098	2596	42124		
TEXAS	41.0	226061	1081	9862	7141	5993	4507	6885	5491	268841		
UTAH	42.0	56484	357	647	203	463	473	801	25553	85181		
VERMONT	43.0	6253	0	569	801	482	386	338	1023	9852		
VIRGINIA	44.0	18470	299	2818	3985	4458	2335	3136	5465	41168		
WASHINGTON	45.0	35078	415	1718	1668	1322	1660	2692	24762	69318		
WEST VIRGINIA	46.0	11898	10	1718	2548	2770	1129	1303	2721	24107		
WISCONSIN	47.0	35753	164	4014	3928	3194	2133	2808	5028	57022		
WYOMING	48.0	71227	0	386	218	232	251	367	25225	97987		
TOTAL		1946617	9539	102175	107452	107475	77115	131928	557773			
		84.0%	0.3%	3.4%	3.5%	3.5%	2.5%	4.3%	18.3%			

NOTE: "AVAILABLE LAND" IS THAT AVAILABLE UNDER THE MOST CONSTRaining CRITERIA, i.e., IF > 1/8 OF THE POPULATION ALLOWED BY A UNIFORM DENSITY CRITERION IS FOUND IN A DOUBLE SECTOR OF 45.0 DEGREES; 1 NUMBERS IN THE COLUMNS REPRESENT THAT LAND UNIQUELY CONSTRAINED BY THE GIVEN FRACTIONAL CRITERION; THIS LAND IS CONSIDERED AVAILABLE IF THE CRITERION WERE RELAXED IF SECTOR CRITERION IS APPLIED. ASSUME THAT UNIFORM CRITERION IS ALSO IN EFFECT. \*\* COMPOSITE OF 5 RADII \*\*

TC-81	TC-6	TC-8	TC-6	TC-1	TC-09	TC-01	TC-01
CX-11	11928E	942641	950001	C89AC	1E0081		TOTAL
TC-62	3W-0	3W-0	3W-0	3W-0	3W-0	3W-0	3W-0
TC-55	055	10B	019	0	2820	0	0 BE
TC-8	84-01	SE-4	1A-8	SE-7	7A-09		0 BE
TC-05	0819	B14	9E95	2E01	12A4C	0 49	
TC-11	1A-6	3D-6	5D-11	3B-6	15-26		
TC-12	4A25E	1A12	2A2E	1A6	15A91	0 99	
TC-6	51-6	51-9	51-6	51-0	32-99		
TC-4	10E9	E8C9	145C	204	28600	0 58	
TC-13	50-91	TE-11	3A-11	3A-0	32-08		
TC-99	1099	1998	0029	0031	22A91	0 99	
TC-01	52-6	52-6	52-6	52-0	32-02		
TC-01	076	1E6	2B2	0	0B99	0 08	
TC-02	53-2	53-2	53-1	53-0	32-09		
TC-55	4802	1E11	9E6	019	2B245	0 28	
TC-5	53-2	53-9	53-5	53-1	31-62		
TC-8	18602	1861	1861	189C1	299C	9A912	0 18
TC-9	52-41	52-11	52-01	51-0	32-68		
TC-2	7A19	B105	5245	088	48600	0 08	
TC-7	52-0	52-1	52-0	52-0	32-49		
TC-22	019	02	129	0	92275	0 60	
TC-8	55-61	5C-21	1E02	1E01	01521	0 8C	
TC-99	5289	198C	1E04	1E01	01521	0 8C	
TC-0	53-2	53-2	53-0	53-0	32-03		
TC-01	0111	02	0E	0	0	0 4C	
TC-6	53-9C	53-9	53-11	53-9	32-52		
TC-55	29941	5269	1A95	0961	08911	0 9C	
TC-12	52-4	52-2	52-1	52-1	32-58		
TC-06	0419	1E10	0827	0821	94975	0 6C	
TC-5	52-6	52-4	52-7	52-0	32-52		
TC-9	0945	0945	0949	0946	09495	0 6C	
TC-5	55-29	5C-01	52-01	52-0	32-62		
TC-22	04111	0999	4265	1E01	12101	0 08	
TC-8	52-6	52-1	52-1	52-0	32-48		
TC-24	024	028	0	0	92129	0 8C	
TC-01	53-21	5C-21	53-21	53-2	32-50		
TC-98	1101	0179	6508	0021	00571	0 1E	
TC-61	5E-62	51-01	55-01	55-0	32-52		
TC-66	0201	0955	0855	0012	01821	0 08	
TC-52	52-2	52-1	52-1	52-0	32-69		
TC-95	9251	9251	5261	192	49498	0 6C	
TC-0	5E-98	5C-7	52-6	52-1	32-50		
TC-6	0247	126	999	78	0	0 8C	
TC-21	51-02	52-6	52-01	52-0	32-49		
TC-11	1061	096	526	1C	00571	0 1E	
TC-81	51-1	50-1	52-0	52-0	32-82		
TC-65	1111	1801	914	2E2	65110	0 9C	
TC-0	52-2	52-2	52-2	52-1	32-68		
TC-95	1112	2E81	6291	818	49569	0 6C	
TC-18	39-0	39-0	39-0	39-0	32-99		
TC-07	9251	9251	5261	192	49498	0 6C	
TC-7	52-9	52-9	52-9	52-9	32-50		
TC-95	9251	9251	5261	192	49498	0 6C	
TC-0	52-62	52-62	52-62	52-62	32-50		
TC-6	0247	1247	928	18	0	0 8C	
TC-1	51-16	51-9	50-6	52-2	32-72		
TC-1	51-9	50-6	52-01	52-1	32-49		
TC-0	52-6	52-6	52-6	52-6	32-50		
TC-62	0201	0955	0855	0012	01821	0 08	
TC-10	52-6	52-6	52-6	52-6	32-50		
TC-62	0201	0955	0855	0012	01821	0 08	
TC-61	51-16	1802	6061	674	00506	0 6C	
TC-0	52-9	52-6	52-01	52-0	32-50		
TC-62	0201	0955	0855	0012	01821	0 08	
TC-11	52-62	52-62	52-62	52-62	32-50		
TC-9	52-62	52-62	52-62	52-62	32-50		
TC-95	9251	9251	5261	192	49498	0 6C	
TC-0	52-62	52-62	52-62	52-62	32-50		
TC-6	0247	1247	928	18	0	0 8C	
TC-1	51-16	51-9	50-6	52-2	32-72		
TC-1	51-9	50-6	52-01	52-1	32-49		
TC-0	52-6	52-6	52-6	52-6	32-50		
TC-62	0201	0955	0855	0012	01821	0 08	
TC-1	51-16	51-9	50-6	52-2	32-72		
TC-1	51-9	50-6	52-01	52-1	32-49		
TC-0	52-6	52-6	52-6	52-6	32-50		
TC-62	0201	0955	0855	0012	01821	0 08	
TC-1	51-16	51-9	50-6	52-2	32-72		
TC-1	51-9	50-6	52-01	52-1	32-49		
TC-0	52-6	52-6	52-6	52-6	32-50		
TC-62	0201	0955	0855	0012	01821	0 08	
TC-1	51-16	51-9	50-6	52-2	32-72		
TC-1	51-9	50-6	52-01	52-1	32-49		
TC-0	52-6	52-6	52-6	52-6	32-50		
TC-62	0201	0955	0855	0012	01821	0 08	
TC-1	51-16	51-9	50-6	52-2	32-72		
TC-1	51-9	50-6	52-01	52-1	32-49		
TC-0	52-6	52-6	52-6	52-6	32-50		
TC-62	0201	0955	0855	0012	01821	0 08	
TC-1	51-16	51-9	50-6	52-2	32-72		
TC-1	51-9	50-6	52-01	52-1	32-49		
TC-0	52-6	52-6	52-6	52-6	32-50		
TC-62	0201	0955	0855	0012	01821	0 08	
TC-1	51-16	51-9	50-6	52-2	32-72		
TC-1	51-9	50-6	52-01	52-1	32-49		
TC-0	52-6	52-6	52-6	52-6	32-50		
TC-62	0201	0955	0855	0012	01821	0 08	
TC-1	51-16	51-9	50-6	52-2	32-72		
TC-1	51-9	50-6	52-01	52-1	32-49		
TC-0	52-6	52-6	52-6	52-6	32-50		
TC-62	0201	0955	0855	0012	01821	0 08	
TC-1	51-16	51-9	50-6	52-2	32-72		
TC-1	51-9	50-6	52-01	52-1	32-49		
TC-0	52-6	52-6	52-6	52-6	32-50		
TC-62	0201	0955	0855	0012	01821	0 08	
TC-1	51-16	51-9	50-6	52-2	32-72		
TC-1	51-9	50-6	52-01	52-1	32-49		
TC-0	52-6	52-6	52-6	52-6	32-50		
TC-62	0201	0955	0855	0012	01821	0 08	
TC-1	51-16	51-9	50-6	52-2	32-72		
TC-1	51-9	50-6	52-01	52-1	32-49		
TC-0	52-6	52-6	52-6	52-6	32-50		
TC-62	0201	0955	0855	0012	01821	0 08	
TC-1	51-16	51-9	50-6	52-2	32-72		
TC-1	51-9	50-6	52-01	52-1	32-49		
TC-0	52-6	52-6	52-6	52-6	32-50		
TC-62	0201	0955	0855	0012	01821	0 08	
TC-1	51-16	51-9	50-6	52-2	32-72		
TC-1	51-9	50-6	52-01	52-1	32-49		
TC-0	52-6	52-6	52-6	52-6	32-50		
TC-62	0201	0955	0855	0012	01821	0 08	
TC-1	51-16	51-9	50-6	52-2	32-72		
TC-1	51-9	50-6	52-01	52-1	32-49		
TC-0	52-6	52-6	52-6	52-6	32-50		
TC-62	0201	0955	0855	0012	01821	0 08	
TC-1	51-16	51-9	50-6	52-2	32-72		
TC-1	51-9	50-6	52-01	52-1	32-49		
TC-0	52-6	52-6	52-6	52-6	32-50		
TC-62	0201	0955	0855	0012	01821	0 08	
TC-1	51-16	51-9	50-6	52-2	32-72		
TC-1	51-9	50-6	52-01	52-1	32-49		
TC-0	52-6	52-6	52-6	52-6	32-50		
TC-62	0201	0955	0855	0012	01821	0 08	
TC-1	51-16	51-9	50-6	52-2	32-72		
TC-1	51-9	50-6	52-01	52-1	32-49		
TC-0	52-6	52-6	52-6	52-6	32-50		
TC-62	0201	0955	0855	0012	01821	0 08	
TC-1	51-16	51-9	50-6	52-2	32-72		
TC-1	51-9	50-6	52-01	52-1	32-49		
TC-0	52-6	52-6	52-6	52-6	32-50		
TC-62	0201	0955	0855	0012	01821	0 08	
TC-1	51-16	51-9	50-6	52-2	32-72		
TC-1	51-9	50-6	52-01	52-1	32-49		
TC-0	52-6	52-6	52-6	52-6	32-50		
TC-62	0201	0955	0855	0012	01821	0 08	
TC-1	51-16	51-9	50-6	52-2	32-72		
TC-1	51-9	50-6	52-01	52-1	32-49		
TC-0	52-6	52-6	52-6	52-6	32-50		
TC-62	0201	0955	0855	0012	01821	0 08	
TC-1	51-16	51-9	50-6	52-2	32-72		
TC-1	51-9	50-6	52-01	52-1	32-49		
TC-0	52-6	52-6	52-6	52-6	32-50		
TC-62	0201	0955	0855	0012	01821	0 08	
TC-1	51-16	51-9	50-6	52-2	32-72		
TC-1	51-9	50-6	52-01	52-1	32-49		
TC-0	52-6	52-6	52-6	52-6	32-50		
TC-62	0201	0955	0855	0012	01821	0 08	
TC-1	51-16	51-9	50-6	52-2	32-72		
TC-1	51-9	50-6	52-01	52-1	32-49		
TC-0	52-6	52-6	52-6	52-6	32-50		
TC-62	0201	0955	0855	0012	01821	0 08	
TC-1	51-16	51-9	50-6	52-2	32-72		
TC-1	51-9	50-6	52-01	52-1	32-49		
TC-0	52-6	52-6	52-6	52-6	32-50		
TC-62	0201	0955	0855	0012	01821	0 08	
TC-1	51-16	51-9	50-6	52-2	32-72		
TC-1	51-9	50-6	52-01	52-1	32-49		
TC-0	52-6	52-6	52-6	52-6	32-50		
TC-62	0201	0955	0855	0012	01821	0 08	
TC-1	51-16	51-9	50-6	52-2	32-72		
TC-1	51-9	50-6	52-01	52-1	32-49		
TC-0	52-6	52-6	52-6	52-6	32-50		
TC-62	0201	0955	0855	0012	01821	0 08	
TC-1	51-16	51-9	50-6	52-2	32-72		
TC-1	51-9	50-6	52-01	52-1	32-49		
TC-0	52-6	52-6	52-6	52-6	32-50		</

1. *ALL INFORMATION PROVIDED*  
2. *S/ S-2 ALL INFORMATION PROVIDED*  
3. *ALL INFORMATION PROVIDED*  
4. *ALL INFORMATION PROVIDED*

TABLE P2.9

TABLE F2.10

POPULATION SECTOR ANALYSIS - TOTAL U.S.  
 DENSITY = 500 / SQ MI \*\*\* "QUAD" SECTOR (90° DEGREES)  
 STATE AREAS IN SQUARE MILES AND % OF STATE

TABULATION		AVAILABLE LAND	UNIFORM DENSITY						RESTRICTED LANDS	
			> 1/4 ALLOWABLE POP		> 1/3 ALLOWABLE POP		> 1/2 ALLOWABLE POP			
ALABAMA	1.0	35988	374	4253	3985	3329	2075	51906		
ARIZONA	2.0	49244	934	1168	1563	2026	59405	114342		
ARKANSAS	3.0	40501	0.81	1.01	1.41	1.81	52.01			
CALIFORNIA	4.0	79062	1612	6282	5800	16115	51492	160363		
COLORADO	5.0	49210	193	1776	1991	2490	28540	104327		
CONNECTICUT	6.0	164	164	482	866	3735	0	5211		
DELAWARE	7.0	1206	56	247	251	425	39	2326		
FLORIDA	8.0	28381	811	4642	5288	7131	13105	59358		
GEORGIA	9.0	36667	309	5259	4053	4429	5867	58604		
IDAHO	10.0	43435	0	926	926	743	37519	83549		
ILLINOIS	11.0	38166	463	4738	3976	7836	1361	56540		
INDIANA	12.0	18991	704	5182	4844	5298	1322	36341		
IDAHO	13.0	48356	193	3291	2152	2075	0	56067		
KANSAS	14.0	73309	434	1949	2335	2046	193	82266		
KENTUCKY	15.0	25080	193	6282	3561	2863	2470	40269		
LOUISIANA	16.0	23745	492	3156	2123	2200	14417	48193		
MAINE	17.0	30224	0	1669	975	849	357	34074		
MARYLAND	18.0	4014	222	1061	1187	4507	145	11156		
MASSACHUSETTS	19.0	1081	261	1139	1033	5115	0	8629		
MICHIGAN	20.0	33852	1409	4902	4223	7472	9479	81837		
MINNESOTA	21.0	52766	164	2566	2142	3229	24926	85913		
MISSISSIPPI	22.0	36459	104	3431	3248	1785	3841	47883		
MISSOURI	23.0	35661	290	3059	2943	3464	4516	69933		
MONTANA	24.0	99491	0	447	704	454	47160	148456		
NEBRASKA	25.0	72365	183	1177	1390	1052	1534	77721		
NEVADA	26.0	86259	434	347	733	589	20255	110617		
NEW HAMPSHIRE	27.0	5520	135	866	656	1090	1197	9466		
NEW JERSEY	28.0	427	261	473	753	3896	0	8010		
NEW MEXICO	29.0	86821	724	782	936	946	31536	121745		
NEW YORK	30.0	19474	1187	5365	5443	8801	9930	50220		
NORTH CAROLINA	31.0	23662	946	7691	4806	5037	8627	50769		
NORTH DAKOTA	32.0	62947	0	485	444	357	6572	71005		
OHIO	33.0	16434	1033	5423	6128	10490	2326	41834		
OKLAHOMA	34.0	3931	253	1303	1463	2515	563			
OREGON	35.0	60489	154	1689	2634	2413	30349	97928		
PENNSYLVANIA	36.0	18721	1380	6002	4507	11117	2551	45278		
RHODE ISLAND	37.0	19	10	58	212	907	0	1206		
SOUTH CAROLINA	38.0	18132	145	3018	2586	2644	2663	31188		
SOUTH DAKOTA	39.0	52785	0	609	405	415	22793	77008		
TENNESSEE	40.0	25244	840	3800	3995	3648	2594	42123		
TEXAS	41.0	229207	1891	9003	10345	12902	5491	268839		
UTAH	42.0	35816	482	1293	965	1071	25553	85180		
VERMONT	43.0	6553	0	1.51	1.51	1.35	30.05			
VIRGINIA	44.0	7533	0.31	5.61	4.93	3.55	10.43			
WASHINGTON	45.0	34692	384	2712	3020	3744	24762	49316		
WEST VIRGINIA	46.0	15459	0	2750	1708	1487	2721	24105		
WISCONSIN	47.0	40935	193	3937	3020	3908	5026	57021		
Wyoming	48.0	71400	0	574	376	405	25225	47985		
		72.93	0.03	0.65	0.43	0.43	25.73			
TOTAL		2015042	21627	145462	122968	177165	557673			
		64.33	0.73	4.85	4.02	5.81	16.33			

NOTE: "AVAILABLE LAND" IS THAT AVAILABLE UNDER THE MOST CONSTRAINING CRITERIA (I.E. IF > 1/4 OF THE POPULATION ALLOWED BY A UNIFORM DENSITY CRITERION IS FOUND IN A "QUAD" SECTOR OF 90° DEGREES) NUMBERS IN THE COLUMNS REPRESENT THAT LAND UNIQUELY CONSTRAINED BY THE GIVEN FRACTIONAL CRITERION. THIS LAND IS CONSIDERED AVAILABLE IF THE CRITERION WERE RELAXED IF SECTOR CRITERION IS APPLIED. ASSUME THAT UNIFORM DENSITY CRITERION IS ALSO IN EFFECT. \*\* COMPOSITE OF 5 RADII \*\*

TABLE F2.11

POPULATION SECTOR ANALYSIS - TOTAL U.S.  
DENSITY = 750 #/50 MI \*\*\* "QUAD" SECTOR (90° DEGREES)  
STATE AREAS IN SQUARE MILES AND % OF STATE

TABULATION	AVAILABLE LAND							
	> 1/4 ALLOWABLE POP		> 1/2 ALLOWABLE POP		> 1/2 ALLOWABLE POP			
					UNIFORM DENSITY			
							RESTRICTED LANDS	
ALABAMA	1.0	37838	482	4950	3706	2856	2075	51907
ARIZONA	2.0	50614	328	1226	1090	1679	59405	114342
ARKANSAS	3.0	41630	23	113	103	155	5203	53259
CALIFORNIA	4.0	93820	1033	5006	5484	1327	51492	160364
COLORADO	5.0	70300	174	1457	1872	1042	28660	104325
CONNECTICUT	6.0	415	193	1020	183	123	275	5211
DELAWARE	7.0	1380	14	280	212	296	39	2326
FLORIDA	8.0	31112	1042	3802	3821	6465	13105	59357
GEORGIA	9.0	40086	280	4912	3848	3912	3867	58405
IDAHO	10.0	43705	0	849	791	485	37519	83549
ILLINOIS	11.0	40791	444	3908	3619	6417	1361	56540
INDIANA	12.0	21346	733	4719	4063	4159	1322	36342
IOWA	13.0	50073	261	2094	1824	1795	0	56067
KANSAS	14.0	76650	135	1911	1631	1747	193	82267
KENTUCKY	15.0	26036	241	6031	3339	2152	2470	40269
LOUISIANA	16.0	27464	10	2702	1708	1853	14417	48154
MAINE	17.0	30494	0	1534	897	791	357	34073
MARYLAND	18.0	4853	0.01	451	243	231	103	11155
MASSACHUSETTS	19.0	1785	376	820	1013	4432	0	8426
MICHIGAN	20.0	36882	222	4429	4092	6533	9479	41837
MINNESOTA	21.0	55529	193	2316	2393	2557	24926	85914
MISSISSIPPI	22.0	37027	145	3310	1978	1943	3841	47984
MISSOURI	23.0	36674	328	3030	2528	2556	4516	69932
MONTANA	24.0	99723	0	447	511	415	47160	148456
NEBRASKA	25.0	72771	357	1245	945	849	1534	77721
NEVADA	26.0	86905	0	550	444	463	20255	110617
NEW HAMPSHIRE	27.0	5819	125	820	560	946	1197	9467
NEW JERSEY	28.0	1110	1	87	878	592	1003	0
NEW MEXICO	29.0	87892	0	936	531	849	31536	121744
NEW YORK	30.0	21954	926	5356	4429	7423	9930	50218
NORTH CAROLINA	31.0	26383	29	475	4632	4333	8427	50789
NORTH DAKOTA	32.0	63362	0	347	425	299	4572	71005
OHIO	33.0	18750	1148	5867	5636	8104	2326	41833
OKLAHOMA	34.0	38132	637	3223	2075	2084	3444	69615
OREGON	35.0	61519	174	1650	2248	1988	30349	47928
PENNSYLVANIA	36.0	22041	280	5192	5201	9013	3951	45278
RHODE ISLAND	37.0	48	10	193	270	465	0	1206
SOUTH CAROLINA	38.0	19628	4.0	83	1601	2243	5683	00
SOUTH DAKOTA	39.0	53162	0	347	328	376	22793	77006
TENNESSEE	40.0	27309	116	5249	3619	3213	2596	42122
TEXAS	41.0	234659	1042	9486	7990	10171	5491	268839
UTAH	42.0	56588	466	724	724	926	25553	85181
VERMONT	43.0	7556	0	482	454	338	1023	9853
VIRGINIA	44.0	23121	447	5211	2078	3445	5665	41167
WASHINGTON	45.0	36332	454	2306	2374	3088	24762	69316
WEST VIRGINIA	46.0	15729	0	73	333	342	453	3573
WISCONSIN	47.0	42315	174	3213	3146	3146	5029	57022
WYOMING	48.0	71786	0	280	328	367	25229	97986
TOTAL		2080704	13849	131258	106457	148018	557673	
		6443	0.5%	4.0%	3.6%	4.9%	18.3%	

NOTE: "AVAILABLE LAND" IS THAT AVAILABLE UNDER THE MOST CONSTRAINING CRITERIA (i.e., IF > 1/4 OF THE POPULATION ALLOWED BY A UNIFORM DENSITY CRITERION IS FOUND IN A "QUAD" SECTOR OF 90° DEGREES). NUMBERS IN THE COLUMNS REPRESENT THAT LAND UNIQUELY CONSTRAINED BY THE GIVEN FRACTIONAL CRITERION. THIS LAND IS CONSIDERED AVAILABLE IF THE CRITERION WERE RELATED IF SECTOR CRITERION IS APPLIED. ASSUME THAT UNIFORM DENSITY CRITERION IS ALSO IN EFFECT. \*\* COMPOSITE OF 5 RADII \*\*

TABLE F2.12

POPULATION SECTOR ANALYSIS - TOTAL U. S.  
 DENSITY = 1500 #/50 MI. \*\*\* QUADE - SECTOR (90.0 DEGREES)  
 STATE AREAS IN SQUARE MILES AND % OF STATE

TABULATION		AVAILABLE LAND						UNIFORM DENSITY RESTRICTED LANDS	
		> 1/4 ALLOWABLE POP		> 1/3 ALLOWABLE POP		> 1/2 ALLOWABLE POP			
		%	SQ. MILES	%	SQ. MILES	%	SQ. MILES		
ALABAMA	1.0	39391	0	4536	3213	2492	2075	51407	
		75.4%	0.0%	8.7%	6.2%	5.2%	4.0%		
ARIZONA	2.0	51820	318	772	584	1438	59405	114342	
		45.3%	0.2%	0.7%	0.5%	1.3%	5.0%		
ARKANSAS	3.0	42470	0	1747	1274	1476	6262	53259	
		79.8%	0.0%	3.3%	2.4%	2.8%	11.8%		
CALIFORNIA	4.0	88259	926	4603	4163	11020	51492	160363	
		55.0%	0.6%	2.4%	2.3%	6.9%	32.1%		
COLORADO	5.0	71275	589	1187	1129	1486	28660	104326	
		68.3%	0.5%	1.1%	1.1%	1.4%	27.5%		
CONNECTICUT	6.0	1255	10	676	598	2673	0	5212	
		24.1%	0.2%	13.0%	11.5%	51.3%	0.0%		
DELAWARE	7.0	1457	0	241	212	367	39	2328	
		63.1%	0.0%	10.4%	9.1%	15.8%	1.7%		
FLORIDA	8.0	34479	29	2345	3397	6002	13105	59357	
		58.1%	0.0%	4.0%	5.7%	10.1%	22.1%		
GEORGIA	9.0	41389	241	4123	3165	3619	5867	58644	
		70.6%	0.4%	7.4%	5.1%	6.2%	10.0%		
IDAHO	10.0	43946	0	772	637	674	37519	83550	
		52.6%	0.0%	0.9%	0.8%	0.8%	44.9%		
ILLINOIS	11.0	42981	454	3464	2634	5645	1361	56539	
		76.0%	0.0%	6.1%	5.7%	10.0%	2.4%		
INDIANA	12.0	23980	175	3889	3204	3812	1322	36342	
		66.0%	0.4%	10.7%	8.8%	10.5%	3.6%		
IOWA	13.0	51299	0	1785	1322	1660	0	56066	
		91.5%	0.0%	3.2%	2.4%	3.0%	0.0%		
KANSAS	14.0	77866	39	1293	1245	1631	193	82267	
		94.7%	0.0%	1.6%	1.5%	2.0%	0.2%		
KENTUCKY	15.0	27097	48	5616	2972	2065	2470	40268	
		67.3%	0.1%	12.9%	7.4%	5.1%	6.1%		
LOUISIANA	16.0	28217	0	2306	1448	1766	14417	48154	
		58.6%	0.0%	10.7%	8.8%	10.5%	3.6%		
MAINE	17.0	30581	0	1474	878	782	357	34074	
		89.7%	0.0%	4.3%	2.6%	2.3%	1.0%		
MARYLAND	18.0	4854	318	1341	1747	2750	145	11155	
		43.5%	2.9%	12.0%	15.7%	24.7%	1.3%		
MASSACHUSETTS	19.0	2863	39	685	1177	4063	0	8627	
		30.9%	0.4%	7.9%	13.6%	47.1%	0.0%		
MICHIGAN	20.0	38571	116	3802	3928	3742	9679	61838	
		62.4%	0.2%	6.1%	6.4%	9.3%	15.7%		
MINNESOTA	21.0	54214	367	2345	1872	2191	24926	85915	
		63.1%	0.4%	2.7%	2.2%	2.5%	29.0%		
MISSISSIPPI	22.0	37703	0	3049	1727	1915	3841	47884	
		78.7%	0.0%	6.5%	3.6%	3.2%	8.0%		
MISSOURI	23.0	57987	222	2721	1930	2557	4516	69933	
		82.9%	0.3%	3.9%	2.8%	3.7%	6.5%		
MONTANA	24.0	100051	0	463	374	405	47160	148455	
		67.4%	0.0%	0.3%	0.3%	0.3%	31.8%		
NEBRASKA	25.0	73909	0	894	618	801	1534	77721	
		95.1%	0.0%	1.1%	0.8%	1.0%	2.0%		
NEVADA	26.0	89446	0	290	251	376	20255	110618	
		80.9%	0.0%	0.3%	0.2%	0.3%	18.3%		
NEW HAMPSHIRE	27.0	6118	0	695	569	886	1197	9467	
		64.6%	0.0%	7.3%	6.0%	9.4%	12.6%		
NEW JERSEY	28.0	1402	145	511	1963	4188	0	8009	
		20.0%	1.8%	6.4%	19.5%	52.3%	0.0%		
NEW MEXICO	29.0	88604	0	569	454	782	31536	121745	
		72.6%	0.0%	0.5%	0.4%	0.6%	25.9%		
NEW YORK	30.0	24907	58	4314	4014	6976	9930	50219	
		49.6%	0.1%	8.6%	8.0%	13.9%	19.8%		
NORTH CAROLINA	31.0	27252	0	6485	4188	4217	8627	50769	
		53.7%	0.0%	12.8%	8.2%	8.3%	17.0%		
NORTH DAKOTA	32.0	63458	0	357	328	290	6572	71005	
		89.4%	0.0%	0.5%	0.5%	0.4%	9.3%		
OHIO	33.0	22861	290	4526	4709	7122	2326	41834	
		54.6%	0.7%	10.8%	11.3%	17.0%	5.6%		
OKLAHOMA	34.0	59878	0	2567	1824	1882	3464	69615	
		86.0%	0.0%	3.7%	2.6%	2.7%	5.0%		
OREGON	35.0	62744	0	1206	1776	1853	30349	97928	
		64.1%	0.0%	1.2%	1.8%	1.9%	31.0%		
PENNSYLVANIA	36.0	23440	318	3172	4709	8087	3551	45277	
		51.8%	0.7%	11.4%	10.4%	17.9%	7.8%		
RHODE ISLAND	37.0	270	0	145	193	598	0	1206	
		22.4%	0.0%	12.0%	16.0%	49.6%	0.0%		
SOUTH CAROLINA	38.0	20159	0	3995	2152	2220	2663	31189	
		64.6%	0.0%	12.8%	8.9%	7.1%	8.5%		
SOUTH DAKOTA	39.0	53297	0	241	309	367	22793	37007	
		89.2%	0.0%	0.3%	0.4%	0.5%	29.8%		
TENNESSEE	40.0	28342	0	4757	3229	3098	2516	42122	
		67.3%	0.0%	11.3%	7.9%	7.4%	6.2%		
TEXAS	41.0	241327	975	6880	5481	9685	5491	268839	
		89.8%	0.4%	2.6%	2.0%	2.2%	2.0%		
UTAH	42.0	57823	0	425	579	801	25553	85181	
		67.9%	0.0%	0.5%	0.7%	0.9%	30.0%		
VERMONT	43.0	7585	0	473	434	338	1023	9853	
		77.0%	0.0%	4.8%	4.4%	3.4%	10.4%		
VIRGINIA	44.0	25090	58	4613	2604	3126	5665	41168	
		60.9%	0.1%	11.2%	6.3%	7.6%	13.8%		
WASHINGTON	45.0	38108	280	1592	1892	2692	2474	69318	
		55.0%	0.4%	2.3%	2.7%	3.9%	35.7%		
WEST VIRGINIA	46.0	16038	0	2625	1416	1303	2721	24106	
		66.5%	0.0%	10.9%	5.9%	5.4%	11.3%		
WISCONSIN	47.0	43319	203	3098	2567	2808	5028	57023	
		72.0%	0.4%	5.4%	4.5%	4.9%	8.8%		
Wyoming	48.0	71412	0	193	290	367	15225	97987	
		73.4%	0.0%	0.2%	0.3%	0.4%	25.7%		
TOTAL		2141133	6178	112074	90981	131928	557673		
		70.4%	0.2%	3.7%	3.0%	4.3%	18.3%		

NOTE: "AVAILABLE LAND" IS THAT AVAILABLE UNDER THE MOST CONSTRAINING CRITERIA, I.E., IF  $\geq 1/4$  OF THE POPULATION ALLOWED BY A UNIFORM DENSITY CRITERION IS FOUND IN A "QUAD" SECTOR OF 90.0 DEGREES. NUMBERS IN THE COLUMNS REPRESENT THAT LAND UNIQUELY CONSTRAINED BY THE GIVEN FRACTIONAL CRITERION. THIS LAND IS CONSIDERED AVAILABLE IF THE CRITERION WERE RELATED IF SECTOR CRITERION IS APPLIED. ASSUME THAT UNIFORM DENSITY CRITERION IS ALSO IN EFFECT. \*\* COMPOSITE OF 5 RADII \*\*

TABLE F2.13

POPULATION SECTOR ANALYSIS - TOTAL U. S.  
DENSITY = 250 /SQ. MI. \*\*\* SINGLE SECTOR (22.5 DEGREES)  
STATE AREAS IN SQUARE MILES AND % OF STATE

## TABULATION

		21/16 POP. IN SECTOR										UNIFORM DENSITY NO POP. CRITERIA NO RESTRICTIONS			
		> 1/8 POP. IN SECTOR			> 1/8 POP. IN SECTOR			> 1/4 POP. IN SECTOR			> 1/3 POP. IN SECTOR				
		> 1/8 POP. IN SECTOR		> 1/8 POP. IN SECTOR	> 1/4 POP. IN SECTOR		> 1/4 POP. IN SECTOR	> 1/3 POP. IN SECTOR		> 1/3 POP. IN SECTOR	> 1/2 POP. IN SECTOR				
		> 1/8 POP. IN SECTOR	> 1/8 POP. IN SECTOR	> 1/8 POP. IN SECTOR	> 1/4 POP. IN SECTOR	> 1/4 POP. IN SECTOR	> 1/4 POP. IN SECTOR	> 1/3 POP. IN SECTOR	> 1/3 POP. IN SECTOR	> 1/2 POP. IN SECTOR	> 1/2 POP. IN SECTOR				
ALABAMA	1.0	16390	20382	24424	30098	35367	41779	44129	49833	51907					
ARIZONA	2.0	44033	48663	45166	48144	49215	50942	51409	54937	114143					
ARKANSAS	3.0	23594	27059	32019	36990	40485	43010	43830	46995	53258					
CALIFORNIA	4.0	4533	5083	6013	6931	7645	8035	8235	8905	9603	10003				
COLORADO	5.0	58527	61123	64095	66624	67224	70380	71719	75666	104326					
CONNECTICUT	6.0	10	10	19	116	183	203	222	251	321					
DELAWARE	7.0	309	366	457	446	1332	1544	1679	2287	2326					
FLORIDA	8.0	10749	15527	19626	25138	28330	32649	34856	44252	59387					
GEORGIA	9.0	16347	20439	25934	33186	36989	43298	47222	5237	56604					
IDAHO	10.0	35686	37809	40019	41736	43058	44380	44824	46031	83575					
ILLINOIS	11.0	11599	15151	23205	31864	37948	42103	43386	55170	56539					
INDIANA	12.0	3020	4482	6296	14002	19011	22908	24472	35020	36342					
IOWA	13.0	20230	28767	36911	43242	48038	51338	52255	54607	56047					
KANSAS	14.0	54574	62580	67801	72008	74624	77576	78358	82073	82284					
KENTUCKY	15.0	7243	7615	8244	8753	9101	9435	9525	9985	10003					
LOUISIANA	16.0	11455	13978	16704	21095	25440	28940	30012	33736	48154					
MARYLAND	17.0	23334	24772	26750	28593	30214	31816	32414	33717	34074					
MASSACHUSETTS	18.0	795	1602	2278	3252	4130	4912	5269	11011	11159					
MICHIGAN	19.0	58	251	415	840	1177	1573	1853	8427	8427					
MINNESOTA	20.0	73	219	4333	973	115	161	178	213	9295	10003				
MISSISSIPPI	21.0	25260	36357	45567	49842	52679	55459	58356	60988	85914					
MISSOURI	22.0	18258	20403	24848	30137	35975	40076	41136	44042	49893					
MONTANA	23.0	32294	36949	44274	51261	55471	59077	60462	63417	64934					
NEBRASKA	24.0	4745	5243	6333	7333	7943	8453	8593	9233	10003					
NEVADA	25.0	92026	94396	96027	98401	99356	100109	100475	101296	148436					
NEW HAMPSHIRE	26.0	2618	30866	37664	46461	5370	6128	6438	6734	8743	10003				
NEW JERSEY	27.0	2895	3262	4193	5143	5845	6473	6734	8743	10003					
NEW MEXICO	28.0	0	27	96	499	865	926	1081	8010	8010					
NEW YORK	29.0	80346	81320	83337	85045	86319	88153	89732	90208	121734					
NORTH CAROLINA	30.0	3703	7440	10629	15025	19734	24115	25852	40289	50219					
NORTH DAKOTA	31.0	6419	9484	12980	18991	23893	30487	33032	42142	50749					
OHIO	32.0	50868	56018	59955	61905	62908	6387	63960	64433	71009					
OKLAHOMA	33.0	1872	4101	7305	12009	16408	20533	21732	29907	41833					
OREGON	34.0	39633	43212	48375	52621	56800	60731	62107	64151	67619					
PENNSYLVANIA	35.0	47944	52139	54918	58450	60208	62114	62420	64978	67926					
RHODE ISLAND	36.0	3590	5324	5613	5975	6153	6355	6485	6493	10003					
SOUTH CAROLINA	37.0	0	0	0	0	29	48	56	1204						
SOUTH DAKOTA	38.0	0	0	0	0	0	545	545	10003	10003					
TENNESSEE	39.0	46918	48723	50412	52149	53795	53384	53794	54214	77007					
TEXAS	40.0	11435	14060	16878	20873	25756	31179	33350	39524	42122					
UTAH	41.0	166143	175519	196522	215919	220396	238751	242408	263349	268839					
VERMONT	42.0	3074	3595	4468	5443	6437	7431	7801	8270	8830	9853				
VIRGINIA	43.0	3955	4535	5523	6535	7541	8105	8345	8945	10003					
WASHINGTON	44.0	4774	4775	12989	17225	21847	27319	28902	35502	41187					
WEST VIRGINIA	45.0	1655	2375	3143	4185	5313	6645	7225	8425	10003					
WISCONSIN	46.0	19591	23343	28043	37486	40733	44622	45866	51994	57022					
Wyoming	47.0	3445	4045	5061	6243	7145	7835	8045	9125	10003					
TOTAL:		1405912	1518712	1681820	1864324	2012560	2150329	2199478	2482289	3039964					
		46.25	50.05	55.25	61.35	66.25	70.75	72.45	81.75	100.05					

NOTE: NUMBERS IN THE COLUMNS REPRESENT THE AMOUNT OF LAND THAT IS CONSIDERED TO BE AVAILABLE IF THE GIVEN CRITERION IS APPLIED. WHENEVER A SECTOR CRITERION IS APPLIED, IT IS ASSUMED THAT A UNIFORM DENSITY CRITERION IS ALSO IN EFFECT. CRITERIA WERE APPLIED TO 3 RADII (2, 3, 10, 20, 30) INDIVIDUALLY AND THE RESULTS COMBINED.

TABLE F2.14

POPULATION SECTOR ANALYSIS - TOTAL U.S.  
DENSITY = 500 /SQ MI \*\*\* SINGLE SECTOR (22.5 DEGREES)  
STATE AREAS IN SQUARE MILES AND % OF STATE

## TABULATION

	31/16 POP IN SECTOR														UNIFORM DENSITY			
	> 1/8 POP IN SECTOR				> 1/8 POP IN SECTOR				> 1/4 POP IN SECTOR				> 1/3 POP IN SECTOR					
	> 1/8 POP IN SECTOR		> 1/8 POP IN SECTOR		> 1/4 POP IN SECTOR		> 1/4 POP IN SECTOR		> 1/3 POP IN SECTOR		> 1/3 POP IN SECTOR		> 1/2 POP IN SECTOR					
															NO POP CRITERIA			
															NO RESTRICTIONS			
ALABAMA	7.0	24028	27629	30726	34798	39121	43936	46503	49833	51407								
ARIZONA	2.0	44800	57326	48723	50421	51463	52467	52911	54937	511343								
ARKANSAS	3.0	30577	31343	35004	39181	42682	44419	45239	46996	52258								
CALIFORNIA	4.0	44566	70976	75859	81996	85152	90565	92756	108871	160564								
COLORADO	5.0	63429	65060	67950	69248	70792	72558	73176	75666	104326								
CONNECTICUT	6.0	10	154	290	647	917	1226	1476	5211	5211								
DELAWARE	7.0	3.03	5.63	12.45	17.65	23.1%	26.3%	30.03	100.03	100.03								
FLORIDA	8.0	20149	23884	26552	31092	33833	36458	39121	46252	59357								
GEORGIA	9.0	25061	28767	32385	36824	41736	44146	48308	52737	58604								
IDAHO	10.0	39758	40501	41727	42885	43868	44853	45287	46031	83150								
ILLINOIS	11.0	19570	33902	32627	38975	43155	46330	47343	55179	56339								
INDIANA	12.0	7080	9375	13437	18074	23632	27274	29722	35020	36342								
IOWA	13.0	32241	34605	40309	48375	51618	53480	53992	56064	56066								
KANSAS	14.0	67296	68662	72616	75666	77932	79487	80027	82073	82266								
KENTUCKY	15.0	16839	18294	20362	22851	26788	32848	35116	37799	40269								
LOUISIANA	16.0	15855	18354	21085	24550	28294	30542	31536	33736	48154								
MAINE	17.0	26229	26557	27954	29230	30658	32241	32866	33717	34074								
MARYLAND	18.0	2123	2393	2963	3985	4880	5809	6504	11011	11155								
MASSACHUSETTS	19.0	434	654	1004	1834	2451	2943	3513	8627	8627								
MICHIGAN	20.0	22581	23922	26494	33630	36455	42093	44487	52156	61837								
MINNESOTA	21.0	3653	3873	4613	5443	6223	6811	7195	8437	100.03								
MISSISSIPPI	22.0	24125	25322	28458	32530	37760	41126	42257	44043	47883								
MISSOURI	23.0	41363	43064	49090	52837	56025	61065	61953	63417	69934								
ONTARIO	24.0	96654	9857	98054	99202	99955	100505	100843	101295	148456								
NEBRASKA	25.0	66527	67724	70213	72298	73736	74788	75135	76187	77721								
NEVADA	26.0	86059	86464	87757	88675	89089	89552	89774	90363	110618								
NEW HAMPSHIRE	27.0	3619	4052	4729	5345	6031	6832	7180	8270	9467								
NEW JERSEY	28.0	125	463	859	1129	1370	1660	2113	8010	8010								
NEW MEXICO	29.0	83801	84225	85383	87246	88142	88954	89262	90208	121744								
NEW YORK	30.0	9669	11927	13150	17261	20496	28960	31488	40289	50219								
NORTH CAROLINA	31.0	12902	13642	18769	22909	27252	33746	37104	42142	50749								
NORTH DAKOTA	32.0	58215	58797	61104	62590	63497	63922	64476	64433	71005								
OHIO	33.0	7160	8859	11995	16029	21558	26501	29018	39907	41833								
OKLAHOMA	34.0	47353	49292	53075	57040	60042	67400	67794	68151	89615								
OREGON	35.0	55246	57331	58923	60992	62349	63442	65164	67978	87828								
PENNSYLVANIA	36.0	9090	11551	14900	19155	23907	28062	30410	41727	45278								
RHODE ISLAND	37.0	0	0	10	46	135	241	299	1204	1206								
SOUTH CAROLINA	38.0	9756	11989	14407	16848	20207	24376	25861	28525	31189								
SOUTH DAKOTA	39.0	50759	50798	51994	53284	53355	53635	53799	54214	77007								
TEXAS	40.0	16656	18026	20603	24009	26313	31357	35879	39526	42122								
	41.0	19454	20519	217704	229844	239098	247416	250446	263348	268399								
UTAH	42.0	53480	54609	55613	56839	57572	58218	58556	59627	85181								
VERMONT	43.0	5308	5308	6080	6765	7643	8135	8482	8830	9853								
VIRGINIA	44.0	12120	14253	16530	20362	242926	29433	31430	35502	41187								
WASHINGTON	45.0	28757	30581	33090	35512	37963	39594	40810	44554	49316								
WEST VIRGINIA	46.0	9399	10393	11551	13471	16144	18962	19918	21384	24106								
WISCONSIN	47.0	26566	26796	33949	36918	43309	46494	48086	51994	57022								
WYOMING	48.0	67662	67663	70619	71352	71844	72134	72356	72781	87986								
TOTAL		54.2%	56.8%	61.3%	65.9%	70.1%	74.0%	75.8%	81.7%	100.0%								

NOTE: NUMBERS IN THE COLUMNS REPRESENT THE AMOUNT OF LAND THAT IS CONSIDERED TO BE AVAILABLE IF THE GIVEN CRITERION IS APPLIED. WHENEVER A SECTOR CRITERION IS APPLIED, IT IS ASSUMED THAT A UNIFORM DENSITY CRITERION IS ALSO IN EFFECT. CRITERIA WERE APPLIED TO 5 RADII (2, 5, 10, 20, 30) INDIVIDUALLY AND THE RESULTS COMPOSITED.

TABLE F2.15

POPULATION SECTOR ANALYSIS - TOTAL U. S.  
DENSITY = 750 #/SQ. MI. \*\*\* SINGLE SECTOR (22.5 DEGREES)  
STATE AREAS IN SQUARE MILES AND % OF STATE

## TABULATION

		2/16 POP IN SECTOR										UNIFORM DENSITY	NO POP CRITERIA	NO RESTRICTIONS			
		> 1/8 POP IN SECTOR			> 1/8 POP IN SECTOR			> 1/8 POP IN SECTOR			> 1/2 POP IN SECTOR						
		> 1/8 POP IN SECTOR			> 1/8 POP IN SECTOR			> 1/8 POP IN SECTOR			> 1/2 POP IN SECTOR						
		> 1/8 POP IN SECTOR			> 1/8 POP IN SECTOR			> 1/8 POP IN SECTOR			> 1/2 POP IN SECTOR						
		> 1/8 POP IN SECTOR			> 1/8 POP IN SECTOR			> 1/8 POP IN SECTOR			> 1/2 POP IN SECTOR						
		> 1/8 POP IN SECTOR			> 1/8 POP IN SECTOR			> 1/8 POP IN SECTOR			> 1/2 POP IN SECTOR						
ALASKA	1.0	21435	30407	33.67	34602	39729	44322	46976	49630	51907							
ARIZONA	2.0	47459	48877	50322	51465	52273	52805	53256	54933	1,4383							
ARKANSAS	3.0	31845	32490	37124	40550	42142	44612	45442	46995	53258							
CALIFORNIA	4.0	71543	75246	80664	85258	89724	93238	95545	108871	140344							
COLORADO	5.0	65380	68961	68640	70406	72028	73099	73803	75646	104326							
CONNECTICUT	6.0	87	257	621	936	1,222	1,653	2,048	2,511	3,211							
DELAWARE	7.0	837	743	975	1,283	15.3	1727	1891	2,287	2,326							
FLORIDA	8.0	25013	27261	30436	32051	34750	37017	39787	44212	59387							
GEORGIA	9.0	26815	31109	34084	37789	42257	46591	49926	52737	58604							
IDAHO	10.0	40839	41254	42335	43087	44052	44901	45345	46030	47550							
ILLINOIS	11.0	25167	27857	35425	40627	44815	47546	48761	53179	56529							
INDIANA	12.0	9843	11857	15537	20439	24936	26516	30561	35020	36342							
IDAHO	13.0	2713	3215	4285	5215	6853	7853	8493	9741	10003							
KANSAS	14.0	69557	71111	74392	76853	78580	79738	80327	82073	82266							
KENTUCKY	15.0	18490	19676	21404	23623	27328	32254	35647	37799	40269							
LOUISIANA	16.0	18943	20425	22407	25457	28622	30841	31884	33736	40154							
MAINE	17.0	26972	26981	28130	29365	30716	32279	32926	33717	34074							
MARYLAND	18.0	2480	2712	3329	4207	5211	6427	7411	11031	11155							
MASSACHUSETTS	19.0	856	1062	1,679	2239	2634	3252	3993	8627	8627							
MICHIGAN	20.0	24067	25978	30533	35049	39353	42894	43625	52159	61837							
MINNESOTA	21.0	3893	4202	4943	5253	5293	5943	6403	6625	7013	10003						
MISSISSIPPI	22.0	25233	26856	29471	30157	36060	41302	42460	44043	47883							
MISSOURI	23.0	43415	44558	50254	52928	59048	61625	62561	65417	69234							
ONTARIO	24.0	612	63	62.75	70.25	78.55	84.41	88.13	93.55	93.55	10003						
NEVADA	25.0	97494	97494	98411	98434	100128	100563	100881	101296	104456							
NEW HAMPSHIRE	26.0	4169	4333	4883	5452	6166	6948	7324	8270	9467							
NEW JERSEY	27.0	482	869	1,110	1,457	1,631	2,036	2,577	8010	8010							
NEW MEXICO	28.0	6	10.81	13.93	18.25	20.43	25.41	32.25	10003	10003							
NEW YORK	29.0	84544	85316	86860	87719	88471	89041	89359	90206	121744							
NORTH CAROLINA	30.0	12091	13829	14994	20776	23828	29380	32467	40299	50219							
NORTH DAKOTA	31.0	15994	17862	20477	22486	27531	32427	37809	42142	50749							
OHIO	32.0	59183	59183	61422	62976	63661	63998	64134	64433	71005							
OKLAHOMA	33.0	9235	10663	13857	16625	23110	26217	31401	39507	41833							
OREGON	34.0	50180	52013	55410	57429	60380	60841	64066	66151	69615							
PENNSYLVANIA	35.0	57398	58759	60245	61818	62999	63999	65591	67578	69299							
RHODE ISLAND	36.0	21	35.25	40.35	46.35	54.25	67.53	74.53	83.03	10003							
SOUTH CAROLINA	37.0	11754	14041	15941	17293	20410	24586	26229	26525	27189							
SOUTH DAKOTA	38.0	51193	51193	52293	53027	53461	53673	53837	54214	57007							
TENNESSEE	39.0	18142	19956	22550	24752	28709	32486	36313	39526	42122							
TEXAS	40.0	208662	214230	224187	235103	243798	250003	253177	263349	268039							
UTAH	41.0	56429	55439	56539	57591	57842	58344	58701	59627	65181							
VERMONT	42.0	5481	5491	6205	6813	7662	8154	8492	8830	9853							
VIRGINIA	43.0	55.63	55.75	63.03	69.13	77.83	82.83	86.23	89.63	10003							
WASHINGTON	44.0	14552	16038	18238	21452	25553	29954	32057	35902	41187							
WEST VIRGINIA	45.0	30774	32742	34653	36440	38677	40125	41466	44554	49316							
WISCONSIN	46.0	29095	30448	35425	39633	43985	47169	48649	51994	57022							
WYOMING	47.0	70262	70262	71072	71568	71912	72163	77144	72761	97986							
TOTAL		1745298	1807107	1929952	2047620	2164506	2271978	2334266	2482287	3039963							
		57.43	59.41	63.53	67.43	71.23	74.73	76.83	81.73	100.03							

NOTE: NUMBERS IN THE COLUMNS REPRESENT THE AMOUNT OF LAND  
THAT IS CONSIDERED TO BE AVAILABLE IF THE GIVEN CRITERION  
IS APPLIED. WHENEVER A SECTOR CRITERION IS APPLIED, IT IS  
ASSUMED THAT A UNIFORM DENSITY CRITERION IS ALSO IN EFFECT.  
CRITERIA WERE APPLIED TO 5 RADII (2, 5, 10, 20, 30) INDIVIDUALLY AND THE RESULTS COMBINED.

TABLE F2.16

POPULATION SECTOR ANALYSIS - TOTAL U. S.  
DENSITY = 1500 #/SQ MI \*\*\* SINGLE SECTOR (22.5 DEGREES)  
STATE AREAS IN SQUARE MILES AND % OF STATE

TABULATION	2/16 POP. IN SECTOR											
	> 1/8 POP. IN SECTOR			> 1/8 POP. IN SECTOR			> 1/8 POP. IN SECTOR			> 1/8 POP. IN SECTOR		
	> 1/8 POP. IN SECTOR			> 1/8 POP. IN SECTOR			> 1/8 POP. IN SECTOR			> 1/8 POP. IN SECTOR		
	> 1/8 POP. IN SECTOR			> 1/8 POP. IN SECTOR			> 1/8 POP. IN SECTOR			> 1/8 POP. IN SECTOR		
	> 1/8 POP. IN SECTOR			> 1/8 POP. IN SECTOR			> 1/8 POP. IN SECTOR			> 1/8 POP. IN SECTOR		
	UNIFORM DENSITY			NO POP. CRITERIA			NO RESTRICTIONS			NO RESTRICTIONS		
ALABAMA	1.0	32482	30106	34624	30281	29845	44438	47140	49233	511907		
ARIZONA	2.0	44666	51087	51820	52244	52544	52017	52206	54923	114342		
ARKANSAS	3.0	33735	34200	37933	40868	43242	44649	45519	46993	53298		
CALIFORNIA	4.0	79513	82286	85837	89079	92215	95303	97851	108971	140365		
COLORADO	5.0	48226	48933	70446	71671	72539	73475	74180	75466	104326		
CONNECTICUT	6.0	386	4403	4778	6078	6455	7045	7111	7230	10003		
DELAWARE	7.0	811	895	1052	1203	1573	1756	1920	2287	2346		
FLORIDA	8.0	29386	31546	32733	33794	35155	37413	40250	44232	59357		
GEORGIA	9.0	32276	33071	33174	36049	42441	46735	49118	52737	58044		
IDAHO	10.0	41804	41813	42643	43232	44081	44911	45555	46330	82550		
ILLINOIS	11.0	29712	30426	37673	42142	45464	48231	49732	55170	58339		
INDIANA	12.0	13279	14832	17997	21259	25244	26796	28208	35020	36342		
IOWA	13.0	36488	39140	46629	50074	52477	53827	54407	56067	5867		
KANSAS	14.0	72500	73002	75791	77204	78754	79853	80442	82073	82266		
KENTUCKY	15.0	20429	21076	22436	24067	27464	33292	35734	37799	40269		
LOUISIANA	16.0	21992	22031	23430	25727	26499	30409	31770	33736	48154		
MAINE	17.0	27194	27194	28226	29375	30726	32279	32935	33717	34074		
MARYLAND	18.0	2779	3156	3706	4931	6002	7045	8260	11011	11155		
MASSACHUSETTS	19.0	1380	1853	2094	2480	2943	3990	4568	8627	8627		
MICHIGAN	20.0	27638	38419	32202	35763	39855	43394	44617	52158	61837		
MINNESOTA	21.0	4473	4603	5213	5785	6453	7023	7513	8433	10003		
MISSISSIPPI	22.0	44632	45094	50508	53114	55536	57943	58797	60980	65914		
MISSOURI	23.0	45345	46088	51830	55709	59415	61847	62640	65417	69934		
MONTANA	24.0	77928	97928	98748	99446	100206	100572	100891	101294	148454		
NEBRASKA	25.0	67605	70153	72286	73475	74498	75048	75386	76187	77721		
NEVADA	26.0	86365	86732	89108	89417	89639	89862	89986	90362	110618		
NEW HAMPSHIRE	27.0	4420	4429	4988	5520	6195	6477	7369	8270	9467		
NEW JERSEY	28.0	1042	1322	1580	1949	2248	2608	2861	8009	8009		
NEW MEXICO	29.0	84165	84849	87564	88027	88967	89108	89427	90208	121744		
NEW YORK	30.0	14841	16303	18966	21713	26036	30330	33243	40289	50219		
NORTH CAROLINA	31.0	19330	19339	21124	23787	27744	34112	37925	42142	50749		
NORTH DAKOTA	32.0	59801	59801	61854	63072	63729	63994	64144	64433	71003		
OHIO	33.0	11812	13838	16984	20111	24492	26882	32385	39507	41833		
OKLAHOMA	34.0	53200	54320	56114	58267	60841	63024	64269	66151	67615		
OREGON	35.0	59949	60448	61355	62223	63063	64105	65726	67579	69928		
PENNSYLVANIA	36.0	15102	15363	17785	20516	24984	30108	33640	41727	45279		
RHODE ISLAND	37.0	10	164	251	280	328	444	608	1204	1204		
SOUTH CAROLINA	38.0	15083	15122	16154	17429	20504	24665	26304	28525	31189		
SOUTH DAKOTA	39.0	51599	51999	52515	53094	53471	53683	53847	54214	57007		
TENNESSEE	40.0	20731	21900	22852	25032	28844	32563	36429	39526	42122		
TEXAS	41.0	219402	223282	237062	239262	245621	251209	258683	263348	268839		
UTAH	42.0	35816	36735	57408	57735	57948	58450	58826	59627	63181		
VERMONT	43.0	5407	5616	6273	6842	7672	8154	8492	8630	9853		
VIRGINIA	44.0	16704	17872	19406	21848	28633	30204	32366	35502	41147		
WASHINGTON	45.0	33920	35143	36342	37348	39034	40416	41982	44554	49316		
WEST VIRGINIA	46.0	11126	11136	11947	13635	14270	19097	20208	21384	24104		
WISCONSIN	47.0	31732	32299	36545	40337	44400	47439	49184	51994	57022		
Wyoming	48.0	70812	70812	71313	71642	71970	72162	72394	72761	79886		
		7235	7235	7283	7315	7343	7343	7343	7433	10003		
TOTAL		1855831	1888350	1987803	2075429	2179879	2284405	2350362	2482286	3019953		
		81.0%	82.1%	85.4%	86.3%	71.7%	75.1%	77.3%	81.7%	100.0%		

NOTE: NUMBERS IN THE COLUMNS REPRESENT THE AMOUNT OF LAND THAT IS CONSIDERED TO BE AVAILABLE IF THE GIVEN CRITERION IS APPLIED. WHENEVER A SECTOR CRITERION IS APPLIED, IT IS ASSUMED THAT A UNIFORM DENSITY CRITERION IS ALSO IN EFFECT. CRITERIA WERE APPLIED TO 9 RADII (2, 5, 10, 20, 30) INDIVIDUALLY AND THE RESULTS COMBINED.

TABLE F2.17

POPULATION SECTOR ANALYSIS - TOTAL U. S.  
DENSITY = 250#/SQ MI \*\*\* DOUBLE SECTOR (45.0 DEGREES)  
STATE AREAS IN SQUARE MILES AND % OF STATE

TABULATION	> 1/8 POP. IN SECTOR		> 1/8 POP. IN SECTOR		> 1/3 POP. IN SECTOR		> 1/3 POP. IN SECTOR		> 1/2 POP. IN SECTOR		UNIFORM DENSITY		CRITERIA NO POP. CRITERIA NO RESTRICTIONS	
ALABAMA	1.0	19617	21201	24617	32742	34975	44129	49833	51567					
ARIZONA	2.0	37 93	40 89	51 73	63 13	76 21	89 03	94 03	101 31					
ARKANSAS	3.0	29 33	37 43	41 31	42 33	43 53	45 13	49 03	100 07					
CALIFORNIA	4.0	24495	45267	67256	46364	49720	51463	54937	114243					
COLORADO	5.0	27 33	38 12	42 47	44 43	50 53	53 73	57 92	100 05					
CONNECTICUT	6.0	10	10	29	106	203	222	3211	5211					
DELAWARE	7.0	644	444	800	1100	1476	1477	207	2326					
FLORIDA	8.0	18790	16200	11420	25 76	30214	3 954	44252	5435					
GEORGIA	9.0	2051	21934	29190	3e23e	42180	49 52	92737	5864					
IDAHO	10.0	36185	36339	41312	4226	4395	~821	46030	83956					
ILLINOIS	11.0	7128	19423	28443	32119	3966	42294	55178	56246					
INDIANA	12.0	4709	4012	11001	16241	21203	24472	35392	36342					
IOWA	13.0	13 08	16 58	20 13	44 71	58 33	67 33	94 43	100 09					
KANSAS	14.0	45166	45981	70179	72556	74854	78358	82073	82246					
KENTUCKY	15.0	14272	14622	18942	23496	30154	33484	37799	40289					
LOUISIANA	16.0	12931	14378	16972	22842	27599	30011	33734	48154					
MAINE	17.0	23 98	29 71	39 41	47 41	57 35	63 35	70 13	100 03					
MARYLAND	18.0	1573	1621	2676	3448	4729	5264	59101	11198					
MASSACHUSETTS	19.0	174	261	349	897	1370	1853	2627	8847					
MICHIGAN	20.0	22562	22899	27086	31449	36178	38739	52156	41837					
MINNESOTA	21.0	34 55	37 05	43 81	50 93	56 51	62 61	84 33	100 05					
MISSISSIPPI	22.0	21954	23357	44830	~381	64434	64384	64998	89914					
MISSOURI	23.0	41061	41745	49109	53422	57951	60062	65417	69734					
MONTANA	24.0	94744	95534	97494	98941	99839	100476	101296	148456					
NEBRASKA	25.0	63 85	64 43	65 73	66 65	67 35	67 73	68 25	100 03					
NEVADA	26.0	85663	86020	87024	87427	88404	89125	90363	110618					
NEW HAMPSHIRE	27.0	3194	3010	4400	5333	6031	6031	6364	8270	9447				
NEW JERSEY	28.0	0	19	116	405	840	1081	8009	8009					
NEW MEXICO	29.0	82063	82486	84370	85380	87091	88732	90208	121744					
NEW YORK	30.0	7469	8231	12477	16936	21923	25562	40289	50219					
NORTH CAROLINA	31.0	8443	9766	14871	21732	29201	33032	42124	50749					
NORTH DAKOTA	32.0	58209	58626	61413	62278	63024	63760	64433	71005					
OHIO	33.0	2351	4632	9187	14184	18473	21732	29507	41833					
OKLAHOMA	34.0	40081	45809	50334	54011	58498	62107	66131	67615					
OREGON	35.0	52149	53056	56221	58617	61046	63420	67379	77926					
PENNSYLVANIA	36.0	53 35	54 23	57 43	60 13	62 35	64 85	69 03	100 03					
RHODE ISLAND	37.0	0	0	0	10	56	66	1206	1206					
SOUTH CAROLINA	38.0	7337	7790	11416	15190	21182	23700	28525	31189					
SOUTH DAKOTA	39.0	48653	49379	51415	52322	52950	53594	54214	77007					
TENNESSEE	40.0	10450	14726	18885	23199	29577	33350	35526	42122					
TEXAS	41.0	181256	184777	204546	219114	232406	242406	263349	268829					
UTAH	42.0	53220	53422	54494	55206	56423	57543	57627	85181					
VERMONT	43.0	4420	4931	6022	7199	7845	8270	8830	9953					
VIRGINIA	44.0	9325	10634	14813	19811	25534	28902	35502	41167					
WASHINGTON	45.0	24974	26564	29741	30987	35782	38253	44554	47314					
WEST VIRGINIA	46.0	7141	7836	11667	14372	17872	1915	21384	24106					
WISCONSIN	47.0	24849	25823	23350	37953	42991	45866	51994	57022					
WYOMING	48.0	67837	67637	70179	70850	71468	72211	72761	97984					
TOTAL		50 81	52 13	58 33	63 33	68 81	72 43	81 73	100 03					

NOTE: NUMBERS IN THE COLUMNS REPRESENT THE AMOUNT OF LAND THAT IS CONSIDERED TO BE AVAILABLE IF THE GIVEN CRITERION IS APPLIED. WHENEVER A SECTOR CRITERION IS APPLIED, IT IS ASSUMED THAT A UNIFORM DENSITY CRITERION IS ALSO IN EFFECT. CRITERIA WERE APPLIED TO 5 RADII (2, 5, 10, 20, 30) INDIVIDUALLY AND THE RESULTS COMBINED.

TABLE F2.18

POPULATION SECTOR ANALYSIS - TOTAL U.S.  
DENSITY = 300 #/SQ MI \*\*\* DOUBLE SECTOR (45° 0 DEGREES)  
STATE AREAS IN SQUARE MILES AND % OF STATE

TABULATION	> 1/8 POP. IN SECTOR		> 1/6 POP. IN SECTOR		> 1/3 POP. IN SECTOR		> 1/2 POP. IN SECTOR		UNIFORM DENSITY	NO POP. CRITERIA	NO RESTRICTIONS
	%	MILES	%	MILES	%	MILES	%	MILES			
ALABAMA	1.0	26652	29371	32008	37540	41357	46203	49833	31907		
	51.28	54.72	62.28	72.45	83.33	89.63	96.33	100.00			
ARIZONA	2.0	47333	47562	49188	50212	51724	52911	54727	314343		
	51.42	41.95	45.61	41.13	45.25	53.33	51.03	100.00			
ARKANSAS	3.0	32917	36738	38542	41344	43840	44239	46996	52258		
	51.71	64.33	72.43	77.63	82.33	84.33	88.23	100.00			
CALIFORNIA	4.0	69422	72057	77465	82462	87731	97556	10371	140364		
	43.31	44.91	48.42	52.13	53.33	51.83	57.43	100.00			
COLORADO	5.0	65214	65552	66.64	67654	71179	73176	75666	104321		
	62.51	63.83	65.45	66.83	68.53	70.13	72.53	100.00			
CONNECTICUT	6.0	29	77	309	685	1168	1476	3211	5211		
	0.63	1.53	5.93	13.13	23.45	29.33	100.00				
DELAWARE	7.0	656	685	1071	1419	1696	1862	2287	2326		
	29.23	29.53	46.15	61.03	73.03	80.33	98.33	100.00			
FLORIDA	8.0	33170	24009	27782	31507	35329	39129	46252	59357		
	39.03	40.43	46.83	53.13	59.33	65.93	77.93	100.00			
GEORGIA	9.0	26342	30154	34981	40154	45394	48308	52737	56604		
	48.43	51.93	51.73	48.93	77.53	82.43	90.03	100.00			
IDAHO	10.0	61099	41446	42665	43637	44477	45287	46031	53550		
	49.23	49.23	51.03	52.23	53.23	54.23	55.83	100.00			
ILLINOIS	11.0	27396	28728	36224	40617	44670	47343	55179	56539		
	48.53	50.83	64.13	71.83	77.03	83.73	97.63	100.00			
INDIANA	12.0	10731	11599	12421	20796	26316	29722	35029	36342		
	29.51	31.93	44.73	57.23	72.43	81.83	96.43	100.00			
IOWA	13.0	26652	40067	46474	49377	52361	53992	56067	56067		
	70.73	71.53	82.93	98.13	93.43	96.33	100.00				
KANSAS	14.0	70397	70684	74035	76167	76445	80027	82073	82266		
	85.63	85.93	90.03	92.63	95.43	97.23	99.83	100.00			
KENTUCKY	15.0	18383	19300	21865	25823	32192	35116	37799	40269		
	45.73	47.93	54.35	64.13	79.93	87.23	93.93	100.00			
LOUISIANA	16.0	18653	18991	23334	26692	30640	31536	33734	48154		
	36.73	39.43	48.53	55.43	62.43	65.53	70.13	100.00			
MAINE	17.0	27474	27474	29056	30397	32077	32868	33717	34074		
	80.63	80.63	85.33	89.23	94.13	96.33	99.03	100.00			
MARYLAND	18.0	2519	2625	3570	4497	5378	6304	11011	11155		
	22.63	23.53	32.03	40.33	50.03	58.33	98.73	100.00			
MASSACHUSETTS	19.0	589	695	1148	2017	2770	3513	8627	8627		
	6.83	8.13	12.33	23.43	32.13	40.73	100.00				
MICHIGAN	20.0	25650	26557	31546	36477	41341	44467	52158	61837		
	41.53	42.93	51.03	59.03	66.93	71.93	84.33	100.00			
MINNESOTA	21.0	47430	47579	50817	53900	59941	57499	60984	65914		
	55.23	55.43	59.13	62.33	65.13	67.13	71.03	100.00			
MISSISSIPPI	22.0	26055	26634	31449	36892	40578	42257	44043	47883		
	54.43	55.63	65.73	72.03	84.73	88.33	92.03	100.00			
MISSOURI	23.0	47164	47546	52630	56433	59772	61953	65417	69934		
	47.43	60.03	73.33	80.73	85.53	88.63	93.53	100.00			
MONTANA	24.0	97803	97803	98884	99540	100312	100843	101296	148456		
	65.93	65.93	66.63	67.13	67.63	67.93	68.23	100.00			
NEBRASKA	25.0	46843	49326	71092	72809	73996	75135	76187	77721		
	89.63	89.23	91.33	93.73	95.23	96.73	98.03	100.00			
NEVADA	26.0	87027	87149	88056	88741	91936	93774	93363	110618		
	78.73	78.83	79.43	80.23	80.43	81.23	81.73	100.00			
NEW HAMPSHIRE	27.0	4254	4410	5230	5954	6716	7180	8270	9467		
	45.03	46.63	55.23	60.93	70.43	75.83	87.43	100.00			
NEW JERSEY	28.0	270	386	847	1177	1612	2113	8010	8010		
	3.43	4.83	10.81	14.73	20.13	24.43	100.03	100.03			
NEW MEXICO	29.0	84360	84543	86048	87531	89577	92622	92026	121744		
	49.33	49.53	70.73	72.03	72.83	73.33	74.13	100.00			
NEW YORK	30.0	11976	12134	17129	22214	27319	31488	40289	50219		
	23.83	24.23	34.13	44.23	54.43	62.73	80.23	100.00			
NORTH CAROLINA	31.0	14984	16859	20912	26248	33273	37104	42142	50769		
	29.53	33.23	41.23	51.73	65.53	73.13	83.03	100.00			
NORTH DAKOTA	32.0	46499	46498	47204	43082	43700	44076	46433	71005		
	85.53	85.53	87.63	89.83	89.73	92.23	90.73	100.00			
OHIO	33.0	9747	10229	13790	18972	24598	29018	35907	41833		
	23.33	24.53	33.03	45.43	58.83	64.43	94.43	100.00			
OKLAHOMA	34.0	50263	50798	54561	58132	61866	63794	66151	69615		
	72.43	73.03	78.43	83.53	88.93	91.63	95.03	100.00			
OREGON	35.0	56588	57697	59637	61277	62928	65166	67579	79728		
	57.83	58.93	60.93	62.63	64.33	66.53	69.03	100.00			
PENNSYLVANIA	36.0	11197	12603	17061	22205	27454	30610	41727	45278		
	25.23	27.83	37.73	49.03	80.63	87.63	92.23	100.00			
RHODE ISLAND	37.0	0	0	10	58	203	299	1206	1206		
	0.03	0.03	0.83	4.83	16.83	28.83	100.03	100.00			
SOUTH CAROLINA	38.0	11010	12082	15372	19348	23971	25881	28525	31189		
	35.43	38.73	49.33	62.03	76.73	83.03	91.53	100.00			
SOUTH DAKOTA	39.0	51608	51608	52419	52940	53509	53799	54214	77007		
	67.03	67.03	68.13	68.73	69.53	69.93	70.43	100.00			
TENNESSEE	40.0	18017	18567	22359	27145	32752	35879	39526	42122		
	42.83	44.13	52.13	64.43	77.83	85.23	93.83	100.00			
TEXAS	41.0	207629	210486	223668	232321	242553	250446	263349	268839		
	77.23	78.33	83.23	88.83	90.23	93.23	98.03	100.00			
UTAH	42.0	54387	54822	55912	56539	57832	58556	59627	85181		
	63.83	64.43	65.63	66.43	67.93	68.73	70.03	100.00			
VERMONT	43.0	5934	5934	6449	7546	8058	8482	8830	9853		
	60.43	60.43	67.53	76.63	81.83	86.13	89.63	100.00			
VIRGINIA	44.0	14195	14977	18528	22356	28738	31430	3502	41167		
	34.53	36.43	45.03	52.23	69.83	76.33	86.23	100.00			
WASHINGTON	45.0	30571	31112	32775	36149	38706	40810	44554	49316		
	44.13	44.93	48.73	52.23	55.83	58.93	64.33	100.00			
WEST VIRGINIA	46.0	10673	10847	13221	13826	18789	19198	21384	24106		
	44.33	45.03	54.83	65.73	77.93	82.63	88.73	100.00			
WISCONSIN	47.0	31334	32019	37442	42035	45683	48086	51994	57022		
	55.03	56.23	65.73	73.73	80.13	84.33	91.23	100.00			
WYOMING	48.0	70233	70233	70976	71516	72018	72356	72761	97986		
	71.73	71.73	72.43	73.03	73.93	74.83	74.33	100.00			
TOTAL	38.03	58.93	63.73	66.23	72.83	75.83	81.73	100.00			

NOTE - NUMBERS IN THE COLUMNS REPRESENT THE AMOUNT OF LAND THAT IS CONSIDERED TO BE AVAILABLE IF THE GIVEN CRITERION IS APPLIED WHENEVER A SECTOR CRITERION IS APPLIED. IT IS ASSUMED THAT A UNIFORM DENSITY CRITERION IS ALSO IN EFFECT. CRITERIA WERE APPLIED TO 9 RADII (2, 3, 10, 20, 30) INDIVIDUALLY AND THE RESULTS COMPOSITED.

TABLE F2.19

POPULATION SECTOR ANALYSIS - TOTAL U. S.  
DENSITY = 730 #/SQ MI \*\*\* DOUBLE SECTOR (45° DEGREES)  
STATE AREAS IN SQUARE MILES AND % OF STATE

TABULATION	> 1/8 POP. IN SECTOR		> 1/6 POP. IN SECTOR		> 1/3 POP. IN SECTOR		> 1/2 POP. IN SECTOR		UNIFORM DENSITY	
									#S POP CRITERIA	
									NO RESTRICTIONS	
ALABAMA	1. 0	29934	30744	34708	36936	44081	44976	49833	51907	
ARIZONA	2. 0	48306	48906	30412	31307	52602	53258	54937	54343	
ARKANSAS	3. 0	25531	36023	27472	42363	44322	45442	46993	53230	
CALIFORNIA	4. 0	75270	76467	81199	85725	91638	95345	100871	140364	
COLORADO	5. 0	67174	67685	69364	70744	72587	73803	75466	104326	
CONNECTICUT	6. 0	134	261	437	1129	1824	2268	3211	5211	
DELAWARE	7. 0	742	840	1216	1305	1718	1891	2087	2326	
FLORIDA	8. 0	3285	3611	5235	6473	7373	8135	9835	10000	
GEORGIA	9. 0	32009	32364	36612	41290	41195	46956	52737	58404	
IDAHO	10. 0	41920	42045	42875	43792	44679	45345	46030	63150	
ILLINOIS	11. 0	32124	32042	38716	42544	44407	48741	51179	56559	
INDIANA	12. 0	13064	13407	18374	23140	27744	30461	35020	36342	
IDAHO	13. 0	42347	43406	48192	51010	53075	54272	56047	56067	
KANSAS	14. 0	72004	72473	73521	77432	79178	80327	82073	82266	
KENTUCKY	15. 0	20312	20506	22890	26444	30829	35447	37785	40269	
LOUISIANA	16. 0	20544	21375	24472	28043	30455	31664	33734	48154	
MAINE	17. 0	27985	27985	29286	30410	32134	32926	33717	34074	
MARYLAND	18. 0	2885	2991	38899	4786	5935	7411	11011	11155	
MASSACHUSETTS	19. 0	859	1033	1862	2441	3185	3495	4827	5827	
MICHIGAN	20. 0	27890	28776	33349	36223	42334	45425	52158	61837	
MINNESOTA	21. 0	48414	48930	51618	54204	56449	58431	60788	69114	
MISSISSIPPI	22. 0	28207	28487	32549	37577	40897	42460	44043	47880	
MISSOURI	23. 0	48983	49234	53519	57353	60805	62561	65417	69934	
MONTANA	24. 0	98179	99076	99929	100447	100861	101296	148454		
NEBRASKA	25. 0	70040	70165	72385	73427	74672	75338	76187	77721	
NEVADA	26. 0	87719	87912	88645	90112	92542	99999	90363	110618	
NEW HAMPSHIRE	27. 0	4641	4787	5275	6080	6880	7324	8270	9467	
NEW JERSEY	28. 0	996	791	1187	1505	1969	2377	3009	3609	
NEW MEXICO	29. 0	85339	85711	87255	98105	98744	99259	90208	121744	
NEW YORK	30. 0	14674	15180	19329	23931	29133	32465	40269	50219	
NORTH CAROLINA	31. 0	18403	18750	22803	27097	30987	37809	42142	50769	
NORTH DAKOTA	32. 0	61123	61223	62704	63439	63777	64134	64433	71005	
OHIO	33. 0	11329	11176	11625	21900	27184	31401	39307	41839	
OKLAHOMA	34. 0	52274	52986	56847	59576	62277	64066	66131	69613	
OREGON	35. 0	58913	59048	60805	62096	63796	65591	67579	97928	
PENNSYLVANIA	36. 0	14378	15044	19220	23353	26905	32713	41727	45278	
RHODE ISLAND	37. 0	0	10	39	164	357	521	1206		
SOUTH CAROLINA	38. 0	13549	14297	16714	20101	24306	26229	28925	31189	
SOUTH DAKOTA	39. 0	91945	91965	52805	53287	53557	53837	54214	77007	
TENNESSEE	40. 0	19937	20593	23893	26148	33136	36313	39526	42124	
TEXAS	41. 0	217173	218524	229554	237950	247590	253177	263348	268839	
UTAH	42. 0	55381	55632	56800	57456	58236	58701	59627	65181	
VERMONT	43. 0	6126	6126	6803	7395	8095	8492	8830	9853	
VIRGINIA	44. 0	16164	16637	20236	24578	29492	32057	35502	41167	
WASHINGTON	45. 0	32192	33022	35290	37644	39748	41466	44554	49316	
WEST VIRGINIA	46. 0	11361	11561	13433	16087	18995	20072	21384	24106	
WISCONSIN	47. 0	34016	34142	36861	42991	46561	48948	51994	57022	
WYOMING	48. 0	70677	70677	71410	71796	72124	72994	72751	97986	
TOTAL		1849701	1849705	2000661	2122471	2252019	2334266	2482266	3039963	
		60. 0%	61. 5%	65. 8%	67. 8%	74. 1%	76. 8%	81. 7%	100. 0%	

NOTE: NUMBERS IN THE COLUMNS REPRESENT THE AMOUNT OF LAND  
THAT IS CONSIDERED TO BE AVAILABLE AT THE GIVEN CRITERION  
IS APPLIED. WHENEVER A SECTOR CRITERION IS APPLIED, IT IS  
ASSUMED THAT A UNIFORM JEWELRY CRITERION IS ALSO IN EFFECT.  
CRITERIA WERE APPLIED TO 2 RADII (2, 5, 10, 20, 30) INDIVI-  
DUALLY AND THE RESULTS COMPOSITED.

TABLE F2.20

POPULATION SECTOR ANALYSIS - TOTAL U. S.  
 DENSITY = 1500 /SQR MI. \*\*\* DOUBLE SECTOR (45.0 DEGREES)  
 STATE AREAS IN SQUARE MILES AND % OF STATE

TABULATION	> 1/8 POP. IN SECTOR		> 1/8 POP. IN SECTOR		> 1/3 POP. IN SECTOR		> 1/2 POP. IN SECTOR		UNIFORM DENSITY NO POP. CRITERIA NO RESTRICTIONS	
	> 1/8 POP. IN SECTOR		> 1/3 POP. IN SECTOR		> 1/2 POP. IN SECTOR					
	> 1/8 POP. IN SECTOR		> 1/3 POP. IN SECTOR		> 1/2 POP. IN SECTOR					
	> 1/8 POP. IN SECTOR		> 1/3 POP. IN SECTOR		> 1/2 POP. IN SECTOR					
	> 1/8 POP. IN SECTOR		> 1/3 POP. IN SECTOR		> 1/2 POP. IN SECTOR					
ALABAMA	1.0	23958	33958	36.234	29726	44384	47140	49833	51907	
ARIZONA	2.0	65.43	65.73	48.0%	70.33	89.21	90.83	81.0%	100.0%	
ARKANSAS	3.0	37201	37471	40407	42905	44448	45119	46995	53258	
CALIFORNIA	4.0	81842	92787	68.79	91.06	94975	97851	106871	140364	
COLORADO	5.0	69229	79441	71.11	72.40	73301	74180	75466	104326	
CONNECTICUT	6.0	590	752	95.95	1351	1999	2336	3211	5211	
DELAWARE	7.0	1013	1623	14.43	18.33	25.93	36.33	44.74	100.0%	
FLORIDA	8.0	31527	31544	23.89	34.914	37143	40250	44252	59357	
GEORGIA	9.0	34293	34605	37.84	47.03	46388	49119	52737	58604	
IDAHO	10.0	42460	42464	43144	42985	44776	45355	46031	83550	
ILLINOIS	11.0	35641	36023	41.04	44.44	47832	49533	51179	56329	
INDIANA	12.0	16110	16869	207.19	244.94	28448	31308	35020	36342	
IDAHO	13.0	49741	49780	44.95	51.88	53384	54407	56067	64087	
KANSAS	14.0	74913	75067	76891	78251	79439	80442	82073	82266	
KENTUCKY	15.0	21598	21915	23922	27367	33032	37734	37799	40249	
LOUISIANA	16.0	23112	23121	25544	28477	30487	31770	33734	48154	
MAINE	17.0	28207	28207	29336	30468	32163	32935	33717	34074	
MARYLAND	18.0	3271	3429	4320	5354	6890	8260	11011	11159	
MASSACHUSETTS	19.0	1824	1911	2364	2906	3351	4364	8627	8627	
MICHIGAN	20.0	31305	31401	35174	39276	43097	44117	52158	61937	
MINNESOTA	21.0	50.65	50.83	56.93	63.53	69.73	75.13	84.35	100.0%	
MISSISSIPPI	22.0	29674	29896	33215	37793	41032	42529	44043	47883	
MISSOURI	23.0	50489	50798	55034	59855	61413	62660	63417	69934	
MONTANA	24.0	98575	98575	99492	100090	105334	106891	101296	148456	
NEBRASKA	25.0	66.43	66.43	67.03	67.43	67.73	68.03	68.23	100.0%	
NEVADA	26.0	88712	88712	92.13	94.13	95.53	96.35	97.03	100.0%	
NEW HAMPSHIRE	27.0	4912	4921	5491	6176	6919	73862	8270	9467	
NEW JERSEY	28.0	91.93	92.03	98.03	105.23	113.13	117.03	87.43	100.0%	
NEW MEXICO	29.0	1187	1332	1489	1959	2625	3621	8010	8010	
NEW YORK	30.0	17515	17939	21268	25534	29905	32292	40289	50219	
NORTH CAROLINA	31.0	20863	23443	27589	31480	37925	42142	50749	54113	
NORTH DAKOTA	32.0	61741	61741	62908	63555	63864	64144	64433	71005	
OHIO	33.0	14552	15247	17258	20477	28332	32383	39907	41833	
OKLAHOMA	34.0	55922	55960	57996	60245	62657	64249	66131	69613	
OREGON	35.0	40399	40660	42049	43043	48008	49724	67576	77228	
PENNSYLVANIA	36.0	17274	17270	20178	24842	29712	33640	41727	45279	
RHODE ISLAND	37.0	46	145	232	318	434	506	1206	1206	
SOUTH CAROLINA	38.0	41.13	41.13	46.63	54.35	67.35	74.73	83.03	100.0%	
SOUTH DAKOTA	39.0	52428	52438	53007	53346	53994	53847	54214	77007	
TENNESSEE	40.0	22398	22591	24810	28651	33770	36429	39526	42122	
TEXAS	41.0	22604	227142	237023	244164	250157	254643	263349	268839	
UTAH	42.0	84.13	84.53	88.21	90.83	93.11	94.73	98.03	100.0%	
VERMONT	43.0	6253	6253	6823	7424	8104	8492	9830	9830	
VIRGINIA	44.0	18470	18769	21587	25579	30031	32344	35502	41167	
WASHINGTON	45.0	35076	35493	37210	38680	40202	41842	44.54	69316	
WEST VIRGINIA	46.0	11808	11908	13426	16173	18943	20082	21384	24106	
WISCONSIN	47.0	35753	35917	39932	43859	47053	49166	51994	57022	
WYOMING	48.0	71227	71227	71613	71912	72143	72394	72761	74786	
TOTAL	64.03	64.35	67.75	71.25	74.85	77.35	81.73	100.0%		

NOTE: NUMBERS IN THE COLUMNS REPRESENT THE AMOUNT OF LAND THAT IS CONSIDERED TO BE AVAILABLE AT THE GIVEN CRITERION IS APPLIED. WHENEVER A SECTOR CRITERION IS APPLIED, IT IS ASSUMED THAT A UNIFORM DENSITY CRITERION IS ALSO IN EFFECT. CRITERIA WERE APPLIED TO 5 RADII (2, 5, 10, 20, 30) INDIVIDUALLY AND THE RESULTS COMPOSITED.

TABLE F.2.21

POPULATION SECTOR ANALYSIS - TOTAL U.S.  
DENSITY = 250 #/SQ MI \*\*\* "QUAD" SECTOR (90° 0 DEGREES)  
STATE AREAS IN SQUARE MILES AND % OF STATE

TABULATION	> 1/4 POP. IN SECTOR							
	> 1/3 POP. IN SECTOR		> 1/2 POP. IN SECTOR		UNIFORM DENSITY		NO POP.	CRITERIA
ALABAMA	1.0	268834	31198	38175	44129	49833	11907	
		53 5%	60 1%	73 1%	80 0%	96 0%	100 0%	
ARIZONA	2.0	47975	45047	49324	51606	54937	114312	
		41 0%	42 0%	43 0%	45 1%	48 0%	100 0%	
ARKANSAS	2.0	37152	37481	41466	42830	44995	51258	
		69 8%	70 4%	77 9%	82 3%	88 2%	100 0%	
CALIFORNIA	4.0	47248	1072	79048	86048	108871	160364	
		42 3%	40 3%	48 7%	53 7%	67 9%	100 0%	
COLORADO	5.0	65956	66440	69316	71714	75666	104326	
		63 2%	63 7%	66 4%	68 7%	72 5%	100 0%	
CONNECTICUT	6.0	29	48	125	222	5211	5211	
		9 7%	0 0%	2 6%	4 3%	10 0%	100 0%	
DELAWARE	7.0	930	945	1511	1879	2287	2326	
		40 2%	41 9%	57 7%	72 2%	98 3%	100 0%	
FLORIDA	8.0	22147	13 19	26862	31836	46232	59257	
		37 2%	38 1%	48 7%	50 7%	77 9%	100 0%	
GEORGIA	9.0	32151	24527	37520	4112	52737	78404	
		54 9%	58 7%	69 1%	78 0%	90 0%	100 0%	
IDAHO	10.0	41585	40045	43467	44824	46030	87556	
		49 7%	50 3%	52 2%	53 6%	55 1%	100 0%	
ILLINOIS	11.0	29249	29445	36417	42786	55179	56739	
		51 7%	53 9%	57 1%	73 7%	97 6%	100 0%	
INDIANA	12.0	123223	12600	19590	24472	35020	36342	
		33 9%	38 0%	33 9%	67 3%	96 4%	100 0%	
IOWA	13.0	42674	42894	49041	52255	56067	56067	
		75 0%	76 5%	87 3%	93 2%	100 0%	100 0%	
KANSAS	14.0	71410	71979	75077	78358	82073	82266	
		84 8%	87 3%	91 2%	95 2%	99 8%	100 0%	
KENTUCKY	15.0	21124	21954	29027	33486	37799	40269	
		52 5%	54 5%	72 1%	80 2%	93 9%	100 0%	
LOUISIANA	16.0	26738	21732	26711	30012	33736	46154	
		43 13	45 13	55 5%	62 3%	70 13	100 0%	
MAINE	17.0	28603	29027	31227	32414	33717	34074	
		83 9%	85 23	91 6%	95 1%	99 0%	100 0%	
MARYLAND	18.0	3059	3310	4314	5249	11011	11135	
		27 4%	29 7%	36 7%	47 2%	98 7%	100 0%	
MASSACHUSETTS	19.0	480	549	1042	1833	8427	8427	
		9 4%	8 6%	12 3%	21 3%	100 0%	100 0%	
MICHIGAN	20.0	28641	29344	34547	36725	52158	61837	
		46 7%	47 5%	55 9%	62 4%	84 3%	100 0%	
MINNESOTA	21.0	50369	50798	53741	56356	60986	85914	
		56 7%	59 13	62 6%	65 6%	71 0%	100 0%	
MISSISSIPPI	22.0	32028	32748	37789	41136	44043	47883	
		64 7%	66 43	76 9%	85 9%	92 0%	100 0%	
MISSOURI	23.0	51314	52572	56443	60042	63417	64934	
		73 42	75 2%	80 7%	85 9%	93 5%	100 0%	
MONTANA	24.0	98674	98674	99900	100474	101294	148456	
		46 6%	46 6%	47 2%	47 7%	66 2%	100 0%	
NEBRASKA	25.0	67567	70445	72143	74015	76187	77721	
		89 53	90 63	92 83	95 23	96 0%	100 0%	
NEVADA	26.0	87194	87390	88615	89189	90343	110618	
		78 8%	79 0%	79 6%	80 8%	91 7%	100 0%	
NEW HAMPSHIRE	27.0	4487	4835	5809	6349	8270	9447	
		47 4%	51 13	61 43	67 33	87 4%	100 0%	
NEW JERSEY	28.0	29	116	540	1081	8010	8010	
		0 4%	1 4%	7 0%	13 5%	100 0%	100 0%	
NEW MEXICO	29.0	84949	85190	86782	88732	90208	121744	
		49 8%	70 0%	71 35	72 9%	74 1%	100 0%	
NEW YORK	30.0	15815	14919	20217	25982	40289	50219	
		25 3%	29 7%	40 3%	50 9%	80 2%	100 0%	
NORTH CAROLINA	31.0	16530	17959	26817	33032	42142	50749	
		32 4%	35 4%	52 8%	65 1%	83 0%	100 0%	
NORTH DAKOTA	32.0	62146	62146	62966	63960	64433	71005	
		87 5%	87 5%	88 7%	90 1%	90 7%	100 0%	
OHIO	33.0	10721	11734	17264	21732	39507	41833	
		25 6%	26 13	41 3%	51 9%	94 4%	100 0%	
OKLAHOMA	34.0	52490	53027	57775	62107	66151	49813	
		75 6%	76 2%	83 0%	89 2%	95 0%	100 0%	
OREGON	35.0	56356	58054	60303	63420	67579	97926	
		57 51	59 31	61 63	64 81	69 0%	100 0%	
PENNSYLVANIA	36.0	11429	13548	19985	24260	41727	45278	
		25 7%	30 0%	44 1%	53 6%	92 2%	100 0%	
RHODE ISLAND	37.0	0	0	39	68	1204	1206	
		0 0%	0 0%	3 2%	5 4%	100 0%	100 0%	
SOUTH CAROLINA	38.0	12535	13549	19860	23700	28523	31189	
		50 2%	43 43	63 7%	76 0%	91 5%	100 0%	
SOUTH DAKOTA	39.0	52226	52226	52853	53596	54214	77007	
		67 8%	67 8%	68 6%	69 6%	70 4%	100 0%	
TENNESSEE	40.0	20779	21867	28332	33350	39526	42122	
		44 8%	51 93	67 3%	79 2%	92 8%	100 0%	
TEXAS	41.0	212496	216363	229921	242408	263349	268839	
		79 13	80 51	85 51	90 23	98 0%	100 0%	
UTAH	42.0	534252	54870	55806	57543	59627	85181	
		63 7%	64 43	65 53	67 6%	70 0%	100 0%	
VERMONT	43.0	6998	6958	7739	8270	8830	9853	
		70 6%	70 6%	78 6%	83 9%	89 4%	100 0%	
VIRGINIA	44.0	16772	18171	24241	26902	35052	41167	
		40 7%	44 13	54 9%	70 2%	86 2%	100 0%	
WASHINGTON	45.0	30967	31469	33020	38253	44554	47316	
		44 7%	45 43	50 53	55 23	84 3%	100 0%	
WEST VIRGINIA	46.0	12451	13558	14736	19107	21384	24104	
		52 3%	54 23	70 3%	79 3%	86 7%	100 0%	
WISCONSIN	47.0	34721	36052	41488	45866	51994	57022	
		40 9%	43 23	73 13	80 4%	91 2%	100 0%	
WYOMING	48.0	70792	70792	71410	72211	72761	97986	
		72 23	72 23	72 9%	73 7%	74 3%	100 0%	
TOTAL		1830731	1870373	2050431	2199478	2482286	3039964	
		80 2%	81 5%	87 4%	72 4%	81 7%	100 0%	

NOTE: NUMBERS IN THE COLUMNS REPRESENT THE AMOUNT OF LAND THAT IS CONSIDERED TO BE AVAILABLE IF THE GIVEN CRITERION IS APPLIED. WHENEVER A SECTOR CRITERION IS APPLIED, IT IS ASSUMED THAT A UNIFORM DENSITY CRITERION IS ALSO IN EFFECT. CRITERIA WERE APPLIED TO 5 RADII (2, 3, 10, 20, 30) INDIVIDUALLY AND THE RESULTS COMPOSITED.

TABLE F2.22

POPULATION SECTOR ANALYSIS - TOTAL U. S.  
DENSITY = 300 #/SQ. MI. \*\*\* "QUAD" SECTOR (90.0 DEGREES)  
STATE AREAS IN SQUARE MILES AND % OF STATE

TABULATION	> 1/4 POP. IN SECTOR		> 1/3 POP. IN SECTOR		> 1/2 POP. IN SECTOR		UNIFORM DENSITY		NO POP. CRITERIA NO RESTRICTIONS	
ALABAMA	1.0	358860	384857	425158	42503	41933	51907			
ARIZONA	2.6	43244	50180	51748	52911	54937	115343			
ARKANSAS	3.8	40101	45368	43367	45236	46996	52330			
CALIFORNIA	4.0	79042	80678	86974	87732	100671	140364			
COLORADO	9.0	67210	67400	71179	73174	75644	104326			
CONNECTICUT	4.0	164	308	611	1476	5211	3211			
DELAWARE	7.0	1204	1264	1412	1842	2287	2326			
FLORIDA	9.0	26381	29191	33633	37121	44252	59357			
GEORGIA	9.0	36487	36994	44255	48308	52737	61404			
IDAHO	10.0	47139	42433	44361	45287	46036	13950			
ILLINOIS	11.0	39166	38649	43307	47343	55179	56539			
INDIANA	12.0	18991	19696	24678	29722	38020	34712			
IOWA	13.0	48354	48519	51840	53992	54664	54666			
KANSAS	14.0	73309	75743	77692	80027	82072	82266			
KENTUCKY	15.0	25900	25272	31594	35116	37799	40249			
LOUISIANA	16.0	25755	26296	29413	31524	33736	48154			
MAINE	17.0	30224	30224	31893	32848	33717	34074			
MARYLAND	18.0	4014	4234	5317	6504	11011	11159			
MASSACHUSETTS	19.0	1061	1341	2480	3513	8427	8427			
MICHIGAN	20.0	33852	35261	40163	44486	52158	61837			
MINNESOTA	21.0	52746	52930	55514	57659	64998	85914			
MISSISSIPPI	22.0	34089	36189	39990	46857	44043	47883			
MISSOURI	23.0	55661	55951	59101	61933	63417	64934			
MontANA	24.0	99471	99471	100136	100842	101294	148456			
NEBRASKA	25.0	72385	72385	73745	75135	76187	77721			
NEVADA	26.0	66259	66673	69041	80774	90343	110618			
NEW HAMPSHIRE	27.0	9520	9655	4523	7180	8070	9467			
NEW JERSEY	28.0	627	886	1361	2113	8610	8610			
NEW MEXICO	29.0	64621	87545	98324	99262	92026	121744			
NEW YORK	30.0	19474	20461	24045	31486	40289	50219			
NORTH CAROLINA	31.0	22642	24607	32299	37104	42142	50749			
NORTH DAKOTA	32.0	62947	62947	63432	64074	64433	71005			
OHIO	33.0	16434	17447	22890	29018	39507	41833			
OKLAHOMA	34.0	54192	57292	61046	63776	64151	64615			
OREGON	35.0	40489	40643	48930	49166	47579	47928			
PENNSYLVANIA	36.0	18721	20101	26103	30610	41727	45278			
RHODE ISLAND	37.0	19	29	67	299	1204	1204			
SOUTH CAROLINA	38.0	18132	18277	23293	25861	26525	31189			
SOUTH DAKOTA	39.0	52785	52785	53370	53799	54214	77007			
TEXAS	40.0	25244	26084	31884	35679	39526	42122			
UTAH	41.0	229207	231098	240102	250446	263349	268839			
VERMONT	43.0	7421	7450	8000	8482	8830	9853			
VIRGINIA	44.0	21635	22272	27724	31430	35502	41167			
WASHINGTON	45.0	34672	35079	37789	40810	44554	49316			
WEST VIRGINIA	46.0	19459	19459	18210	19918	21384	24106			
WISCONSIN	47.0	40935	41126	45045	48086	51994	57022			
WYOMING	48.0	71800	71400	71979	72356	72761	97986			
TOTAL		2010462	2036470	2182154	2305121	2482286	3039943			
		66.3%	67.0%	71.8%	75.8%	81.7%	100.0%			

NOTE: NUMBERS IN THE COLUMNS REPRESENT THE AMOUNT OF LAND THAT IS CONSIDERED TO BE AVAILABLE IF THE GIVEN CRITERION IS APPLIED. WHENEVER A SECTOR CRITERION IS APPLIED, IT IS ASSUMED THAT A UNIFORM DENSITY CRITERION IS ALSO IN EFFECT. CRITERIA WERE APPLIED TO 5 RADII (2, 5, 10, 20, 30) INDIVIDUALLY AND THE RESULTS COMBINED.

TABLE F2.23

POPULATION SECTOR ANALYSIS - TOTAL U. S.  
DENSITY = 750 A/SQ MI \*\*\* "QUAD" SECTOR (90.0 DEGREES)  
STATE AREAS IN SQUARE MILES AND % OF STATE

TABULATION	> 1/4 POP. IN SECTOR						
	> 1/3 POP. IN SECTOR						
	> 1/2 POP. IN SECTOR						
	UNIFORM DENSITY		NO POP. CRITERION		NO RESTRICTIONS		
ALABAMA	1.0	37936	39320	43271	25976	49833	31907
ARIZONA	2.0	30614	50942	52168	93258	94937	11433
ARKANSAS	3.0	41130	42235	42907	45442	46475	53258
CALIFORNIA	4.0	93620	64852	37561	93545	108671	140364
COLORADO	5.0	70300	70474	71931	73003	73666	104328
CONNECTICUT	6.0	413	408	1671	2246	5211	5211
DELAWARE	7.0	1280	1299	1477	1891	2387	2387
FLORIDA	8.0	31112	32154	39554	39767	44252	59257
GEORGIA	9.0	40086	40364	43878	46926	52737	56634
IDAHO	10.0	43703	43703	44954	45343	46030	63390
ILLINOIS	11.0	40791	41234	49143	48741	53179	54520
INDIANA	12.0	21346	20279	24798	37047	35030	34342
IOWA	13.0	50093	50354	52448	54772	56047	56047
KANSAS	14.0	56490	76785	78494	80327	82072	82246
KENTUCKY	15.0	34030	42277	38308	39447	37799	40248
LOUISIANA	16.0	27846	27474	30176	31864	32734	48154
MARYLAND	17.0	30494	30494	32028	32996	33717	34074
MARYLAND	18.0	4437	4584	5615	7411	11011	11155
MASSACHUSETTS	19.0	1789	2162	2980	3945	6227	6227
MICHIGAN	20.0	3075	2915	3443	4435	100.0%	100.0%
MINNESOTA	21.0	34682	37104	41534	45425	52158	51837
KANSAS	22.0	53529	53722	54038	54431	54988	55914
MISSISSIPPI	23.0	37087	37178	40488	42460	44643	47865
MISSOURI	24.0	54474	57003	60033	62761	63417	64934
NEBRASKA	25.0	97723	97723	100370	100881	101296	148454
NEVADA	26.0	88903	88903	89455	89999	90363	110618
NEW HAMPSHIRE	27.0	9819	9744	4765	7324	8270	9467
NEW JERSEY	28.0	1110	1204	1768	2977	8010	8010
NEW MEXICO	29.0	13.95	15.15	22.01	32.25	100.0%	100.0%
NEW YORK	30.0	31798	72375	7305	7343	74.15	100.0%
NORTH CAROLINA	31.0	34363	34412	33177	37805	42142	50749
NORTH DAKOTA	32.0	43342	43342	43709	44134	44433	71005
OHIO	33.0	19730	19998	25745	31401	39507	41833
OKLAHOMA	34.0	38138	38748	41992	44066	46151	49415
OREGON	35.0	41919	41492	43343	45991	47579	47928
PENNSYLVANIA	36.0	42.05	43.05	44.75	47.05	49.05	100.0%
RHODE ISLAND	37.0	46	50	251	321	1204	1204
SOUTH CAROLINA	38.0	19628	19428	22903	24229	26525	31189
SOUTH DAKOTA	39.0	53162	53119	53309	53837	54214	57007
TENNESSEE	40.0	27309	27425	32494	34313	39524	42122
TEXAS	41.0	234439	235701	245187	233177	263349	266839
UTAH	42.0	34368	37052	39777	38701	39427	85181
VERMONT	43.0	7536	7354	8038	8492	8830	9853
VIRGINIA	44.0	23121	23748	28979	32057	35502	41167
WASHINGTON	45.0	26332	36789	39092	41466	44334	49216
WEST VIRGINIA	46.0	13729	13729	18403	20072	21364	24104
WISCONSIN	47.0	42315	42487	45702	48648	51994	57022
WYOMING	48.0	71784	71784	72044	72394	72761	97986
TOTAL		2080704	2094548	2229812	2334266	2482268	3039964
		68.43	68.92	73.25	76.83	81.73	100.0%

NOTE: NUMBERS IN THE COLUMNS REPRESENT THE AMOUNT OF LAND  
THAT IS CONSIDERED TO BE AVAILABLE IF THE GIVEN CRITERION  
IS APPLIED. WHENEVER A SECTOR CRITERION IS APPLIED, IT IS  
ASSUMED THAT A UNIFORM DENSITY CRITERION IS ALSO IN EFFECT.  
CRITERIA WERE APPLIED TO 3 RADII (2, 5, 10, 20, 30) INDIVIDUALLY  
AND THE RESULTS COMBINED.

TABLE F2.24

POPULATION SECTOR ANALYSIS - TOTAL U.S.  
DENSITY = 1500 /sq mi \*\*\* "QUAD" SECTOR (90° 0 DEGREES)  
STATE AREAS IN SQUARE MILES AND % OF STATE

TABULATION	> 1/4 POP. IN SECTOR		> 1/3 POP. IN SECTOR		> 1/2 POP. IN SECTOR		UNIFORM DENSITY		NO POP. CRITERIA		NO RESTRICTIONS	
ALABAMA	1.0	39391	39391	43927	47140	49653	51907					
ARIZONA	2.0	51820	52129	52911	53500	54937	57113					
ARKANSAS	3.0	42499	42499	44245	45519	46702	52558					
CALIFORNIA	4.0	60259	60125	53768	57851	108871	140344					
COLORADO	5.0	71275	71864	73051	74180	77464	103353					
CONNECTICUT	6.0	1255	1264	1440	2338	3211	3211					
DELAWARE	7.0	1467	1567	1706	1820	2287	2324					
FLORIDA	8.0	34479	34508	34953	40250	4225	57257					
GEORGIA	9.0	41389	41630	45953	49118	52753	58401					
IDAHO	10.0	43945	43946	44718	45578	46330	52950					
ILLINOIS	11.0	42981	43430	44899	45933	55179	56534					
INDIANA	12.0	23780	24115	26004	31218	35020	36342					
IOWA	13.0	51279	51299	53085	54402	54647	54647					
KANSAS	14.0	77846	77904	79198	80442	82073	82266					
KENTUCKY	15.0	27097	27145	32762	35734	37799	40264					
LOUISIANA	16.0	28217	29217	30523	31970	33734	48154					
MAINE	17.0	30561	30581	32057	32938	33717	34074					
MARYLAND	18.0	48554	5172	53154	56260	11011	11155					
MASSACHUSETTS	19.0	24663	2702	3387	4344	5627	5627					
MICHIGAN	20.0	38571	38687	42489	44417	52158	61837					
MINNESOTA	21.0	34214	34580	56725	58777	60988	89914					
MISSISSIPPI	22.0	37703	37703	40800	42328	44043	47883					
MISSOURI	23.0	57907	58209	60730	62860	65417	69934					
MONTANA	24.0	100051	100051	100514	100891	101296	148454					
NEBRASKA	25.0	73909	73909	74768	75386	76187	77721					
NEVADA	26.0	89446	89446	89735	89984	90363	110618					
NEW HAMPSHIRE	27.0	41118	41118	4813	7362	8270	9457					
NEW JERSEY	28.0	1402	1747	2258	3621	8009	8009					
NEW MEXICO	29.0	86404	86404	86473	87427	92028	121744					
NEW YORK	30.0	24907	24963	27678	32293	40289	50219					
NORTH CAROLINA	31.0	27252	27252	33736	37925	42142	50747					
NORTH DAKOTA	32.0	9372	9372	6651	7475	8339	1003					
OHIO	33.0	22861	23150	27676	32385	39307	41833					
OKLAHOMA	34.0	55778	55978	62445	64249	64151	64151					
OREGON	35.0	62758	62744	63951	65736	67979	71720					
PENNSYLVANIA	36.0	23340	23758	26931	33440	41727	45278					
RHODE ISLAND	37.0	270	270	415	408	1204	1204					
SOUTH CAROLINA	38.0	20199	20199	21154	24306	26525	31189					
SOUTH DAKOTA	39.0	53297	53297	53938	53847	54214	77007					
TENNESSEE	40.0	28342	28342	33100	34429	37526	42122					
TEXAS	41.0	241327	242302	249182	254664	263349	268839					
UTAH	42.0	57823	57823	58247	58826	59627	85181					
VERMONT	43.0	7585	7585	8058	8492	8830	9853					
VIRGINIA	44.0	25970	25148	29761	32364	35902	41167					
WASHINGTON	45.0	38108	38108	39980	41862	44554	49316					
WEST VIRGINIA	46.0	16038	16038	18443	20082	21384	24104					
WISCONSIN	47.0	43319	43521	46172	47184	51994	57022					
Wyoming	48.0	71912	71912	72105	72394	72761	77996					
		7345	7345	7345	7345	7425	10003					
TOTAL		7072	7072	7735	8178	10003						

NOTE: NUMBERS IN THE COLUMNS REPRESENT THE AMOUNT OF LAND THAT IS CONSIDERED TO BE AVAILABLE IF THE GIVEN CRITERION IS APPLIED. WHEREVER A SECTOR CRITERION IS APPLIED, IT IS ASSUMED THAT A UNIFORM DENSITY CRITERION IS ALSO IN EFFECT. CRITERIA WERE APPLIED TO 3 RADII (2, 3, 10, 20, 30) INDIVIDUALLY AND THE RESULTS COMBINED.

TABLE F3.1

POPULATION CASE 1 AND  
ENVIRONMENTAL SUITABILITY LEVELS \*\*\*  
STATE AREAS IN SQUARE MILES AND % OF STATE

TABULATION	LOW SUITABILITY			MEDIUM SUITABILITY			HIGH SUITABILITY			DEN. IN LAND RESTRICTED RESTRICTED LAND	
		MEDIUM-LOW	MEDIUM-HIGH		MEDIUM-HIGH	HIGH	DENSITY	RESTRICTED	DEN. IN LAND RESTRICTED RESTRICTED LAND		
ALABAMA	48	5250	9830	15758	8956	10942	184	1911	5190		
	0%	101	175	321	171	211	0%	4			
ARIZONA	2113	3631	15517	27532	1721	4244	181	57704	114047		
	2%	35	143	245	25	25	2%	503			
ARKANSAS	4687	8958	6321	8975	14195	581	181	6177	52211		
	9%	135	125	173	271	11	0%	12%			
CALIFORNIA	40520	18772	9052	10594	4844	27086	321	4821	160364		
	2%	101	61	71	31	19	2%	30%			
COLORADO	29876	13616	10554	12487	3763	529	81	27762	104327		
	2%	135	101	121	41	21	2%	27%			
CONNECTICUT	0	106	29	26	0	504	0	0	5211		
	0%	23	11	11	0%	97	0%	0%			
DELAWARE	0	0	0	0	1216	1071	0	0	2326		
	0%	0%	0%	0%	521	481	0%	2%			
FLORIDA	0	0	0	1889	29008	15556	2045	11010	59356		
	0%	0%	0%	33	491	26	31	19%			
GEORGIA	2220	2625	4890	12873	15739	12971	598	5246	58621		
	4%	41	121	221	275	21	11	6%			
IDAHO	10451	7554	11561	9920	4150	2393	212	37307	83550		
	13%	91	141	121	55	31	0%	45%			
ILLINOIS	27999	1457	3792	11831	19618	13681	97	1264	56534		
	5%	39	71	213	351	281	0%	2%			
INDIANA	0	434	1216	6321	12506	14542	48	1274	36342		
	0%	15	21	171	341	403	0%	4%			
IDAHO	10	9023	4603	25756	10094	6591	0	0	56067		
	0%	161	81	461	181	121	0%	0%			
KANSAS	15546	26354	15208	12275	3614	5915	0	193	82265		
	19%	321	181	161	71	0%	0%	0%			
KENTUCKY	3277	8376	9418	9547	917	10113	521	1949	40266		
	8%	211	231	141	21	251	1%	5%			
LOUISIANA	0	0	347	4475	20226	481	3604	10806	48152		
	0%	0%	15	171	421	145	71	22%			
MAINE	1197	8309	4574	14784	1602	3232	0	357	34077		
	4%	241	133	431	51	101	0%	1%			
MARYLAND	0	212	627	39	3435	6687	24	116	11155		
	0%	21	61	0%	311	601	0%	1%			
MASSACHUSETTS	203	647	0	347	0	7430	0	0	8627		
	2%	71	0%	41	0%	861	0%	0%			
MICHIGAN	0	0	0	11580	22533	1846	251	4429	61836		
	0%	0%	0%	191	361	291	0%	15%			
MINNESOTA	0	232	1390	25630	26180	7537	704	24222	85915		
	0%	0%	21	301	301	91	1%	2%			
MISSISSIPPI	0	2355	4294	18036	13249	6108	145	306	47882		
	0%	51	91	381	281	131	0%	8%			
MISSOURI	11406	12400	13529	10171	9235	8666	68	4449	69934		
	16%	181	193	151	131	121	0%	6%			
MONTANA	13286	27107	29483	23276	6311	1631	241	46918	148455		
	9%	181	201	161	41	111	0%	32%			
NEBRASKA	14157	12236	16019	13365	17196	3213	10	1525	77721		
	18%	161	211	171	221	41	0%	2%			
NEVADA	37365	18844	27570	4719	463	1399	425	19831	110619		
	34%	171	251	41	0%	114	0%	18%			
NEW HAMPSHIRE	13202	3474	656	58	0	2760	0	1197	9467		
	0%	371	71	15	0%	291	0%	13%			
NEW JERSEY	0	68	0	627	7315	0	0	0	8010		
	0%	1%	0%	81	411	0%	0%	0%			
NEW MEXICO	46631	17939	18758	4449	222	2509	1052	30484	121744		
	38%	151	151	41	0%	21	1%	29%			
NEW YORK	2654	4989	7189	5443	753	19261	415	9515	50219		
	9%	101	141	111	17	381	11	7%			
NORTH CAROLINA	2274	485	1911	6630	14778	16164	1124	7499	30769		
	9%	11	41	131	281	321	2%	15%			
NORTH DAKOTA	8241	12207	17476	14147	31194	1166	24	6543	71005		
	12%	171	251	201	161	21	0%	9%			
OHIO	1119	3735	1737	3204	7778	21934	608	1718	41833		
	3%	91	41	81	191	521	11	41%			
OKLAHOMA	7923	11329	19570	12709	7016	7604	290	3175	69416		
	11%	161	281	181	101	111	0%	31%			
OREGON	251	19869	12256	24926	4101	6178	232	30118	97929		
	0%	201	131	251	41	6%	0%	31%			
PENNSYLVANIA	1226	9196	7035	1554	376	22340	174	3377	45278		
	3%	201	161	31	12	49%	0%	7%			
RHODE ISLAND	0	0	0	68	0	1139	0	0	1207		
	0%	0%	0%	61	0%	9%	0%	0%			
SOUTH CAROLINA	1090	5365	11348	965	984	8772	212	2451	31187		
	7%	171	361	31	31	281	1%	8%			
SOUTH DAKOTA	11686	7334	11368	11947	10605	1274	184	22629	77007		
	15%	101	191	161	141	21	0%	29%			
TENNESSEE	4275	9563	7865	9368	125	12130	357	2239	42122		
	10%	231	191	131	0%	291	1%	5%			
TEXAS	53065	41331	26730	80916	29442	28663	347	5143	268827		
	20%	151	101	311	111	111	0%	21%			
UTAH	16521	12941	12420	10084	4815	2847	917	24636	85161		
	19%	151	151	121	61	31	1%	29%			
VERMONT	3621	2538	1013	68	0	1390	125	897	9852		
	3%	261	102	12	0%	141	1%	9%			
VIRGINIA	3532	1959	1776	9942	7545	11049	830	4835	41166		
	9%	51	41	231	181	273	2%	12%			
WASHINGTON	5182	7468	10403	8926	3908	8646	357	24405	69315		
	7%	111	151	131	61	121	1%	35%			
WEST VIRGINIA	3503	8145	4424	48	68	5192	48	2670	24106		
	15%	341	181	0%	221	0%	11%				
WISCONSIN	116	3428	5674	2001	12246	10239	241	4786	57021		
	0%	61	101	351	211	181	0%	8%			
WYOMING	17411	13520	20564	15464	4445	1148	154	25071	47966		
	18%	141	211	161	51	13	0%	26%			
TOTAL	381599	382005	400812	531017	373547	413209	23024	534650			
	13%	121	121	121	121	14%	1%	18%			

\*\*\* POPULATION CASE 1 COMPOSITE

RADIUS 0 - 2 MILES/DENSITY 100 PERSONS PER SQUARE MILE  
RADIUS 2 - 30 MILES/DENSITY 250 PERSONS PER SQUARE MILE

POPULATION CASE 1 IS 1% IN THE AMOUNT OF LAND IT CONSTRAINS

TABLE F3.2

POPULATION CASE 2 AND  
ENVIRONMENTAL SUITABILITY LEVELS \*\*\*  
STATE AREAS IN SQUARE MILES AND % OF STATE

TABULATION	LOW SUITABILITY			MEDIUM SUITABILITY			HIGH SUITABILITY			DENSITY RESTRICTIONS		
				MEDIUM-LOW			MEDIUM-HIGH					
										DEN 1	NAL RE 2	ESTIMATED LAND
ALABAMA	58	4301	10596	18594	10683	2986	32	2026	5182			
ARIZONA	2171	4082	15855	28912	1756	2171	52	58674	11444			
ARKANSAS	4979	7392	7373	4942	15411	1986	45	6215	5774			
CALIFORNIA	44392	18924	9968	11948	6562	16977	184	49951	11114			
COLORADO	30946	13925	11040	13143	3956	2604	154	26117	10412			
CONNECTICUT	301	131	111	131	41	20	0	271	0			
DELAWARE	0	0	0	0	0	0	0	0	0			
FLORIDA	0	0	0	0	0	0	0	0	0			
GEORGIA	2994	3426	916	15247	17418	4854	161	5664	56504			
IDAHO	10702	7748	11918	10239	4516	886	46	37452	6251			
ILLINOIS	3081	1959	4420	13452	23758	8309	10	1351	5654			
INDIANA	62	31	82	242	425	155	0	25				
IDAHO	0	902	1776	8714	18345	5684	10	1312	36343			
KANSAS	0	15	51	242	501	181	0	41				
KANSAS	15758	27194	15778	14359	6610	2774	0	0	5056			
KENTUCKY	192	33	192	173	81	35	0	0	822			
Louisiana	112	281	282	192	32	72	0	61				
MARYLAND	0	0	376	4671	23961	2526	1293	13124	48152			
MAINE	1235	8444	4860	16415	1727	1013	0	357	34074			
MARYLAND	42	251	142	481	51	32	0	15				
MASSACHUSETTS	0	32	81	32	441	411	0	15				
MICHIGAN	791	1457	135	917	0	5327	0	0	862			
MICHIGAN	0	173	21	112	0	621	0	0				
MINNESOTA	0	0	0	241	472	135	0	161				
MISSISSIPPI	0	291	1640	27753	27831	3513	116	24810	85514			
MISSOURI	12186	13500	14832	11086	10665	3744	0	4516	69932			
MISSOURI	172	193	212	162	143	35	0	65				
MISSOURI	13423	27319	29992	23440	6514	408	39	47121	148454			
NEBRASKA	14205	12275	16444	14012	18065	1187	0	1534	77724			
NEVADA	37732	18846	27618	4999	492	476	20	20226	110616			
NEW HAMPSHIRE	341	172	252	53	0	12	0	181				
NEW JERSEY	1457	4186	946	454	0	1226	0	1197	9486			
NEW MEXICO	0	318	29	32	1679	5415	0	135	0			
NEW YORK	47196	18181	18702	4490	304	1129	203	31334	121746			
NORTH CAROLINA	3242	6361	16210	9303	1428	9225	77	9552	50214			
NORTH DAKOTA	2943	1255	3590	10876	17843	5636	241	838	50770			
OHIO	4448	4719	2770	6147	13259	11165	241	2084	41837			
OKLAHOMA	8125	11821	21925	14166	775	2536	68	339	49615			
OREGON	290	20999	13413	29244	5076	2557	20	30320	87921			
PENNSYLVANIA	0	213	142	261	51	32	0	311				
RHODE ISLAND	32	283	231	81	21	261	0	83				
SOUTH CAROLINA	1544	4671	14552	1361	1255	2943	24	2634	31186			
SOUTH DAKOTA	11782	7420	11484	12053	10895	540	48	85755	77057			
TEXAS	53934	42865	27866	91617	33136	13628	77	5414	26822			
UTAH	17833	13192	12516	10113	4825	1148	34	25186	85182			
VERMONT	211	193	151	125	61	11	0	201				
VERMONT	4150	2847	1322	87	0	425	56	945	965			
VIRGINIA	4352	2542	2316	12043	4940	4304	251	3414	41167			
WASHINGTON	7044	8624	11590	9341	4244	4256	77	24685	89314			
WEST VIRGINIA	101	121	173	135	61	61	0	361				
WISCONSIN	161	411	231	15	15	75	0	115				
WYOMING	17727	13616	20709	15556	4642	511	24	25146	87451			
TOTAL	402791	414226	439406	59990	441227	190657	7374	550321				
	131	141	143	202	15%	62	0	181				

\*\*\* POPULATION CASE 2 COMPOSITE

RADIUS 0 - 8 MILES/DENSITY 290 PERSONS PER SQUARE MILE  
RADIUS 2 - 30 MILES/DENSITY 300 PERSONS PER SQUARE MILE

POPULATION CASE 1 IS 8% IN THE AMOUNT OF LAND IT CONSTRAINS

TABLE F3.3

POPULATION CASE 3 AND  
ENVIRONMENTAL SUITABILITY LEVELS \*\*\*  
STATE AREAS IN SQUARE MILES AND % OF STATE

POPULATION	LOW SUITABILITY			MEDIUM-LOW			MEDIUM SUITABILITY			MEDIUM-HIGH			HIGH SUITABILITY			DENSYITY RESTRICTION			LAND USE RESTRICTION		
ALABAMA	56	6,396	10,771	18,627	10,676	2,943	14	2051	5,151	4	4	4	14	2051	5,151	14	2051	5,151	4	4	4
ARIZONA	2171	4,062	15,674	29,268	18,111	1,177	421	5,879	11,403	4	4	4	14	2051	5,151	4	4	4	4	4	4
ARKANSAS	5047	7,421	7,538	9,891	15,117	1,612	46	6,215	8,728	4	4	4	14	2051	5,151	4	4	4	4	4	4
CALIFORNIA	46,201	19,23	10,055	12,273	7,044	1,212	13,97	50,29	16,114	4	4	4	14	2051	5,151	4	4	4	4	4	4
COLORADO	31,295	3,992	11,213	13,192	3,995	1,982	124	28,735	1,437	4	4	4	14	2051	5,151	4	4	4	4	4	4
CONNECTICUT	31	102	454	336	0	3,41	2	2	5,112	4	4	4	14	2051	5,151	4	4	4	4	4	4
DELAWARE	0	0	0	0	0	0	0	0	0	0	0	0	0	0	0	0	0	0	0	0	0
FLORIDA	0	0	0	1,930	3,742	6,900	55	12,555	5,935	4	4	4	14	2051	5,151	4	4	4	4	4	4
GEORGIA	26,15	34,84	9,42	15,536	17,640	2,976	164	57,0	5,601	4	4	4	14	2051	5,151	4	4	4	4	4	4
IDAHO	10,712	7,768	11,927	10,30x	4,574	743	44	3,747	8,254	4	4	4	14	2051	5,151	4	4	4	4	4	4
ILLINOIS	33,49	21,42	4,574	12,693	24,675	6,785	10	12,55	5,935	4	4	4	14	2051	5,151	4	4	4	4	4	4
INDIANA	63	43	82	24	44	32	0	0	0	4	4	4	14	2051	5,151	4	4	4	4	4	4
IDAHO	0	511	1,805	8,907	19,319	4,478	10	13,12	3,634	4	4	4	14	2051	5,151	4	4	4	4	4	4
KANSAS	0	0	0	0	0	0	0	0	0	0	0	0	0	0	0	0	0	0	0	0	0
KENTUCKY	4,261	10,326	11,561	7,874	14,95	2,162	68	24,03	4,247	4	4	4	14	2051	5,151	4	4	4	4	4	4
LOUISIANA	0	0	0	0	0	0	0	0	0	0	0	0	0	0	0	0	0	0	0	0	0
MARYLAND	12,25	8,444	4,883	16,511	17,27	9,17	0	3,57	34,074	4	4	4	14	2051	5,151	4	4	4	4	4	4
MASSACHUSETTS	830	1,554	376	11,110	0	4,77	0	0	0	4	4	4	14	2051	5,151	4	4	4	4	4	4
MICHIGAN	0	0	0	0	0	0	0	0	0	0	0	0	0	0	0	0	0	0	0	0	0
MINNESOTA	0	0	0	0	0	0	0	0	0	0	0	0	0	0	0	0	0	0	0	0	0
MISSISSIPPI	0	0	0	0	0	0	0	0	0	0	0	0	0	0	0	0	0	0	0	0	0
MISSOURI	12,294	13,799	14,957	11,264	10,152	3,011	0	4,516	4,993	4	4	4	14	2051	5,151	4	4	4	4	4	4
MONTANA	13,71	27,339	29,492	23,440	4,552	5,02	29	47,121	14,885	4	4	4	14	2051	5,151	4	4	4	4	4	4
NEBRASKA	14,205	12,275	16,530	14,089	18,150	4,17	0	1524	7,772	4	4	4	14	2051	5,151	4	4	4	4	4	4
NEVADA	37,789	18,646	27618	5,086	4,92	5,31	10	20,246	11,061	4	4	4	14	2051	5,151	4	4	4	4	4	4
NEW HAMPSHIRE	1457	4,217	1,623	940	0	1,033	0	0	0	4	4	4	14	2051	5,151	4	4	4	4	4	4
NEW JERSEY	153	453	312	62	0	3,375	0	0	0	4	4	4	14	2051	5,151	4	4	4	4	4	4
NEW MEXICO	47,227	18,248	18,760	4,738	3,18	9,17	154	31,382	12,174	4	4	4	14	2051	5,151	4	4	4	4	4	4
NEW YORK	3291	6,697	10,504	10,007	15,134	8,251	77	9,813	3,021	4	4	4	14	2051	5,151	4	4	4	4	4	4
NORTH CAROLINA	2,962	12,93	3,696	11,377	18,046	4,767	22	8,405	5,076	4	4	4	14	2051	5,151	4	4	4	4	4	4
NORTH DAKOTA	63	33	73	221	361	95	0	0	0	4	4	4	14	2051	5,151	4	4	4	4	4	4
OHIO	1448	4,806	2,865	7,739	14,157	8,473	212	2113	4,183	4	4	4	14	2051	5,151	4	4	4	4	4	4
OKLAHOMA	8135	11,860	22,031	14,301	7,604	2,220	68	3,987	6,961	4	4	4	14	2051	5,151	4	4	4	4	4	4
OREGON	299	21,027	13,500	25,273	9,423	2,055	19	30,330	9,792	4	4	4	14	2051	5,151	4	4	4	4	4	4
PENNSYLVANIA	2374	13,673	10,876	3,966	13,22	9,446	39	35,13	4,527	4	4	4	14	2051	5,151	4	4	4	4	4	4
RHODE ISLAND	0	0	0	0	0	0	0	0	0	0	0	0	0	0	0	0	0	0	0	0	0
SOUTH CAROLINA	1,572	7,073	14,803	13,70	12,55	2,451	24	2,634	3,118	4	4	4	14	2051	5,151	4	4	4	4	4	4
SOUTH DAKOTA	1,162	7,450	11,503	12,063	10,924	4,54	48	22,745	7,700	4	4	4	14	2051	5,151	4	4	4	4	4	4
TENNESSEE	5415	13,336	9,399	7,266	2,90	3,300	48	2,546	4,212	4	4	4	14	2051	5,151	4	4	4	4	4	4
TEXAS	54,069	4,316	28,053	9,313	34,084	10,924	39	54,52	26,684	4	4	4	14	2051	5,151	4	4	4	4	4	4
UTAH	18,046	13,192	12,526	10,113	4,825	9,26	232	25,232	8,151	4	4	4	14	2051	5,151	4	4	4	4	4	4
VERMONT	4,178	2,856	1,341	87	0	3,67	48	9,75	4,981	4	4	4	14	2051	5,151	4	4	4	4	4	4
VIRGINIA	4,362	2,586	2,355	12,217	10,207	3,686	193	5,472	4,116	4	4	4	14	2051	5,151	4	4	4	4	4	4
WASHINGTON	7,527	8,056	11,725	9,399	4,439	3,406	48	24,714	8,931	4	4	4	14	2051	5,151	4	4	4	4	4	4
WEST VIRGINIA	3,647	10,007	5,722	125	1,83	1,390	10	2,712	2,410	4	4	4	14	2051	5,151	4	4	4	4	4	4
WISCONSIN	116	421	243	13	15	61	0	111	0	4	4	4	14	2051	5,151	4	4	4	4	4	4
Wyoming	17,737	13,625	20,719	15,572	4,661	4,04	24	25,14	8,781	4	4	4	14	2051	5,151	4	4	4	4	4	4
TOTAL	406,711	418,199	44,843	80,4417	45,2510	13,6612	348	55,21%	0	4	4	14	2051	5,151	4	4	4	4	4	4	
	13%	14%	15%	20%	15%	51	0	18													

\*\*\* POPULATION CASE 3 COMPOSITE

RADIUS 0 - 2 MILES/DENSITY 250 PERSONS PER SQUARE MILE

RADIUS 2 - 30 MILES/DENSITY 750 PERSONS PER SQUARE MILE

POPULATION CASE 1 IS 3% IN THE AMOUNT OF LAND IT CONSTRAINS

TABLE F3.4

POPULATION CASE 4 ANY  
ENVIRONMENTAL SUITABILITY LEVELS \*\*\*  
STATE AREAS IN SQUARE MILES AND % OF STATE

TABULATION	LOW SUITABILITY										HIGH SUITABILITY									
	MEDIUM LOW					MEDIUM SUITABILITY					HIGH SUITABILITY					DENSITY RESTRICTIVE				
ALABAMA	58	6514	11030	18175	11300	1756	10	2065	7142											
ARIZONA	2210	4092	15980	29481	1805	1370	116	39240	118144											
ARKANSAS	5105	7546	7604	10065	15739	936	10	6144	30210											
CALIFORNIA	47283	19686	16200	12506	7363	1831	152	50421	160364											
COLORADO	31499	14002	11252	19270	4024	1660	36	28603	104321											
CONNECTICUT	405	1322	940	434	0	2509	0	6	5210											
DELAWARE	0	0	0	0	2007	280	0	39	2346											
FLORIDA	0	0	0	2046	39025	9162	425	12680	59358											
GEORGIA	2644	3667	4930	15990	17930	2577	47	5771	56606											
IDAHO	10779	7797	12014	10354	4642	444	10	37510	87550											
ILLINOIS	3484	2200	4690	14070	25486	9250	10	1251	56541											
INDIANA	0	321	1930	9283	20275	3011	0	1322	36342											
IDAHO	10	9457	5018	28477	11918	1187	0	0	56067											
KANSAS	15807	27493	5874	14716	4730	1226	0	193	8226											
KENTUCKY	4555	10490	11773	9145	1544	1293	29	2441	40270											
LOUISIANA	0	0	376	7025	24993	1341	742	13674	48152											
MAINE	1245	8453	4941	16878	1756	444	0	357	34074											
MARYLAND	0	338	1604	743	5761	3165	0	143	1115											
MASSACHUSETTS	917	1840	540	1439	0	4092	0	0	8627											
MICHIGAN	0	0	0	15931	31237	4970	0	9479	61837											
MINNESOTA	0	261	1776	28518	28477	1939	39	24867	89915											
MISSISSIPPI	0	2615	4873	20429	15072	1052	0	3641	47803											
MISSOURI	12516	13983	19112	11218	10325	2123	0	4516	69933											
ONTARIO	13529	27349	30649	23449	8581	299	0	47160	148455											
NEBRASKA	14214	12264	16627	17137	18316	4068	0	1334	77720											
NEVADA	37847	18846	27638	9105	311	419	10	20246	110618											
NEW HAMPSHIRE	1476	4271	1071	714	0	427	0	1197	9466											
NEW JERSEY	0	311	145	154	2316	4860	0	0	8009											
NEW MEXICO	47324	18976	18664	4777	318	627	48	31486	121744											
NEW YORK	316	867	1120	11001	1689	8193	10	9420	50216											
NORTH CAROLINA	3078	1446	4092	12284	18386	2454	87	8540	50769											
NORTH DAKOTA	67336	12361	17544	14321	11341	3049	6	6972	71006											
OHIO	1467	8008	3117	8347	19015	4552	106	2220	41832											
OKLAHOMA	8145	11976	22330	14494	7720	1486	19	3445	69815											
OREGON	299	21085	13780	25322	9474	1419	0	30349	97926											
PENNSYLVANIA	2548	14904	11890	4336	1913	8774	14	3532	45278											
RHODE ISLAND	0	0	390	405	0	911	0	0	1206											
SOUTH CAROLINA	1520	7344	15170	1486	1293	1983	0	2663	31189											
SOUTH DAKOTA	11870	7488	11532	12072	11001	251	0	22793	77007											
TENNESSEE	6137	13996	9554	7678	357	1891	14	2577	42122											
TEXAS	34204	43367	29778	84445	34856	8203	39	9452	268840											
UTAH	18200	13211	12564	10113	4835	714	174	25379	85100											
VERMONT	4227	2934	1370	87	0	212	10	1013	9853											
VIRGINIA	4467	2692	2432	12574	10425	2972	68	5997	41167											
WASHINGTON	7602	8193	11870	9534	4555	2566	29	24733	69316											
WEST VIRGINIA	4072	10249	6012	139	203	743	0	2721	24107											
WISCONSIN	173	421	251	11	12	35	0	111												
WYOMING	17775	13655	20736	15623	4690	280	14	25204	47985											
TOTAL	810763	423912	450935	616864	465237	114556	3933	354346												

\*\*\* POPULATION CASE 4 COMPOSITE

RADIUS 0 - 2 MILES/DENSITY 500 PERSONS PER SQUARE MILE  
RADIUS 2 - 30 MILES/DENSITY 750 PERSONS PER SQUARE MILE

POPULATION CASE 1 IS 4% IN THE AMOUNT OF LAND IT CONSTRAINS

TABLE F3.5

POPULATION CASE 5 AND  
ENVIRONMENTAL SUITABILITY LEVELS \*\*\*  
STATE AREAS IN SQUARE MILES AND % OF STATE

POPULATION	LOW SUITABILITY			MEDIUM SUITABILITY			HIGH SUITABILITY			DENSITY RESTRICTIONS		
				MEDIUM-HIGH						DEN. * LAND RESTRICTED		
ALABAMA	96	6552	11117	19232	11425	1426	0	2278	5180			
ARIZONA	0%	121	213	375	221	21	0%	41	41			
ARKANSAS	2210	4092	15990	29938	1805	1024	60	59276	118345			
CALIFORNIA	2%	41	141	261	21	11	0%	52	52			
CONNECTICUT	3124	7568	7662	10067	15815	714	1%	5244	5225			
DELAWARE	10%	141	141	191	305	31	0%	121	121			
FLORIDA	49572	19851	10306	12825	7652	8656	5%	50413	162063			
GEORGIA	31756	14030	11426	13278	4072	1061	4%	2812	104325			
IDAHO	30%	131	111	131	41	11	0%	21	21			
ILLINOIS	473	1534	647	801	0	1796	1	5	5211			
INDIANA	0	2%	121	151	101	34	0%	55	55			
KANSAS	0	0	0	0	2062	222	0	26	234			
KENTUCKY	2644	36%	10290	16222	18046	1920	4%	5019	58615			
Louisiana	10790	7807	12034	10374	4861	357	10	37510	82551			
MARYLAND	12%	91	141	121	61	0%	0%	45	45			
MASSACHUSETTS	3442	2248	4913	14185	20383	4034	10	1251	56536			
MISSOURI	19607	27628	15084	14871	7054	820	0	192	82261			
NEBRASKA	5255	10499	11879	8200	1560	926	2%	2441	40285			
NEW HAMPSHIRE	0	0	276	7025	2322	1013	511	1304	48153			
NEW JERSEY	0%	0%	15	151	53	21	1%	29	29			
NEW MEXICO	1245	8433	4841	18994	1796	328	0	357	34074			
NEW YORK	10	9457	5057	28193	12034	917	0	0	56088			
NORTH CAROLINA	2078	14118	15237	11221	10264	1573	0	4516	69933			
OKLAHOMA	12574	14118	15237	11221	10264	1573	0	4516	69933			
PENNSYLVANIA	17556	27347	30649	23478	4991	232	0	47160	148451			
PENNSYLVANIA	87	181	203	161	41	0%	0%	30	30			
RHODE ISLAND	14214	12284	16427	14214	18345	902	0	1534	77720			
TEXAS	37886	18644	27336	5201	311	280	10	20246	110418			
VERMONT	341	172	231	51	321	21	0%	0%	0%			
WISCONSIN	1474	4371	1081	954	0	482	0	0	8426			
WISCONSIN	163	463	111	91	0%	51	0%	132	132			
WISCONSIN	87	590	396	357	3561	3019	0	0	8010			
WISCONSIN	291	153	162	41	441	381	0%	0%	863			
WISCONSIN	75	141	223	221	41	101	0%	203	203			
WISCONSIN	3076	1467	2027	12993	18606	1986	6%	8940	50749			
WISCONSIN	61	31	81	251	271	41	0%	173	173			
WISCONSIN	0%	221	141	261	61	15	0%	311	311			
WISCONSIN	12581	17553	14394	11390	212	0	4572	71005				
OHIO	1486	9018	3146	4935	13701	4632	10%	2220	41834			
OKLAHOMA	8145	11976	22444	14626	7768	1129	14	3443	49615			
OKLAHOMA	121	173	321	211	111	21	0%	51	51			
OREGON	299	21085	13866	85341	9935	1033	0	30349	47729			
PENNSYLVANIA	0%	221	141	261	61	15	0%	311	311			
PENNSYLVANIA	2615	15072	12446	9006	2660	4912	14	3932	45277			
RHODE ISLAND	61	331	261	111	41	111	0%	61	61			
SOUTH CAROLINA	1649	7421	15401	1196	1292	1245	0	285	31188			
SOUTH DAKOTA	11896	7488	11541	12072	11030	183	0	22742	77005			
TENNESSEE	151	103	151	161	141	0%	0%	301	301			
TEXAS	54223	43483	29370	95516	29840	5876	29	5442	266839			
UTAH	18793	13201	12564	10112	4825	521	48	25505	85180			
VERMONT	221	151	193	121	61	15	0%	301	301			
VIRGINIA	4487	2702	2441	12804	11069	1496	48	5416	41167			
WASHINGTON	113	71	61	311	271	51	0%	141	141			
WEST VIRGINIA	121	121	171	141	71	37	0%	361	361			
WISCONSIN	4072	10248	8195	1455	203	521	0	2721	24105			
WISCONSIN	17785	17604	20748	15633	4709	203	14	25206	81987			
WYOMING	181	141	211	161	51	0%	0%	281	281			
TOTAL	614601	426238	455642	626314	476153	83339	1801	555775				
	14%	14%	151	211	161	31	0%	181	181			

\*\*\* POPULATION CASE 5 COMPOSITE

RADIUS 0 - 2 MILES/DENSITY 300 PERSONS PER SQUARE MILE  
RADIUS 2 - 30 MILES/DENSITY 1500 PERSONS PER SQUARE MILE

POPULATION CASE 5 DEN. IN THE AMOUNT OF LAND IT CONSTRATES

TABLE F3.6

ENVIRONMENTAL SUITABILITY AND POPULATION CASES 1 - 5 \*\*\*  
 HIGH SUITABILITY  
 STATE AREAS IN SQUARE MILES AND % OF STATE

TABULATION	AVAILABLE LAND	POPULATION									
		POPULATION CASE 1	POPULATION CASE 2	POPULATION CASE 3	POPULATION CASE 4	POPULATION CASE 5	OTHER CRITERIA	STATE AREA	% STATE AREA	AVAILABLE LAND	% AVAILABLE LAND
ALABAMA	29984	1676	193	427	125	425	3767	2071	51854		
	171	2%	0%	1%	0%	1%	7%	4%			
ARIZONA	1727	24	46	0	0	10	53120	59421	114341		
	2%	0%	0%	0%	0%	0%	4%	5%			
ARKANSAS	14195	1216	106	222	125	328	37801	6263	53251		
	275	2%	0%	0%	0%	1%	5%	1%			
CALIFORNIA	4844	1718	482	318	290	733	100487	51494	160367		
	3%	1%	0%	0%	0%	0%	6%	3%			
COLORADO	3763	192	2%	3%	4%	8%	71507	28682	104320		
	4%	0%	0%	0%	0%	0%	8%	2%			
CONNECTICUT	0	0	0	0	0	0	5211	0	5211		
	0%	0%	0%	0%	0%	0%	100%	0%			
DELMARVA	1211	647	46	97	56	224	0	3%	2327		
	52%	29%	2%	4%	3%	10%	0%	2%			
FLORIDA	29009	7556	859	1602	1168	3928	2133	13105	59254		
	493	12%	1%	3%	2%	7%	4%	2%			
GEORGIA	15739	1679	241	270	116	376	34315	5867	58663		
	275	3%	0%	0%	0%	1%	5%	1%			
IDAHO	4150	367	56	48	19	116	41254	37519	83571		
	9%	0%	0%	0%	0%	0%	4%	4%			
ILLINOIS	19618	6140	917	811	897	3281	25515	1361	56547		
	35%	7%	2%	1%	2%	6%	4%	2%			
INDIANA	12506	5830	975	955	608	1891	12246	1322	36381		
	34%	16%	3%	3%	2%	5%	34%	4%			
IOWA	10094	1790	193	241	116	482	43550	0	36066		
	18%	2%	0%	0%	0%	1%	78%	0%			
KANSAS	8674	926	174	174	97	299	74720	192	8227		
	7%	1%	0%	0%	0%	0%	91%	0%			
KENTUCKY	917	425	154	46	116	318	35821	2470	40295		
	2%	1%	0%	0%	0%	1%	29%	6%			
LOUISIANA	20226	3735	415	618	328	955	7459	14417	48155		
	42%	5%	1%	1%	1%	2%	15%	30%			
MAINE	1802	125	0	29	0	0	31961	357	34074		
	5%	0%	0%	0%	0%	0%	94%	1%			
MARYLAND	3435	1476	540	309	480	1100	3464	145	11154		
	21%	13%	9%	3%	6%	10%	31%	1%			
MASSACHUSETTS	0	0	0	0	0	0	0	0	0	0	0
	0%	0%	0%	0%	0%	0%	100%	0%			
MICHIGAN	25533	42886	946	1370	945	2647	17109	9679	41837		
	36%	10%	2%	2%	2%	5%	26%	16%			
MINNESOTA	26180	1630	347	299	980	897	31334	84926	85913		
	30%	8%	0%	0%	0%	1%	36%	2%			
MISSISSIPPI	13249	133	232	241	125	463	28371	3641	47802		
	2%	3%	0%	1%	0%	1%	39%	8%			
MISSOURI	9225	830	81	183	2%	143	94660	4514	69933		
	12%	1%	0%	0%	0%	0%	7%	6%			
MONTANA	6311	203	39	29	10	58	94647	47160	148457		
	4%	0%	0%	0%	0%	0%	64%	32%			
NEBRASKA	17196	966	87	164	29	106	57736	1534	77720		
	22%	1%	0%	0%	0%	0%	74%	2%			
NEVADA	443	29	0	14	0	0	89851	20255	110617		
	0%	0%	0%	0%	0%	0%	81%	18%			
NEW HAMPSHIRE	0	0	0	0	0	0	8270	1197	9467		
	0%	0%	0%	0%	0%	0%	87%	13%			
NEW JERSEY	427	1052	367	270	1245	1862	2567	0	8010		
	9%	13%	5%	3%	16%	23%	32%	0%			
NEW MEXICO	222	87	10	0	14	0	89870	31536	121744		
	0%	0%	0%	0%	0%	0%	74%	26%			
NEW YORK	793	676	106	154	87	1216	37297	4930	50219		
	1%	13%	0%	0%	0%	2%	74%	20%			
NORTH CAROLINA	14378	3444	203	940	222	531	28003	8627	50768		
	28%	7%	0%	1%	0%	1%	45%	17%			
NORTH DAKOTA	31194	290	39	29	48	145	52699	6572	71006		
	14%	0%	0%	0%	0%	0%	74%	9%			
OHIO	7778	5481	897	859	485	2364	81442	2326	41832		
	19%	13%	2%	2%	2%	6%	91%	6%			
OKLAHOMA	7016	540	25	116	48	116	38267	3464	69616		
	10%	1%	0%	0%	0%	0%	84%	5%			
OREGON	4101	475	347	251	161	753	60891	30349	97926		
	4%	13%	0%	0%	0%	1%	62%	31%			
PENNSYLVANIA	376	704	241	192	145	454	39613	3551	43277		
	1%	2%	0%	0%	0%	1%	87%	8%			
RHODE ISLAND	0	0	0	0	0	0	1206	0	1206		
	0%	0%	0%	0%	0%	0%	100%	0%			
SOUTH CAROLINA	994	270	0	29	0	29	27203	2663	31188		
	0%	0%	0%	0%	0%	0%	87%	9%			
SOUTH DAKOTA	10605	290	29	77	29	48	43136	22792	77007		
	14%	0%	0%	0%	0%	0%	56%	30%			
TEXAS	29442	3676	946	772	784	2413	225096	9491	26884		
	11%	13%	0%	0%	0%	1%	84%	2%			
UTAH	4615	10	0	10	0	0	54793	25553	85181		
	6%	0%	0%	0%	0%	0%	64%	30%			
VERMONT	0	0	0	0	0	0	8830	1023	9852		
	0%	0%	0%	0%	0%	0%	90%	10%			
VIRGINIA	7595	2345	357	328	444	1399	23037	3665	41166		
	18%	6%	1%	1%	1%	3%	56%	14%			
WASHINGTON	3906	366	145	116	48	309	39642	24762	49316		
	6%	1%	0%	0%	0%	0%	57%	36%			
WEST VIRGINIA	66	116	0	14	0	29	21150	2721	24106		
	0%	0%	0%	0%	0%	0%	88%	11%			
WISCONSIN	12246	2421	367	326	357	1033	25213	5029	57023		
	21%	8%	1%	1%	1%	2%	62%	6%			
WYOMING	4449	193	19	29	19	38	87994	25225	97984		
	9%	0%	0%	0%	0%	0%	89%	26%			
TOTAL	373547	47677	11262	12729	10914	31971	187416P	597673			
	12%	2%	0%	0%	0%	1%	65%	18%			

NOTE: "AVAILABLE LAND" IS THAT AVAILABLE UNDER THE MOST CONSTRAINING POPULATION CRITERIA. THE NUMBERS IN THE POPULATION CASE COLUMNS REPRESENT THAT LAND UNIQUELY DEMONSTRATED BY THAT CRITERION.

TABLE F3.7

ENVIRONMENTAL SUITABILITY AND POPULATION CASES 1 - 5 \*\*\*  
 CCC - MEDIUM-HIGH SUITABILITY  
 STATE AREAS IN SQUARE MILES AND % OF STATE

TABULATION	AVAILABLE LAND	POP/CASE 1				POP/CASE 2				POP/CASE 3				OTHER SUITABILITIES	
															% STATELINE LAND
ALABAMA	15758	2837	232	34*	51	427	30176	2071	51496						
	20%	5%	0%	1%	0%	11	5%	4%	6%						
ARIZONA	27902	1399	56	212	37	917	24123	5441	114342						
	24%	1%	0%	0%	0%	11	5%	5%	5%						
ARKANSAS	8775	868	46	174	0	125	36877	621	50246						
	17%	2%	0%	0%	0%	0%	0%	0%	0%						
CALIFORNIA	10596	1322	357	232	318	704	93347	51491	163361						
	7%	1%	0%	0%	0%	0%	0%	0%	0%						
COLORADO	12487	656	48	39	46	71	8201	2867	104371						
	12%	1%	0%	0%	0%	0%	0%	0%	0%						
CONNECTICUT	29	174	135	81	367	917	3477	0	5214						
	13	3%	3%	2%	7%	18	6%	0%	0%						
DELAWARE	0	0	0	0	0	0	0	0	0						
	0%	0%	0%	0%	0%	0%	0%	0%	0%						
FLORIDA	14889	241	0	114	10	917	44127	13107	59338						
	33	0%	0%	0%	0%	0%	0%	0%	0%						
GEORGIA	12873	234	260	854	232	917	35531	567	58601						
	22%	4%	0%	1%	0%	0%	0%	0%	0%						
IDAHO	9720	318	48	48	19	68	35597	37119	83544						
	12%	0%	0%	0%	0%	0%	0%	0%	0%						
ILLINOIS	11831	1621	941	376	116	290	40704	1361	56540						
	21%	3%	0%	0%	0%	0%	0%	0%	0%						
INDIANA	6321	9343	143	376	106	290	25341	1322	36342						
	17%	7%	0%	0%	0%	0%	0%	0%	0%						
IOWA	25755	2055	212	454	116	347	27126	0	56088						
	46%	4%	0%	1%	0%	0%	0%	0%	0%						
KANSAS	13575	954	135	222	154	281	66492	192	82278						
	16%	0%	0%	0%	0%	0%	0%	0%	0%						
KENTUCKY	5997	1959	318	270	175	290	29230	8470	40268						
	14%	5%	0%	0%	0%	0%	0%	0%	0%						
LOUISIANA	6475	396	19	125	0	98	26653	14417	48172						
	13%	1%	0%	0%	0%	0%	0%	0%	0%						
MARYLAND	14784	1621	97	367	116	290	16444	357	34074						
	3%	0%	0%	1%	0%	0%	0%	0%	0%						
MASSACHUSETTS	347	569	193	326	460	1525	4999	0	8627						
	4%	7%	0%	4%	8%	18%	58%	0%	0%						
MICHIGAN	11580	3435	328	608	902	656	35049	9879	61821						
	19%	6%	0%	1%	1%	1%	57%	16%	0%						
MINNESOTA	28450	2104	374	386	183	396	31892	24926	85914						
	30%	2%	0%	0%	0%	0%	0%	0%	0%						
MISSISSIPPI	18036	2036	48	290	77	281	23276	3861	47887						
	38%	4%	0%	1%	0%	0%	0%	0%	0%						
MISSOURI	11171	917	116	147	203	434	33432	4516	64924						
	15%	1%	0%	0%	0%	0%	0%	0%	0%						
ONTARIO	23276	164	0	2%	10	97	77721	47160	14845						
	16%	0%	0%	0%	0%	0%	0%	0%	0%						
NEBRASKA	13365	647	77	48	77	241	61731	1534	77720						
	17%	1%	0%	0%	0%	0%	0%	0%	0%						
NEVADA	4719	280	87	19	97	174	84986	20255	110619						
	4%	0%	0%	0%	0%	0%	0%	0%	0%						
NEW HAMPSHIRE	98	396	87	174	145	376	7035	1197	5468						
	13%	4%	0%	2%	2%	4%	74%	13%	0%						
NEW JERSEY	0	46	19	68	203	318	7334	97	8010						
	0%	1%	0%	1%	0%	0%	0%	0%	0%						
NEW MEXICO	6449	241	48	39	19	97	85316	31536	121745						
	4%	0%	0%	0%	0%	0%	0%	0%	0%						
NEW YORK	3443	3660	704	994	579	2384	26327	9930	30219						
	11%	8%	0%	2%	1%	5%	52%	20%	0%						
NORTH CAROLINA	6630	4244	902	907	309	473	26574	8427	50770						
	13%	8%	0%	2%	1%	2%	56%	17%	0%						
NORTH DAKOTA	14147	174	0	0	39	19	50055	4572	71006						
	20%	0%	0%	0%	0%	0%	0%	0%	0%						
OHIO	3204	2943	1992	608	1177	1793	29186	2326	41833						
	8%	7%	4%	3%	4%	4%	4%	4%	4%						
OKLAHOMA	12709	1457	135	193	145	318	51192	3464	64614						
	18%	2%	0%	0%	0%	0%	0%	0%	0%						
OREGON	24426	318	29	48	19	46	42171	30348	47928						
	25%	0%	0%	0%	0%	0%	0%	0%	0%						
PENNSYLVANIA	1554	1980	425	544	473	1370	35329	3551	45276						
	3%	4%	0%	1%	0%	0%	0%	0%	0%						
ROHDE ISLAND	68	145	114	77	48	294	434	0	1207						
	6%	12%	10%	6%	6%	25%	36%	0%	0%						
SOUTH CAROLINA	965	396	10	116	10	8	26443	2663	31140						
	3%	1%	0%	0%	0%	0%	0%	0%	0%						
SOUTH DAKOTA	11947	104	10	10	0	29	42113	22793	77006						
	16%	0%	0%	0%	0%	0%	0%	0%	0%						
TENNESSEE	5568	1515	203	405	87	318	31430	25%	42124						
	13%	4%	0%	1%	0%	0%	0%	0%	0%						
TEXAS	83916	7701	1486	1341	1071	2634	16519	5471	268838						
	31%	3%	1%	0%	0%	1%	61%	2%	0%						
UTAH	10084	24	0	0	0	0	49514	25553	8518						
	12%	0%	0%	0%	0%	0%	0%	0%	0%						
VERMONT	68	19	0	0	10	19	8714	1023	4852						
	13%	0%	0%	0%	0%	0%	0%	0%	0%						
VIRGINIA	9592	2451	174	357	232	405	22291	5665	41187						
	23%	6%	0%	1%	1%	1%	54%	14%	0%						
WASHINGTON	8926	415	58	135	10	58	34952	24762	67314						
	13%	1%	0%	0%	0%	0%	50%	36%	0%						
WEST VIRGINIA	48	87	0	0	10	0	21240	2721	24104						
	0%	0%	0%	0%	0%	0%	0%	0%	0%						
WISCONSIN	80091	2663	521	482	241	618	27317	5026	57021						
	35%	5%	1%	1%	0%	0%	1%	48%	0%						
WYOMING	15464	87	10	48	10	77	57051	25225	97988						
	16%	0%	0%	0%	0%	0%	50%	26%	0%						
TOTAL	531013	62975	10433	12468	9432	22862	1833105	559672							
	17%	2%	0%	0%	0%	0%	13%	40%	18%						

NOTE: "AVAILABLE LAND" IS THAT AVAILABLE UNDER THE MOST CONSTRAINING POPULATION CRITERIA. THE NUMBERS IN THE POPULATION CASE COLUMNS REPRESENT THAT LAND UNIBLY CONSTRAINED BY THAT CRITERION.

TABLE F3.8

ENVIRONMENTAL SUITABILITY AND POPULATION CASES 1 - 2 \*\*\*  
 CCCCC MEDIUM SUITABILITY DDD  
 STATE AREAS IN SQUARE MILES AND % OF STATE

TABULATION	AVAILABLE LAND										OTHER SUITABILITY	POPULATION AREA		
	POPCASE 1		POPCASE 2		POPCASE 3		POPCASE 4		POPCASE 5					
ALABAMA	8830	1766	125	244	87	270	36445	2175	51428					
	175	3%	0%	1%	0%	1%	74%	4%						
ARIZONA	15517	338	14	106	10	56	36997	5405	114443					
	14%	0%	0%	0%	0%	0%	34%	5%						
ARKANSAS	6321	1052	125	47	58	125	36206	6265	52244					
	12%	2%	0%	0%	0%	0%	74%	12%						
CALIFORNIA	4052	917	87	145	106	232	46324	5142	140561					
	6%	1%	0%	0%	0%	0%	61%	32%						
COLORADO	10634	386	174	39	174	222	64018	2866	304327					
	10%	0%	0%	0%	0%	0%	61%	2%						
CONNECTICUT	24	249	125	87	106	135	4424	0	5215					
	1%	8%	2%	2%	2%	3%	85%	0%						
DELAWARE	0	0	0	0	0	0	226	3%	2026					
	0%	0%	0%	0%	0%	0%	98%	2%						
FLORIDA	0	0	0	0	0	0	46254	13105	59351					
	0%	0%	0%	0%	0%	0%	76%	22%						
GEORGIA	6890	2306	270	443	270	521	42016	5857	5823					
	12%	4%	0%	1%	0%	1%	72%	10%						
IDAHO	11561	357	10	87	19	58	32939	37519	82551					
	14%	0%	0%	0%	0%	0%	41%	45%						
ILLINOIS	3792	627	134	116	125	270	50093	1361	56536					
	7%	13%	0%	0%	0%	0%	89%	2%						
INDIANA	1216	560	29	125	39	19	30302	1322	36342					
	3%	2%	0%	0%	0%	0%	91%	4%						
LOUISIANA	4603	338	19	56	39	68	50942	0	56067					
	8%	11%	0%	0%	0%	0%	91%	0%						
KANSAS	13206	569	10	87	10	77	66112	193	82266					
	18%	1%	0%	0%	0%	0%	80%	0%						
KENTUCKY	9418	2036	106	212	106	222	25698	2470	40266					
	23%	5%	0%	1%	0%	1%	84%	8%						
Louisiana	347	29	0	0	0	0	33260	14417	48153					
	1%	0%	0%	0%	0%	0%	89%	20%						
MAINE	4574	309	0	56	0	39	28738	357	34075					
	13%	11%	0%	0%	0%	0%	84%	1%						
MARYLAND	627	270	48	36	10	68	4930	145	11156					
	6%	2%	0%	1%	0%	1%	89%	1%						
MASSACHUSETTS	0	125	241	164	406	1110	6269	0	8627					
	0%	2%	3%	2%	7%	7%	74%	0%						
MICHIGAN	0	0	0	0	0	0	52158	9479	61827					
	0%	0%	0%	0%	0%	0%	84%	18%						
MINNESOTA	1390	251	47	39	48	134	59010	24926	85915					
	2%	0%	0%	0%	0%	0%	84%	2%						
MISSISSIPPI	4294	425	48	87	39	56	39073	3841	47885					
	9%	11%	0%	0%	0%	0%	82%	8%						
MISSOURI	13329	1893	125	154	125	911	44949	4516	66932					
	19%	24%	0%	1%	0%	1%	71%	8%						
MONTANA	29683	309	0	77	0	29	71190	47160	148456					
	20%	0%	0%	0%	0%	0%	48%	32%						
NEBRASKA	16019	425	106	77	0	145	59415	1534	77221					
	21%	13%	0%	0%	0%	0%	76%	2%						
NEVADA	27570	48	0	19	0	10	62715	20255	110617					
	25%	0%	0%	0%	0%	0%	57%	18%						
NEW HAMPSHIRE	636	290	77	48	10	10	7180	1197	94668					
	7%	3%	1%	1%	0%	0%	76%	13%						
NEW JERSEY	0	29	48	48	251	482	7131	0	8009					
	0%	0%	1%	1%	3%	81	89%	0%						
NEW MEXICO	18258	44	56	106	48	232	71043	31526	121745					
	15%	0%	0%	0%	0%	0%	58%	26%						
NEW YORK	7189	3020	299	511	444	560	26265	4930	50216					
	14%	6%	1%	1%	1%	1%	56%	20%						
NORTH CAROLINA	1911	1679	106	376	116	270	37664	8627	50769					
	4%	31%	0%	1%	0%	1%	74%	17%						
NORTH DAKOTA	17476	58	0	10	10	19	46860	6752	71005					
	25%	0%	0%	0%	0%	0%	66%	9%						
OHIO	1737	1033	116	232	29	386	35975	2326	41834					
	4%	2%	0%	1%	0%	1%	86%	6%						
OKLAHOMA	19570	2355	106	299	164	369	43087	3484	89614					
	28%	3%	0%	0%	0%	0%	62%	5%						
OREGON	12256	1156	87	280	106	154	53528	30249	87926					
	13%	1%	0%	0%	0%	0%	55%	31%						
PENNSYLVANIA	7035	3223	618	973	406	1891	27277	3551	45276					
	16%	7%	1%	2%	1%	4%	60%	8%						
RHODE ISLAND	0	125	135	29	87	58	772	0	1206					
	0%	10%	11%	2%	7%	51	64%	0%						
SOUTH CAROLINA	11348	3204	251	367	232	743	12381	2663	31189					
	36%	10%	1%	1%	1%	2%	40%	9%						
SOUTH DAKOTA	11348	114	19	29	10	39	42634	22923	77008					
	15%	0%	0%	0%	0%	0%	55%	30%						
TENNESSEE	7845	1399	125	154	106	318	29548	25%	42121					
	19%	3%	0%	0%	0%	0%	70%	8%						
TEXAS	26730	1235	87	222	116	270	23466	5491	268839					
	10%	0%	0%	0%	0%	0%	87%	2%						
UTAH	12420	47	10	39	0	19	47044	25553	85182					
	15%	0%	0%	0%	0%	0%	55%	3%						
VERMONT	1013	308	16	29	28	36	7373	1023	9853					
	10%	3%	0%	0%	0%	0%	75%	10%						
VIRGINIA	1776	540	29	77	10	39	33222	5665	41166					
	4%	1%	0%	0%	0%	0%	80%	14%						
WASHINGTON	10403	1187	135	145	135	222	32327	24762	69316					
	15%	2%	0%	0%	0%	0%	47%	3%						
WEST VIRGINIA	4429	1197	47	290	183	328	1486	2721	24101					
	18%	5%	0%	1%	1%	1%	64%	11%						
WISCONSIN	5674	704	10	125	10	87	45374	5026	57022					
	10%	1%	0%	0%	0%	0%	80%	4%						
WYOMING	20564	145	10	19	10	10	52004	25225	67007					
	21%	0%	0%	0%	0%	0%	53%	2%						
TOTAL	400812	38592	4440	7094	4713	31165	2015474	557673						
	13%	1%	0%	0%	0%	0%	86%	18%						

NOTE: "AVAILABLE LAND" IS THAT AVAILABLE UNDER THE MOST CONSTRAINING POPULATION CRITERIA. THE NUMBERS IN THE POPULATION CASE COLUMNS REPRESENT THAT LAND UNIQUELY CONSTRAINED BY THAT CRITERION.

TABLE F3.9

ENVIRONMENTAL SUITABILITY AND POPULATION CASES 1 - 5 \*\*\*  
 C = MEDIUM-Low SUITABILITY  
 STATE AREAS IN SQUARE MILES AND % OF STATE

TAXASATION	AVAILABLE LAND					OTHER SUITABILITY CRITERIA	SUITABILITY INDEX
	POPCASE 1	POPCASE 2	POPCASE 3	POPCASE 4	POPCASE 5		
ALABAMA	5250	1052	97	116	39	318	42962
	101	21	01	01	01	11	821
ARIZONA	3831	251	0	10	0	10	5802
	31	01	01	01	01	44	521
ARKANSAS	8936	434	29	125	14	48	39387
	131	11	01	01	01	74	121
CALIFORNIA	16772	2152	299	463	164	574	8844
	101	11	01	01	01	55	321
COLORADO	13614	304	46	10	46	304	61304
	131	01	01	01	01	59	21
CONNECTICUT	106	569	347	299	212	540	313
	21	111	73	62	41	701	60
DELAWARE	0	0	0	0	0	0	228
	01	01	01	01	01	98	21
FLORIDA	0	0	0	0	0	0	46252
	01	01	01	01	01	781	221
GEORGIA	2621	801	56	183	29	58	42962
	41	11	01	01	01	84	101
IDAHO	7954	212	0	29	10	48	38196
	91	01	01	01	01	461	451
ILLINOIS	1457	902	183	56	48	77	52834
	31	11	01	01	01	93	21
INDIANA	438	46	10	10	29	29	34441
	111	01	01	01	01	951	41
IDAHO	9023	264	0	46	0	19	46252
	168	11	01	01	01	831	01
KANSAS	26354	840	104	173	125	174	54272
	321	11	01	01	01	661	01
KENTUCKY	8376	1911	39	184	10	77	27223
	213	91	01	01	01	681	61
LOUISIANA	0	0	0	0	0	0	3073
	01	01	01	01	01	701	301
MAINE	8309	135	0	10	0	10	25254
	845	01	01	01	01	741	11
MARYLAND	212	104	0	14	0	10	10462
	25	11	01	01	01	961	131
MASSACHUSETTS	447	811	97	87	56	77	4651
	71	91	11	11	11	791	01
MICHIGAN	0	0	0	0	0	0	39198
	01	01	01	01	01	841	161
MINNESOTA	232	14	0	10	0	0	60727
	01	01	01	01	01	711	201
MISSISSIPPI	2355	193	39	29	10	0	41418
	51	01	01	01	01	861	81
MISSOURI	12400	3100	299	183	135	264	91000
	181	21	01	01	01	651	69932
MONTANA	27107	212	19	10	14	14	73909
	181	01	01	01	01	501	321
NEBRASKA	12236	39	0	10	0	10	63863
	161	01	01	01	01	801	21
NEVADA	18644	0	0	0	0	0	71516
	171	01	01	01	01	651	181
NEW HAMPSHIRE	3474	714	29	154	0	47	3862
	371	81	01	21	05	11	401
NEW JERSEY	68	251	87	104	39	116	7344
	12	31	11	11	01	921	01
NEW MEXICO	17939	241	68	48	10	77	71825
	151	01	01	01	01	591	261
NEW YORK	4989	1592	116	270	232	579	32511
	101	31	01	11	01	11	691
NORTH CAROLINA	485	947	39	154	19	145	40530
	11	11	01	01	01	801	171
NORTH DAKOTA	12907	139	10	29	0	29	32023
	171	01	01	01	01	731	71
OHIO	3725	984	87	207	10	77	34412
	91	21	01	01	01	821	61
OKLAHOMA	11320	492	39	114	0	121	34050
	161	11	01	01	01	781	91
OREGON	19849	1129	29	98	0	48	46443
	201	11	01	01	01	471	311
PENNSYLVANIA	9196	3437	840	811	569	1033	25621
	201	81	21	21	11	21	571
RODE ISLAND	0	0	0	0	0	0	1206
	01	01	01	01	01	101	01
SOUTH CAROLINA	3643	1505	203	870	77	378	20725
	171	31	11	11	01	861	91
SOUTH DAKOTA	7330	116	0	39	0	39	46696
	101	01	01	01	01	22793	77007
TENNESSEE	9543	3484	890	940	154	463	25013
	231	81	11	11	01	591	61
TEXAS	41331	1534	231	231	116	482	219363
	151	11	01	01	01	821	21
UTAH	12941	851	0	10	0	19	46407
	191	01	01	01	01	941	301
VERMONT	2536	309	10	77	10	98	36229
	21	31	01	11	01	991	101
VIRGINIA	1954	569	29	734	10	87	32713
	51	11	01	01	01	791	141
WASHINGTON	7480	540	29	125	29	77	34255
	111	11	01	01	01	321	361
WEST VIRGINIA	8145	1756	104	212	24	116	11020
	341	71	01	11	01	461	111
WISCONSIN	3628	174	0	29	0	29	46134
	61	01	01	01	01	841	41
WYOMING	12520	97	14	19	29	14	99056
	141	01	01	01	01	601	261
TOTAL	382005	32221	3981	5715	2328	6812	2049232
	13%	11	01	01	01	671	181

NOTE: "AVAILABLE LAND" IS THAT AVAILABLE UNDER THE MOST CONSTRAINING POPULATION CRITERIA. THE NUMBERS IN THE POPULATION CASE COLUMNS REPRESENT THAT LAND UNIQUELY CONSTRAINED BY THAT CRITERION.

TABLE F3.10

ENVIRONMENTAL SUITABILITY AND POPULATION CASES 1 - 5 AND  
CITY - LOW SUITABILITY: 20%  
STATE AREAS IN SQUARE MILES AND % OF STATE

## TABULATION

	AVAILABLE LAND					OTHER SUITABILITY RELATIVES LANDS				
	POPCASE 1	POPCASE 2	POPCASE 3	POPCASE 4	POPCASE 5					
ALABAMA	46	10	0	0	0	0	48777	2075	51804	
ARIZONA	0%	0%	0%	0%	0%	0%	9%	4%		
ARKANSAS	2113	36	0	39	0	10	52718	39407	114343	
CALIFORNIA	4680	290	48	56	14	87	41781	4243	53256	
CONNECTICUT	9%	13	0%	0%	0%	0%	7%	12%		
COLORADO	29876	1119	249	203	261	286	43527	26440	104327	
DELAWARE	0%	11	0%	0%	0%	0%	4%	2%		
FLORIDA	0%	0%	0%	0%	0%	0%	0%	2%		
GEORGIA	2220	374	14	24	0	10	80083	9847	58604	
IDAHO	10451	231	10	48	19	48	35184	37514	83551	
ILLINOIS	2799	482	48	135	10	114	91570	1361	36541	
INDIANA	0%	13	0%	0%	0%	0%	4%	2%		
IZZIA	107	0	0	0	0	0	35020	1322	36342	
KANSAS	15546	212	19	29	0	14	86247	143	82265	
KENTUCKY	3377	1004	0	174	0	14	33225	2470	40264	
LOUISIANA	0%	21	0%	0%	0%	0%	8%	6%		
MARYLAND	1197	39	0	10	0	0	32472	257	34075	
MASSACHUSETTS	203	869	39	87	10	34	7462	0	8629	
MICHIGAN	0%	71	0%	13	0%	0%	8%	0%		
MINNESOTA	0%	0%	0%	0%	0%	0%	86156	9679	41837	
MISSISSIPPI	0%	0%	0%	0%	0%	0%	60780	2426	85914	
MISSOURI	11406	782	104	222	96	135	82708	4514	69933	
MONTANA	13206	135	48	36	24	29	87709	47160	148456	
NEBRASKA	14157	48	0	10	0	0	61972	1534	77721	
NEVADA	27365	367	38	56	39	47	32380	80255	110619	
NEW HAMPSHIRE	1322	125	0	14	0	0	471	18%		
NEW JERSEY	1413	13	0%	0%	0%	0%	6794	1197	9467	
NEW MEXICO	44831	367	39	97	29	77	42778	3153	121744	
NEW YORK	2654	869	48	125	48	48	36757	9930	50214	
NORTH CAROLINA	2374	869	14	116	0	48	38796	8627	80769	
NORTH DAKOTA	8641	87	0	10	0	0	86095	8572	71005	
OHIO	1119	306	0	14	19	10	36011	2326	41832	
OIDAHOMA	7923	803	16	10	0	0	913	81		
OREGON	113	0%	0%	0%	0%	0%	831	31		
PENNSYLVANIA	1826	1033	116	174	48	145	38967	3551	45280	
RHODE ISLAND	0	0	0	0	0	0	1204	0	1204	
SOUTH CAROLINA	1090	454	29	77	19	10	26846	2643	31188	
SOUTH DAKOTA	11486	106	29	48	29	39	42277	22793	77007	
TENNESSEE	4275	1496	145	222	29	318	33042	2594	42123	
TEXAS	53065	868	175	135	19	47	209029	9491	86886	
UTAH	16521	1312	212	154	193	482	40752	25553	85179	
VERMONT	3921	329	29	48	0	24	4574	1023	9852	
VIRGINIA	3532	820	10	125	0	48	30948	9655	41168	
WASHINGTON	5182	1862	482	299	482	1206	35029	24762	89314	
WEST VIRGINIA	3503	444	0	125	0	48	17264	2721	24105	
WISCONSIN	116	21	0	11	0%	0%	72%	11%		
WYOMING	17611	116	10	36	10	34	54937	25225	87981	
TOTAL	38199	21192	3919	6054	3842	10512	2057177	357672		
	13%	13%	0%	0%	0%	0%	48%	18%		

NOTE: "AVAILABLE LAND" IS THAT AVAILABLE UNDER THE MOST CONSTRAINING POPULATION CRITERIA. THE NUMBERS IN THE POPULATION CASE COLUMNS REPRESENT THAT LOAD UNIMINIMIZED BY THAT CRITERION.

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