



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

JERSEY CENTRAL POWER & LIGHT COMPANY

DOCKET NO. 50-219

OYSTER CREEK NUCLEAR GENERATING STATION, UNIT NO. 1

AMENDMENT TO PROVISIONAL OPERATING LICENSE

Amendment No. 34
License No. DPR-16

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Jersey Central Power & Light Company (the licensee) dated November 15, 1978, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

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cc w/enclosures:

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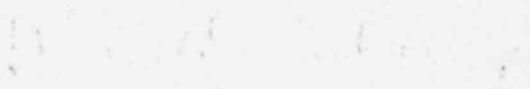
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and paragraph 3.D of Provisional Operating License No. DPR-16 is hereby amended to read as follows:

B. Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 34, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION


Dennis L. Ziemann, Chief
Operating Reactors Branch #2
Division of Operating Reactors

Attachment:
Changes to the Technical
Specifications

Date of Issuance: November 24, 1978

ATTACHMENT TO LICENSE AMENDMENT NO. 34

PROVISIONAL OPERATING LICENSE NO. DPR-16

DOCKET NO. 50-219

Revise Appendix A Technical Specifications by removing the pages identified below and inserting the enclosed pages. The revised pages are identified by amendment number and contain vertical lines indicating the area of change.

REMOVE

3.4-3A
4.3-5
4.3-8
6-26

INSERT

3.4-3A
4.3-5
4.3-8
6-26

F. Fire Protection System

1. The fire protection system shall be operable at all times with fuel in the reactor vessel except as specified in Specification 3.4.F.2.
2. If the fire protection system becomes inoperable during the run mode, the reactor may remain in operation provided both core spray system loops are operable with no inoperable components.

Bases:

This specification assures that adequate emergency core cooling capability is available when the core spray system is required. Based on the loss of coolant analysis for the worst line break, a core spray of at least 3400 gpm is required within 35 seconds to assure effective core cooling*(1). Thus, if one loop becomes inoperable, the operable loop is capable of providing cooling to the core and the reactor may remain in operation for a period of 7 days provided repairs can be completed.

*Core Spray System 2 is required to deliver 3640 gpm.

Change No. 7
Amendment No. 34

TABLE 4.3.1

EXAMINATION SCHEDULE OF REACTOR COOLANT SYSTEM

Component	Sample	Extent	Inspection Process (See Note 1)	Inspection Frequency (See Note 2)
		100% safe end to pipe weld	RT & VT	a
7. Circumferential weld head to head flange	One	10% of weld length including 2 intersects with longitudinal welds	RT & VT	a
8. Longitudinal weld on head from flange weld to cap	One	Entire length	RT & VT	a
9. Integrally welded internal vessel components:				
Core spray piping	One	Entire accessible surfaces and welds	VT	a
Core spray sparger	One	Entire accessible surfaces and welds	VT	e
Shroud support ring	Partial	Any accessible surface	VT	a
Liquid poison sparger	Partial	Any accessible surface and/or welds	VT	a
10. Cladding on head	2 patches	Surface	VT	a

TABLE 4.3.1EXAMINATION SCHEDULE OF REACTOR COOLANT SYSTEMNOTES:

1. UT Ultrasonic examination
RT Radiographic examination (UT acceptable alternate for RT)
VT Examination by viewing
2. a. Inspect same sample twice during first 5 years of operation
b. 100% inspect partial sample during at least two inspections such that 100% of the studs are inspected during the first 5 years of operation
c. Inspect partial sample during at least two inspections such that 10% of the penetrations are inspected during the first 5 years of operation
d. Normal maintenance observations - Examination by viewing, where accessible, during maintenance.
e. Full inspections of the accessible surfaces and welds of both spargers and the repair assembly on core spray sparger no. 2 shall be carried out during each of the next five refueling outages beginning in 1979, subsequent inspections will be conducted at 5 year intervals.
3. The examination schedule of Table 4.3.1, extent of examination, inspection process, and inspection frequency shall be reviewed after the fourth year of operation and a revised specification for subsequent inservice inspection developed.

- (b) If levels of radioactive materials in environmental media as determined by an environmental monitoring program indicate the likelihood of public intakes in excess of 1% of those that could result from continuous exposure to the concentration values listed in Appendix B, Table II, Part 20 estimates of the likely resultant exposure to individuals and to population groups, and assumptions upon which estimates are based shall be provided.
 - (c) If statistically significant variations of offsite environmental concentrations with time are observed, correlation of these results with effluent release shall be provided.
 - (d) Results of required leak tests performed on sealed sources if the tests reveal the presence of 0.005 microcuries or more of removable contamination.
- d. Inoperable fire protection equipment (3.12)
 - e. Core Spray Sparger Inservice Inspection (Table 4.3.1-9)

Prior to startup of each cycle, a special report presenting the results of the inservice inspection of the Core Spray Spargers during each refueling outage shall be submitted to the Commission for review.