BOSTON EDISON
Pilgrim Nuclear Power Station
Rocky Hill Road
Plymouth, Massachusetts 02360

E. S. Kraft, Jr.
Vice President Nuclear Operations
and Station Director

March 15, 1994
BECo Ltr. #94-029

U.S. Nuclear Regulatory Commission
Attention: Document Control Desk
Washington, DC 20555

Docket No. 50-293
License No. DPR-35

FEBRUARY 1994 MONTHLY REPORT

In accordance with PNPS Technical Specification 6.9.A.2, a copy of the Operational Status Summary for Pilgrim Nuclear Power Station is attached for your information and planning. Should you have any questions concerning this report please contact me directly.

for E. [illegible]ling

WJM/dmc/9417

Attachment

cc: Mr. Thomas T. Martin
Regional Administrator, Region I
U.S. Nuclear Regulatory Commission
475 Allendale Road
King of Prussia, PA 19406

Senior Resident Inspector

JE24

AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-293
DATE March 15, 1994
COMPLETED BY: W. Munro
TELEPHONE (508) 830-8474

MONTH February 1994

DAY	AVERAGE DAILY POWER LEVEL (MWe Net)	DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1	666	17	565
2	665	18	479
3	666	19	485
4	665	20	485
5	666	21	485
6	665	22	482
7	665	23	54
8	665	24	0
9	666	25	0
10	665	26	0
11	585	27	0
12	665	28	0
13	665		
14	664		
15	664		
16	665		

This format lists the average daily unit power level in MWe-Net for each day in the reporting month, computed to the nearest whole megawatt.

BOSTON EDISON COMPANY
PILGRIM NUCLEAR POWER STATION
DOCKET NO. 50-293

OPERATIONAL SUMMARY FOR FEBRUARY 1994

The unit started the reporting period at 100 percent core thermal power (CTP). On February 11, 1994, a brief power reduction to approximately 47 percent CTP was made to facilitate a backwash of the main condenser to de-ice the intake bay. Following the backwash reactor power was increased to 100 percent CTP where it was maintained until February 17, 1994. On February 17, 1994, at approximately 1450 hours, while performing the Main Steam Isolation Valve (MSIV) Quarterly Test, the AO-203-1D inboard isolation valve failed the closure time requirement, rendering the valve inoperable and requiring the valve to be closed. The outboard MSIV AO-203-2D was also closed. Reactor power was reduced to approximately 75 percent CTP at 1800 hours and was maintained at that level until 2300 hours on February 22, 1994, when a controlled manual shutdown of the unit was commenced for a planned maintenance outage. On February 23, 1994, at 0553 hours the main turbine generator was manually tripped off line. At 1254 hours the Reactor Mode Switch was placed in the SHUTDOWN position, and by 2003 hours the unit was in cold shutdown condition. The unit remained in cold shutdown for the remainder of the reporting period to facilitate the MSIV outage maintenance activities.

SAFETY RELIEF VALVE CHALLENGES
MONTH OF FEBRUARY 1994

Requirement: NUREG-0737 T.A.P. II.K.3.3

There were no safety relief valve challenges during the reporting period.

An SRV challenge is defined as anytime an SRV has received a signal to operate via reactor pressure, auto signal (ADS) or control switch (manual). Ref. BECo ltr. #81-01 date 01/05/81.

REFUELING INFORMATION

The following refueling information is included in the Monthly Report as requested in an NRC letter to BECo, dated January 18, 1978:

For your convenience, the information supplied has been enumerated so that each number corresponds to equivalent notation utilized in the request.

1. The name of this facility is Pilgrim Nuclear Power Station, Docket Number 50-293.

2. Scheduled date for next refueling shutdown: April 1, 1995

3. Scheduled date for restart following next refueling: May 26, 1995

4. Due to their similarity, requests 4, 5, & 6 are responded to collectively under #6.

5. See #6.

6. The new fuel loaded during the 1993 refueling outage was of the same design as loaded in the previous refueling outage and consisted of 140 assemblies.

7. (a) There are 580 fuel assemblies in the core.

   (b) There are 1629 fuel assemblies in the spent fuel pool.

8. (a) The station is presently licensed to store 2320 spent fuel assemblies. The actual usable spent fuel storage capacity is 2320 fuel assemblies.

   (b) The planned spent fuel storage capacity is 2320 fuel assemblies.

9. With present spent fuel in storage, the spent fuel pool now has the capacity to accommodate an additional 691 fuel assemblies.

MONTH FEBRUARY 1994

PILGRIM NUCLEAR POWER STATION MAJOR SAFETY RELATED MAINTENANCE

SYSTEM	COMPONENT	MALFUNCTION	CAUSE	MAINTENANCE	CORRECTIVE ACTION TO PREVENT RECURRENCE	ASSOCIATED LER
MAIN STEAM (MS) SYSTEM	MAIN STEAM ISOLATION VALVE (MSIV) AO203-10 INBOARD ISOLATION VALVE.	MSIV FAILED QUARTERLY SURVEILLANCE. VALVE WOULD NOT CLOSE (PR94.9070).	ROOT CAUSE UNDER INVESTIGATION.	REPLACED 4-WAY VALVE ASSEMBLY (MODEL #C1540-4H). CHANGED OUT ALL THREE SOLENOIDS USING SOLENOID KIT (#K-C6930-010). REPLACED 2-WAY VAVLE ASSEMBLY (MODEL #C-5577).	TO BE DETERMINED.	N/A
SALT SERVICE WATER (SSW) SYSTEM	"C" SSW PUMP DISCHARGE BLOCK VALVE (LOOP "A" ISOLATION VALVE) MO-3808	VALVE FAILED CLOSURE TIME DURING SURVEILLANCE AND WAS DECLARED INOPERABLE.	ROOT CAUSE UNDER INVESTIGATION. SUSPECT STEM BINDING DUE TO CORROSION.	COMPENSATORY MEASURES TAKEN INCLUDED CLOSING VALVE MO-3813 TO ENSURE SEPARATION OF THE SSW LOCPS.	PERFORM DIAGNOSTIC TESTING, AND PROBABLE REPLACEMENT OF THE MO-3808 VALVE.	N/A
SALT SERVICE WATER (SSW) SYSTEM.	SSW PUMP P-208-E BREAKER B1444.	BROKEN HANDLE ON BREAKER. (PR94.9082).	ROOT CAUSE UNDER INVESTIGATION.	BREAKER REPLACED IN KIND.	TO BE DETERMINED.	N/A

MONTH FEBRUARY 1994

PILGRIM NUCLEAR POWER STATION MAJOR SAFETY RELATED MAINTENANCE

SYSTEM	COMPONENT	MALFUNCTION	CAUSE	MAINTENANCE	CORRECTIVE ACTION TO PREVENT RECURRENCE	ASSOCIATED LER
MAIN STEAM (MS) SYSTEM	MAIN STEAM ISOLATION VALVE (MSIV) AO203-10 INBOARD ISOLATION VALVE.	MSIV FAILED QUARTERLY SURVEILLANCE. VALVE WOULD NOT CLOSE (PR94.9070).	ROOT CAUSE UNDER INVESTIGATION.	REPLACED 4-WAY VALVE ASSEMBLY (MODEL #C5140-200). CHANGED OUT ALL THREE SOLENOIDS USING SOLENOID KIT (#K-C6930-010). REPLACED 2-WAY VAVLE ASSEMBLY (MODEL #C5577).	TO BE DETERMINED.	N/A
SALT SERVICE WATER (SSW) SYSTEM	"C" SSW PUMP DISCHARGE BLOCK VALVE (LOOP "A" ISOLATION VALVE) MO-3808	VALVE FAILED CLOSURE TIME DURING SURVEILLANCE AND WAS DECLARED INOPERABLE.	ROOT CAUSE UNDER INVESTIGATION. SUSPECT STEM BINDING DUE TO CORROSION.	COMPENSATORY MEASURES TAKEN INCLUDED CLOSING VALVE MO-3813 TO ENSURE SEPARATION OF THE SSW LOOPS.	PERFORM DIAGNOSTIC TESTING, AND PROBABLE REPLACEMENT OF THE MO-3808 VALVE.	N/A
SALT SERVICE WATER (SSW) SYSTEM.	SSW PUMP P-208-E BREAKER B1444.	BROKEN HANDLE ON BREAKER. (PR94.9082).	ROOT CAUSE UNDER INVESTIGATION.	BREAKER REPLACED IN KIND.	TO BE DETERMINED.	N/A

OPERATING DATA REPORT

DOCKET NO. 50-293
DATE March 15, 1994
COMPLETED BY: W. Munro
TELEPHONE (508) 830-8474

OPERATING STATUS

NOTES

1. Unit Name: Pilgrim I
2. Reporting Period: February 1994
3. Licensed Thermal Power (MWt): 1998
4. Nameplate Rating (Gross MWe): 678
5. Design Electrical Rating (Net MWe): 655
6. Maximum Dependable Capacity (Gross MWe): 696
7. Maximum Dependable Capacity (Net MWe): 670
8. If Changes Occur in Capacity Ratings (Item Number 3 Through 7) Since Last Report, Give Reasons:

None

9. Power Level To Which Restricted, If Any (Net MWe): None
10. Reasons For Restrictions, If Any: N/A

		This Month	Yr-to-Date	Cumulative
11.	Hours In Reporting Period	672.0	1416.0	186048.0
12.	Number of Hours Reactor Was Critical	590.4	1334.4	114275.9
13.	Reactor Reserve Shutdown Hours	0.0	0.0	0.0
14.	Hours Generator On-Line	533.9	1277.9	110066.9
15.	Unit Reserve Shutdown Hours	0.0	0.0	0.0
16.	Gross Thermal Energy Generated(MWH)	986064.0	2464560.0	193609296.0
17.	Gross Electrical Energy Generated (MWH)	339050.0	850530.0	65496464.0
18.	Net Electrical Energy Generated (MWH)	326292.0	818973.0	62952667.0
19.	Unit Service Factor	79.4	90.2	59.2
20.	Unit Availability Factor	79.4	90.2	59.2
21.	Unit Capacity Factor (Using MDC Net)	72.5	86.3	50.5
22.	Unit Capacity Factor (Using DER Net)	74.1	88.3	51.7
23.	Unit Forced Outage Rate	0.0	0.0	11.7

24. Shutdowns scheduled over next 6 months (type, date, and duration of each) - None
25. If shutdown at end of report period, estimated date of startup - March 4, 1994

UNIT SHUTDOWNS AND POWER REDUCTIONS DOCKET NO: 50-293

DOCKET NO: 50-293
NAME: Pilgrim I
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REPORT MONTH FEBRUARY, 1994

NO.	DATE	TYPE 1	DURATION (HOURS)	REASON 2	METHOD OF SHUTTING DOWN REACTOR 3	LICENSE EVENT REPORT #	SYSTEM CODE 4	COMPONENT CODE 5	CAUSE & CORRECTIVE ACTION TO PREVENT RECURRENCE
1	2-23-94	S	138.1	B	1	N/A	N/A	N/A	Main Steam Isolation Valve Maintenance Outage.

1	2	2	3	4&5
F-FORCED S-SCHED	A-Equip Failure B-Main or Test C-Refueling D-Regulatory Restriction E-Operator Training & License Examination	F-Admin G-Oper Error H-Other	1-Manual 2-Manual Scram 3-Auto Scram 4-Continued 5-Reduced Load 9-Other	Exhibit F & H Instructions for Preparations of Data Entry Sheet Licensee Event Report (LER) File (NUREG-1022)