Leadership in Science and Technology



October 1, 1993

Mr. Jack Strosnider U.S. Nuclear Regulatory Commission **MS 7D4** One White Flint North 11555 Rockville Pike Rockville, MD 20852

Reports for NRC Review of the Industry Steam Generator SUBJECT: Degradation Specific Management (SGDSM) Initiative

Dear Jack:

The purpose of this letter is to provide copies of specific reports requested by Mr. Ken Karwoski of the NRC staff on September 13, 1993, for use in the NRC review of the industry Steam Generator Degradation Specific Management (SGDSM) initiative.

Enclosed are 15 copies of each of the / reports listed below. Please note that the reports listed in items 5, 6 and 7 bel w are proprietary.

- "Steam Generator Progress ...eport, Revision 9," EPRI Report, Research 1. Project S405-3, August 1995.
- "European Plugging Cateria for Defects at Tube Support Plates," EPRI 2. Report, Research Project S404-30, February 1991.
- 3. "Steam Generator PWSCC In Service Leakage Experience in Belgium," LABORELEC Report M00-90-022/R-PH/SD, Revision 1, August 21, 1990.
- "Belgian Approach to Steam Generator Tube Plugging for Primary Water 4. Stress Corrosion Cracking," EPRI Report, NP-6626-SD, March 1990.
- 5. "Steam Generator Tube Integrity, Volume 1: Burst Test Results and Validation of Rupture Criteria (Framatome Data)," EPRI Report, NP-6865-L, Volume 1, June 1991.

This report contains proprietary information. Therefore, a letter requesting the report be withheld from public disclosure and an affidavit describing the basis for withholding this information from the public and signed by E?RI, the owner of the information, is provided in Attachment 1.

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2222 See attached Headquarters: 3412 Hillview Avenue, Post Office Box 10412, Paio Alto, CA 94303, USA • (415) 855-2000 • Telex: 82977 EPRI UF • Fax: (415) 855-2954 Washington Office: 2000 L Street, NW. Suite 805, Washington, DC 20036, USA • (202) 872-9222 • Fax: (202) 296-6040 "Steam Generator Tube Integrity, Volume 2: Leak-Before-Break Analysis for Primary Water Stress Corrosion Cracking Near the Tubesheet (Framatome Data)," EPRI Report, NP-6865-L, Volume 2, June 1991.

This report contains proprietary information. Therefore, a letter requesting the report be withheld from public disclosure and an affidavit describing the basis for withholding this information from the public and signed by EPRI, the owner of the information, is provided in Attachment 2.

 "Steam Generator Tubing Outside Diameter Stress Corrosion Cracking at Tube Support Plates - Database for Alternate Repair Limits, Volume 1: 7/8 Inch Diameter Tubing," EPRI Draft Report, NP-7480-L, Volume 1, Revision 1, September 1993.

This report contains proprietary information. Therefore, a letter requesting the report be withheld from public disclosure and an affidavit describing the basis for withholding this information from the public and signed by EPRI, the owner of the information, is provided in Attachment 3.

One additional report is in preparation and will transmitted to you shortly. This additional report is a companion report to item 6 above and is data for ODSCC for 3/4 inch diameter tubing (NP-7480-L, Volume 2).

Please contact me if you have any questions on this information.

Sincerely,

David A. Steininger

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cc: Ken Karwoski (NRC)