

## UNITED STATES NUCLEAR REGULATORY COMMISSION WASHINGTON, D. C. 20555

N\$0-25%

December 5, 1978

Docket Nos. 50-254 and 50-265

> Mr. Cordell Reed Assistant Vice President Commonwealth Edison Company P. O. Box 767 Chicago, Illinois 60690

Dear Mr. Reed:

Your submittals of November 6, 1976 and February 21, 1978 relating to instrument setpoints for the Quad Cities Units Nos. 1 and 2 are being reviewed by our staff. In order to complete our review, you are requested to provide within 60 days of receipt of this letter, the additional information identified in the enclosure.

Sincerely,

Thomas A. Ippolito, Chief Operating Reactors Branch #3 Division of Operating Reactors

Enclosure: Request for Additional Information

cc w/enclosure: See page 2

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cc Mr. D. R. Stichnoth President Iowa-Illinois Gas and Electric Company 206 East Second Avenue Davenport, Iowa 52801

Mr. John W. Rowe Isham, Lincoln & Beale Counselors at Law One First National Plaza, 42nd Floor Chicago, Illinois 60603

Mr. Nick Kalivianakas Plant Superintendent Quad-Cities Nuclear Power Station 22710 - 206th Avenue - North Cordova, Illinois 61242

Anthony Z. Roisman Natural Resources Defense Council 917 15th Street, N. W. Washington, D. C. 20005

Moline Public Library 504 - 17th Street Moline, Illinois 61265

## QUAD CITIES UNITS NO. 1 AND 2 INSTRUMENT SETPOINT CHANGES

- 1. With regard to the proposed increase in the ECCS-ADS interlock setpoint pressure from 75-100 psig to 100-150 psig, provide the basis
  for assuming that the LPCI pump reaches 150 psig before the time
  assumed in the most recent LOCA-ECCS analysis involving ADS initiation.
  If it cannot be demonstrated that the time for reaching 150 psig is
  earlier than the time assumed for ADS initiation then the increase
  in PCI due to the additional delay must be evaluated.
- With regard to the proposed reduction in the main steamline low pressure isolation setpoint from ≥850 psig to ≥825 psig, provide the results (i.e., △MCPRs) for each full type of the pressure regulator failure transient analysis in order to demonstrate that this currently non-limiting event remains non-limiting even with the proposed change.
- 3. In connection with the proposed increase in the main steamline high flow isolation set from 120% to 140% of rated steam flow, since the largest small break which will not result in MSIV closure (due to high steam flow signal) will increase, it may now be possible that a steam line break smaller than a complete severence of a main steam line will result in a larger total blowdown and hence radiological consequences. Accordingly, provide the basis for the continued use of the FSAR assumption that the most severe break among the spectrum of credible steam line breaks occuring outside the drywall is the complete severence of a main steam line.