

U.S. NUCLEAR REGULATORY COMMISSION
OFFICE OF INSPECTION AND ENFORCEMENT

Region I

Report No. 50-219/78-22

Docket No. 50-219

License No. DPR-16 Priority - Category C

Licensee: Jersey Central Power and Light Company
Madison Avenue at Punch Bowl Road
Morristown, New Jersey 07960

Facility Name: Oyster Creek Nuclear Generating Station

Inspection at: Forked River, New Jersey

Inspection conducted: September 5-7, 1978

Inspectors: *J. Hyman FOR* 9/28/78
W. E. Coleman, Reactor Inspector date signed
G. Kalman 9/27/78
G. Kalman, Reactor Inspector date signed

Approved by: *J. Hyman FOR* 9/28/78
D. L. Cahton, Chief, Nuclear Support date signed
Section No. 1, RO&NS Branch

Inspection Summary:

Inspection on September 5-7, 1978 (Report No. 50-219/78-22)

Areas Inspected: Routine, unannounced inspection by regional based inspectors of refueling preparations, document review, and surveillance of pipe support and restraint systems. The inspection involved 39 inspector-hours onsite by two NRC regional based inspectors.

Results: Of the three areas inspected, no items of noncompliance were found in two areas; two apparent items of noncompliance were found in one area, (Infraction - failure to inspect two snubbers during periodic snubber inspections, T.S. paragraph 4.5.Q.6; Infraction - failure to follow procedure during snubber inspections, T.S. paragraph 6.8.1).

DETAILS

1. Persons Contacted

- *J. Carroll, Station Superintendent
- *K. Fickeissen, Technical Engineer
- M. Laggart, Engineer
- A. Lewis, Document Center Supervisor
- *J. Molnar, Maintenance Engineer
- A. Rone, Technical Supervisor
- **D. Ross, Manager Nuclear Generation
- L. Smialek, Health Physicist
- *J. Sullivan, Assistant Station Superintendent

The inspectors also contacted several other licensee employees during the course of the inspection, including control room operators, engineering staff and office personnel.

* denotes those present at the exit interview.

** participated in exit interview by means of speakerphone.

2. Licensee Action on Previous Inspection Findings

(Closed) Unresolved item (219/77-23-01): Surveillance test omissions. The inspector discussed with a licensee representative the conduct of surveillance testing during the upcoming refueling outage in view of the change to T.S. 1.24 surveillance requirements. The licensee stated that surveillance tests will be performed during the outage on all instrumentation required to be operable by T.S. Section 3.1.

(Closed) Unresolved item (219/77-23-02): RWM program checks. Procedure 1001.5 Revision 1 was reviewed and noted to now contain requirements for RWM program checks.

(Closed) Unresolved item (219/77-23-03): Test case checks for Core Performance Program. Procedure 1001.12 Revision 4 requires test cases be run for the CLAP Program. Procedure 1001.20 Revision 0 requires test cases be run for the GPU-NFAP Program.

(Closed) Unresolved item (219/77-23-04): Power Distribution Measurement Procedure Revisions. The following items are now incorporated in Revision 4 of Procedure 1001.12.

-- A data sheet to record plant parameters.

-- A data sheet to document engineering review of completed GPU-FNAP.

Procedure 1001.33 Revision 0 requires performance of the GPU-FWAP program every 15 full power days to provide data for the CLAP program.

(Closed) Unresolved item (219/78-06-06): Deficiencies observed during inspector's tour. The inspector conducted another tour, similar to the one previously conducted. No deficiencies of the type noted in the previous report were observed. The inspector also discussed corrective action of these items with the cognizant plant engineer. The inspector had no further questions on this item.

(Closed) Followup item (219/77-09-15): TIP Shear Valve Squib Replacement. The licensee has determined the shelf life of the explosive charges in the TIP Shear Valves. A licensee representative stated that these charges will be replaced during the upcoming outage.

(Closed) Infraction (219/78-06-05): Failure to incorporate special installation requirements in the procedure. The inspector reviewed documentation (Memo: Lang to Molnar dated June 16, 1978) which states that the connecting pin clearance on the five snubbers in question was measured to be within the .015 inch limit. This completes the licensee's corrective action on this item.

(Closed) Unresolved item (219/77-09-02): Hydraulic Snubber visual inspection procedure deficiencies. The inspector reviewed Procedure 675.1.001 Revision 0, Inspection of Bergen-Patterson Hydraulic Snubbers. The inspector reviewed this procedure for technical adequacy and for the existence of a definitive acceptance criteria. The inspector had no further questions on this item.

(Closed) Infraction (219/78-06-01): Failure to perform snubber inspection within the required time. The inspector reviewed the attendance records and outline of the Technical Specification training given to maintenance supervision personnel. The snubber surveillance procedure has been included in the 600 series procedure as part of the plant Surveillance Program. This completes the licensee's corrective action on this item.

(Closed) Infraction (219/78-06-06): Failure to perform snubber inspections after failures were identified and failure to report subject failures. The inspector reviewed the visual inspection results dated May 8, 1978, May 31, 1978, and August 10, 1978. These inspections indicate that the licensee is inspecting the accessible safety related snubbers at the frequency required by TS 4.5.Q.1. The licensee has submitted a report dated May 19, 1978. This completes the licensee's corrective action on this item.

(Open) Infraction (219/78-06-02): Failure to follow procedure. The inspector reviewed procedure 675.1.001 Revision 0, Inspection of Bergen-Patterson Hydraulic Snubbers for inclusion of explicit acceptance criteria and required actions. Training records were reviewed for the snubber inspection training given to maintenance personnel. The inspector had no further questions on these corrective actions.

The inspector reviewed the results of the snubber visual inspection completed August 10, 1978. The inspector noted that eleven snubbers were recorded to have accumulator levels outside the procedurally required action range, however, no action was taken to return the accumulator levels to the required position. This is contrary to the requirements of Procedure 775.1.003, which was in effect at the time of the inspection. This item of noncompliance is designated (219/78-22-02).

Review of item (219/78-06-02) will continue after the licensee's response to the additional item of noncompliance.

3. Surveillance of Pipe Support and Restraint Systems

a. Scope

- (1) The inspector reviewed the following procedures for technical adequacy with emphasis on changes made since the last inspection of these procedures.
 - Procedure 675.1.001, Revision 0, Inspection of Bergen Patterson Hydraulic Snubbers
 - Procedure 775.1.001, Revision 0, Rebuilding of Bergen Patterson Hydraulic Snubbers.
 - Procedure 775.1.004, Revision 3, Replacement of Bergen Patterson Hydraulic Snubbers.
 - Procedure 775.1.005, Revision 0, Functional Testing of Bergen Patterson Hydraulic Snubbers.
 - Procedure 775.1.006, Revision 0, Inspection and Testing of Pacific Scientific Snubbers Type PSA-10.
 - Procedure 775.1.008, Revision 0, Inspection and Adjustment of Spring Type Supports.

- (2) The inspector toured accessible areas of the Reactor Building looking at the condition of several hydraulic snubbers, spring hangers, component supports, and attachments.
- (3) The results of the past three hydraulic snubber visual inspections were also reviewed by the inspector.

b. Findings

- (1) The inspector discussed with the cognizant engineer the adequacy of the hydraulic snubber functional test procedure. The licensee's representative stated that the functional testing procedure will be changed when the Bergen Patterson testing machine arrives on site. The new functional testing procedure will be reviewed as part of a previous unresolved item number 219/77-09-04.
- (2) The inspector noted no deficiencies during the Reactor Building tour.
- (3) During review of the completed hydraulic snubber visual inspection dated August 10, 1978, the inspector noted that two snubbers (Nos. 23 and 24 of the Core Spray System Elevation 51) which are listed in TS Table 3.5.1 and are required to be inspected at the frequency established for all safety related snubbers in the "accessible during reactor operation" category were not inspected. The failure to inspect these snubbers is in noncompliance with TS 4.5.Q.2. The licensee representative stated that these snubbers had not been inspected since January 13, 1978 and as such a total of three required inspections had been missed.

The licensee's representative stated that the reason for not inspecting these snubbers was that they were located inside a high radiation area; however, the licensee's Technical Specifications give no relief from the visual inspection requirement based on radiation levels. This noncompliance is designated as 219/78-22-01.

4. Preparation for Refueling

a. Documentation Reviewed

- (i) Procedure No. 205.0, Revision 1, May 15, 1977, Reactor Refueling;

- (2) Procedure No. 205.1, Revision 5, March 31, 1978, Receiving and Processing New Fuel;
- (3) Procedure No. 205.4, Revision 1, June 15, 1977, Core Offloading;
- (4) Procedure No. 205.5, Revision 2, June 15, 1977, Core Reloading;
- (5) Procedure No. 205.7, Revision 6, July 19, 1977, Control Cell Unloading/Reloading;
- (6) Procedure No. 205.11, Revision 0, April 21, 1977, Fuel Assembly Removal/Insertion in Fuel Preparation Machines;
- (7) Procedure No. 205.22, Revision 1, June 27, 1977, Control Rod Blade Removal/Installation;
- (8) Procedure No. 205.41, Revision 0, April 21, 1977, In-Core Fuel Sipping;
- (9) Procedure No. 1001.24, Revision 1, March 8, 1978, Core Verification;
- (10) Procedure No. 535, Revision 0, April 1, 1977, Inadvertent Reactor Criticality;
- (11) Procedure No. 215, Revision 6, May 17, 1977, Examination of Irradiated Exxon Fuel Assemblies; and,
- (12) New Fuel Receipt Documents and Inspection Reports for 150 Fuel Assemblies.

b. Scope

Preparatory activities related to the forthcoming refueling were observed and supporting documentation was reviewed. The schedule and scope of activities during the planned refueling outage were ascertained through discussions with facility personnel. The outage related procedures (denoted in a. above) were reviewed and their content was checked for compliance with Technical Specification requirements.

c. Findings

The inspection of prerefueling activities, including the document review associated with the receipt and inspection of 150 new fuel assemblies, was conducted and no discrepancies were noted.

The inspector noted that differences exist between the Standard Technical Specifications and Oyster Creek's Technical Specifications in the area of manning requirements during fuel handling. The inspector discussed these differences with the licensee's representatives at the exit meeting and stated that this item would receive further review by IE.

5. Exit Interview

The inspectors met with licensee representatives (denoted in Paragraph 1) at the conclusion of the inspection to summarize the inspection scope and findings. The topic of direct SRO supervision during core alterations was discussed. The two items of noncompliance were identified and discussed during the exit interview and during subsequent telephone communications.