U. S. NUCLEAR REGULATORY COMMISSION NRC FORM 366 (7.77) LICENSEE EVENT REPORT CONTROL BLOCK: (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION) N Y J A F 1 2 0 0 - 0 0 0 - 0 0 0 3 4 1 1 1 1 1 0 5 1 0 LICENSEE CODE CON'T REPORT L 6 0 5 0 0 0 3 3 3 7 1 2 1 3 8 2 3 1 2 2 9 8 SOURCE 60 61 DOCKET NUMBER 68 69 EVENT DATE 74 75 REPORT DATE 29 0 1 EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10) During normal operation, while conducting surveillance testing of the 0 2 High Pressure Turbine Steam High Flow Containment Isolation Switches, 03 23DPIS-76 and 77, as required by T.S. 4.2-2 difficulty was experienced 0 4 in initiating the trip (<106" of H20) on DPIS-76 and in venting the low 0 5 pressure side of DPIS-77. Investigation of the difficulty was immedi-0 6 lately initiated and the HPCI system declared inoperable. The event did 0 7 not represent a significant hazard to the public health and safety. COMP SYSTEM CAUSE CAUSE VALVE SUBCODE COMPONENT CODE SUBCODE |S 15 S | F | (11 2 (16) Di 12 Z (13) N S T R Y (14) 0 9 REVISION OCCURRENCE SEQUENTIAL REPORT TYPE NO REPORT NO. CODE EVENT YEAR LER RO 0 1812 03 L REPORT 0 1514 NUMBER 30 28 PRIME COMP. NPRD-4 COMPONENT EFFECT ON PLANT ATTACHMENT SUBMITTED SHUTDOWN METHOD ACTION FUTURE HOURS (22) FORMSUB SUPPLIER MANUFACTURER B | 0 | 8 |0 Y 24 0 0 0 N (23) N (25) Z (21) 0 (26)(18) CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27) Investigation revealed the instrument surveillance procedure that 1 0 governs this test was incomplete in that the procedure did not note the presence of installed snubbers in the low side lines of the differentlial pressure switches. See attachment for additional details. 1 4 80 METHOD OF FACILITY (30) DISCOVERY DESCRIPTION (32) OTHER STATUS % POWER DISCOVERY B (31) 1 0 0 29 NA E (28) 5 80 ACTIVITY CONTENT LOCATION OF RELEASE (36) AMOUNT OF ACTIVITY (35 RELEASED OF RELEASE NA Z 33 Z 34 NA 6 80 PERSONNEL EXPOSURES DESCRIPTION 39 I O I 80 PERSONNEL INJURIES DESCRIPTION (41) NUMBER 0 0 40 NA 80 LOSS OF OR DAMAGE TO FACILITY (43) DESCRIPTION TYPE B301110295 B21229 PDR ADOCK 05000333 PDR ADOCK 05000333 Z (42) NA 80 PUBLICITY NRC USE ONLY DESCRIPTION (45 S N 44 SUED NA 69 68 80 NAME OF PREPARER Hartford Keith 315-342-3840 PHONE ..

POWER AUTHORITY OF THE STATE OF NEW YORK JAMES A. FITZPATRICK NUCLEAR POWER PLANT

DOCKET NO. 50-333

ATTACHMENT TO LER 82-054/03L-0

PAGE 1 of 1

The technicians performing the test were using very small pressure changes to determine the instrument trip point. The snubbers associated with this instrument caused several time constants to expire before the instrument responds. This action caused the technicians to assume the instrument was not responding correctly. The procedure that governs this test was revised to reflect the presence of the snubbers and their effect on the operation of the switch. No further corrective action is required.