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YANKEE ATOMIC ELECTRIC COMPANY

B.3.2.1



20 Turnpike Road Westborough, Massachusetts 01581

WYR 78-103

November 27, 1978

United States Nuclear Regulatory Commission Washington, D. C. 20555

Attention: Office of Nuclear Reactor Regulation

Reference: (a) License No. DPR-3 (Docket No. 50-29) (b) YAEC Letter to USNRC, dated February 28, 1978 (c) USNRC Letter to YAEC, dated September 6, 1978

Dear Sir:

Subject: Systematic Evaluation Program (SEP)

In response to your letter (Reference c), we hereby submit a complete tabulation of the Yankee Rowe environmental qualification of safety-related electrical equipment (SEP Topic 111-12). A list of the appropriate references to this tabulation is also enclosed.

Revisions to this information were made as necessary. In cases where it was necessary to separate combined items because of differing operating times or environmental conditions, the numbering system was preserved by using the original item number and adding a letter (e.g.: 1A, 1B, 1C, etc.).

During the current refueling shutdown, the ECCS system was modified, resulting in the deletion of a number of items from the original list submitted in Reference (b). This original list, and the completed tabulation, have been modified to reflect these and other changes to date. A copy of this updated original list is also attached.

To the test of our knowledge, the tabulation is complete and accurate. Any further changes to equipment, which are to be made during the remainder of the current refueling outage, will be minor in nature. Revised pages reflecting these changes will be forwarded when the information is available.

We trust this information is acceptable to you; however, should you have any questions, please contact Mr. David A. Hansen of this office.

Very truly yours,

YANKEE ATOMIC ELECTRIC COMPANY

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Robert H. Groce Licensing Engineer

RTT/em Enclosure

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Reactor: Yankee Rowe	644 - Miter		Systematic Evaluation Program								
	SEP		Time	Env	ironmen	t	Qual.				
Equipment Type	Reference	Loc.	Needed	Parameter	Spec.	Qual.	Method	Reference	Recarks		
1A Solenoid Valve ASCO/ HV-202-301-3-U-125 HV-202-303-16	1-4/4, 12	0	< 0.2	Temp. Pr.(Psia) RH Chem. Rad. Sub.	90 [°] F Amb Amb No 10 No	356 [°] F Amb Amb > 10 ⁴	V. Data Exper Exper - Eval.	11 - - 45 -			
1B Solenoid Valve ASCO/ 8211B56	1-8/3	0	Long	Temp. Pr.(Psia) RH Chem. Rad. Sub.	90°F Amb. Amb. No 4 10 No	104 [°] F Amb. Amb. - 10 ⁴	V. Data Exper Exper - Eval -	10 - 45 -			
2A Solenoid Valve Atkomatic/ 31820	1-4/5	0	< 0.2	Temp. Pr.(Psia) RH Chem. Rad. Sub.	90 [°] F Amb Amb No 10 No	500° F Amb Amb - $2x10^{6}$ -	V. Data Exper Exper - Eval	63 - - 45 -			
2B Solenoid Valve Atkomatic 30,000 Series	1-7/7	I	Long	Temp. Pr.(Psia) RH Chem. Rad. Sub.	244 [°] F 47 100% No 2x10 ⁶ No	500 ⁰ F 47 100% 	V. Data Test Spec - Spec -	63 65 78 - 78 -			
3 DELETED				Temp. Pr.(Psia) RH Chem. Rad. Sub.							

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Reactor: Yankee Row	e				5	Systematic	Evaluatio	n Program	
	SEP Submittal	1	Time	En	vironmen	nt	Qual.		1
Equipment Type	Reference	Lo.	Needed	Parameter	Spec.	Qual.	Method	Reference	Remarks
4A Motor Operated Valve Limitorque/ SMA-2	1-2/17	I	Long	Temp. Pr.(Psia) RH Chem. Rad. Sub.	244 [°] F 47 100% No 2x10 ⁶ No	340°F 47 100% 	Eval Test Eval - Eval	31,32,40,70 	
4B Motor Operated Valve Limitorque/ SMA-000 SMA-1	1-4/7 1-2/18	0	Long	Temp. Pr.(Psia) RH Chem. Rad. Sub.	90°F Amb Amb No 10 No	340° F Amb Amb - 2x10 ⁸ -	Eval Exper Exper - Eval -	31,32,40,70 - - - - - -	
4C Motor Operated Valve Limitorque/ SA-3	1-5/7	I	Long	Temp. Pr.(Psia) RH Chem. Rad. Sub.	95 [°] F <20 <100% No 6x10 No	320 [°] F 105 100% - 2x10 ⁸	Test Test Test Test -	31,32,37,40,7 37 37 37 37 37 37	Only operates in normal environment inside containment.
5 Motor Operated Valve Limitorque/ SMB-1	1-3/2	0	Long	Temp. Pr.(Psia) RH Chem. Rad. Sub.	90 [°] F Amb Amb No 10 4 No	320°F 105 100% - 2x10 ⁸	Test Test - Test -	31,32,37,40,7 37 37 - 37 - 37 -	0
6 Motor, Pump EMMCO/ Type 1C	1-2/16	0	Long	Temp. Pr.(Psia) RH Chem. Rad. Sub.	90 [°] F. Amb Amb No 10 [°] No	$104^{\circ}F$ Amb Amb 70°	Spec Exper Exper Eval	71 - - -	

Reactor: Yankee Row	e				S	ystematic	Zvaluation	Program	
	SEP		Time	Env	nvironment Ou				
Equipment Type	Reference	Loc.	Needed	Parameter	Spec.	Qual.	Method	Reference	Remarks
7 Motor, Pump GE/ 5K404AK174	1-3/1	0	Long	Temp. Pr.(Psia) RH Chem. Rad. Sub.	90 [°] F Amb Amb No ₄ 10 No	$104^{\circ}F$ Amb Amb > 10 ⁴	Spec Exper Exper Eval	71 	
8 DELETED				Temp. Pr.(Psia) RH Chem. Rad Su'.					
9 Motor, Pump Westinghouse/ 18N6512 18N8216	1-6/1,2	0	Long	Temp. Pr.(Psia) RH Chem. Rad. Sub.	90 [°] F Amb Amb No < 10 No	$104^{\circ}F$ Amb Amb	V. Data Exper Exper - Eval	72,73,74	
10A Motor, Pump Westinghouse/ 19N3044 19N3369 19N3370	1-4/9 1-5/2 1-7/3	0	Long	Temp. Pr.(Psia) RH Chem. Rad. Sub.	90 [°] F Amb Amb No 10 No	$104^{\circ}F$ Amb Amb > 10 ⁴ -	V. Data Exper Exper - Eval	70,72,75,76	
10B Motor, Pump Westinghouse/ 19N3044	1-5/6	0	Long	Temp. Pr.(Psia) RH Chem. Rad. Sub.	100 [°] F Amb Amb No 2x10 ² No	104 [°] F Amb Amb >10 ⁴	V. Data Exper Exper Eval	70, 72	

Reactor: Yankee Row	e				S	ystematic	Evaluation	Program	
	SEP		Time	Env	ironnen	t	Qual.		
Equipment Type	Reference	Loc.	Needed	Parameter	Spec.	Qual.	Method	Reference	Remarks
11 Penetration, Elec. Chicago Bridge and Iron/ No Part #	1-9/8	I	Long	Temp. Pr.(Psia) RH Chem. Rad. Sub.	244 [°] F 47 90% No < 10 ⁶ No	257 [°] F 55 90% -7 10 ⁷	Test Test Eval Hdbk	5,6,33(T.R.) 33 33 - 46 -	
12A Terminal Block Marathon/ TB-6000 Series	1-9/6	I	Long	Temp. Pr.(Psia) RH Chem. Rad. Sub.	95 [°] F <20 90% No <1∂ Nc	< 275 [°] F 47 90% - 2x10 ⁷	Test Test - Test -	8 47 47,65 47 - 47 -	Not required for LOCA environment.
12B Terminal Block Marathon/ TB-6000 Series	1-9/6	0	Long	Temp. Pr.(Psia) RH Chem. Rad. Sub.	$95^{\circ}F$ Amb Amb No $< 10^4$ No	< 275 [°] F 47 90% - 2x10 ⁷	Test Test - Test -	8 47 47 47 47 47	No required for LOCA environment.
12C Terminal Block/ Westinghouse/ 542247	1-9/7	I	Long	Temp. Pr.(Psia) RH Chem. Rad. Sub.	244 [°] F 47 90% No <10 ⁶ No	340 [°] F 47 90% - 2x10 ⁷	Test Test - Test -	8,77 47 47 47 - 47 - 47 -	
13 Cable Mineral Insulated/ General Cable/ No Part #	1-10/3	1/0	Long	Temp. Pr.(Psia) RH Chem. Rad. Sub.	244 ^o F 47 100% No 2x10 ⁶ No	482°F 47 100% - 2x10 ¹⁰	V. Data Test V. Data - V. Data	52 65 52 - 52 -	

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Systematic Evaluation Program

		SEP Submittal		Time	Env	vironnen	t	Qual.		
	Equipment Type	Reference	Loc.	Needed	Parameter	Spec.	Qual.	Method	Reference	Remarks
	14 Cable Polyethylene/PVC Simplex PVC Shielded No Mfr. Listed/ No Part #	1-10/4,8	I	Long	Temp. Pr.(Psia) RH Chem. Rad. Sub.	244 [°] F 47 100% No No No	260°F 47 100% 	Test Test Test Hdbk	41 65 41 - 42 -	
	15 Cable Butyl. Rubber/FVC No Mfr. Listed/ No Part #	1-10/5	I	Long	Temp. Pr.(Psia) RH Chem. Rad. Sub.	244 ⁰ F 47 100% No No No	250°F 47 100% 	Hdbk Test Hdok - Hdbk	42 65 42 - 42 -	
	<pre>16 Cable Silicone Rubber/ Asbestos Rockbestos/ XLP/Neoprene or Hypalon</pre>	1-10/6,7	I	Long	Temp. Pr.(Psia) RH Chem. Rad. Sub.	244 [°] F 47 100% No 2x10 ⁶ No	250°F 47 100% 	Test Test Test Test	39 39 39 - 39 -	
	17A Cable Coaxial/ No Mfr. Listed/ RG/149/U	1-10/9	I	<0.1	Temp. Pr.(Psia) RH Chem. Rad. Sub.	115 [°] F < 20 <100% No 2x10 No	> $185^{\circ}F$ 47 100% - 2x10 ⁷ -	Spec Test Eval - Exper	48 65 - - 68 -	Not required for LOCA conditions.
	17B Cable Coaxial/ No Mfr. Listed/ RG-391/U	1-10/9	I	\$ 9.1	Temp. Pr.(Psia) RH Chem. Rad. Sub.	115 [°] F. < 20 < 100% No No No	230 [°] F 47 100% - 2x10 ⁷	Spec Test Eval Exper	48 65 - - 68 -	

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Reactor: Yankee Ro	we				S	ystematic	Evaluation	Program	
	SEP Submittal		Time	Env	ironmen	t	Qual.		
Equipment Type	Reference	Loc.	Needed	Parameter	Spec.	Qual.	Method	Reference	Remarks
18 Transmitter, Level Bailey/ BR-116X	1-6/5	I	<10	Temp. Pr.(Psia) RH Chem. Rad. Sub.	244 [°] F 47 100% No 10 No	266°F 47 100% 	Eval Test V. Data - Spec -	49 65 14 	
19 Detector, Level Bailey/ BR-117XX Model XNX	1-1/11	I	< 0.1	Temp. Pr.(Psia) RH Chem. Rad. Sub.	244 ⁰ F 47 100% No 10 No	266 [°] F 47 100% 	Eval Test Eval - Spe	12 49 65 - - 50 -	
20 Detector Pressure Bailey/ KIX	1-1/10	I	< 0.2	Temp. Pr.(Psia) RH Chem. Rad. Sub.	244 ⁰ F 47 100% No 10 ⁶ No	266 [°] F 47 100% 	Eval Test Eval - Spec -	13 49 65 - 50 -	
21 Detector, △ P Barton/ 244	1-1/5	I	< 0.1	Temp. Pr.(Psia) RH Chem. Rad. Sub.	950F < 20 < 100% No 8x10 ⁵ No	200°F 47 100% 	V. Data Test V. Data - Eval	18 65 69 - 45 -	Nctrequired for LOCA or MSLB
				Temp, Pr.(Psia) RH Chem. Rad. Sub.			-		

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Reactor: Yankee Rowe	2	stematic	Evaluation	Program					
	SEP		Time	Env	ironmen		Qual.		
Equipment Type	Reference	Loc.	Needed	Parameter	Spec.	Qual.	Method	Reference	Remarks
22 DELETED				Temp. Pr.(Psia) RH Chem. Rad. Sub.					
23 Transmitter, Level Fischer-Porter/ 1302495J-BC-BBB-NS	1-1/12	I	2	Temp. Pr.(Psia) RH Chem. Rad. Sub.	244 [°] F 47 100% No 2x10 ⁶ No	302 [°] F 70 100% -7 10 ⁷	Test Test Test Test	34 34 34 - 35 -	
24 Transmitter, Level GE/ 555	1-3/3	0	Lóng	Temp, Pr.(Psia) RH Chem. Rad. Sub.	90 [°] F Amb Amb No ₄ 10 No	200°F 1500 100% >104 -	Hdbk Hdbk Spec - Eval	15 15 51 - 45 -	
25A Transmitter, Pressure Rosemount/ 1151-GP	1-10/10	0	Long	Temp. Pr.(Psia) RH Chem. Rad. Sub.	90 [°] F Amb Amb No 10 No	200° F Amb Amb > 10^4	V. Data Exper Exper - Eval -	25 - - 45 -	
25B Transmitter, Pressure Rosemount/ 1152 GP	1-5/8	I	Long	Temp. Pr.(Psia) RH Chem. Rad. Sub.	244°F 47 100% No <10 No	316 [°] F 85 100% - 5x10 ⁶	Test Test Test Test	24 24 24 - 24 -	

Reactor: Yankee Ro	we				S	ystematic	Evaluation	Program	
	SEP		Time	Env	ironmen	t	0.121		
Equipment Type	Reference	Loc.	Needed	Parameter	Spec.	Qual.	Method	Reference	Remarks
26 Switch Static-O-Ring/ 7821-100	1-3/11	I	< 0.2	Temp. Pr.(Psia) RH Chem. Rad. Sub.	244 [°] F 47 100% No 2x10 ² No	$280^{\circ}F$ 51 100% - 4x10 ²	Test Test - Test -	66, 67 66, 67 66, 67 	
27 Switch GEMS/ LS800	1-8/2	0	Long	Temp. Pr.(Psia) RH Chem. Rad. Sub.	90 ⁰ F Amb Amb No 10 No	$180^{\circ}F$ 150 100% - >10 ⁴	V. Data V. Data V. Data - Eval -	16 16 16 - 45 -	
28 Switch Magnetrol/ 0-751-X1-SP-SiM3	1-3/6	0	< 0.1	Temp. Pr.(Psia) RH Chem. Rad. Sub.	90 ⁰ F Amb Amb No 10 No	$100^{\circ}F$ 515 Amb	V. Data V. Data Exper Eval	21 21 - 45 -	
29 Switch Static-O-Ring 12N-S4-04-X2 12NNL4-C	1-3/9 1-5/1	0	< 0.1	Temp. Pr.(Psia) RH Chem. Rad. Sub.	90 [°] F Amb Amb ^N ··4 10 [°] No	180° F Amb Amb > 10^4 -	V. Data Exper Exper - Eval -	3 - - 45 -	
30A Battery C&D Battery/ KU-15	1-8/1	0	Long	Temp. Pr.(Psia) RH Chem. Rad. Sub.	90 [°] F Amb Amb No 10 [°] No	$110^{\circ} F$ Amb Amb	V. Data Exper Exper - Eval	53 - - 45 -	

Reactor: Yankee Rowe					Sy	stematic	Evaluation	Program	
	SEP		Time	Env	ironment		Qual.	Server and	
Equipment Type	Reference	Loc.	Needed	Parameter	Spec.	Qual.	Method	Reference	Remarks
30B Battery Exide/ FOP-15 FOP-19	1-8/5,6	0	Long	Temp. Pr.(Psia) RH Chem. Rad. Sub.	100 [°] F Amb Amb No 1 No	$110^{\circ} F$ Amb Amb -4 -10^{4}	V. Data Exper Exper - Eval -	57 - - 45 -	
31A Battery Charger C&D Battery/ ARR130H	1-8/10	0	Long	Temp. Pr.(Psia) RH Chem. Rad. Sub.	90 ⁰ F Amb Amb No 10 No	$122^{\circ}F$ Amb Amb -4 -	V. Data Exper Exper - Eval -	54 - - 45 -	
31B Battery Charger EPCO/ 70-D	1-8/8,9	0	Long	Temp. Pr.(Psia) RH Chem. Rad. Sub.	100°F Amb Amb No 1 No	$100^{\circ} F$ Amb Amb - > 10^{4} -	Exper Exper Exper - Eval -	- - 45 -	
32 Diesel Generator Delco/ E-4859VB	1-8/1	. 0	Long	Temp. Pr.(Psia) RH Chem. Rad. Sub.	100 [°] F Amb 90% No 10 No	104 [°] F Amb 90% > 10 ⁴	Eval Exper Exper - Eval -	7 - - 45 -	
33 Indicator Analogic Measurometer/ PI2445	1-10/12	0	Long	Temp. Pr.(Psia) RH Chem. Rad. Sub.	90 [°] F Amb Amb No 0.1 No	$ \begin{array}{c} 140^{\circ} F \\ Amb \\ 95\% \\ - 4 \\ 710^{4} \\ - \end{array} $	V. Data Exper V. Data Eval	9 - 9 - -	

Reactor: Yankee Rowe Systematic Evaluation Program									
	SEP		Time	Env	ironment	t	Qual.		
Equipment Type	Reference	Loc.	Needed	Parameter	Spec.	Qual.	Method	Reference	Remarks
34 Indicator GE/ 180	1-3/5	0	Long	Temp. Pr.(Psia) RH Chem. Rad. Sub.	90 [°] F Amb Amb No 0.1 No	150°F Amb Amb - > 10 ⁴ -	V. Data V. Data Exper - Eval -	15 15 - - -	
35 Indicator Westinghouse/ KX24	1-1/2	0	Long	Temp. Pr.(Psia) RH Chem. Rad. Sub.	90 ⁰ F Amb Amb No 0.1 No	120°F Amb Amb > 10 ⁴	V. Data Exper Exper - Eval -	43,44 28 - - - -	
36 Motor, Fan Westinghouse/ 72Y51238	1-7/6	I	Long	Temp. Pr.(Psia) RH Chem. Rad. Sub.	244 [°] F 47 100% No 2x10 ⁶ No	275 ⁰ F 65 100% - 10 ⁸ -	Test Test - Test -	1,2,30	
37 Switch Electro Switch/ 28204A-2	1-1/15	0	<0.1	Temp. Pr.(Psia) RH Chem. Rad. Sub.	90 [°] F Amb Amb No 0.1 No	$140^{\circ} F$ Amb 100% > 10^{4}	Test Exper Test - Eval -	64 - 64 - -	
38 Switch GE/ CR2940U%200R CR244QU207 SBM	1-4/6,15 1-9/9	0	<0.1	Temp. Pr.(Psia) RH Chem. Rad. Sub.	90 ⁰ F Amb Amb No 0.1 No	90° F Amb Amb - $>10^{4}$	Exper Exper Exper - Eval -	-	

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	SEP Submittal		Tine	Env	vironmen	t	Qual.		
Equipment Type	Reference	Loc.	Needed	Parameter	Spec.	Qual.	Method	Reference	Remarks
39A Switch Westinghouse/ OT OT211 WL	1-2/8 1-3/7 1-4/1 1-10/1	0	∠ 0.1	Temp. Pr.(Psia) RH Chem. Rad. Sub.	90°F Amb Amb No 0.1 No	$90^{\circ} F$ Amb Amb $> 10^4$	Exper Exper Exper - Eval -	-	
39B Switch Westinghouse / 803-A448601	1-4/13	0	ζ0.1	Temp. Pr.(Psia) RH Chem. Rad. Sub.	90 ⁰ F Amb Amb No <i>i</i> No	$90^{\circ}F$ Amb Amb - $>10^{4}$	Exper Exper Exper Eval	-	
40A Coolant Control & Instrumentation Westinghouse/ Type FN	1-1/1 1-2/13,14	0	Long	Temp. Pr.(Psia) RH Chem. Rad. Sub.	110 [°] F Amb Amb No 0.1 No	158°F 30 100% >10 -4	Spec Spec Spec Eval	28,43,44 50 50 50 -	
40B Coolant Control & Instrumentation Westinghouse/ Type FN	1-1/3,4 1-2/3,4,5, 11,12	0	< 0.1	Temp. Pr.(Psia) RH Chem. Rad. Sub.	110 ⁰ F Amb Amb No 0.1 No	158°F 30 100% > 104 -	Spec Spec Spec - Eval -	28,43,44 50 50 50	
41 Power Supply GE / 570-04	1-3/4	0	Long	Temp. Pr.(Psia) RH Chem. Rad. Sub.	90 [°] F Amb Amb No 0.1 No	$120^{\circ}F$ Amb 100% - $> 10^{4}$ -	V. Data Exper V. Data - Eval -	15 	

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	SEP		Time	Env	ironment		Qual.		
Equipment Type	Reference	Loc.	Needed	Parameter	Spec.	Qual.	Method	Reference	Remarks
42 Power Supply Lambda/ LM262 LCS	1-1/13 1-5/9 1-10/11	0	Long	Temp. Pr.(Psiz) RH Chem. Rad. Sub.	90 ⁰ F Amb Amb No 0.1 No	160°F Amb 95% 	Spec Spec Spec - Eval -	19 51,55 51,55 51,55 - - -	
43A Relay Westinghouse/ SG,HFA,WL 1875-524 No Part No.	1-3/10 1-4/2 1-9/10,11,15 1-10/2	0	<0.1	Temp. Pr.(Psia) RH Chem. Rad. Sub.	100°F Amb Amb No 0.1 No	100°F Amb Amb > 10 ⁴ -	Exper Exper Exper Eval	-	
43B Relay Westinghouse/ 1876-072	1-1/7	0	< 0.1	Temp. Pr.(Psia) RH Chem. Rad. Sub.	100 ⁰ F Amb Amb No 1 No	$100^{\circ} F$ Amb Amb - >10 ⁴ -	Exper Exper Exper - Eval -		
44A Relay Couch Ordnance/ 4C37B	1-1/8	0	< 0.1	Temp. Pr.(Psia) RH Chem. Rad. Sub.	100°F Amb Amb No 1 No	$125^{\circ}F$ Amb Amb -4 -10^{4}	V. Data Exper Exper - Eval -	58 - - - -	
44B Relay Agastat/ 2422-PA	1-1/9	0	<0.1	Temp. Pr.(Psia) RH Chem. Rad. Sub.	100°F Amb Amb No 0.1 No	$165^{\circ}F$ Amb Amb - > 10^4 -	V. Data Exper Exper - Eval -	56 - - - -	

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Reactor: Yankee Row	e				S	ystematic	Evaluation	Program	
	SEP Submittel	1	Time	Env	ironnen	t	Oual.		
Equipment Type	Reference	Loc.	Needed	Parameter	Spec.	Qual.	Method	Reference	Recarks
44C Relay Agastat/ ND 22-0	1-2/6	0	< 0.1	Temp. Pr.(Psia) RH Chem. Rad. Sub.	90 ⁰ F Amb Amb No 0.1 No	165 ⁰ F Amb Amb 	V. Data Exper Exper - Eval -	56 - - - -	
44D Relay Agastat/ 7012FD 2412PD	1-4/3 1-9/16	0	< 0.2	Temp. Pr.(Psia) RH Chem. Rad. Sub.	90°F Amb Amb No 0.1 No	165 [°] F Amb Amb >10 ⁴ -	V. Data Exper Exper - Eval -	56 - - - -	
45 Relay Allen-Bradley/ 200E441Z1	1-3/8	0	< 0.2	Temp. Pr.(Psin) RH Chem. Rad. Sub.	90 ⁰ F Amb Amb No 0.1 No	$90^{\circ}F$ Amb Amb - $>10^{4}$ -	Exper Exper Exper - Eval		
46A Relay GE/ HGA11J52	1-4/14	0	< 0.2	Temp. Pr.(Psia) RH Chem. Rad. Sub.	90 ⁰ F Amb Amb No 0.1 No	90° F Amb Amb - $>10^{4}$	Exper Exper Exper - Eval		
46B Relay GE/ HFA51A HFA54E HGA11J	1-9/12,13, 14,17	0	< 0.1	Temp. Pr.(Psia) RH Chem. Rad. Sub.	90 [°] F Amb Amb No 0.1 No	90°F Amb Amb - >10 ⁴ -	Exper Exper Exper Eval		

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Reactor: Yankee Rowe

Systematic Evaluation Program

	SEP Submittal		Time . Needed	Environment			Qual.		
Equipment Type	Reference	Loc.		Parameter	Spec.	Qual.	Method	Reference	Remarks
47 Relay, Solid State GEMS/ SI-28196	1-8/4	0	< 0.1	Temp. Pr.(Psia) RH Chem. Rad. Sub.	90°F Amb Amb No 10 ⁴ No	120°F 150 100% ~ 10 ⁴	V. Data V. Data V. Data - Eval -	17 17 17 45 -	
48 Bistable Rochester/ ET-215R	1-1/14	0	ζ0.1	Temp. Pr.(Psia) RH Chem. Rad. Sub.	110°F Amb Amb No 0.1 No	150°F Amb Amb > 10 ⁴	V. Data Exper Exper Eval	22,23	
49 Alarm Sigma/ 9222-20ED-2-VB11M	1-1/16	0	< 0.1	Temp. Pr.(Psia) RH Chem. Rad. Sub.	110 ⁰ Amb Amb No 0.1 No	130° Amb Amb > 10 ⁴	V. Data Exper Exper - Eval -	26	
50 Alarm Acromag/ 371-S-8	1-2/10	0	< 0.1	Temp. Pr.(Psia) RH Chem. Rad. Sub.	110 ⁰ F Amb Amb No 0.1 No	122°F Amb Amb 	V. Data Exper Exper Exper	59 - - - -	
51 Ion Chamber Westinghouse/ WL6377 WL6937A	1-2/1,2	I	<0.1	Temp. Pr.(Psia) RH Chem. Rad. Sub.	110°F < 20 < 100% No 4x10 ⁷ No	176 [°] F > 20 100% - 2x10 ¹¹	V. Data Test V. Data - Spec -	27 65 27 50,68	

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	SEP		Time	Env	ironmen	t	Qual.		
Equipment Type	Reference	Loc.	Needed	Parameter	Spec.	Qual.	Method	Reference	Remarks
52 Thermal Converter Sangamo/ CW-10	1-2/9	0	< 0.1	Temp. Pr.(Psia) RH Chem. Rad. Sub.	110°F Amb Amb No 0.1 No	110°F Amb Amb - > 10 ⁴ -	Test Exper Exper - Eval -		
53 Recorder Leeds & Northrup/ Speedomax M	1-5/10	0	Long	Temp. Pr.(Psia) RH Chem. Rad. Sub.	90 [°] F Amb Amb No 0.1 No	$122^{\circ}F$ Amb Amb	V. Data Exper Exper - Eval -	20	
54 Radiation Monitor Nuclear Research/ TA900	1-5/11	0	Long	Temp. Pr.(Psia) RH Chem. Rad. Sub.	90 ⁰ F Amb Amb No 0.1 No	90°F Amb Amb >10 ⁴	Exper Exper Exper - Eval -		
55 Transformer Westinghouse/ ECI	1-1/6	0	< 0.1	Temp. Pr.(Psia) RH Chem. Rad. Sub.	100 [°] F Amb Amb No 1 No	131°F Amb Amb > 10 ⁴	V. Data Exper Exper - Eval -	61 - - - -	
56 Circuit Breaker Westinghouse/ DB25	1-2/7	0	< 0.1	Temp. Pr.(Psia) RH Chem. Rać. Sub.	100 ⁰ F Amb Amb No 1 No	$104^{\circ}F$ Amb Amb - $>10^{4}$	V. Data Exper Exper Eval	62 - - - -	

Reactor: Yankee Ro	we				5	Systematic	Evaluation	Program	
	Submittal		Time	Env	vironmen	nt	Qual.		
Equipment Type	Reference	Loc.	Needed	Parameter	Spec.	Qual.	Method	Reference	Remarks
57A Battery Switchboard Westinghouse/ No Part No.	1-8/11,12	0	Long	Temp. Pr.(Psia) RH Chem. Rad. Sub.	100 ⁰ F Amb Amb No 1 No	104°F Amb Amb >10 ⁴	Eval Exper Exper Eval	-	
57B Battery Switchboard Westinghouse/ CDP, G-10P, WFB-2W	1-8/13,14,15	0	Long	Temp. Pr.(Psia) RH Chem. Rad. Sub.	90°F Amb Amb No 10 ⁴ No	$104^{\circ}F$ Amb Amb -4 > 10^{4}	Eval Exper Exper - Eval -		
58 Emergency Buss GE/ AKD-5	1-8/16,17 1-9/1	0	Long	Temp. Pr.(Psia) RH Chem. Rad. Sub.	90 [°] F Amb Amb No 10 ⁴ No	$104^{\circ}F$ Amb Amb - $>10^4$ -	Eval Exper Exper Eval		
59A Emergency MCC Westinghouse/ Type W	1-9/3	0	Long	Temp. Pr.(Psia) RH Chem. Rad. Sub.	90 [°] F Amb Amb No 10 No	$104^{\circ}F$ Amb Amb $>10^{4}$	Eval Exper Exper - Eval -		
59B Emergency MCC Westinghouse/ YS-1738	1-9/2	0	Long	Temp. Pr.(Psia) RH Chem. Rad. Sub.	100 ^o F Amb Amb No 1 No	104°F Amb Amb >10 ⁴	Eval Exper Exper Eval	-	

YANKEE ROWE

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ID # EOUIPMENI (DESC.-FUNC.)

MFR MODEL, TYPE OR CAT # CLASS LOC ENV REQT ENV QUAL -

Reactor Protection System

	Linear Amp RPS	WEST FN	IE	MCR	Al	B1
	Indicator RPS	WEST KX24		MCR	Al	B1
	Bistable RPS	WEST FN	IE	MCR	Al	B1
	Relay and Control Panel RPS	WEST FN	IE	MCR	Al	B1
MC-FD-	Diff. Pressure Detector MC Loop Flow	BARTN 244	IE	VC	A.2	B2
AOCT	Current Xformer MCP Current Phase	WEST ECI	IE	SWGR	A2	В3
	Current Relay MC Flow Scram	WEST 1876-072	IE	SWGR	A2	В3
K	Relay MC Flow Scram	COUCH C378	IE	SWCR	A2	B4
K-TD	Time Delay Relay Delay Scram 200 ms	AGASI 2422-PA	IE	SWGR	A2	В4
MC-PD-	Pressure Detector MC & Pressurizer	BAILY KIX	IE	VC	A2	B5
PR-LD-	Level Detector Pressurizer Level	BAILY BR117XX MODEL XNX	IE	VC	A2	B6
LT-1003	Level Xmitter SG	FP 13D2495J-BC-BBB-NS	IE	VC	A2	в7
	Power Supply for Xmitter LT-1003	LAMBD LM262	۳E	MCR	Al	B8
LA-1003	Bistable Reactor Trip	RIS E -215R	IE	MCR	Al	В9
	Test Switch for Xmitter LT-1003	ES 28204A-2		MCR	A1	B10
LIC-1003	Meter Relay Alarm Unit	SIGMA 9222-20ED-2-VB11M		MCR	Al	B11

10 #	EOUIPMENT (DESCFUNC.)	MFR MODEL, TYPE OR CAT #	CLASS	LOC	ENV REQT	ENV

PAB A4

B13

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QUAL.

CH-3,4,& 5	Comp Ion Chamber	WEST WL6377	IE	VC	A2	B12
CP-6,7,8 8	Ionization Chamber	WEST WL6937A	1E	VC	A2	812
	Intermediate Range Panel	WEST TYPE FN	IE	MCR	A1	81
	Power Range Panel	WEST TYPE FN	IE	MCR	A1	81
	Bistable Panel	WEST TYPE FN	IE	MCR	A2	B1
62CX	Relay Trip BFP	AGAST ND22-0		MCR	A1	83
BK1	Breaker Rod Scram Bkr	WEST DB25	IE	SWGR	A3	в3
	PB Switch Manual Scram	WEST OT	IE	MCR	A1	B3
TC-420	Thermal Converter/Permissive 15MWE	SANG CW-10	IE	MCR	A1	83
BS-420	Current Alarm Unit/Permissive 15 MWE	ACRO 871-5-8		MCR	A1	83
	Alarm & Scram Panel	WEST FN	IE	MCR	A1	81
	Scram Amplifier	WEST FN	IE	MCR	A1	B1
	Current Reference	WEST FN	IE	MCR	A1	B1
	AC-DC Panel Power Supply	WFST FN	IE	MCR	A1	81
Safety Inject	tion System					
	DELETED					
SI-PP-LP	Motor Low Press SI Pump	EMMCO TYPE 1C	IE	PAB	A4	B3
CS-MOV-535	Motor Oper. LPSI Path	Limit SMA-2		vc	A6	B13
CS-MOV-533	Motor Oper. LPSI Path	Limit SMA-1		PAB	A4	B13

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ID #	EQUIPMENT (DESCFUNC.)	MFR MODEL, TYPE OF CAT #	CLASS	LOC	ENV REQT	ENV OUAL
SI-PP-HP	Motor High Press SI Pump	GE 5K404AK174	IE	PAB	A4	33
SI-MOV-1	Motor Oper. SI Accumulator Isol	LIMIT SMB-1	IE	PAB	A4	813
LI-SI-1	Electronic Transmitter Level SI TK	GE 555		PAB	A4	814
	Power Supply Level SI TK	GE 570-04		MCR	A1	B15
	Vertical Edgewise Meter Level SI TK	GE 180		MCR	A1	816
SI-LS-	Level Switch SI Accumulator	MAGNT 0-751-X1-SP-SIM3	IE	PAB	A4	B17
SI-WL-	Switch SI Initiation Circuit	WEST WL	IE	MCR	A1	33
SI-1-X	Relay Multiplier	A/B 200E441Z1	IE	MCR	A1	B3
SI-PS-	Pressure Switch VC Pressure	STAT 12N-S4-04-X2	IE	PAB	A4	B18
SI-PS-X	Relay SI Initiation Circuit	WEST SG	IE	MCR	A1	В3
SI-PS-14	Pressure Switch SI Initiation	STAT 7821-100	IE	VC	A6	B19
	DELETED					
	DELETED					
	DELETED					

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10 4	EOUIPYEST (DESCFUNC.)	MFR MODEL, TYPE OR CAT #	CLASS	LOC	ENV REOT	ENV OUAL
	Switch Auto-Manual Bypass Selector	WEST OT	IE	MCR	Al	83
LSX-	Relay Vent Accumulator	WEST HFA	IE	MCP	A1	83
TDC-	Relay Pressurize Accumulator	AGAST 7012FD	IE	MCR	Al	83
NS-SOV	SOV Pressurize Accumulator	ASCG HV-202-301-3-U-125	IE	PAB	A4	820
NS-SOV-	SOV Vent Accumulator	ATKOM 31820	IE	PAB	A4	B21
	Switch Test Sw	GE CR294OUT200R		MCR	Al	83
PU-MOV-541	Motor Oper. Cont Isol	LIMIT SMA-000		PAB	A4	B13
	DELETED		199			
Charging and	Volume Control System					
D_15	Note Concroit System					
F-13-	Motor Charging Pump	WEST 19N3044		PAB	A4	В3
	DELETED					
	DELETED					
Containment	Isolation System					
CIS-SOV-	SOV Cont Isol	ASCO #7-202-303-16	IE ,	PAB	A4	B20
CIS-WL-	WL Switch Cont Isol Act	WEST 803-A448601	IE	PAB	A4	в3
CIS-K9-	Relay Cont Isol Act	GE HGA11352	IE	MCR	Al	33
CIS-5801	Switch/Manual Cont Isol & Valve Test	GE CR294QU207	IE	MCR	A1	В3

		전화 가슴이 그 매력적으로				
ID #	EQUIPMENT (DESCFUNC.)	MFR MODEL, TYPE OF CAT #	CLASS	LOC	ENV REQT	ENV OUAL
C1S-PS	Pressure Switch Cont Isol Act	STAT 12NNL4-C	ĬF	PAR	14	818
Component Coo	ling System			1110		010
component core	TTUE SYSCEE					
p2/)	Motor Component Cooling Pump	WEST 19N3369		PAB	A4	B3
Chemical Shut	down System					
	DELETED					
	DELETED					
	DELETED					
Service Water	System					
P-6-	Motor Service Water Pump	WEST 19N3044		SCRN	A7	B3
Main Coolant :	System					
MC-MOV-	Motor Oper. Main Coolant Inlet/Outlet	LIMIT SA-3		VC	A6	B13
MC-PI	Press Xmitter LTOP	ROSMT 1152-GP		VC	A6	B23
	Power Supply LTOP	LAMBD LCS-A-03		MCR	A1	B8
	Recorder LTOP	L&N Speedomax M		MCR	A1	823
RM-	Emerg Radiation Monitor/High Range	NRC TA900		MCR	A1	B24

ID 4	EOUIPYENT (DESCFUNC.)	MFR MODEL, TYPE OR CAT	# CLASS	LOC	ENV REOT	ENV OUAL
Feedw	ater System					
P-1 -	Motor Boiler Feed Pump	WEST 18N6512		тв	А9	B3
P-4-	Notor Condensate Pump	WEST 18N8216		TB	А9	В3
SG-SE	V- Pneum Cntrl Sta F W Flow	BAILY AM		MCR	A1	B22

BAILY KP-1320A

BAILY BR-116X

SG-PIT-

SG-LT-

Press Xmitter Stm Gen

Level Xmitter Stm Gen

Main Coolant Drain & Sampling Systems

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DELETED

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822

B25

TB

VC

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A9

A6

Page 6

ID A	EOUIPMENT (DESCFUNC.)	MFR MODEL, TYPE OF CAT #	CLASS	LOC	ENV REOT	ENV QUAL
LPST, Shut	down & Spent Fuel Pit Cooling System					
	DELETED					
	DELETED					
P-21	Motor Spent Fuel Transfer Pump	WEST 19N3370		PAB	A4	В3
Pressure Co	ontrol and Relief System					
	DELETED					
	DELETFD					
Post Accide	ent Hydrogen Vent System					
FN-18-	Motor Post Accident Recirc Fan	WEST 72Y51238		vc	A6	B26
HV-SOV-	SOV Hydrogen Sample	ATKOM URAMIC 30,000 Series		vc	A6	821
Primary Put	np Seal Water System					

DELETED

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ID * EOUIPMENT (DESC.-FUNC.)

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MFR MODEL, TYPE OR CAT # CLASS LOC ENV REOT ENV DUAL

1.

Electrical Equipment

Generator Emergency Diesel Gen	DELCO E-4859VB	IE	PAB	A5	B27
Level Sw Level Cntrl Day Tnk	GEMS LS 800	IE	PAB	A5	B28
SOV Diesel Gen Day Tnk Fill Valve	ASCO 8211856	IE	PAB	A5	B20
Relay Solid State/Level Cntrl Day Tnk	GEMS 28196	IE	PAB	A5	B28
No 1 Battery	EXIDE FOP-15	IE	SWGR	A3	В3
No 2 Battery	EXIDE FOP-19	IE	SWGR	A3	в3
No 3 Battery	C&D KU-15	IE	PAB	A5	B3
No.1 Battery Charger - MG Set	EPC0 70-D	IE	SWGR	A3	В3
No 2 Battery Charger - MG Set	EPC0 70-D	IE	SWGR	A3	В3
No 3 Battery Charger - Static	C&D ARR130H	IE	PAB	A5	B3
No 1 Battery Switchboard	WEST	IE	SWGR	A3	B3
No 2 Rattery Switchboard	WEST	IE	SWGR	A3	B3
No 3 Battery Switchboard	WEST CDP	IE	PAB	A5	B3
No 3A Battery Switchboard	WEST G-10P	IE	PAB	A5	В3
No 3B Battery Switchboard	WEST WFB-2W	IE	PAB	A5	B3
480 Volt Emergency Bus 1	GE AKD-5	IE	PAB	άŝ	B3
480 Volt Emergency Bus 2	GE AKD-5	IE	PAB	A5	B3
Connector Coaxial	Type HN	IE	VC*	A2	B12
	Generator Emergency Diesel Gen Level Sw Level Cntrl Day Tnk SOV Diesel Gen Day Tnk Fill Valve Relay Solid State/Level Cntrl Day Tnk No 1 Battery No 2 Battery No 3 Battery No 1 Battery Charger - MG Set No 2 Battery Charger - MG Set No 3 Battery Charger - Static No 1 Battery Switchboard No 3 Battery Switchboard No 3 Battery Switchboard No 3A Battery Switchboard No 3B Battery Switchboard Ano 3D Battery Switchboard	Generator Emergency Diesel GenDELCO E-4859VBLevel Sw Level Cntrl Day TnkGEMS LS 800SOV Diesel Gen Day Tnk Fill ValveASCO 8211B56Relay Solid State/Level Cntrl Day TnkGEMS 28196No 1 BatteryEXIDE FOP-15No 2 BatteryEXIDE FOP-19No 3 BatteryC&D KU-15No 1 Battery Charger - MC SetEPCO 70-DNo 2 Battery Charger - MC SetEPCO 70-DNo 3 Battery Charger - StaticC&D ARR130HNo 1 Battery SwitchboardWESTNo 2 Rattery SwitchboardWESTNo 3 Battery SwitchboardWEST G-10PNo 3A Battery SwitchboardWEST WEST G-10PNo 3B Battery SwitchboardWEST WEST G-10PNo 3B Battery SwitchboardWEST G-10PNo 3B Battery SwitchboardWEST WEST G-10PNo 3A Battery SwitchboardWEST WEST G-10PNo 3B Battery Switchboard <td>Generator Emergency Diesel GenDELCO E-4859VBTELevel Sw Level Cntrl Day TnkGEMS LS 800IESOV Diesel Gen Day Tnk Fill ValveASCO 8211856IERelay Solid State/Level Cntrl Day TnkGEMS 28196IENo 1 BatteryEXIDE FOP-15IENo 4 BatteryEXIDE FOP-19IENo 3 BatteryC&D KU-15IENo 1 Battery Charger - MG SetEPCO 70-DIENo 2 Battery Charger - MG SetEPCO 70-DIENo 1 Battery SwitchboardWESTIENo 1 Battery SwitchboardWESTIENo 3 Battery SwitchboardWEST G-10PIENo 3B Battery SwitchboardWEST WFB-2WIENo 3B Battery SwitchboardWEST WFB-2WIEA60 Volt Emergency Bus 1GE AKD-5IEConnector CoaxialType HNIE</td> <td>Generator Emergency Diesel GenDELCO E-4859VBIEPABLevel Sw Level Cntrl Day TnkGEMS LS 800IEPABSOV Diesel Gen Day Tnk Fill ValveASCO 8211856IEPABRelay Solid State/Level Cntrl Day TnkGEMS 28196IEPABNo 1 BatteryEXIDE FOP-15IESWGRNo 2 BatteryC&D KU-15IEPABNo 1 BatteryC&D KU-15IEPABNo 1 Battery Charger - MG SetEPCO 70-DIESWGRNo 2 Battery Charger - MG SetEPCO 70-DIESWGRNo 3 Battery Charger - StaticC&D ARR130HIEPABNo 1 Battery SwitchboardWESTIESWGRNo 3 Battery SwitchboardWEST CDPIESWGRNo 3 Battery SwitchboardWEST WFB-2WIEPABNo 3B Battery SwitchboardWEST WFB-2WIEPABNo 3B Battery SwitchboardWEST WFB-2WIEPABNo 3B Battery SwitchboardWEST WFB-2WIEPABNo 3B Battery SwitchboardWEST WFB-2WIEPABA60 Volt Emergency Bus 1GE AKD-5IEPAB480 Volt Emergency Bus 2GE AKD-5IEPABConnector CoaxialType HNIEVC*</td> <td>Generator Emergency Diesel GenDELCO E-4859VBIEPABA5Level Sw Level Cntrl Day TnkGENS LS 800IEPABA5SOV Diesel Gen Day Tnk Fill ValveASCO 8211856IEPABA5Relay Solid State/Level Cntrl Day TnkGENS 28196IEPABA5No 1 BatteryEXIDE FOP-15IESWGRA3No 4 BatteryEXIDE FOP-19IESWGRA3No 3 BatteryC&D KU-15IEPABA5No 1 Battery Charger - MG SetEPCO 70-DIESWGRA3No 2 Battery Charger - MG SetEPCO 70-DIESWGRA3No 3 Battery Charger - StaticC&D ARR130HIEPABA5No 1 Battery SwitchboardWESTIESWGRA3No 3 Battery SwitchboardWEST CDPIEPABA5No 3 Battery SwitchboardWEST G-10PIEPABA5No 3 Battery SwitchboardWEST G-10PIEPABA5No 3 Battery SwitchboardWEST WEST G-10PIEPABA5No 16 EMERGENCY SwitchboardWEST WEST G-10PIEPAB</td>	Generator Emergency Diesel GenDELCO E-4859VBTELevel Sw Level Cntrl Day TnkGEMS LS 800IESOV Diesel Gen Day Tnk Fill ValveASCO 8211856IERelay Solid State/Level Cntrl Day TnkGEMS 28196IENo 1 BatteryEXIDE FOP-15IENo 4 BatteryEXIDE FOP-19IENo 3 BatteryC&D KU-15IENo 1 Battery Charger - MG SetEPCO 70-DIENo 2 Battery Charger - MG SetEPCO 70-DIENo 1 Battery SwitchboardWESTIENo 1 Battery SwitchboardWESTIENo 3 Battery SwitchboardWEST G-10PIENo 3B Battery SwitchboardWEST WFB-2WIENo 3B Battery SwitchboardWEST WFB-2WIEA60 Volt Emergency Bus 1GE AKD-5IEConnector CoaxialType HNIE	Generator Emergency Diesel GenDELCO E-4859VBIEPABLevel Sw Level Cntrl Day TnkGEMS LS 800IEPABSOV Diesel Gen Day Tnk Fill ValveASCO 8211856IEPABRelay Solid State/Level Cntrl Day TnkGEMS 28196IEPABNo 1 BatteryEXIDE FOP-15IESWGRNo 2 BatteryC&D KU-15IEPABNo 1 BatteryC&D KU-15IEPABNo 1 Battery Charger - MG SetEPCO 70-DIESWGRNo 2 Battery Charger - MG SetEPCO 70-DIESWGRNo 3 Battery Charger - StaticC&D ARR130HIEPABNo 1 Battery SwitchboardWESTIESWGRNo 3 Battery SwitchboardWEST CDPIESWGRNo 3 Battery SwitchboardWEST WFB-2WIEPABNo 3B Battery SwitchboardWEST WFB-2WIEPABNo 3B Battery SwitchboardWEST WFB-2WIEPABNo 3B Battery SwitchboardWEST WFB-2WIEPABNo 3B Battery SwitchboardWEST WFB-2WIEPABA60 Volt Emergency Bus 1GE AKD-5IEPAB480 Volt Emergency Bus 2GE AKD-5IEPABConnector CoaxialType HNIEVC*	Generator Emergency Diesel GenDELCO E-4859VBIEPABA5Level Sw Level Cntrl Day TnkGENS LS 800IEPABA5SOV Diesel Gen Day Tnk Fill ValveASCO 8211856IEPABA5Relay Solid State/Level Cntrl Day TnkGENS 28196IEPABA5No 1 BatteryEXIDE FOP-15IESWGRA3No 4 BatteryEXIDE FOP-19IESWGRA3No 3 BatteryC&D KU-15IEPABA5No 1 Battery Charger - MG SetEPCO 70-DIESWGRA3No 2 Battery Charger - MG SetEPCO 70-DIESWGRA3No 3 Battery Charger - StaticC&D ARR130HIEPABA5No 1 Battery SwitchboardWESTIESWGRA3No 3 Battery SwitchboardWEST CDPIEPABA5No 3 Battery SwitchboardWEST G-10PIEPABA5No 3 Battery SwitchboardWEST G-10PIEPABA5No 3 Battery SwitchboardWEST WEST G-10PIEPABA5No 16 EMERGENCY SwitchboardWEST WEST G-10PIEPAB

ID # EOUIPMENT (DESC.-FUNC.)

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MFR MODEL, TYPE OR CAT # CLASS LOC ENV REQT ENV DUAL"

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3

ID 4	EQUIPMENT (DESCFUNC.)	MFR MODEL, TYPE OR CAT #	CLASS	LOC	ENV REOT	ENV QUAL
	Selector Switch for MOV-1	WEST 0T211	IE	MCR	A1	33
	Relay Aux Hand Reset (40 Stage)	WEST WL	IE	MCR	A1	93
	Cable Mineral Insulated		IE	VC*	A11	831
	Cable Polyethylene/PVC		IE	VC*	A11	B31
	Cable Butyl Rubber/PVC		IE	VC*	A11	B31
	Cable Silicone Rubber/Asbestos		IE	VC	A11	831
	Cable XLP/Neoprene (or Hypalon)		IE	VC*	A11	831
	Cable PVC Shielded		IE	VC*	A11	B31
	Cable Coaxial		IE	VC*	A11	B12
Instrumenta	tion					
PT-227	Pressure Transmitter VC Press.	ROSMT 1151-GP		PAB	A4	B32
	Power Supply	LAMBD LCS		MCR	A1	B8

ANALO PI2455

MCR A1

B33

Indicator

Page 10

Key to Abbreviations Used on Equipment List

Manufacturers

1. 11

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WEST	Westinghouse
BARTN	Barton
AGAST	Agastat
BAILY	Bailey
FP	Fischer & Porter
LAMBD	Lambda
ES	Electro Switch
RIS	Rochester Instruments
SANG	Sangamo
ACRO	Acromas
LIMIT	Limitorque
EMMCO	Electric Machinery
GE	General Electric
MAGNT	Magnetrol
A/B	Allen Bradley
ASCO	Automatic Switch
ATKOM	Atkomatic Valve
ROSMT	Rosemount
C&D	C&D Battery
EPCO	Electronic Products
MARAT	Marathon
CBICO	Chicago Bridge & Iron
MMM	Manning Maxwell and Moore
COUCH	Couch Ordnance
SIGMA	Sigma Instrument
STAT	Static 'O' Ring
EXIDE	Exide Battery
GEMS	GEMS
L&N	Leeds & Northrup
NRC	Nuclear Research

Location	
MCR	Main Control Room
VC	Vapor Container
SWGR	Switchgear Room
PAB	Primary Auxiliary Building
SCRN	Screen Well House
TB	Turpine Building
VC*	Equipment, such as cable which is
	located in the vapor container, but
	also is used throughout the plant

EOUIPMENT LIST ATTACHMENT A ENVIRONMENTAL REQUIREMENTS

The environment in the main control room is air conditioned with temperatures maintained at 700 ± 100 F. Equipment is mounted in cabinets, on racks within control boards, or mounted on a control board, and there are no adverse vibrational effects. The room has been protected from the environmental effects of piping system breaks.

The bounding environmental condition for this equipment is the occurrence of loss of offsite power, wherein the source of power for the air conditioning system is lost. However, in 17 years of plant operation, this event has occurred only once during the great Northeast blackout in 1965, and lasted for only 20 minutes. Therefore, it can reasonably be assumed that power will be restored within this time frame, as each transmission line from the plant is connected directly to a hydro-electric plant.

A test was run in August 1971 where the control room air conditioning was turned off for a period of six hours on a hot humid day. Test results indicate that equipment mounted on the control board and panels will not exceed 90°F and equipment mounted on racks within the main control board will approach 110°F. Based on the test conditions, it is assumed that control room relative humidity will not exceed 75% and pressure is atmospheric.

The 30-day integrated LOCA dose will not exceed 0.1R.

Λ.2

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The attached Table A.1 presents the transients and the protection provided by the reactor protection system for these transients.

Projection system equipment completes its function within a few seconds of the trip initiation. Environmental requirements for each reactor protection detector function are as follows:

a. High Neutron Flux Power Range and High Startup Rate

The neutron detectors for these channels are located in sealed thimbles in the neutron shield tank, which provides a controlled environment at a temperature of 100°F to 110°F and low relative humidity. Containment pressure under normal operating conditions is maintained at less than 20 psia. Coaxial cables are run from the detectors to the control room through a sealed drybox (105-115°F) around the top of the reactor vessel, and via conduits and penetrations (75-95°F along the route). Connectors are of the standard coaxial cable type. None of the transients requiring protection from these channels affect the normal environmental conditions of the detectors because of the double seal between the thimble and the containment air space.

The neutron detectors are positioned axially opposite the reactor core, so radiation levels under normal power conditions are higher than those in general containment areas. The integrated gamma dose for the two-core operational life of the cables in the thimbles is 2 \times 107R .

h. Reactor Coolant Pump Low Current Flo;

The redundant current transformers and equipment for this protection are located in the controlled environment of the switchgear room (see Section A.3) and the normal environment is unaffected by the loss of flow transient.

Backup protection for the loss of flow transient is provided by four differential pressure detectors, with one connected across the primary side of each steam generator. The signal from the detector is transmitted through mineral insulated cables, and in cables in conduit via penetrations to the control room. The environme... of the detectors and cables within the containment or outside the containment is unaffected by the transient. These detectors are located in the containment loop area. See Section A.6 for environmental conditions. The 40 year integrated radiation dose is calculated to be less than 106R.

c. Pressurizer Low Pressure and Low Main Coolant Pressure

These channels provide redundant reactor protection for events which cause low main coolant system pressure. The low main coolant pressure channel also provides one of the initiation signals for the safety injection actuation. See Section A.6 for normal and DBE environmental conditions in containment.

The signals from these detectors are transmitted through mineral insulated cables and cables in conduits via penetrations to the control room.

d. Pressurizer High Water Level (Wide Range)

This detector is located in the pressurizer compartment and the signal is transmitted to the control room through a mineral insulated cable and cable in conduit via a penetration. Normal conditions in the compartment are given in Section A.6. For reactor protection functions, the environment at the detector does not change. This detector and the identical narrow range detector are not required during the LOCA or main steam line break DBE.

e. Steam Generator Low Water Level

One of these detectors monitors the water level in each steam generator. With the exception of steam line break within containment, none of the transients requiring protection will affect the normal environment of the detectors or cables. For steam line break inside containment this channel provides a backup trip. See Section A.6 for normal and DBE conditions in containment.

f. Turbine Trip

This trip is initiated by reactor protection relay circuits located within the main control room, which actuate a turbine trip solenoid coil which is part of the turbine equipment. The loss of load transient does not affect the environment of the protection circuitry. Although this trip provides a backup function, no credit is taken for it in the safety analysis.

g. Manual Reactor Trip

Manual reactor trip is actuated by three pushbutton switches located on the main control board. Since manual trip can be actuated at any time, the limiting environmental conditions occur during loss of offsite power where air conditioning is lost in the main control room.

A.3 The switchgear room is located on the second floor level of the turbine building and contains the following equipment: 2400V switchgear, 2400/480V transformers, 480V switchgear and motor control centers, main coolant pump current (low flow) trip cabinets, vital a-c motor generator set and vital bus cabinet, 125V d-c No. 1 and No. 2 batteries (enclosed), battery chargers, d-c switchboard buses, reactor scram breakers, and safety injection system valve contactors. The time of operation of the power equipment is determined by the requirements of the equipment to which it is connected. The reactor protection equipment operates within seconds of trip initiation.

The environment of the area is controlled and is protected against the effects of pipe breaks in the turbine building and LOCA radiation levels. The 30 day integrated LOCA dose is less than 1R. The limiting environmental condition in this area is caused by the loss of offsite power where a loss of ventilation would occur. However, at the same time, the 2400/480V transformers would lose their excitation and cease being a heat source. Since power should be restored within 20 minutes, based on previous experience, and that there are no other significant heat sources, the effect of the event on the environment is not significant. This was demonstrated during the 1965 Northeast blackout in that no equipment operational abnormalities occurred. Based on this analysis, the conditions in the switchgear room are not anticipated to exceed 100°F or 75% relative humidity.

A.4 Equipment located in the Primary Auxiliary Building which is associated with the ECCS and containment isolation systems, provides protection for the LOCA and main steam line break DBE's. Since this equipment is outside containment, it is not subjected to the steam and pressure environmental conditions. Being located adjacent to the containment, it is subject to LOCA 30-day accumulated radiation dose of less than 10⁵ R, using the source term based on 100% of the noble gases and 50% of the halogens uniformly dispersed in the containment atmosphere, and taking minimal credit for the shielding effects of the PAB Structure. If it is assumed that the safety injection system operates, this radiation dose level will be greatly reduced because fuel damage will be minimized. Therefore, this level

has been reduced to 104R and all other calculated dose levels have been reduced by a factor of ten throughout Attachments A and B, and still will be conservatively higher than the dose rates expected using realistic assumptions.

In the general areas of the first and second floors of the PAB, including the accumulator, safety injection pumps, component cooling pumps, and instrumentation; the normal radiation level is 0.001R/hr. In the cubicle area where most pumps and valves are located, the normal levels vary from 0.01 to 0.03 R/hr, giving a total integrated exposure to date of less than $5 \times 10^3 R$ for this equipment and cables. Valve PU-MOV-541 is located in an area of the PAB where the radiation level is 0.8 R/hr, and the integrated dose to date is approximately $10^5 R$.

The normal temperature variations in these areas of the Primary Auxiliary Building are from 65°T to 90°F. Relative humidity and pressure vary in accordance with normal variations in ambient weather conditions. A broken pipe in the PAB could cause a relative humidity higher than 75%; however, none of the equipment is required for this DBE and it will not occur coincident with LOCA.

Specific operational requirements are as follows:

 $\underline{SI-MOV-1}$ Operation is required during the injection phase of the LOCA. Valve operation occurs within the first few minutes of the LOCA.

<u>CS-MOV-533</u> This valve is redundant to CS-MOV-53^c which is inside containment. At the completion of the injection phase of a LOCA, either valve is closed, which prevents the LPSI pump from running out, thereby preventing NPSH problems.

 $\underline{PU-MOV-541}$ This value operates on the containment isolation signal within seconds after the LOCA.

P-15-1,2,3 The charging pumps are used for emergency boration, if required, following main steam line breaks.

LI-LS The accumulator level switch actuates the NS-SOV- solenoid valves in the nitrogen system within minutes of a LOCA.

NS-SOV-45, 46, 47, 56, 57, 539 & 540 These values are part of the accumulator control system. Operation is required during the first minutes of a LOCA.

SI-PP-LP1, 2, 3 & HP1, 2, 3 The low pressure and high pressure safety injection pumps are required during the injection phase of a LOCA and a rupture of the 24" steam header which is outside the vapor container.

Operation during the LOCA is for the first minutes up to several hours. Operation following a steam line break may be at any time prior to cold shutdown. SI-LI-1 The safety injection tank level transmitter provides an indication of tank level which is required up to several hours after the LOCA.

<u>CIS-PS</u> These redundant pressure switches operate within seconds when containment pressure reaches the containment isolation setpoint. Operation of either switch causes containment isolation actuation via operation of CIS-WL switches and CIS-SOV solenoids.

SI-PS These redundant pressure switches operate within seconds when containment pressure reaches the containment isolation setpoint and initiate safety injection.

P-21 The spent fuel transfer pump is used to cool the spent fuel pit water, and it operates continuously.

<u>P-22A & B</u> The component cooling water pumps transfer heat from the spent fuel pit pool cooling water to the service water.

- A.5 Equipment located in the Primary Auxiliary Building which is associated with the emergency power system provides protection for the loss of offsite power event. Since the local ventilation is operated by the diesel generators under this condition, there is no change in environment for this equipment. The normal temperature variations at this general location are from 65°F to 90°F. Within the diesel generator rooms the temperature varies from 55°F tr 100 °F. Normal radiation level is 0.001 R/hr. The 30 day integrated LOCA dose is <10⁴R. Relative humidity and pressure vary with normal atmospheric conditions.
- A.6 Equipment located within the containment which is operated during the LOCA DBE is subject to the steam and pressure conditions shown in Figures 1 and 2. No valves or motors have to operate during the injection phase. CS-MOV-535 may be required during the recirculation phase and is exposed to the peak transient conditions, but does not have to be operated for at least twenty minutes, when conditions are considerably less severe.

Detailed calculations of the containment transient resulting from rupture of a main steam line inside containment have not been completed at this time. Scoping analyses have, however, been performed and indicate that the peak containment pressure will be less than that resulting from the LOCA DBE. These scoping analyses also indicate that although the peak vapor space temperature may exceed that occurring during the LOCA DBE due to superheating of the blowdown fluid, it will decrease below the LOCA DBE values in the long term.

In areas where safety related equipment is located, normal temperature variations are from 70° to 95°F, containment pressure is maintained at less than 20 psia which is the containment isolation trip setpoint, and relative humidity is less than 100%. Radiation levels at power operation are approximately 0.04 to 2R/hr in the general loop areas, 0.25R/hr near the pressurizer, and 0.001R/hr outside the biological shield wall. The loop cutout valves, which are mounted on the loop

piping are at 20R/hr which gives them an integrated dose to date of 3 x 106R. The integrated doses to date in general areas are approximately 3 x 10⁵R in the loops, 4 x 10⁴R near the pressurizer, and 2 x 10²R outside the shield wall. The 30-day integrated dose from the LOCA is <2 x 10⁶R at the center of the containment and <10 ⁶R outside the shield wall.

The valve inside the vapor container which is required to operate during the recirculation phase will not be submerged when the maximum amounts of water are injected.

Specific operational requirements are as follows: <u>CS-MOV-535</u> This valve is redundant to valve CS-MOV-533 which is located outside containment. At the completion of the injection phase of a LOCA, either valve is closed, which prevents the LPSI pump from running out, thereby preventing NPSH problems.

FN-18, 1, 2, 3 The post accident recirculation fans may be required to operate during the recirculation phase for months after the LOCA. They will not be started until one hour after the LOCA.

HV-SOV-1 & 2 These are the post-accident hydrogen sample valves. Periodic operation of these valves is required from several days to several months following a LOCA.

MC-MOV-301, 302, 309, 310, 318, 319, 325 & 326 The main coolant loop cutout valves are required for a steam line break outside the vapor container to isolate the affected steam generator. Valve operation is not required for a steam line break inside the vapor container therefore, the environmental conditions do not change when operation is required.

SI-PS-14 The pressure switch monitors main coolant pressure and initiates one of the redundant safety injection actuation signals within seconds after the LOCA DBE. It is located outside the biological shield wall.

<u>MC-PI-LTOP</u> The low temperature overpressurization (LTOP) pressure transmitter is connected at the same location as the main coolant pressure detector. It normally functions under normal operating conditions for LTOP, but provides long term pressure indication during LOCA conditions. This channel was installed in 1977, so the integrated dose to date is less than 2×10^4 R.

SG-LT-(WR) The steam generator level transmitters are located in the main coolant loops and are required for a few minutes during the initial stages of the main steam line break to determine the affected steam generator. If the break is outside containment, there are no changes in transmitter environment.

Table A.2 summarizes the operational requirements of the equipment within containment.

A.7 The service water pumps are located in the screen well house where

normal temperatures vary from 50-100°F. The pumps provide cooling water for spent fuel pit cooling. The limiting conditions for these pump motors occur during the loss of offsite power when ventilation is lost. As shown in Section A.1, power should be restored within approximately 20 minutes, and for this amount of time, there are no significant environmental changes. The 30 day accumulated LOCA radiation dose is less than 200R.

A.8 Deleted (Included in Section A.6)

A.9 The steam driven emergency boiler feed pump provides secondary side makeup in event of a loss of offsite power occurring simultaneously with any steam line break. If power is not lost, the boiler feed pumps and/or condensate pumps may be used following a steam line break. In event of loss of offsite power, the condensate pumps are capable of being operated by the diesel generators as a backup to the steam driven pump. A further backup is provided by manual connection of spool pieces to provide charging pump discharge flow to the feedwater header.

These pumps are in the turbine building and have been protected against the effects of a steam line break. Normal temperatures in this area vary from 70° to 90° F. Pressure and humidity vary with normal atmospheric conditions. This is no radiation dose from a steam line break.Operation of the pumps may be required for several days following the DBE.

Also located in this general area are the pneumatic steam generator pressure transmitters which provide pressure indication during the DBF.

Reference: See References B.1.2 and B.1.3 in Section B.1.

- A.10 Deleted (Included in Section A.6)
- A.11 For cables, terminal blocks and penetrations in the containment, the bounding environmental conditions and length of operation are determined by the requirements of the equipment to which they are connected. Since the cables outside of containment are generally the same types that are used inside containment, the bounding environmental conditions are much less severe. Except for mineral insulated cables, all cables, penetrations, and terminal blocks in containment are installed in conduits or protective boxes. There are no spliced connections in containment. For environmental conditions within the containment, refer to Section A.6.

TABLE A.1

RPS TRIPS POTENTIALLY ACTIVATED

	TRANSIENT		*RPS TRIPS POTENTIALLY ACTIVATED								
		1	2	3	4	5	6	7	8	9	
1	Loss of Feedwater Flow				х	x			x		
2	Rod Group Withdrawal	x			X	x			Х	X	
3	Loss of Load				х	X	X		x		
4	Loss of Main Coolant Flow		x						x		
5	Loss of Offsite Power		x		x	x	x		x		
6	Main Coolant System Depressurization	1.2		x				x	X		
7	Excessive Load Increase	x				x			X	x	
8	Boron Dilution	x			x	x			x	X	
9	Control Rod Drop	x		x				x	x		
10	Isolated Loop Startup	x			x	x			x	X	
11	Steam Generator Tube Rupture			x				x	x		
12	Rod Ejection	x		x				x	x	X	
13	Steamline Rupture	x		x		x		x	X	>	
14	LOCA			x				x	x		
* RPS	Trip Functions										
1.,	High Neutron Flux Power Range	7.	Low	Mair	n Coo	lant	Pre	ssur	е		
2.	Reactor Coolant Pump Low Current Flow (2/4)) 8.	Manu	ial P	leact	or T	rip				
3.	Pressurizer Low Pressure	9.	High	n Sta	irtup	Rat	e				
4.	Pressurizer High Water Level										

6. Turbine Trip

5. Steam Generator Low Water Level

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TABLE A.2 EQUIPMENT IN CONTAINMENT 0

	Operational Requirements									
		LOCA		Steam Lin	e Break					
Function/Equipment	Inj. Phase	Recirc. Phase	H2 Control	Inside VC	Outside VC					
LPSI Flow Path										
CS-MOV-535		х								
Recirculation of Ha										
FN-18-1,2,3 2			Х							
Hydrogen Sample										
HV-SOV-1,2			х							
Main Coolant Loop Cutout										
MC-MOV-301,302,309,310,318,					х					
319,325,326										
Safety Injection Actuation										
MC-PD-9, SI-PS-14	х				х					
Indication										
Pressure MC-PI-LTOP	х	х		х	х					
SG Level SG-LT-(WR, each SG)				x	x					
and the second se				•	4					



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EQUIPMENT LIST ATTACHMENT B ENVIRONMENTAL QUALIFICATION

B.1

R. 2

1.4

Components of the reactor protection system located in the control room are the same type of components that were used in contemporary U.S. Navy nuclear submarines and are qualified for military usage. Although no qualification test data is available; the equipment is of rugged military type construction, uses simple and straightforward magnetic amplifier circuits, and has been tested for compatibility and overall system performance, and has been tested to 158°F, 100% relative humidity, and 30 psia. (See Reference B.1.4). The equipment is located in a controlled air conditioned environment protected from the environmental effects of a steam line break and from LOCA radiation levels, which are calculated to be less than 0.1R. The accuracies specified for the basic circuit units are based on an ambient temperature variation of 80° F + 40°F. The bounding environmental conditions for the equipment result from the loss of offsite power wherein the air conditioning equipment loses its power source. The original plant design did not include control room air conditioning, so the equipment did operate for about 10 years under the conditions which would result from this transient and has operated satisfactorily during and since these conditions. The air conditioning was installed for operator comfort, rather than for equipment performance. The protection system equipment is under a periodic maintenance and testing program wherein equipment is calibrated and operationally tested during each refueling outage to verify continued satisfactory operation.

An evaluation of the materials used in electrical, instrumentation, and control systems shows that all equipment is operable with 10^4 P integrated radiation dose. (See Reference B.1.6)

References: B.1.1 Westinghouse, Descriptive Bulletin 57-452, "Type FN Reactor Coolant Control and Instrumentation."

- B.1.2 "Report on Effects of a Piping System Break Outside Containment at Yankee Nuclear Power Station, Rowe, Massachusetts" July 1973.
- B.1.3 "Report on Effects of a Piping System Break Outside Containment, Supplemental Information, at Yankee Nuclear Power Station, Rowe, Massachusetts" September 1973.
- B.1.4 MIL-N-19900, General Requirements-Nuclear Propulsion Control and Instrumentation Equipment

B.1.5 Couch Ordnance Data Sheet

B.1.6 Reference Book, "Effects of Radiation on Materials and Components", Kircher and Bowman.

The main coolant flow transmitters are specified for operation from

 $60^{
m OF}$ to $200^{
m OF}$ and 100% relative humidity, but they only have to operate under normal containment environmental conditions. The containment integrated leak rate conditions of 47 psia had no adverse effect on the transmitters. No damage from radiation should occur below an integrated dose of 2 x $10^{
m 6R}$ (Reference B.1.6). In 17 years of operation with maintenance, calibration and testing performed at each refueling, these transmitters have continued to perform reliably.

Reference: B.2.1

- 2.1 (Reference deleted)
- B.2.2 (Reference deleted)
- B.2.3 (Reference deleted)
- B.2.4 Barton Product Bulletin 199-2-1
- B.2.5 Report, YAEC, No. 1074, "Reactor Containment Integrated Leak Rate Test, May 1974.
- B.2.6 Barton Drawning No. D1-244-901X-5
- Certain electrical equipment was included in the original Yankee 8.3 plant design because of its previously proven reliability in utility applications. These components are in the general areas of protective relays, timers and auxiliary relays, transformers, circuit breakers, motors, hatteries, control switches, and other power circuit controlling equipment. This equipment had to have a demonstrated reliability because of the large economic penalties which are incurred when failures occur in the switching large blocks of energy in power generating stations, substations, and switchyards. Components with this background of reliability were deemed satisfactory for safety applications, installed in environments suitable to their application, provided with appropriate periodic and preventive maintenance, and have continued to perform reliably through 17 years of continuous operation. When reliability has been demonstrated in this manner, similar equipment has been specified for use in design changes and new installations.

The specifications for the motors and electrical power equipment indicate that this type of equipment is designed with a temperature rating of 40° C (104° F). The motors are specified for "power house treatment" to allow operation in areas where high humidity or sprays could occur. An evaluation of the insulating materials in electrical equipment using Reference B.1.6 shows that they will withstand an integrated radiation dose much greater than 10^{4} R before reaching the threshold of damage.

As noted in Section A.4, some motors are installed in the PAB in cubicles in an area where radiation levels are higher than normal because of the fluids in the systems. The integrated dose to date for these motors after 17 years of operation is 5×10^3 R. When the LOCA 30-day integrated dose of <10⁴R in the PAB is superimposed on this, the total dose is approximately 10⁴R which will not damage the motors or their cables. Motors associated with motor operated

valves are discussed in Section B.13.

Breakers, motors, transformers, switchgear, batteries, motor control centers, M-G sets, and protective relaying are provided with continuing periodic maintenance and testing programs to assure continuing proper operation and to detect any conditions which would indicate potential wear and other types of incipient failures. Any components which have known tailure rates, such as M-G set brushes and bearings, are replaced on periodic intervals in advance of anticipated failure. Vibration readings are taken monthly on the diesel generators, motor generator sets, and all major motors outside of the vapor container, and readings are analyzed and plotted to provide continuous trends which will indicate potential failure.

The capacities of the station batteries have been demonstrated by discharge test and are equal to or greater than the capacities required to supply their maximum loads. Periodic battery service tests on a refueling interval have demonstrated that the batteries will supply their anticipated loads after a loss of offsite power.

References: B.3.1 C &

C & D Catalog, Section 12-300

- B.3.2 C & D Catalog, Section 5-460-1
- B.3.3 Agastat Catalog SR-3
- B.3.4 Exide Catalogue Section 50.00
- B.3.5 Acromag CAtalog 2-001.00
- B.3.6 Westinghouse Data Sheet 44-060
- R.3.7 Westinghouse Data Sheet AD-33-760
- B.3.8 Specification, Stone & Webster, SM31-3.55, General Design Requirements - General Service Induction Motors-575V Max.
- 8.3.9 Specification, Stone & Webster, SM32-3.55, General Design Requirements - General Service Induction Motors-2400 and 4160V
- B.3.10 Specification Sheet Boiler Feed Motor
- B.3.11 Specification sheet Condensate Motor
- B.3.12 Specification Sheet P-19, 21, 23
- 8.3.13 Specification Sheet-Main Coolant Charging Pump Motors
- B.4 The reactor coolant pump low current trip cabinets are located in the controlled environment of the switchgear room and use components similar to those previously discussed in Section B.1 and B.3.

The pressurizer pressure and main coolant pressure transmitters provide reactor protection for events which cause main coolant system depressurization. In addition, the main coolant pressure detector provides actuation of one of the safety injection trains at a pressure below the reactor protection system pressure setpoint.

8,5

The bounding environmental conditions for these detectors and their cables during the LOCA DBE is shown in Figures 1 and 2 of Attachment A. The safety injection low pressure setpoint is reached within 10 seconds; therefore, the reactor protection scram point is reached in a shorter time period.

The ambient temperature at the transmitter location varies from 75° to $95^{\circ}F$ and the maximum specification allowable continuous ambient temperature of the transmitter is $158^{\circ}F$. Because of the large mass of the steel detector housing, 170 ± 5 pounds, and the very short time that the temperature is above $\overline{158^{\circ}F}$, the internal temperature of the housing should not exceed the $158^{\circ}F$ ambient temperature allowable for normal operation. Therefore, there will be no effect on the transmitter internal components before the reactor protection and safety injection actuation functions are initiated. An evaluation of transmitter components indicates that the limiting factor is the Class B coll which is rated at $266^{\circ}F$. (Reference B.5.2).

The containment pressure will have no effect on transmitter operation because the housing has been tested to an internal test pressure of 4500 psig. The containment integrated leak test pressure of 47 psia has no effect on transmitter operation.

These detectors are part of the Westinghouse Type FN equipment, and, similar to the remaining circuit components in the control room (see Section 8.1), are constructed to military specifications. Although no qualification test data is available, shock and vibration tests to military standards were approved by BUSHIPS letters. The transformer winding is encapsulated and the electrical housing is explosion proof and qualified watertight in accordance with MIL-STD-108 and MIL-F-2036. MIL-N-19900 requires operability to an integrated dose of 3 x 10^{12} R. (Reference 8.1.4)

From the detector to a junction box outside the concrete shield wall, the signal is carried in mineral insulated cable. From the junction box to the penetration, cable with polyethylene insulation and a polyvinylchloride jacket is run in steel conduit. For discussion of the environmental qualification of the cables, see Section B.31.

Indication of pressure through the DBE is provided by the independent low temperature overpressurization channel which is qualified for LOCA conditions (See Section B.23).

These channels and the pressurizer wide range level channel (Section B.6) have been maintained, calibrated, and tested at each refueling since initial plant operation. In addition, operators perform a check of each channel at least twice per day to verify continued proper operation.

Reference: B.5.1 Bailey Meter Co., Drawing No. H6610184, Rev. A, Title "Electrical Pressure Transmitter"

4 1

- B.5.2 Handbook, "Reference Data for Radio Engineers, Fifth Edition", Table of Insulating Material Classes
- B.6 For all of the transients for which the pressurizer wide range level detector provides a reactor protection signal, there is no change in detector environment. The level detectore (wide range and naarrow range) are constructed to the same military specifications as the pressure detectors discussed in Section B.5. The only difference is that the level detectors are of the differential pressure type. The cable arrangement is the same from detector to penetration.

Reference: B.6.1 Bailey Meter Co., Drawing No. H6610183, Rev. A, Title "Electrical Diff. Pressure Trans. Low Hd."

B.7 The narrow range steam generator level transmitter is designed for post accident operation with high temperature specifications, epoxy paint, special seals, and radiation hardened transistors, and specified to be tested to Combustion Engineering Test Procedure, ICE-TP-1. Test conditions were one hour at 302°F and 70 psia, ramp down for two hours to, etc. then for five hours at, 228°F and 20 psia. Analysis of the steam line break indicates that these transmitters will perform their function within two minutes after a large break.

Separate radiation testing was performed to qualify the transmitter to at least ten megarads of gamma exposure. There is no radiation dose ass inted with the transient and the 40 year integrated dose at the detector location is less than 2 x 10^6 R.

The cable used to connect the transmitter to the penetration has crosslinked polyethylene insulation, shielded, with a Hypalon jacket, and is in conduit.

Reference: B.7.1 Test Procedure, ICE-TP-1, Combustion Engineering.

B.7.2 Engineering Report, "Nuclear Radiation Investigation," Fischer and Porter Company, September 14, 1970.

B.8 Lambda power supplies are designed to meet military environmental specifications and for continuous operation from -20°C to +71°C (160 °F) ambient temperature and 95% relative humidity. These requirements far exceed the environmental requirements of actual usage.

Specific specifications are as follows:

LC Series: MIL-STD-810B, MIL-E-5272C, MIL-1-6181D

LM Series: MIL-STD-810B, MIL-1-16910, MIL-E-5272C, MIL-T-4807A, MIL-E-4970A

Since the power supplies are connected to instruments which provide

continuous indication in the main control room, any deviations in performance are readily detected.

Reference: B.8.1 Lambda Specification Sheets for LC and LM Series Power Supplies

- B.8.2 MIL-STD-810, "Environmental Test Methods"
- B.8.3 MIL-E-5272, "General Specification for Environmental Testing, Aeronautical and Associated Equipment

B.9 These bistables are specified for operation in ambient temperatures ranging from -30° to 150°F which allows adequate margin over the actual conditions in the control room. A periodic testing program is in effect on a monthly basis to allow detection of any deviations in performance. Since installation in 1972, no abnormalities in performance have been detected.

Reference: B.9.1 Rochester Instrument Systems, Bulletin No. 214

B.9.2 Rochester Instrument Systems, Bulletin No. 401

8.10 This switch was installed in 1972 when the steam generator low level trip channels were added, and is of the same rugged construction as equipment discussed in Section B.3. Although designed for heavy duty application on power switching circuits, it is used only for switching signals in the milliampere range, and only during channel test. Therefore, it does not have to function during the time when protection is required. Testing to 140°F and 100% relative humidity is required per Reference B.10.1

Reference: B.10.1 Specification, UL1054, "Special Use Switches"

8.11 These meter relays are not required for the reactor protective function, but provide an alarm to indicate power supply or channel failure. Indicators similar to this have been used extensively at both Connecticut Yankee and Maine Yankee plants for protective functions and have a demonstrated reliability for providing a continuous indication function. They are specified for operation from 60° to 130°F.

Reference: B.11.1 Sigma Specification Sheet, Series 9220

B.12 Neutron detectors used for power range applications are designed to operate in high neutron and gamma fields for an exposure of 2 x 10"R per Reference B.1.4. In the first few years of operation, the symptoms of initial radiation damage to the detector, coaxial connectors, and coaxial cables located within the detector thimbles and dry box were evidenced by the occurrence of a few instantaneous "pips" on the normally stable power range level indications. Inspection of the detectors and cables which were in the thimbles (Reference B.12.3) revealed that they had been exposed to an accumulated dose of 2 x 10⁷R. With relative in-thimble radiation exposure thus determined, a program of periodic replacement of inthimble and dry box components was established to preclude failure due to radiation damage.

The controlled temperature of the neutron shield tank keeps the ion chambers well within their specified temperature ratings of at least $80^{\circ}C$ (176°F). The original cable, RC-149/U was designed to operate at $185^{\circ}F$, while the replacement cables, RC-391/U are designed for $230^{\circ}F$. Both cables meet MIL-C-17 (Reference B.12.2)

The coaxial cables between the loop area and the penetrations are subject to comparatively insignificant radiation levels, so routine replacement is not required.

- Reference: B.12.1 Westinghouse, Descriptive Bulletin 57-451, "Type FN Reactor Nuclear Control Equipment."
 - B.12.2 Specification, MIL-C-17, "General Specification for Cables, RF, Flexible and Semirigid"
 - B.12.3 Report, "Inspection of Channel 6 Ion Chamber and Cables", April 1963.
- B.13 Motor operator CS-MOV-535 is installed in the vapor container and was specified to have a totally enclosed motor with Class H insulation, suitable for operation in an area with 50-75 R/hr continuous radiation dose rate. PU-MOV-541, which is installed in a PAB area with high radiation levels (see Section A.4), is also specified to these requirements. (Reference B.13.1). Similar motor operators having Class H insulation have been successfully tested under post accident steam conditions (Reference B.13.2). The test conditions for these motor operators exceed expected DBE conditions in the vapor container.

The remaining motor operators outside the vapor container are located in the PAB (Section A.4) and are not subject to the severe environmental conditions which result from the LOCA or main steam line break. These motor operators have Class B insulation as a minimum. Similar motor operators have been successfully tested under steam temperature and pressure conditions (Reference B.13.3). Motor operators with Class B insulation have also been successfully operated after irradiation to 2 x 10^{8} R. (Reference B.13.4) Since there are no steam conditions in the PAB and the LOCA radiation dose plus integrated operational dose is less than 10^{5} R, these motor operators are adequately qualified for their environmental conditions.

Reference:

B.13.1 Specification #YS-621, Stainless Steel Stem Leakoff Manual and Motor Operated Valves in Radioactive Service.

P.13.2 Qualification Test of Limitorque Valve Operators in Simulated Reactor Containment Post Accident Steam Environment, FIRL Report #F-C3441, Sept., 1972

- B.13.3 "Performance Qualification Tests of Four Valve Motor Operators", FIRL Report #F-C 4124, April, 1975
- B.13.4 WCAP-7410-L, Volume I of II, "Environmental Testing of Engineered Safety Features Related Equipment", December, 1970.
- B.13.5 Letter, Stone & Webster to YAEC, "Specification (YS-371) for Valve Operators", September 1957
- B.14 The safety injection tank level transmitter is specified with a pressure rating of 1500 psig, an operating temperature range of -20° to 200° F, and meets MIL-E-5272 (Reference B.8.3) humidity test (24 hour cycle 68° to 158°F at 100% relative humidity), which far exceeds its application and environmental conditions. The 30-day integrated LOCA radiation doze of <10⁴ R will have no effect on the transmitter within the operational period of several hours. (Reference B.1.6).

Reference: B.14.1 General Electric Measurement & Control Handbook, Section 8042A

B.15 The GE power supply was selected for this application because of extensive reliable use at the Vermont Yankee plant in similar applications and for its compatibility with the safety injection tank level transmitter. It is rated for ambient temperature ranges of 40° to 120°F and relative humidity of 100%.

Reference: B.15.1 General Electric Measurement and Control Handbock, Section 8063-2C.

B.16 This indicator is used extensively at the Vermont Yankee plant for indication on protective channels and has provided reliable service. It is specified for operation in an ambient temperature range of -4° to 150°F, allowing adequate margin for its environment.

Reference: B.16.1 General Electric Apparatus Handbook, Section 8075, page 104.

B.17 The Magnetrol level switch assembly is made primarily of 304 stainless steel and is rated for 1000F at 500 psig. The switch element mounted in the assembly is rated for 2500F. The PAB temperature and pressure are unaffected by the LOCA environment, allowing adequate margin for assured operation. Since this operates within seconds after a LOCA, radiation dose is not a factor.

These switches have been tested at each refueling for five years without a failure.

Reference: B.17.1 Magnetrol Product Data, Model 751 Series Liquid Level Controls

B.18 Being located in the PAB, these vapor container pressure switches are not subject to a change in temperature or pressure conditions during the LOCA or steam line break events. The switching element is specified for operation between - 650F and 1800F, allowing considerable margin over the normal (or accident) environmental temperature variation of 65 to 90°F.

Radiation level will increase during the LOCA event, but the switch operates at 5 psig containment pressure and will have completed its function before any increase in radiation in the PAB. These switches are similar to those qualified by GE for use in the Vermont Yankee plant protection systems. Information on qualification and operational experience at Vermont Yankee has been submitted in the referenced letter.

Reference:

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P.18.1 Letter, YAEC to USAEC, subject: Additional Information for Proposed Change No. 111, dated January 16, 1974.

B.18.2 Static 'O' Ring Product Data

8.19 The Static-O-Ring main coolant low pressure safety injection actuation switch was installed in 1978 and has been tested to 2800F at 51 psia and 100% relative humidity. The pressure switch operates within ten seconds after the LOCA DBE. Because the pressure switch is located outside the biological shield wall and the switch operates in such a short time period, the switch internals should not be adversely affected by the temperature increase. There will be no increase in radiation level in that short a time after the initiation of the transient. The integrated dose at this location is less than 200R and the switch has performed satisfactorily after a test exposure of 400R.

This pressure switch is redundant to the main coolant pressure channel signal and diverse to the two vapor container high pressure actuation signals which are discussed in Sections B.5 and B.18.

Reference: B.19.1 Certification No. 44296-1, Wyle Labs, November 1, 1978

B.19.2 report No. 44296-2, Wyle Labs, November 2, 1978

B.20 The ASCO Solenoid values with Class H coils are designed to operate in a temperature environment from o^oC to 180^oC (356^oF). The remaining ASCO Solenoid Values are rated at 104^oF. Since these values are all in the PAB, the design exceeds the temperature conditions. Experience during operation to date has shown that the variations in ambient relative humidity and atmospheric pressure have no effect on these values.

Reference: P.20.1 ASCO Product Bulletin 8211

B.20.2 ASCO Product Bulletin 8302

B.21 The Atkomatic values are specified for explosion proof construction, sealed for waterproofing, with a Class H insulated coil, and specified to operate satisfactorily to 275°F. The value seat and coil insulation are specified to an integrated exposure of 8 x 10 $^{7}\mathrm{R}.$ Vendor data indicates that the values are designed for operation to 500°F.

In addition, the valves have been operationally tested monthly for a period of five years without a failure to operate, and have been exposed to the containment integrated leak test pressure of 47 psia.

Reference: B.11.1 Atkomatic Catalog

B.21.2 YAEC Specification YA-729-Y

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The Bailey pneumatic control stations for the feedwater system can be manually adjusted to position the main feedwater values or the bypass values if required after a steam line break. This pneumatic equipment consists of a regulating value which adjusts the air pressure on copper tubing which is terminated at a bellows or similar air-tight device. It is non-electrical, almost all metallic and not subject to the failure modes of electrical equipment. For 17 years of continuous operation, the equipment has been given routine maintenance, and has provided essentially trouble-free, reliable service.

The steam generator pressure transmitters monitor the pressure of the steam lines and transmit a pneumatic signal from the turbine building to a pressure gage in the control room. The transmitter also provides direct local readout, which can be used for backup indication.

B.23 The low temperature overpressurization (LTOP) pressure transmitter was installed in 1977, and is specified to maintain ± 0.75% accuracy after sequential exposure to steam pressure of 70 psig, 316°F for one hour; 55.4 psig, 303°F for seven hours; and 6 psig, 230°F for 42 hours. Radiation performance is ± 2% after 5 megarads total integrated gamma dose. The 40 year integrated dose at the detector location is less than 10°F. Documentation reports are on file at Rosemount. Crosslinked polyethylene cables are run from the detector to a penetration in conduit.

The recorder, which was also installed in 1977, allows trending of main coolant pressure throughout the transient with a range of 0-3000 psig. It is of the latest L&N recorder des'gn, continually monitored to allow detection of any deterioration in performance and suitable for operation in ambient temperatures from 16° to 122°F.

Reference: B.23.1 Rosemount Product Data Sheet 2235.

B.23.2 Leeds & Northrup Co. Manual, 177823

8.24 The high range emergency radiation monitor provides an indication of the radiation level in containment after the LOCA DBE. The detector is mounted high on the turbine room wall, above the control room roof elevation, and the circuitry and readout is in the control room. The readout circuitry is under continuous surveillance by the control room operator and the detector and cables are tested monthly to assure proper operation. The only change in environment during the DBE is at the detector and the coaxial cable in the turbine building. The 120-day integrated dose is less than 1R which will have no adverse effect.

B.25 These transmitters are similar in style to the pressurizer level transmitters discussed in Section B.6, except that they are rated for 1500 psia, rather than 3000 psia, and are of weatherproof construction, rather than explosion proof. The Buna-N O-rings which provide the housing seals are rated for operation up to 212°F: however, these seals are not exposed to the external environment. A scoping evaluation indicates that for the time the transmitter has to operate the temperature of the housing will not cause the O-ring to fail. Since there is no radiation dose increase associated with the steam line break and continued operation is required for only a few minutes, there is reasonable assurance that the transmitters will provide a signal adequate to allow the operators to obtain a comparison of steam generator levels.

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Reference: B.25.1 Bailey Product Specification E21-7.

B.26 The recirculation fan motors have been designed to operate in an atmosphere of steam and dissolved boric acid at a temperature of 275°F and a pressure of 50 psig to an integrated dose of 10⁸R. A prototype motor has been qualification tested at the Franklin Institute Research Laboratory and documentation is on file at YAEC.

Reference: B.26.1 Specification YA-719-Y (rev. 4), Specification for Fag Motors for Hydrogen Control System.

B.26.2 Letter, Westinghouse to YAEC, dated 3/9/72

B.26.3 Letter, Westinghouse to YAEC, dated 3/27/73

B.27 The referenced letter contains a complete description of the standby diesel generators as well as information regarding their satisfactory performance to date in their periodic testing and maintenance programs. If loss of a-c occurs in coincidence with the LOCA DBE, the 30 day integrated dose of less than 10⁴R will have no effect on the diesel-generator, (Reference B.1.6)

Reference: B.27.1 Letter, WYR 78-5, YAEC to NSNRC, dated January 20, 1978, Subject: Standby Diesel Generator Ouestionnaire

B.28 The Gems level switch is designed for fuel oil day tank applications. In the fuel oil environment, the switch assembly will function between 0°F and 230°F, up to a pressure of 150 psi. The solid state relay that is used in conjunction with the level switch is a completely encapsulated unit, free from the affects of oil, dust, humidity and vibration. In addition, the system has been tested at each refueling interval for five years with no failures.

Reference: B.28.1 Gems Technical Bulletin, Series LS-800

B.28.2 Gems Product Data, Model 28196

8.29 Terminal blocks used inside containment are enclosed in protective boxes and their environmental qualification has been discussed with NRC and addressed in the referenced letters.

Terminal blocks which are used throughout the plant outside containment and are exposed to much less severe environmental conditions.

Reference: B.29.1 Letter, YAEC to USNRC, Subject: IE Bulletin #78-02, Terminal Block Oualification, dated February 14, 1978.

- B.29.2 Letter, USNRC to YAEC, Subject: Summary of Meeting 2/1/78 concerning Yankee Rowe, dated February 3, 1978
- B.29.3 Letter, Westinghouse to YAEC, January 31, 1978
- B.30 The references contain qualification information for the penetration assemblies at Yankee Rowe.

Reference:

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B.30.1 YAEC letter to USNRC dated November 30, 1977, WYR 77-116

- B.30.2 YAEC letter to USNRC dated December 1, 1977, WYR 77-118
- B.30.3 Test Report: Associated Nucleonics, Inc., AN-115, dated May 15, 1959
- R.30.4 Letter, YAEC to USNRC, dated 12/12/77
- B.31 The equipment within the containment to which cables are connected consists of a value motor, recirculation fan motors, and instruments. Equipment which was originally installed in the plant was wired in a consistent manner. Instrumentation wiring within the loop compartments and the pressurizer compartment consists of mineral insulated cable to a junction box outside the biological shield wall; and from there, polyethylene insulated cable with a PVC jacket in conduit to the penetration. Similarly, the power and control wiring for value motors consists of mineral insulated cable within the biological shield wall; and from there, either polyethylene or butyl rubber insulated cable with a PVC jacket in conduit to the penetration.

Instrumentation and motors which were installed later in plant life were connected by cables in conduit from the equipment via enclosed junction boxes to the penetrations. This equipment is specifically the recirculation fan motors, the LTOP transmitter, the steam generator narrow range level transmitters for reactor protection, the main coolant pressure switch for safety injection initiation, and the solenoid valves for hydrogen sampling. PVC shielded cables were specified for these installations, unless otherwise noted in the discussion of the environmental qualifications of the equipment to which it is connected. Otherwise, it may be assumed that the cables connected to equipment are those of the initial installation mentioned above.

The mineral insulated cable used within the biological shield wall is specified to meet or exceed the National Electric Code requirements for MI cable. It is generally rated for operation at temperatures of 482° F and is not susceptible to radiation damage for doses exceeding 2 x 10^{10} R. After some problems during initial power operation due to moisture at the fittings, the MI cables have operated essentially trouble-free for 17 years and have shown no evidence of radiation effects.

The polyethylene insulated cable with PVC jacket has been successfully tested in a steam environment at 260°F for a period a period of 45 minutes. This allows margin over the momentary peak anticipated LOCA temperature of 2440F. Test results are on file at YAEC (Reference B.31.4). Butyl rubber insulation is generically rated for service up to 250°F which equals or exceeds the service rating of the polyethylene and PVC insulation material. Since the polyethylene and PVC have been satisfactorily tested and shown to have at least a 16°F margin over the anticipated momentary peak LOCA temperature, this provides reasonable assurance that the butyl rubber insulation will also operate satisfactorily under these conditions. The tests were performed in an open steam environment; whereas the cables are actually installed in steel conduits and junction boxes, which provide protection from the steam environment. The threshold of radiation damage for butyl rubber is 2×10^{6} R and for polyethylene and PVC, the threshold is $2 \times 10^7 R$. These cables are located outside the biological shield wall, so the integrated dose after 17 years is only 2 x 10^2 R. The 120-day integrated LOCA dose outside the shield wall is less than 10^{6} R, so there is adequate margin for these cables to perform their function before radiation damage even begins to take effect. Operating experience for 17 years has shown these cables to be satisfactory for their application.

Cables with silicone rubber insulation and asbestos braid overall and cables with crosslinked polyethylene (XLP) insulation have been tested in accordance with IEEE-383 to environmental conditions which far exceed the conditions expected within the containment. Documentation of test results is on file at YAEC. (Reference B.31.2)

All cables have been exposed to the integrated leak rate pressure of 47 psia with no adverse effect. (Reference B.2.5)

Reference:

B.31.1 U.S. Department of Commerce Report #APEX-261, "Radiation Damage to Elastomers, Organic Liquids, and Plastics".

B.31.2 FIRL Test Report #F-C3798, "Oualification of Class IE Electric Cables for Nuclear Power Generating Stations per IEEE 383", 1974

B.31.3 Standard Handbook for Electrical Engineers,

1957.

B.31.4 Collyer Technical Report #67-2, "Study of 1000V Control Cable in Steam Environment", 1967.

B.31.5 General Cable Catalog, "Mineral Insulated Cable"

B.32 The Rosemount 1151GP transmitter is the same basic unit as the 1152GP model discussed in Section B.23. Amplifier operation is designed for a temperature range of 20°F to 200°F, which far exceeds the conditions in the PAB. The transmitter is rated for 100 psig and is calibrated to a span of 60 psig. This allows considerable margin over the LOCA peak pressure of 32 psig.

Reference: B.32.1 Rosemount Product Data Sheet 2260

B.33 The Analogic PI2455 indicator has solid-state circuitry and an LED digital display. The operating temperature range is 140F to 1400 F, which exceeds the environmental conditions in the control room.

Reference: B.33.1 Analogic Bulletin No. 16-100056 Rev. 2

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