

LICENSEE EVENT REPORT

CONTROL BLOCK: 1 (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

01 C T M N S 2 0 0 - 0 0 0 0 0 - 0 0 3 4 1 1 1 1 1 4 5
7 8 9 14 15 25 26 30 37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 55 56 57 58 59 60 61 62 63 64 65 66 67 68 69 70 71 72 73 74 75 76 77 78 79 80
LICENSEE CODE LICENSE NUMBER LICENSE TYPE CAT

CON'T
01
7 8
REPORT SOURCE L 0 5 0 0 0 3 3 6 7 1 0 2 5 7 8 8 1 1 2 1 7 8 9
60 61 62 63 64 65 66 67 68 69 70 71 72 73 74 75 76 77 78 79 80
DOCKET NUMBER EVENT DATE REPORT DATE

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES 10
02 Testing on the channel A reactor protection system (RPS) core protection
03 calculator indicated a grounded input that was traced to the signal for
04 Pressurizer Pressure within the containment boundary. The channel A trips
05 for High Reactor Coolant System Pressure and Thermal Margin/Low Pressure
06 were bypassed and the grounded signal was isolated from the RPS. There
07 are no consequences from this event as the plant was operated in accord-
08 ance with Technical Specifications, section 3.3.1.1, action 2-b.
7 8 9 80

09 I A E A I N S T R U X Z
7 8 9 10 11 12 13 14 15 16 17 18 19 20
SYSTEM CODE CAUSE CODE CAUSE SUBCODE COMPONENT CODE COMP. SUBCODE VALVE SUBCODE

17 7 8 0 2 9 0 3 L 0
21 22 23 24 25 26 27 28 29 30 31 32
LER/RO REPORT NUMBER EVENT YEAR SEQUENTIAL REPORT NO. OCCURRENCE CODE REPORT TYPE REVISION NO.

F X Z Z 0 0 0 0 Y Y A Z 9 9 9 9
33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48
ACTION TAKEN FUTURE ACTION EFFECT ON PLANT SHUTDOWN METHOD HOURS ATTACHMENT SUBMITTED NPRD-4 FORM SUB. PRIME COMP. SUPPLIER COMPONENT MANUFACTURER

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS 27
10 The source of the ground is not known at this time. The grounded signal
11 was isolated from the reactor protection system with the temporary
12 addition of an isolation amplifier, and the channel restored to service.
13 At the next available reactor shutdown, the ground will be cleared and
14 the isolation amplifier removed.
7 8 9 80

15 E 1 0 0 NA B Surveillance test and inspection
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 55 56 57 58 59 60 61 62 63 64 65 66 67 68 69 70 71 72 73 74 75 76 77 78 79 80
FACILITY STATUS % POWER OTHER STATUS METHOD OF DISCOVERY DISCOVERY DESCRIPTION

16 Z Z NA NA
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 55 56 57 58 59 60 61 62 63 64 65 66 67 68 69 70 71 72 73 74 75 76 77 78 79 80
ACTIVITY CONTENT RELEASED OF RELEASE AMOUNT OF ACTIVITY LOCATION OF RELEASE

17 0 0 0 Z NA
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 55 56 57 58 59 60 61 62 63 64 65 66 67 68 69 70 71 72 73 74 75 76 77 78 79 80
PERSONNEL EXPOSURES NUMBER TYPE DESCRIPTION

18 0 0 0 NA
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 55 56 57 58 59 60 61 62 63 64 65 66 67 68 69 70 71 72 73 74 75 76 77 78 79 80
PERSONNEL INJURIES NUMBER DESCRIPTION

19 Z NA
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 55 56 57 58 59 60 61 62 63 64 65 66 67 68 69 70 71 72 73 74 75 76 77 78 79 80
LOSS OF OR DAMAGE TO FACILITY TYPE DESCRIPTION

20 Z NA
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 55 56 57 58 59 60 61 62 63 64 65 66 67 68 69 70 71 72 73 74 75 76 77 78 79 80
PUBLICITY ISSUED DESCRIPTION

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7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 55 56 57 58 59 60 61 62 63 64 65 66 67 68 69 70 71 72 73 74 75 76 77 78 79 80
NAME OF PREPARER PHONE: NRC USE ONLY

ATTACHMENT

LER 78-29/3L-0

Millstone Unit 2

Docket No. 50-336

EVENT DESCRIPTION

During steady state operation, surveillance testing on the channel A reactor protection system (RPS) core protection calculator indicated a system ground. Investigation showed the ground was on the signal for Pressurizer Pressure and was located within the containment boundary. The channel A reactor trips for High Reactor Coolant System Pressure and Thermal Margin/Low Pressure were bypassed and the grounded signal was isolated from the RPS. There are no probable consequences from this event since the plant was operated in accordance with action statement 2-b of section 3.3.1.1 of the Technical Specifications.

CAUSE DESCRIPTION

The source of the ground is not known at this time, but is either on the signal cable or the channel A Pressurizer Pressure transmitter. The grounded signal was isolated from the reactor protection system with the temporary addition of an isolation amplifier. Channel calibration checks showed the reactor protection system to be fully isolated from the grounded signal and the channel was restored to service. At the next available reactor shutdown, the ground will be cleared, the isolation amplifier removed and the channel restored to normal operation.