LICENSEE E	VENT REPORT
CONTROL BLOCK:	(PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)
0 1 M A Y K R 1 2 0 0 - 0 0 0 1	
7 8 60 61 DOCKET NUMBER 68 EVENT DESCRIPTION AND PROBABLE CONSEQUENCES 10	7 1 1 10 11 17 18 8 11 11 15 17 18 9 69 EVENT DATE 74 75 REPORT DATE 80
0 2 During refueling while removing guide	tubes an auxiliary set of latches
closed on the no. 23 control rod (C.R.) drive shaft, thus withdrawing	
the C.R. The withdrawal of a C.R. con-	stitutes an umplanned reactivity
o s insertion of >0.5% delta K/K (T.S. 6.9	.4.a.[4]). There were no adverse
o 6 effects upon public health or safety or	r plant personnel. Source counts
remained stable, M.C.S. was borated to	>7.0% shutdown and no other rods
0 8 were withdrawn during this event.	80
SYSTEM CAUSE CAUSE SUBCODE  R B 11 D 12 Z 13 X  SEQUENTIAL	COMPONENT CODE SUBCODE
17) REPORT   7 8     0 2 9   24 26	CODE TYPE NO. 1 1 28 29 30 31 32
	RS (22) ATTACHMENT NPRO-4 PRIME COMP. COMPONENT MANUFACTURER  0 0 1 Y 23 N 24 Z 25 Z 9 9 9 9 26
1 The cause of this event was an imprope	r setting of the manipulator
handling tool load cell controller. T	he controller was immediately
readjusted, plant procedures were revi	sed, and Refueling Supervisors
and Shift Supervisors were reminded of	their responsibilities for
adherence to procedures and Technical	Specifications.
FACILITY STATUS % POWER OTHER STATUS 30 M	DISCOVERY DESCRIPTION (32)  D (31) Operator/Inspector Observation  45  D (32)  D (32)  D (32)  D (32)  D (32)  D (32)
RELEASED OF RELEASE AMOUNT OF ACTIVITY (35)  1 6 2 33 Z 34 NA  PERSONNEL EXPOSURES	NA LOCATION OF RELEASE (36)
NUMBER TYPE DESCRIPTION (39)  NA  PERSONNEL INJURIES	80
NUMBER DESCRIPTION (41) NA	
7 8 9 11 12 LOSS OF OR DAMAGE TO FACILITY 43 TYPE DESCRIPTION	781121 0325 S
1 9 Z 42 NA	80
PUBLICITY ISSUED DESCRIPTION 45 NA	NRC USE ONLY
NAME OF PREPARER Richard Aron	PHONE: 416-625-6140 9
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LER 50-29/78-29/01T-0
YANKEE ATOMIC ELECTRIC COMPANY
Rowe, Massachusetts

## EVENT DESCRIPTION AND PROBABLE CONSEQUENCES

During refueling operations in Mode 6 the V.C. manipulator crane operator was in the process of removing core internals in preparation for the fuel handling operations. A guide tube was latched on the manipulator handling tool. Upon withdrawing the guide tube, two auxiliary sets of latches closed on the No. 23 control rod drive shaft withdrawing the control rod. The withdrawal of the control rod constitutes an unplanned reactivity insertion of greater than 0.5%  $\Delta K/K$  (T.S. 6.9.4.a[4]). Main coolant parameters during this event: boron 2094 ppm; temperature 80°F. Based on stuck rod calculations for Core XIII, it is estimated that at 68°F rod 23 is worth between 2.23%  $\Delta \rho$  and 2.13%  $\Delta \rho$ .

During this event the source count rate channels were stable, the main coolant system was borated to > 7% shutdown margin and no other rods were withdrawn. During and after refueling operation the process of withdrawing a control rod is a normal surveillance activity for verifying shutdown margin and rod operability tests. There were no adverse effects upon the health and safety of the public or plant personnel as a result of this event.

## CAUSE DESCRIPTION AND CORRECTIVE ACTIONS

The cause of this event was an improper setting of the manipulator handling tool load cell controller. The controller was set to trip all latches at 50 pounds greater than base load. The guide tube weighs approximately 100 pounds; therefore the latches on all the fingers closed causing the tool to withdraw the control rod train along with the guide tube. The weight of the control rod train is approximately equivalent to the interference weight normally seen during guide tube removal; therefore, the operator was unaware he had the rod drive train until it was visible.

The I & C Department was notified and adjusted the controller for handling guide tubes, weight tubes and drive shafts.

Plant procedures were updated to include the set points on the controller for various core components, and precautions were inserted pertaining to control rod withdrawal. The Shift Supervisors and Refueling Supervisors were issued a memo to exercise extreme caution during the movements of core components.