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MAR 0 1 1994

Docket No. 50-160 License No. R-97

Georgia Institute of Technology ATTN: Dr. Ratib A. Karam, Director Neely Nuclear Research Center 225 North Avenue Atlanta, GA 30332

Gentlemen:

SUBJECT: MANAGEMENT MEETING SUMMARY

This letter refers to the Management Meeting held at our request on February 16, 1994. The meeting concerned activities at your Georgia Tech Research Reactor (GTRR) facility. The issues discussed at the meeting included the safety and training programs at your facility and other items of mutual interest. The meeting was beneficial in that the NRC gained a better understanding of your programs and everyone gained a better understanding of what can be expected during the conversion to Low Enriched Fuel and during the license renewal process.

A list of attendees and a meeting summary are enclosed.

In accordance wit. Section 2.790 of the NRC's "Rules of Practice," Part 2, Title 10, Code of Federal Regulations, a copy of this letter and its enclosures will be placed in the NRC Public Document Room.

Should you have any questions concerning this matter, please contact us.

Sincerely,

ORIGINAL SIGNED BY

J. Philip Stohr, Director Division of Radiation Safety and Safeguards

Enclosures: 1. List of Attendees

2. Meeting Summary

cc w/encls: (See page 2)

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Georgia Institute of Technology

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cc w/encls: Dr. G. Poehlein, Vice President for Interdisciplinary Programs Georgia Institute of Technology 225 North Avenue Atlanta, GA 30332

Dr. William Vernetson Director of Nuclear Facilities Department of Nuclear Engineering Sciences University of Florida 202 Nuclear Sciences Center Gainesville, FL 32611

Pedro B. Perez, Associate Director Nuclear Reactor Program North Carolina State University P. O. Box 7909 Raleigh, NC 27695-7909

Dr. R. U. Mulder, Director Reactor Facility University of Virginia Charlottesville, VA 22901

Harold Reheis, Director Department of Natural Resources 205 Butler Street, SE, Suite 1252 Atlanta, GA 30334

Mayor of the City of Atlanta 55 Trinity Avenue, SW Suite 2400 Atlanta, GA 30335

bcc w/encls: A. Adams, NRR J. Caldwell, NRR D. Collins, RII C. Bassett, RII Document Control Desk

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ENCLOSURE 1

MANAGEMENT MEETING ATTENDEES

Georgia Institute of Technology

R. A. Karam, Director, Neely Nuclear Research Center

R. D. Ice, Manager, Office of Radiation Safety

W. E. Statham, Manager, Reactor Operations

Nuclear Regulatory Commission

J. P. Stohr, Director Division of Radiation Safety and Safeguards (DRSS)

E. J. McAlpine, Section Chief, Radiation Safety Projects Section (RSPS), Nuclear Material Safety and Safeguards Branch (NMSS), DRSS

C. H. Bassett, Fuel Facility Inspector, RSPS, NMSS, DRSS

Nuclear Regulatory Commission - Headquarters

- M. M. Mendonca, Senior Project Manager, Non-Power Reactor & Decommissioning Project Directorate, Division of Operating Reactor Support, Office of Nuclear Reactor Regulation (NRR)
- R. A. Spence, Reactor Systems Engineer, Reactor Operations Analysis Branch, Division of Safety Programs, Office for Analysis & Evaluation of Operational Data (AEOD)
- D. Westall, Australian Assignee, NRR

ENCLOSURE 2

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MANAGEMENT SUMMARY

On February 16, 1994, the Region II DRSS Division Director and staff held a management meeting with representatives from Georgia Institute of Technology (Georgia Tech). M. Mendonca and D. Westall from NRR and R. Spence from AEOD in Headquarters also attended the meeting. The purpose of the meeting was to discuss the status of the licensee's non-power reactor safety and training programs and the future of reactor operations at Georgia Tech. The licensee gave a presentation detailing the progress that had been made during the period since the last management meeting. Various equipment upgrades were discussed including: 1) reactor security upgrade, 2) repair of the cooling tower, 3) instrumentation in the reactor console, and 4) and non-intrusive flow meters to measure the flow for the primary and secondary cooling systems.

The licensee also indicated that the staffing level at the facility was adequate. Other issues were also discussed by those present including: 1) the pending license renewal, 2) the conversion to low enriched uranium (LEU) fuel, 3) the need for the licensee to have more local and state involvement in emergency response training, and 4) the potential impact of a sewer line failure near the facility. Georgia Tech indicated that an emergency response tabletop exercise with state and local participation was scheduled for the week of February 21-25, 1994. The licensee also indicated that they would further evaluate the potential consequences of a sewer line failure and would provide an additional response to the NRC concerning their conclusions. (During a followup telephone call, it was agreed to provide the information by March 31, 1994.)

Although usage of the facility currently is low, the licensee discussed several projects that could increase the usage and bring in additional revenue. The major project that would help bring this about is the Boron/Neutron Capture Therapy (BNCT) program. Georgia Tech is currently designing a filter to be used in this program to provide the proper spectrum of neutrons for this type of therapy. The licensee has the facilities to support this program, a shielded room adjacent to the reactor, but does not yet have a license for this work.

The meeting proved to be very beneficial and, as a result, several issues were raised, including questions on licensing issues dealing with the conversion to LEU fuel.

During the meeting a recent event was also discussed. It dealt with a problem which resulted when an SRO failed to turn a recorder on following a check-out of the recorder during a training session on reactor operations. The recorder prints out, among other indications, indications of the temperature of the thermal shield and of the bismuth block. These signals have scrams associated with them on indication of high temperature in either the thermal shield or the bismuth block. These scrams are not listed in the Technical Specifications as being required for reactor operation but the licensee considers them part of the reactor safety system and safety related instrumentation and, thus considers that the problem was a reportable occurrence. A written report was provided to the NRC on February 18, 1994.