

AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-368

UNIT ANO-2

11-13-78

DATE

COMPLETED BY R. E. Ideker

TELEPHONE 501/371/4355

MONTH October

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1	0
2	0
3	0
4	0
5	0
6	0
7	0
8	0
9	0
10	0
11	0
12	0
13	0
14	0
15	0
16	0

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
17	0
18	0
19	0
20	0
21	0
22	0
23	0
24	0
25	0
26	0
27	0
28	0
29	0
30	0
31	0

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INSTRUCTIONS

On this format, list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt.

OPERATING DATA REPORT

DOCKET NO. 50-368  
 DATE 11-13-78  
 COMPLETED BY R.E. Ideker  
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OPERATING STATUS

1. Unit Name: Arkansas Nuclear One - Unit 2
2. Reporting Period: October 1-31, 1978
3. Licensed Thermal Power (MWt): 2815
4. Nameplate Rating (Gross MWe): 958
5. Design Electrical Rating (Net MWe): 912
6. Maximum Dependable Capacity (Gross MWe): NA
7. Maximum Dependable Capacity (Net MWe): NA
8. If Changes Occur in Capacity Ratings (Items Number 3 Through 7) Since Last Report, Give Reasons:  
NA

Notes Not in Commercial  
Operation

9. Power Level To Which Restricted, If Any (Net MWe): NA
10. Reasons For Restrictions, If Any: None

	This Month	Yr.-to-Date	Cumulative
11. Hours In Reporting Period	<u>NA</u>	<u>NA</u>	<u>NA</u>
12. Number Of Hours Reactor Was Critical	<u>"</u>	<u>"</u>	<u>"</u>
13. Reactor Reserve Shutdown Hours	<u>"</u>	<u>"</u>	<u>"</u>
14. Hours Generator On-Line	<u>"</u>	<u>"</u>	<u>"</u>
15. Unit Reserve Shutdown Hours	<u>"</u>	<u>"</u>	<u>"</u>
16. Gross Thermal Energy Generated (MWH)	<u>"</u>	<u>"</u>	<u>"</u>
17. Gross Electrical Energy Generated (MWH)	<u>"</u>	<u>"</u>	<u>"</u>
18. Net Electrical Energy Generated (MWH)	<u>"</u>	<u>"</u>	<u>"</u>
19. Unit Service Factor	<u>"</u>	<u>"</u>	<u>"</u>
20. Unit Availability Factor	<u>"</u>	<u>"</u>	<u>"</u>
21. Unit Capacity Factor (Using MDC Net)	<u>"</u>	<u>"</u>	<u>"</u>
22. Unit Capacity Factor (Using DER Net)	<u>"</u>	<u>"</u>	<u>"</u>
23. Unit Forced Outage Rate	<u>"</u>	<u>"</u>	<u>"</u>

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):  
NA

25. If Shut Down At End Of Report Period, Estimated Date of Startup: NA
  26. Units In Test Status (Prior to Commercial Operation):
- |                      | Forecast        | Achieved |
|----------------------|-----------------|----------|
| INITIAL CRITICALITY  | <u>11/23/78</u> | <u>-</u> |
| INITIAL ELECTRICITY  | <u>12/11/78</u> | <u>-</u> |
| COMMERCIAL OPERATION | <u>2/15/79</u>  | <u>-</u> |

UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 50-368  
 UNIT NAME ANO-2  
 DATE 11-13-78  
 COMPLETED BY R.E. Ideker  
 TELEPHONE 501/371/4355

REPORT MONTH October

No.	Date	Type <sup>1</sup>	Duration (Hours)	Reason <sup>2</sup>	Method of Shutting Down Reactor <sup>3</sup>	Licensee Event Report #	System Code <sup>4</sup>	Component Code <sup>5</sup>	Cause & Corrective Action to Prevent Recurrence
NA	NA	NA	NA	H	NA	NA	NA	NA	Unit is on Hot Functional Testing

<sup>1</sup>  
 F: Forced  
 S: Scheduled

<sup>2</sup>  
 Reason:  
 A-Equipment Failure (Explain)  
 B-Maintenance or Test  
 C-Refueling  
 D-Regulatory Restriction  
 E-Operator Training & License Examination  
 F-Administrative  
 G-Operational Error (Explain)  
 H-Other (Explain)

<sup>3</sup>  
 Method:  
 1-Manual  
 2-Manual Scram.  
 3-Automatic Scram.  
 4-Other (Explain)

<sup>4</sup>  
 Exhibit G - Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NURIG-0161)

<sup>5</sup>  
 Exhibit I - Same Source

REFUELING INFORMATION

DATE: October 1978

1. Name of facility. Arkansas Nuclear One - Unit 2
2. Scheduled date for next refueling shutdown. 03-01-80
3. Scheduled date for restart following refueling. 06-01-80
4. Will refueling or resumption of operation thereafter require a technical specification change or other license amendment?  
If answer is yes, what, in general, will these be?  
If answer is no, has the reload fuel design and core configuration been reviewed by your Plant Safety Review Committee to determine whether any unreviewed safety questions are associated with the core reload (Ref. 10 CFR Section 50.59)?  
Yes - Description of effects of new core loading  
\_\_\_\_\_  
\_\_\_\_\_  
\_\_\_\_\_
5. Scheduled date(s) for submitting proposed licensing action and supporting information. 1-01-80
6. Important licensing considerations associated with refueling, e.g., new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures.  
None  
\_\_\_\_\_  
\_\_\_\_\_  
\_\_\_\_\_  
\_\_\_\_\_
7. The number of fuel assemblies (a) in the core and (b) in the spent fuel storage pool. a) 177 b) 0
8. The present licensed spent fuel pool storage capacity and the size of any increase in licensed storage capacity that has been requested or is planned, in number of fuel assemblies.  
present 486 increase size by 0
9. The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity.

DATE: March 1988

OPERATING SUMMARY - OCTOBER, 1978

UNIT II

The unit remained shut down until 10/1/78, when the heatup began to continue the Post-Core Hot Functional Test.

On 10/5/78, the pressurizer code relief valve developed a leak during setpoint testing. The valve was temporarily gagged, allowing the valve to cool and reseal (Reference R.O. 50-368/78-8).

On 10/6/78, an Emergency Feedwater valve's hydraulic pump motor failed, making the valve inoperable. Repairs were made and the emergency feedwater flowpath was reestablished (Reference R.O. 50-368/78-9).

The unit was cooled down on 10/14/78 to allow the change out of a partially shorted upper gripper solenoid coil on a control drive element assembly. Heatup began on 10/29/78 and reached Mode 3 operation two days later. Post-Core Hot Functional Testing continued the remainder of the month.

OPERATING SUMMARY - SEPTEMBER, 1978

UNIT II

Post-core Hot Function Testing began on 9/4/78. On 9/12/78, the condensate storage tank level had reduced to approximately 35% due to an abnormally high use rate. It was not brought to >80% within the required 7 days. A contractor was brought in to assist in making the required condensate (Reference R.O. 50-368/78-3).

On 9/16/78, following a Unit I plant trip, breakers feeding both engineering safeguard busses tripped when off-site power supply for both units transferred to Start-Up transformer #2. The emergency diesel generators started, but inverter malfunctions caused the inverters to transfer to their alternate source position before the diesel generators were at speed and operable. In the alternate source position the inverters are aligned exclusively to the diesel generators. Since the alternate source of power to the inverters was not operable, vital instrumentation was without power and a full actuation of the engineering safeguards systems resulted. This included: 1) Reactor Building Spray System actuation, wetting down all equipment, 2) Isolation of component cooling water to the reactor coolant pump seals, 3) Sodium Hydroxide admitted into the Shutdown cooling system. All equipment and components in the Reactor Building was inspected and cleaned, the reactor coolant pump seals were replaced as necessary, and an engineering evaluation of the high chloride concentration in the Reactor Coolant Systems was performed (Reference R.O. 50-368/78-4, 78-5, 78-6 and 78-7).

The Unit remained shut down the rest of the month.