UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO AMENDMENT NO. 146 TO FACILITY OPERATING LICENSE NO. DPR-63

NIAGARA MOHAWK POWER CORPORATION

NINE MILE POINT NUCLEAR STATION UNIT NO. 1

DOCKET NO. 50-220

1.0 INTRODUCTION

By letter dated December 22, 1993, Niagara Mohawk Power Corporation (the licensee or NMPC) submitted a request for changes to the Nine Mile Point Nuclear Station Unit No. 1, Technical Specifications (TSs). The requested changes would revise TS 3.4.4.e (Emergency Ventilation System). TS 3.4.4.e currently permits fuel handling operations to continue during refueling for up to 7 days with one circuit of the emergency ventilation system inoperable, provided all active components of the other emergency ventilation system circuit are operable. The proposed revision would permit fuel handling operations to continue during refueling beyond 7 days with one circuit of the emergency ventilation system inoperable, provided the remaining emergency ventilation system circuit is operable and in operation. The licensee stated that the proposed revision is consistent with recently issued Amendment No. 47 to the Nine Mile Point Unit 2 TSs and with the NRC's Improved Standard Technical Specifications, NUREG-1433.

2.0 EVALUATION

TS 3.4.4.e currently permits reactor operation and fuel handling to continue for up to 7 days when one emergency ventilation system circuit is inoperable, provided that during such 7 days all active components of the other emergency ventilation system circuit are operable. The proposed change would remove the 7-day limit on fuel handling operations during refueling, provided the remaining emergency ventilation system circuit is operable and in operation.

The emergency ventilation system is provided to filter particulates and iodines from the reactor building atmosphere prior to exhausting to the stack and release to the environment during secondary containment isolation conditions. The emergency ventilation system is composed of two 100 percent capacity circuits which are normally maintained in a standby status. Each circuit starts automatically upon detection of high radiation levels in the exhaust duct of the normal reactor building ventilation system, or from high radiation at the refueling platform during refueling operations. Each circuit contains a filter bank for removal of particulates and halogens and has a

rated flow capacity of 1600 cfm with the reactor building at a negative pressure of 0.25-inch water gauge relative to the outside atmosphere. This negative pressure ensures that air discharged from the reactor building is filtered before its release to the environment so as to minimize the release of radioactivity to the environment.

During normal operations (both reactor operations and refueling), the reactor building is ventilated by its normal reactor building ventilation system. The normal ventilation system maintains the reactor building at a negative pressure of at least 0.25-inch water gauge with respect to the outside environment, but this system does not have the capability to filter radioactivity from the discharged air. If radioactive materials are released into the reactor building atmosphere and their concentrations exceed a predetermined limit at the radiation detectors in the exhaust duct, the normal ventilation system is automatically stopped and isolated. The operable emergency ventilation system circuits are then automatically started. Operation of a single circuit of the emergency ventilation system will reestablish the required negative pressure in the reactor building and provide a filtered release to the environment.

The proposed change would remove the 7-day limit on continuing fuel handling operations during refueling with one circuit of the emergency ventilation system inoperable, provided the remaining circuit is operable and in operation. The proposed change is acceptable since placing the operable circuit in operation ensures that its safety function (filtering of the reactor building atmosphere before release to the environment) is being accomplished. The proposed change is also consistent with the NRC staff's current position which permits fuel handling operations to continue during refueling with one operable emergency ventilation system circuit, provided the operable circuit is in operation. The NRC staff's current position is reflected in the guidance provided in the NRC's Improved Standard Technical Specifications for Boiling Water Reactors (NUREG-1433) and satisfies the Commission's Final Policy Statement on TS (58 FR 39132). The NRC staff is considering this change as a potential TS line-item improvement.

3.0 STATE CONSULTATION

In accordance with the Commission's regulations, the New York State official was notified of the proposed issuance of the amendment. The State official had no comments.

4.0 ENVIRONMENTAL CONSIDERATION

The amendment changes a requirement with respect to installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20. The NRC staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no

significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendment involves no significant hazards consideration, and there has been no public comment on such finding (59 FR 4940). Accordingly, the amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendment.

5.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor:
Donald S. Brinkman

Date: March 8, 1994

Docket No. 50-220

March 8, 1994

Mr. B. Ralph Sylvia
Executive Vice President, Nuclear
Niagara Mohawk Power Corporation
301 Plainfield Road
Syracuse, New York 13212

Dear Mr. Sylvia:

SUBJECT: ISSUANCE OF AMENDMENT FOR NINE MILE POINT NUCLEAR STATION UNIT NO. 1 (TAC NO. M88497)

The Commission has issued the enclosed Amendment No. 146 to Facility Operating License No. DPR-63 for the Nine Mile Point Nuclear Station Unit No. 1 (NMP-1). The amendment consists of changes to the Technical Specifications (TSs) in response to your application transmitted by letter dated December 22, 1993.

The amendment revises TS 3.4.4.e (Emergency Ventilation System) to permit fuel handling operations to continue during refueling beyond 7 days with one circuit of the emergency ventilation system inoperable, provided the remaining emergency ventilation system circuit is operable and in operation. The change to TS 3.4.4.e is consistent with the NRC's Improved Standard Technical Specifications, NUREG-1433 and is being considered by the NRC staff as a potential TS line-item improvement.

A copy of the related Safety Evaluation is enclosed. A Notice of Issuance will be included in the Commission's next regular biweekly Federal Register notice.

Sincerely,
Original signed by:

Donald S. Brinkman, Senior Project Manager
Project Directorate I-1
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Enclosures:
1. Amendment No. 146 to DPR-63
2. Safety Evaluation

cc w/enclosures:
See next page
Distribution:
See attached sheet

* 94-038

LA:PDI-1	PM:PDI-1	BC:OTSB	BC:SPLB	OGC	D:PDI-1
CVogan	DBrinkman:smm	CGrimes	CMcCracken		RACapra
2/17/94	2/17/94	2/18/94	2/22/94	2/22/94	3/8/94

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