



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20535

PORTLAND GENERAL ELECTRIC COMPANY

THE CITY OF EUGENE, OREGON

PACIFIC POWER AND LIGHT COMPANY

DOCKET NO. 50-344

TROJAN NUCLEAR PLANT

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No.34
License No. NPF-1

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Portland General Electric Company, the City of Eugene, Oregon, Pacific Power and Light Company (the licensee) dated January 6, 1977, as supplemented and amended April 26, August 29, September 27, October 4 and 10, November 8 and 10, December 15 and 21, 1977 and January 18, 1978, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and paragraphs 2.C.(2) and 2.C.(6) of Facility License No. NPF-1 are hereby amended to read as follows:

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"(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 34, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

"(6) Spent Fuel Pool Modification

The licensee is authorized to modify the spent fuel pool as described in the application dated January 6, 1977 as supplemented and amended April 26, August 29, September 27, October 4 and 10, November 8 and 10, December 15 and 21, 1977 and January 18, 1978.

Since spent fuel is now being stored in the spent fuel pool, upon commencement of work on either the existing racks or the new racks in the spent fuel pool in conjunction with replacement of the existing racks with new racks:

- (a) the water in the spent fuel pool shall contain at least 2000 ppm boron and shall be maintained at this boron concentration until completion of the rack replacement; and
- (b) spent fuel stored in the spent fuel pool must have decayed at least 60 days from the time it was last removed from the reactor.

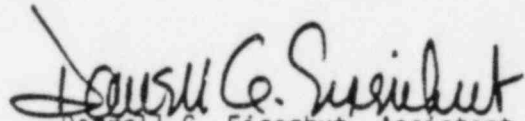
During the time that boron is required in the spent fuel pool, it shall be verified to meet the concentration requirement by chemical analysis at least 3 times per 7 days, with a maximum time interval between samples of 72 hours.

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Prior to final conversion to the modified rack design, fuel may be stored, as needed, in either the modified storage racks described in Technical Specification 5.6.1.2 or in the unmodified storage racks (or both) which are designed and shall be maintained with a nominal 21-inch center-to-center distance between fuel assemblies placed in the storage racks to ensure a K_{eff} of <0.95 with the storage pool filled with unborated water, including a conservative allowance for uncertainties as described in Section 4.3.2.7 of the FSAR."

3. This license amendment is effective as of its date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Darrell G. Eisenhut, Assistant Director
for Systems and Projects
Division of Operating Reactors

Attachment:
Changes to the Technical
Specifications

Date of Issuance: November 3, 1978

ATTACHMENT TO LICENSE AMENDMENT NO. 34

FACILITY OPERATING LICENSE NO. NPF-1

DOCKET NO. 50-344

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages. The revised pages are identified by Amendment number and contain vertical lines indicating the area of change. The corresponding overleaf pages are also provided to maintain document completeness.

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REFUELING OPERATIONS

CRANE TRAVEL - FUEL BUILDING

LIMITING CONDITION FOR OPERATION

3.9.7 Loads carried over the spent fuel pool and the heights at which they may be carried over racks containing fuel shall be limited in such a way as to preclude impact energies over 240,000 in.-lbs., if the loads are dropped.

APPLICABILITY: With fuel assemblies and water in the storage pool.

ACTION:

With the requirements of the above specification not satisfied, place the crane load in a safe condition. The provisions of Specification 3.0.3 are not applicable.

SURVEILLANCE REQUIREMENTS

4.9.7 The potential impact energy due to dropping the crane's load shall be determined to be $< 240,000$ in.-lbs. prior to moving each load over racks containing fuel.

DESIGN FEATURES

- a. In accordance with the code requirements specified in Section 5.2 of the FSAR, with allowance for normal degradation pursuant to the applicable Surveillance Requirements,
- b. For a pressure of 2485 psig, and
- c. For a temperature of 650°F, except for the pressurizer which is 680°F.

VOLUME

5.4.2 The total water and steam volume of the reactor coolant system is 13,104 \pm 100 cubic feet at a nominal T_{avg} of 584.7°F.

5.5 EMERGENCY CORE COOLING SYSTEMS

5.5.1 The emergency core cooling systems are designed and shall be maintained in accordance with the original design provisions contained in Section 6.3 of the FSAR with allowance for normal degradation pursuant to the applicable Surveillance Requirements.

5.6 FUEL STORAGE

CRITICALITY

5.6.1.1 The fuel storage racks for new fuel are designed and shall be maintained with a nominal 21-inch center-to-center distance between fuel assemblies placed in the storage racks to ensure a K_{eff} of ≤ 0.95 with the storage pit filled with unborated water, including a conservative allowance for uncertainties as described in Section 4.3.2.7 of the FSAR.

5.6.1.2 The storage racks for spent fuel are designed and shall be maintained with a nominal 13.3-inch center-to-center distance between fuel assemblies placed in the storage racks to ensure a K_{eff} of ≤ 0.95 with the storage pool filled with unborated water and with new fuel containing not more than 44.3 grams of U-235 per axial centimeter of active fuel assembly. The criticality analysis includes a conservative allowance for uncertainties as described in Section 3.1.2 of PGE-1013.