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**POWERING
MICHIGAN'S PROGRESS**

Big Rock Point Nuclear Plant, 10269 US-31 North, Charlevoix, MI 49720

Patrick M Donnelly
Plant Manager

March 1, 1994

Nuclear Regulatory Commission
Document Control Desk
Washington, DC 20555

Dear Sir:

Enclosed please find the statistical data for the Big Rock Point Nuclear Plant covering the period of February 1, 1994 through February 28, 1994.

Sincerely,

P M Donnelly
Plant Manager

Enclosures

cc: Administrator Region III, Nuclear Regulatory Commission
DRHahn, Department of Public Health
JRPadgett, Michigan Public Service Commission
Raben, Michigan Department of Labor
MPCass, American Nuclear Insures
FYost, Research Services Utility Data Institute
INPO Records Center
DWJoos, P24-117B
GCWithrow, Big Rock Point
NRC Resident Inspector, Big Rock Point
Document Control, Big Rock Point, 740/22*35*10
File

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9403140239 940228
PDR ADOCK 05000155
R PDR

A CMS ENERGY COMPANY

IEPA
1/1

Operating Status Report - Nuclear Regulatory Commission

Prepared by: James R. Jorgensen Date: 3/1/1994
 Reviewed by: J.C. Mott Date: 3/3/94
 (Reactor Engineer or Alternate)

Copy to be sent to the following by the seventh day of each month:

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 Washington, DC 20555

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Routing Copy:

DW Joos, P26-117B
 SM McIntyre, P13-121
 GC Withrow, Big Rock Point Plant
 RC Joba, Big Rock Point Plant
 NRC Resident Inspector, Big Rock Point Plant
 Document Control Center, Big Rock Point Plant
 DCC 740*22*35*10
 (740*22*10*04) Cross Reference



NUCLEAR OPERATIONS DEPARTMENT
Unit Shutdowns and Power Reductions

Report Month <i>February, 1994</i>	Docket Number <i>55-150</i>	Unit <i>Big Rock Point Plant</i>	Date <i>March 1, 1994</i>	Completed by <i>JB JOHNSTON</i>	Telephone <i>(616) 547-8223</i>
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Number	Date	Type ¹	Duration (Hours)	Reason ²	Method of Shutting Down Reactor ³	Licensee Event Report Number	System Code ⁴	Component Code ⁵	Cause and Corrective Action To Prevent Recurrence
<i>94-01</i>	<i>02/08/94</i>	<i>F</i>	<i>0.0 Hrs</i>	<i>A</i>	<i>4</i>	-----	-----	-----	<i>On 02/08/94 at 12:02; The #1 reactor feedwater pump tripped due to a failed coil on the minimum flow valve relay scheme. To prevent a reactor trip the #1 reactor recirculating pump was tripped per operating procedures. After repairs were completed the plant returned to normal power operations.</i>

¹F = Forced
S = Scheduled

²Reason:
A = Equipment Failure (Explain)
B = Maintenance of Test
C = Refueling
D = Regulatory Restriction
E = Operator Training and License Examination
F = Administrative
G = Operational Error (Explain)
H = Other (Explain)

³Method:
1 = Manual
2 = Manual Scram
3 = Automatic Scram
4 = Other (Explain)

⁴Exhibit G = Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

⁵Exhibit I = Same Source

GREYBOOK OPERATING DATA REPORT

DOCKET NO. 50-155 DATE: 3 / 1 / 94
BY: JR JOHNSTON
PHONE: 616-547-6537, EXT 223

OPERATING STATUS

- 1. UNIT NAME: BIG ROCK POINT NUCLEAR PLANT
- 2. REPORTING PERIOD: 2 / 94
- 3. LICENSED THERMAL POWER (MWT): 240
- 4. NAMEPLATE RATING (GROSS MWE): 75
- 5. DESIGN ELECTRICAL RATING (NET MWE): 72
- 6. MAXIMUM DEPENDABLE CAPACITY (GROSS MWE): 71.0
- 7. MAXIMUM DEPENDABLE CAPACITY (NET MWE): 67.0
- 8. IF CHANGES OCCUR IN CAPACITY RATINGS (ITEMS 3 THRU 7) SINCE LAST REPORT, GIVE REASONS:

NOTES:

- 9. POWER LEVEL TO WHICH RESTRICTED, IF ANY (NET MWE):
- 10. REASONS FOR RESTRICTIONS, IF ANY:

	THIS MONTH	YEAR-TO-DATE	CUMULATIVE
11. HOURS IN REPORTING PERIOD	672.0	1416.0	271075.0
12. NUMBER OF HOURS REACTOR WAS CRITICAL	672.0	1416.0	196534.4
13. REACTOR RESERVE SHUTDOWN HOURS	0.0	0.0	0.0
14. HOURS GENERATOR ON-LINE	672.0	1416.0	193495.2
15. UNIT RESERVE SHUTDOWN HOURS	0.0	0.0	0.0
16. GROSS THERMAL ENERGY GENERATED (MWH)	149675.0	320539.0	37223530.0
17. GROSS ELECTRICAL ENERGY GENERATED (MWH)	47848.0	102245.0	11824360.0
18. NET ELECTRICAL ENERGY GENERATED (MWH)	45400.5	97063.2	11186323.2
19. UNIT SERVICE FACTOR	100.0%	100.0%	71.4%
20. UNIT AVAILABILITY FACTOR	100.0%	100.0%	71.4%
21. UNIT CAPACITY FACTOR (USING MDC NET)	100.8%	102.3%	61.3%
22. UNIT CAPACITY FACTOR (USING DER NET)	93.8%	95.2%	57.3%
23. UNIT FORCED OUTAGE RATE	0.0%	0.0%	11.2%

24. SHUTDOWNS SCHEDULED OVER NEXT 6 MONTHS (TYPE, DATE, & DURATION OF EACH): MAINTENANCE, 03/02/94, 10 DAYS.

25. IF SHUT DOWN AT END OF REPORT PERIOD, ESTIMATED DATE OF STARTUP:

(2/94) - CYCLE 27

DAY	AVERAGE DAILY POWER(MWT)	(MWEN)
1	226.25	59.21
2	227.08	59.13
3	228.33	58.89
4	226.92	58.68
5	225.46	58.48
6	224.96	58.32
7	227.56	58.59
8	160.71	46.01
9	173.67	51.81
10	222.71	57.32
11	224.17	57.83
12	225.83	58.52
13	226.38	58.45
14	227.42	58.98
15	226.38	58.99
16	226.83	58.62
17	228.38	59.50
18	228.71	59.64
19	229.21	59.94
20	229.33	59.80
21	229.21	59.50
22	227.96	59.25
23	226.04	58.95
24	229.33	59.64
25	228.29	59.32
26	227.50	59.08
27	230.21	59.74
28	227.96	59.41

Refueling Information Request

1. Facility Name: Big Rock Point Plant
2. Scheduled date for next refueling shutdown: September, 1994.
3. Scheduled date for restart following shutdown: November, 1994.
4. Will refueling or resumption of operation thereafter require a technical specification change or other license amendment? No

If yes, explain:

If no, has the reload fuel design and core configuration been reviewed by Plant Safety Review Committee to determine whether any unreviewed safety questions as associated with the core reload (Reference 10 CFR, Section 50.59)? Yes

If no review has taken place, when is it scheduled?

5. Scheduled date(s) for submittal of proposed licensing action and supporting information:
6. Important licensing considerations associated with refueling, eg. new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design new operating procedures:
7. Number of fuel assemblies in: core 84 ; spent fuel pool storage 316; new fuel storage 0 .
8. Present licensed spent fuel pool storage capacity: 441
Size of any increase in licensed storage capacity that has been requested or planned (in number of fuel assemblies): 0
9. Projected date of last refueling that can be discharged to spent fuel pool assuming the present license capacity: Last total core off load 1996; Last refueling 1999.