

UNITED STATES NUCLEAR REGULATORY COMMISSION REGION II 101 MARIETTA STREET, N.W. ATLANTA, GEORGIA 30303

Report Nos. 50-369/82-43 and 50-370/82-31

Licensee: Duke Power Company

422 South Church Street Charlotte, NC 28242

Facility Name: McGuire Units 1 and 2

Docket Nos. 50-369 and 50-370

License Nos. NPF-9 and CPPR-84

Inspectors: a. of Agnatoris for 12.03-82

W. Orders of Gratoris for Date Signed

A. Ignatomis 12-03-82
Date Signed

P. Hopkins Date Signed

Approved by:

J. C. Bryant, Section Chief, Division of Date Signed

Project and Resident Programs

SUMMARY

Inspection on October 20 - November 20, 1982

Areas Inspected

This routine, announced inspection involved 93 inspector-hours on site in the areas of operational safety verification maintenance and surveillance.

Results

Of the 3 areas inspected, no violations or deviations were identified.

DETAILS

1. Persons Contacted

Licensee Employees

*M. McIntosh, Station Manager

G. Cage, Superintendent of Operations

E. Estep, Project Engineer
*M. Sample, Project Engineer

B. Barron, Operations Engineer, Unit 2

G. Gilbert, Operations Engineer
*D. Mendezoff, Licensing Engineer

C. Van Vynckt, Staff Engineer

Other licensee employees contacted included superintendents, operating engineers, shift supervisors, reactor operators, unit coordinators, station group supervisors, planners, technicians, mechanics, specialists, security, office personnel, corporate design engineers, training and QA personnel.

*Attended exit interview

2. Exit Interview

The inspection scope and findings were summarized on November 20, 1982, with those persons indicated in paragraph 1 above. The station manager acknowledged the findings.

3. Licensee Action on Previous Enforcement Matters

Not inspected.

4. Unresolved Items

Unresolved items were not identified during this inspection.

5. Plant Operations

The inspectors reviewed plant operations throughout the report period, October 20 - November 20, to verify conformance with regulatory requirements, technical specifications and administrative controls. Control room logs, shift supervisors' logs, shift turnover records and equipment removal and restoration records for the units were routinely perused. Interviews were conducted with plant operations, maintenance, chemistry, health physics, and performance personnel on day and night shifts.

Activities within the control rooms were monitored during all shifts and at shift changes. Actions and/or activities observed were conducted as prescribed in the Station Directives. The complement of licensed personnel

on each shift met or exceeded the minimum required by technical specifications. Operators were responsive to plant annunciator alarms and appeared to be cognizant of plant conditions.

Plant tours were taken throughout the reporting period on a frequent basis. During the plant tours, ongoing activities, housekeeping, security, equipment status and radiation control practices were observed.

McGuire Unit 1 began the report period operating at 50% power and maintained that power level until 7:00 p.m. on October 23 when the unit was taken off-line to facilitate the installation of lower containment cooling spray nozzles. The unit was placed back on line at 11:59 a.m. on October 24, power was escalated to 50% and was maintained until November 5 at 6:35 p.m. The licensee began reducing power at that time after determining that both containment spray systems were seismically inoperable. Later that evening, one of the containment spray systems was repaired and the unit was returning to power when at 10:29 p.m. the unit tripped from approximately 20% power as a result of an electrical power spike in the supply to the nuclear power range instrumentation. The plant responded as designed and expected. On November 6, after recovering from the trip, and in preparation for placing the unit on line, it was determined that the main turbine bearing oil pump was inoperable. At that time, the licensee decided to shut the unit down and enter a 17 day outage to facilitate eddy current examination of the steam generators and effect necessary repairs.

Examination of the steam generators resulted in the plugging of 6 tubes; 5 in A generator and 1 in C generator.

At the end of the report period, preparations are underway for unit restart with a forecasted on line pate of November 22.

6. Surveillance Testing

The surveillance tests detailed below were analyzed and/or witnessed by the inspector to ascertain procedural and performance adequacy.

The completed test procedures examined were analyzed for embodiment of the necessary test prerequisites, preparations, instructions, acceptance criteria and sufficiency of technical content.

The selected tests witnessed were examined to ascertain that current written approved procedures were available and in use, that test equipment in use was calibrated, that test prerequisites were met, system restoration completed and test results were adequate.

The selected procedures perused attested conformance with applicable Technical Specifications, having received the required administrative review and having been performed within the surveillance frequency prescribed.

Procedure	Title
PT-1-A-4600-10	Loose Parts Monitoring System
PT-0-A-3207A-03A	Nuclear Instrument System
PT-1-A-4150-01B	Reactor Coolant Leakage
PT-1-A-44C3-01B	Nuclear Service Water Performance
PT-0-A-4600-14-A	Nuclear Instrumentation System
PT-1-A-4600-01	RCCA Movement Test
0P-0-A-6150-07	Incore Instrumentation/Flux Map

No violations or deviations were identified within the areas inspected.

7. Maintenance Activities

Maintenance activities were observed and/or reviewed throughout the report period to ascertain that the work was being performed by qualified personnel, that activities were accomplished employing approved procedures or the activity was within the skill of the trade. Limiting conditions for operation were examined to ensure that technical specification requirements were satisfied. Activities, procedures, and work requests were examined to ensure adequate fire protection, cleanliness control and radiation protection measures were observed, and equipment was properly returned to service.

Acceptance criteria employed for this review included but was not limited to:

STATION DIRECTIVES
ADMINISTRATIVE POLICY MANUAL
TECHNICAL SPECIFICATIONS
TITLE 10 CFR.

Detailed below are 9 maintenance activities which were observed and/or reviewed during the report period:

WORK REQUEST	EQUIPMENT
63941	NC Pump D
62823	Reactor Building Pressure Transmitter
92081	Auxiliary Shutdown Panel

110692	Pressurizer Spray Valves
35336	Hydrogen Analyzer
92025	Reactor Trip Switchgear
109957	Valve 2 NC-112
109812	Valve 2 NV-829
107570	Valve 1 NV-137

No violations or deviations were identified.