

# UNITED STATES NUCLEAR REGULATORY COMMISSION WASHINGTON, D. C. 20555

October 20, 1978

Docket No. 50-244

LICENSEE: Rochester Gas & Electric Corporation (RG&E)

FACILITY: R. E. Ginna Atomic Power Station

SUBJECT: SUMMARY OF MEETING HELD ON OCTOBER 4, 1978

Representatives of the Nuclear Regulatory Commission, Rochester Gas & Electric Corporation (RG&E), and other members of the Systematic Evaluation Program (SEP) and Utility Owners Group met in Bethesda, Maryland, NRC offices on October 4, 1978. The purpose of the meeting was to discuss a program for review and evaluation of "Pipe Breaks Inside the R. E. Ginna Containment Structure." The draft criteria for the SEP review methods was outlined in an NRC letter to RG&E dated September 7, 1978. Attached is a list of attendees.

At the suggestion of RG&E Representatives, the objectives of the meeting were to:

- 1. Clarify statements in the September 7, 1978, NRC letter and attachment.
- Review schedules for accomplishing the "pipe breaks inside containment" objectives and to coordinate this activity with TAP A.2.
- 3. Consider technical details.

The results of the meeting follow:

o Several aspects of pipe breaks inside containment were discussed and will continue to be discussed in the future to assure a common understanding of the topic. These aspects are, deviations from current standard review plan; fatigue analysis; other approaches to the postulated pipe breaks inside containment to be provided for NRC consideration by the Owners Group; and the usefulness of the original piping system analysis.

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- 2 -Meeting Summary October 20, 1978 o The asymmetric LOCA loads will be evaluated in the generic task A-2 (NUREG 0410 January 1, 1978) and will be coordinated with the SEP review of Ginna. RG&E does not possess the original pipe design calculations. The vendor and architect engineer for the Ginna plant have been requested to provide duplicates of the pipe stress calculations that were made prior to plant construction. The vendor representative reported that a search for the original piping design calculations is in progress. RG&E estimated that a period of 60 to 90 days is required to collect and submit this information for NRC review. o The Mechanistic Approach, using Standard Review Plan section 3.6.2 discussed in Item 3 approach A in the enclosure to the NRC letter dated September 7, 1978, is preferred by the Owners Group because it could provide a basis for limiting the number of postulated pipe break locations. o RG&E believes the schedule for completing Topic III-5.A, "Effects of Pipe Break on Structures, Systems and Components Inside Containment" published in NUREG 0485, September 22, 1978, is ambitious. The Owners Group recommended a schedule stretchout to allow the group more time for discussions and consideration of possible alternate proposals. o The Owners Group expressed concern for the potentially large costs associated with additional pipe stress calculations and the resources for accomplishing such calculations. o RG&E suggested that new direction might result from re-examining the relationship between calculational effort and resultant modifications for the high energy piping outside containment. It is believed that the modifications did not result from the calculated results. RG&E will follow-up on this view.

RG&E agreed, at the conclusion of the meeting, to provide their schedule for the resolution of Topic III 5.A within a few weeks.

K. N. Jabbour

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Systematic Evaluation Program Branch

Division of Operating Reactors

Attachment:

List of Attendees

cc: See next page

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## LIST OF ATTENDEES

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#### MPR Assoc.

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## MEETING SUMMARY DISTRIBUTION

Docket NRC PDR Local PDR ORB#2 Reading NRR Reading H. Denton V. Stello B. Grimes D. Eisenhut D. Davis O. Ziemann P. Check G. Lainus A. Schwencer R. Reid T. Ippolito V. Noonan G. Knighton STS Group J. Shea OELD 01&E(3) H. Smith K. Jabbour S. Hosford R. Fraley, ACRS(16)

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