



UNITED STATES
NUCLEAR REGULATORY COMMISSION
REGION II
101 MARIETTA ST., N.W., SUITE 3100
ATLANTA, GEORGIA 30303

Report No. 50-302/82-26

Licensee: Florida Power Corporation
3201 34th Street, South
St. Petersburg, FL 33733

Facility Name: Crystal River 3

Docket No. 50-302

License No. DPR-72

Inspection at Crystal River site near Crystal River, Florida

Inspector: *E. H. Brooks*
E. H. Brooks

11-18-82
Date Signed

Approved by: *Frank Jape*
F. Jape, Section Chief
Engineering Inspection Branch
Division of Engineering and Technical Programs

11/18/82
Date Signed

SUMMARY

Inspection on November 1-4, 1982

Areas Inspected

This routine, announced inspection involved 25 inspector-hours on site in the areas of pipe supports and restraints and types B and C containment leakage rate testing.

Results

Of the two areas inspected, no violations or deviations were identified.

REPORT DETAILS

1. Persons Contacted

Licensee Employees

- *E. M. Howard, Director, Site Nuclear Operation
- T. C. Lutkehaus, Plant Manager
- *Q. B. DuBois, Assistant Plant Manager
- *K. R. Wilson, Nuclear Licensing Specialist
- *S. E. Primo, Performance Engineer
- *D. W. Bienkowski, Performance Engineer

NRC Resident Inspector

- *T. Stetka

*Attended exit interview

2. Exit Interview

The inspection scope and findings are summarized on November 4, 1982, with those persons indicated in paragraph 1 above. The licensee was informed of the inspection findings listed below. The licensee acknowledged the inspection findings with no dissenting comments.

Inspector Followup Item 302/82-26-01, Snubber augmented in service inspection and schedule, paragraph 6.

3. Licensee Action on Previous Enforcement Matters

Not inspected.

4. Unresolved Items

Unresolved items were not identified during this inspection.

5. Surveillance - Containment Leakage Rate Testing - Types B and C Tests (61720)

The inspector reviewed the licensee's documentation for containment leakage rate testing. Following are the dates for the most recent containment integrated leakage rate test (ILRT) and subsequent types B and C (penetrations and isolation valves) leak tests:

ILRT

June 29-30, 1980

B and CMarch-June 1980
October - December 1981

The inspector reviewed the following surveillance procedures:

SP-179 Containment Leakage Tests - Types "B" and "C" Revision 16, approval date - 10-26-82

SP-181 Containment Air Lock Test - (Semiannual), Approval Date - 10-23-81

The total allowable leakage rate for types B and C tests is 248,656 SCCM (0.6 La). The documented results of type C tests conducted October - December 1981, showed that "as found" leakage exceeded 248,656 SCCM. The "as left" (after repair) was 95,138 SCCM. Although the licensee has provided summary reports of the results of type C tests as required by Appendix J to 10 CFR 50 as well as licensee event reports, containment integrity could not be readily determined due to the complexity of containment isolation valve leakage paths. The licensee has revised surveillance procedure SP-179 to include "as found" and "as left" path leakage in all future isolation valve test reports. This procedure change will indicate the status of containment integrity rather than just a summary of isolation valve leakages.

6. Surveillance - Routine Pipe Supports and Restraints (61729)

The inspector reviewed the licensee's hydraulic seismic restraints (snubber) surveillance and functional testing program. The licensee completed a refueling and maintenance outage during December 1981. During inspection and testing of the snubbers, the licensee identified potentially generic defects such as cracked bushings, cracked radial bearings, and contaminated hydraulic fluid. Functional testing results also showed that approximately 20 percent of the snubbers were not within the acceptance criteria for lockup. The details of the inspection and testing are included in inspection report 50-302/81-29 dated December 10, 1981.

Subsequent to the December 1981 outage the licensee has removed and rebuilt all safety related hydraulic snubbers. The licensee proposed that since all snubbers were rebuilt and functionally tested that the inspection interval be designated as 18 months \pm 25% in accordance with the hydraulic snubber inspection schedule, Table 4.7-4 of the Crystal River Technical Specifications. Following discussions with the NRR staff, the licensee agreed to reduce the interval for visual inspection to 12 months \pm 25% for safety related snubbers inside the reactor building. The Technical Specifications have been changed accordingly (Amendment 49).

During an October 1982 unscheduled, cursory inspection of approximately 50% of the snubbers inside containment, the licensee found 19 snubbers with low levels of hydraulic fluid, including one snubber with an empty gauge glass. The inspector related a concern to the licensee that an additional generic

problem might exist with regard to inadequate seals, and the probability that snubber reservoirs may become empty during operation in an extended 18 month scheduled inspection period. The licensee stated that the type of snubber (Power Piping) installed at Crystal River Nuclear Plant is frequently found to be defective in other plants also, but that hydraulic fluid leakage is common to all snubbers. The inspector stated that the findings of this inspection and the need for augmented inservice inspection of snubbers would be reviewed by the NRC staff.

The results of the next scheduled inspection of all snubbers will be considered in determining the need for augmented inservice inspection and related changes to the surveillance schedule listed in the Technical Specifications. This item is identified as an inspector followup item (302/82-26-01).