

ENCLOSURE

GENERIC ISSUE MANAGEMENT CONTROL SYSTEM
FIRST QUARTER FY-94 UPDATE

OFFICE OF NUCLEAR REGULATORY RESEARCH
FEBRUARY 1994

DATA ELEMENTS

Management and control indicators used in GIMCS are defined as follows:

1. Issue No. - Generic Issue Number
2. Title - Generic Issue Title
3. Identification Date - Date the issue was identified
4. Prioritization Date - The date that the prioritization evaluation was approved by the RES Director
5. Type - Generic Safety (GSI), Licensing (LI), or Regulatory Impact (RI)
6. Priority - High (H) or Medium (M)
7. Task Manager - Name of assigned individual responsible for resolution
8. Office/Division/Branch - The Office, Division, and Branch of the Task Manager who has lead responsibility for resolving the issue.
9. Action Level
 - Active - Technical assistance funds appropriated for resolution and/or Task Manager actively pursuing resolution
 - Inactive - No technical assistance funds appropriated for resolution, Task Manager assigned to more important work, or no Task Manager assigned
 - Resolved - All necessary work has been completed and no additional resources will expended
10. Status - Coded summary as follows:
 - NR - (Nearly-Resolved);
 - 3A - (Resolved with requirements);
 - 3B - (Resolved with No requirements);
 - 5 - (Licensing or Regulatory Impact issue that has been assigned resources for completion)
11. TAC Number - Task Action Control (TAC) number assigned to the issue

12. Resolution Date - Scheduled resolution date for the issue
13. Work Authorization - Who or what authorized work to be done on the issue
14. FIN - Financial identification number assigned to contract (if any) for technical assistance
15. Contractor - Contractor name
16. Contract Title - Contract Title (if contract issue)
17. Work Scope - describes briefly the work necessary to technically resolve and complete the generic issue
18. Status - Describes current status of work
19. Affected Documents - Identifies documents into which the technical resolution will be incorporated
20. Problem/Resolution - Identifies problem areas and describes what actions are necessary to resolve them
21. Milestones - Selected significant milestones:
 - (a) the "original" scheduled dates reflect the original Task Action Plan plus additional milestone dates added during task resolution;
 - (b) changes in the original scheduled dates are listed under "Current";
 - (c) actual completion dates are listed under "Actual"

GENERIC ISSUE MANAGEMENT CONTROL SYSTEMDESCRIPTION

The Generic Issue Management Control System (GIMCS) provides appropriate information necessary to manage the resolution of safety-related as well as non-safety-related generic issues. For the purpose of this management control system, resolved is defined as the point where the staff's final resolution product is issued and a memorandum summarizing the resolution is issued to the EDO. For safety-related issues, this generally occurs when the resolution has been incorporated into one or more of the following documents:

- | | |
|--------------------------------|------------------------|
| (a) Commission Order | (e) Regulatory Guide |
| (b) NRC Policy Statement | (f) Generic Letter |
| (c) Rule | (g) Bulletin |
| (d) Standard Review Plan (SRP) | (h) Information Notice |

GIMCS is part of an integrated system of reports and procedures that is designed to manage generic safety issues through the stages of prioritization and resolution (development of new criteria, management review and approval, public comments, and incorporation into the regulations, as appropriate). The priority evaluation for each issue listed in this report is contained in NUREG-0933, "A Prioritization of Generic Safety Issues." The procedure and criteria for adding new generic issues to the system are outlined in RES Office Letter No. 1. (Prior to the NRC reorganization of April 12, 1987, NRR Office Letter No. 40 was used for identifying new issues.) Guidance for resolving issues is provided in RES Office Letter No. 3.

GIMCS provides the proposed schedules for managing and controlling the resolution of: (1) issues that are ranked HIGH-priority; (2) issues that are ranked MEDIUM-priority; (3) issues for which possible resolutions have been identified for evaluation; (4) issues for which technical resolutions are available (as documented by memorandum, analysis, NUREG, etc.); and (5) other issues designated to receive resources for resolution. Issues ranked as either LOW or DRCP are not allocated resources for resolution and, therefore, are not tracked in GIMCS.

Some prioritized generic issues may not have resources allocated for resolution; these issues are listed in GIMCS as inactive issues. Generally, these will be MEDIUM-priority issues that have no safety deficiency demanding high-priority attention, but have potential for safety improvements or reduction in uncertainty of analysis that may be substantial and worthwhile. Efforts for the resolution of these issues will be planned over the next several years, but on a basis that will not interfere with work on the resolution of HIGH-priority generic issues or other high-priority work. Thus, some (MEDIUM-priority) generic issues may be inactive until such time as resources become available to resolve them. The detailed schedule for resolving generic issues is monitored by GIMCS.

LEGEND

ANPRM	-	Advance Notice of Proposed Rulemaking
BNL	-	Brookhaven National Laboratory
BTP	-	Branch Technical Position
DTR	-	Draft Technical Resolution
EPRI	-	Electric Power Research Institute
FIN	-	Financial Identification Number
FRN	-	Federal Register Notice
FTR	-	Final Technical Resolution
GL	-	Generic letter
GSI	-	Generic Safety Issue
H	-	High-priority GSI
IEB	-	Inspection & Enforcement Bulletin
IN	-	Information Notice
INEL	-	Idaho Nuclear Engineering Laboratory
LI	-	Licensing Issue
M	-	MEDIUM-priority GSI
NR	-	Nearly-Resolved GSI
ORNL	-	Oak Ridge National Laboratory
PNL	-	Pacific Northwest Laboratories
PRA	-	Probabilistic Risk Assessment
RAI	-	Request for Additional Information
RG	-	Regulatory Guide
RI	-	Regulatory Impact
S	-	Subsumed in Another Issue (No.)
SOW	-	Statement of Work
SRP	-	Standard Review Plan
STS	-	Standard Technical Specification
T/A	-	Technical Assistance
TAP	-	Task Action Plan
TBD	-	To be Determined
TI	-	Temporary Instruction
TS	-	Technical Specification
USI	-	Unresolved Safety Issue

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TABLE 1
GENERIC SAFETY ISSUES SCHEDULED FOR RESOLUTION,
ISSUES RESOLVED AND ISSUES INTEGRATED
FY-83 THROUGH FY-94 (1ST QUARTER)

ISSUE TYPE	FY-83				FY-84				FY-85			
	GIs Start +	New Issues -	GIs Resolved =	End	GIs Start +	New Issues -	GIs Resolved =	End	GIs Start +	New Issues -	GIs Resolved =	End
USI	16	0	2	14	14	0	2	12	12	0	0	12
HIGH	24	2	0	26	26	1	1	26	22*	41	6	57
MEDIUM	31	2	0	33	33	4	3	34	28*	1	10	19
NR	20	3	4	19	19	5	9	15	15	11	7	19
TOTAL:	91	7	6	92	92	10	15	87	77	53	23	107

ISSUE TYPE	FY-86				FY-87					
	GIs Start +	New Issues -	GIs Resolved =	End	ISSUE TYPE	GIs Start +	New Issues -	GIs Resolved =	GIs Integrated =	End
USI	12	0	1	11	USI	11	0	2	0	9
HIGH	57	(16)*	3	38	HIGH	38	4	3	7	32
MEDIUM	19	7	2	24	MEDIUM	24	1	4	5	16
NR	19	(3)*	3	13	NR	13	0	1	1	11
TOTAL:	107	(12)*	9	86	TOTAL:	86	5	10	13	68

*Extensive revisions to Human Factors issues resulted in priority changes.

TABLE 1 (Continued)
GENERIC SAFETY ISSUES SCHEDULED FOR RESOLUTION,
ISSUES RESOLVED AND ISSUES INTEGRATED
FY-83 THROUGH FY-94 (1ST QUARTER)

ISSUE TYPE	FY-88				
	GIs Start	+ New Issues	- GIs Resolved	- GIs Integrated	= End
USI	9	0	5	0	4
HIGH	32	1	6	3	24
MEDIUM	16	2	2	3	13
NR	11	1	3	0	9
TOTAL:	68	4	16	6	50

ISSUE TYPE	FY-89				
	GIs Start	+ New Issues	- GIs Resolved	- GIs Integrated	= End
USI	4	0	4	0	0
HIGH	24	1	9	0	16
MEDIUM	13	1	3	1	10
NR	9	0	2	0	7
TOTAL:	50	2	18	1	33

ISSUE TYPE	FY-90				
	GIs Start	+ New Issues	- GIs Resolved	- GIs Integrated	= End
HIGH	16	0	2	0	14
MEDIUM	10	1	2	0	9
NR	7	0	3	0	4
TOTAL:	33	1	7	0	27

ISSUE TYPE	FY-91				
	GIs Start	+ New Issues	- GIs Resolved	- GIs Integrated	= End
HIGH	14	2	2	0	14
MEDIUM	9	3	1	0	11
NR	4	1	1	0	4
TOTAL:	27	6	4	0	29

ISSUE TYPE	FY-92				
	GIs Start	+ New Issues	- GIs Resolved	- GIs Integrated	= End
HIGH	14	0	4	0	10
MEDIUM	11	1	2	0	10
NR	4	2	1	0	5
TOTAL:	29	3	7	0	25

ISSUE TYPE	FY-93				
	GIs Start	+ New Issues	- GIs Resolved	- GIs Integrated	= End
HIGH	10	0	7	0	3
MEDIUM	10	0	3	0	7
NR	5	2	0	0	7
TOTAL:	25	2	10	0	17

TABLE 1 (Continued)
GENERIC SAFETY ISSUES SCHEDULED FOR RESOLUTION,
ISSUES RESOLVED AND ISSUES INTEGRATED
FY-83 THROUGH FY-94(1ST QUARTER)

FY-94

ISSUE TYPE	GIs Start	+	New Issues	-	GIs Resolved	-	GIs Integrated =	End
HIGH	3		1		0		0	4
MEDIUM	7		0		1		0	6
NR	7		0		1		0	6
TOTAL:	17		1		2		0	16

Table 1A
SUMMARY OF GENERIC SAFETY ISSUES SCHEDULED FOR RESOLUTION,
ISSUES RESOLVED, AND ISSUES INTEGRATED
CUMULATIVE TOTALS
FY-83 THROUGH FY-94 (1ST QUARTER)

ISSUE TYPE	GIs Start	+	New Issues	= Sub-Total	-	GIs Resolved	-	GIs Integrated**	= Remainder
USI	16		0	16		16		0	0
HIGH	24		33*	57		43		10	4
MEDIUM	31		17	48		33		9	6
NR	20		22*	42		35		1	6
TOTAL:	91		72	163		127		20	16

* Extensive revisions to Human Factors issues resulted in priority changes in FY-85 and FY-86.

** GIs Integrated

FY-87:
 Total (13) Issues 48, 49, and A-30 into Issue 128
 Issue 65 into Issue 23
 Issues 68; 122.1.a; 122.1.b; 122.1.c; and 125.II.1.b into Issue 124
 Issues I.B.1.1(6) and I.B.1.1(7) into Issue 75
 Issue B-6 into Issue 119.1
 Issue 67.7 into 135

FY-88:
 Total (6) Issue 77 into A-17
 Issues I.D.5(5), II.B.5(1), II.B.5(2), II.B.5(3), and II.F.5 were integrated into the Research Activities Program and were reclassified as Licensing Issues.

FY-89:
 Total (1) Issue 131 was integrated into the External Events IPE Program.

TABLE 2
NUMBER OF GENERIC SAFETY ISSUES RESOLVED
AND SCHEDULED FOR RESOLUTION
FY-83 THROUGH FY-94 (1ST QUARTER)

RESOLVED

ISSUE PRIORITY	FY-83	FY-84	FY-85	FY-86	FY-87	FY-88	FY-89	FY-90	FY-91	FY-92	FY-93	FY-94	TOTAL
USI	2	2	0	1	2	5	4	0	0	0	0	0	16
HIGH	0	1	6	3	3	6	9	2	2	4	7	0	43
MEDIUM	0	3	10	2	4	2	3	2	1	2	3	1	33
NR	4	9	7	3	1	3	2	3	1	1	0	1	35
Total:	6	15	23	9	10	16	18	7	4	7	10	2	127

SCHEDULED FOR RESOLUTION

ISSUE PRIORITY	FY-94	FY-95	FY-96	FY-97	TBD	TOTAL
HIGH	1	1	1	0	1	4
MEDIUM	2	3	0	1	0	6
NR	4	0	0	0	2	6
Total:	7	4	1	1	3	16

TABLE 3
GENERIC SAFETY ISSUES RESOLVED BY FISCAL YEAR

Issue Number	Title	Priority	Resolution Product	Date Approved for Resolution	Date Resolved
<u>FY-83</u>					
A-11	Reactor Vessel Materials Toughness	USI	GL 82-26	01/79	10/82
A-16	Steam Effects on BWR Core Spray Distribution	NR	MPA D-12	NRR-OP FY83	03/29/83
A-39	Determination of Safety Relief Valve (SRV) Pool Dynamic Loads and Temperature Limits for BWR Containment	USI	SRP Revision	01/79	10/82
B-53	Load Break Switch	NR	SRP Revision	NRR-OP FY83	07/28/83
II.E.5.1	(B&W) Design Evaluation	NR	No. Req.	NRR-OP FY83	03/21/83
IV.C.1	Extend Lessons Learned From TMI to Other NRC Programs	NR	No. Req.	NRR-OP FY83	04/15/83
<u>FY-84</u>					
12	BWR Jet Pump Integrity	Medium	No. Req.	NRR-OP FY83	09/25/84
20	Effects of Electromagnetic Pulse on Nuclear Plant Systems	NR	NUREG/CR-3069	NRR-OP FY83	11/15/83
40	Safety Concerns Associated With Breaks in the BWR Scram System	NR	MPA B-65	07/18/83	12/27/83
45	Inoperability of Instruments Due to Extreme Cold Weather	NR	SRP Revision	09/08/83	03/23/84

TABLE 3 (Continued)
GENERIC SAFETY ISSUES RESOLVED BY FISCAL YEAR

<u>Issue Number</u>	<u>Title</u>	<u>Priority</u>	<u>Resolution Product</u>	<u>Date Approved for Resolution</u>	<u>Date Resolved</u>
<u>FY-84 (Cont.)</u>					
50	Reactor Vessel Level Instrumentation in BWRs	NR	MPA F-26	07/28/83	09/06/84
69	Make-Up Nozzle Cracking in B&W Plants	NR	MPA B-43	12/06/83	09/27/84
A-1	Water Hammer	USI	SRP Revision	01/79	03/15/84
A-12	Steam Generator and Reactor Coolant Pump Supports	USI	SRP Revision	01/79	10/83
B-10	Behavior of BWR Mark III Containments	High	SRP Revision	NRR-OP FY83	09/10/84
B-26	Structural Integrity of Containment Penetrations	Medium	No Req.	NRR-OP FY83	09/27/84
B-60	Loose Parts Monitoring Systems (LPMS)	NR	GL	NRR-OP FY83	09/25/84
I.A.1.4	Long-Term Upgrading of Operating Personnel	NR	New Rule	NRR-OP FY83	01/01/84
II.A.1	Siting Policy Reformulation	Medium	No Req.	NRR-OP FY83	09/20/84
II.E.5.2	(B&W) Reactor Transient Response Task Force	NR	NUREG-0667	NRR-OP FY83	09/28/84
III.D.2.5	Offsite Dose Calculation Manual	NR	NUREG/ CR-3332	NRR-OP FY83	01/17/84

TABLE 3 (Continued)
GENERIC SAFETY ISSUES RESOLVED BY FISCAL YEAR

Issue Number	Title	Priority	Resolution Product	Date Approved for Resolution	Date Resolved
<u>FY-85</u>					
22	Inadvertent Boron Dilution Events	NR	GL 85-05	11/05/82	10/15/84
A-41	Long Term Seismic Program	Medium	No Req.	NRR-OP FY83	10/10/84
B-19	Thermal-Hydraulic Stability	NR	GL	01/03/85	05/21/85
B-54	Ice Condenser Containments	Medium	NUREG/ CR-4001	NRR-OP FY83	10/22/84
B-58	Passive Mechanical Failures	Medium	No Req.	NRR-OP FY83	07/09/85
C-11	Assessment of Failure and Reliability of Pumps and Valves	Medium	No Req.	NRR-OP FY83	07/09/85
I.A.2.2	Training and Qualifications of Operating Personnel	High	Policy Statement (No Req.)	NRR-OP FY83	06/24/85
I.A.2.6(4)	Operator Workshops	Medium	No Req.	NRR-OP FY83	09/25/85
I.A.2.7	Accreditation of Training Institutions	Medium	Policy Statement (No Req.)	NRR-OP FY83	06/24/85
I.A.3.4	Licensing of Additional Operations Personnel	Medium	Policy Statement (No Req.)	NRR-OP FY83	02/12/85
I.G.2	Scope of Test Program	Medium	No Req.	NRR-OP FY83	10/05/84

TABLE 3 (Continued)
GENERIC SAFETY ISSUES RESOLVED BY FISCAL YEAR

Issue Number	Title	Priority	Resolution Product	Date Approved for Resolution	Date Resolved
<u>FY-85 (Cont.)</u>					
II.B.6	Risk Reduction For Operating Reactors at Sites With High Population Densities	High	No Req.	NRR-OP FY83	09/25/85
II.B.8	Rulemaking Proceedings on Degraded Core Accidents (a) Hydrogen Rule (b) Severe Accidents	High	- Rule/ Policy Statement	- NRR-OP FY83 NRR-OP FY83	- 07/19/85 08/12/85
II.C.1	Interim Reliability Evaluation Program	High	No Req.	NRR-OP FY83	07/09/85
II.C.2	Continuation of Interim Reliability Evaluation Program	High	No Req.	NRR-OP FY83	09/25/85
II.E.2.2	Research on Small-Break LOCAs and Anomalous Transients	Medium	No Req.	NRR-OP FY83	07/25/85
III.A.1.3(2)	Maintain Supplies of Thyroid-Blocking Agent for Public	NR	Policy Statement	NRR-OP FY83	08/15/85
III.A.3.4	Nuclear Data Link	Medium	No Req.	NRR-OP FY83	06/26/85
III.D.2.3(1)	Develop Procedures to Discriminate Between Sites/Plants	NR	ESRP Revision	NRR-OP FY83	08/28/85
III.D.2.3(2)	Discriminate Between Sites and Plants that Require Consideration of Liquid Pathway Interdiction Techniques	NR	ESRP Revision	NRR-OP FY83	08/28/85
III.D.2.3(3)	Establish Feasible Method of Pathway Interdiction	NR	ESRP Revision	NRR-OP FY83	08/28/85

TABLE 3 (Continued)
GENERIC SAFETY ISSUES RESOLVED BY FISCAL YEAR

Issue Number	Title	Priority	Resolution Product	Date Approved for Resolution	Date Resolved
<u>FY-85 (Cont.)</u>					
III.D.2.3(4)	Prepare a Summary Assessment	NR	ESRP Revision	NRR-OP FY83	08/28/85
IV.E.5	Assess Currently Operating Plants	High	No Req.	NRR-OP FY83	09/25/85
<u>FY-86</u>					
3	Setpoint Drift in Instrumentation	NR	Reg Guide Rev. (No Req.)	NRR-OP FY83	05/19/86
14	PWR Pipe Cracks	NR	No Req.	NRR-OP FY83	10/04/85
36	Loss of Service Water	NR	SRP Revision (No Req.)	02/15/84	05/13/86
61	SRV Discharge Line Break Inside the Wetwell Airspace of BWR Mark I and Mark II Containments	Medium	No Req.	11/30/83	08/08/86
A-43	Containment Emergency Sump Performance	USI	SRP Revision (Req.)	01/79	10/85
I.C.9	Long-Term Plan for Upgrading of Procedures	Medium	No Req.	NRR-OP FY83	06/07/85
III.D.3.1	Radiation Protection Plans	High	No Req.	NRR-OP FY83	05/19/86
HF1.2	Engineering Expertise on Shift	High	No Req.	10/01/84	10/28/85
HF1.3	Guidance on Limits and Conditions of Shift Work	High	No Req.	10/01/84	06/26/86

TABLE 3 (Continued)
GENERIC SAFETY ISSUES RESOLVED BY FISCAL YEAR

Issue Number	Title	Priority	Resolution Product	Date Approved for Resolution	Date Resolved
<u>FY-87</u>					
91	Main Crankshaft Failures in Transamerica Delaval Diesel Generators	NR	NUREG-1216 (No Req.)	07/85	09/87
A-46	Seismic Qualification of Equipment in Operating Plants	USI	GL 87-02 (Req.)	02/81	02/87
A-49	Pressurized Thermal Shock	USI	Rule/Reg. Guide 1.154 (Req.)	12/81	02/87
I.A.2.6(1)	Long-Term Upgrading of Training and Qualifications - Revise Reg. Guide 1.8	High	Reg. Guide 1.8 (Req.)	10/82	05/87
I.A.3.3	Requirement for Operator Fitness	High	No Req.	12/82	01/87
I.A.4.2(1)	Research on Training Simulators	High	Reg. Guide 1.149, Rev. 1 (Req.)	10/84	05/87
I.B.1.1	<u>Organization and Management Long-Term Improvements</u>	-	-	-	-
I.B.1.1(1)	Prepare Draft Criteria	Medium	No Req.	12/82	01/87
I.B.1.1(2)	Prepare Commission Paper	Medium	No Req.	12/82	01/87
I.B.1.1(3)	Issue Requirements for the Upgrading of Management and Technical Resources	Medium	No Req.	12/82	01/87
I.B.1.1(4)	Review Responses to Determine Acceptability	Medium	No Req.	12/82	01/87

TABLE 3 (Continued)
GENERIC SAFETY ISSUES RESOLVED BY FISCAL YEAR

Issue Number	Title	Priority	Resolution Product	Date Approved for Resolution	Date Resolved
<u>FY-88</u>					
86	Long Range Plan for Dealing with Stress Corrosion Cracking in BWR Piping	NR	NUREG-0313, Rev. 2, GL 88-01 (Req.)	10/84	01/88
93	Steam Binding of Auxiliary Feedwater Pumps	High	GL 88-03 (No Req.)	10/84	02/88
I.D.4	Control Room Design Standard	Medium	No Req.	NRR-OP FY83	03/88
II.E.4.3	(Containment) Integrity Check	High	NUREG-1273 (No. Req.)	NRR-OP FY83	03/88
B-5	Ductility of Two-Way Slabs and Shells and Buckling Behavior of Steel Containments	Medium	No Req.	NRR-OP FY83	04/88
HF8	Maintenance and Surveillance Program	High	Policy Statement (No Req.)	03/85	05/88
I.A.4.2(4)	Review Simulators for Conformance	High	Rule (Req.)	NRR-OP FY83	05/88
A-44	Station Blackout	USI	Rule/Reg. Guide 1.155 (Req.)	01/79	06/88
43	Reliability of Air Systems	High	GL 88-14 (Req.)	12/87	09/88

TABLE 3 (Continued)
GENERIC SAFETY ISSUES RESOLVED BY FISCAL YEAR

Issue Number	Title	Priority	Resolution Product	Date Approved for Resolution	Date Resolved
<u>FY-88 (Cont.)</u>					
66	Steam Generator Requirements	NR	NUREG-0844 (No Req.)	11/83	09/88
102	Human Error in Events Involving Wrong Unit or Wrong Train	NR	NUREG-1192 (No Req.)	02/85	09/88
125.II.7	Reevaluate Provision to Automatically Isolate Feedwater from Steam Generator During a Line Break	High	NUREG-1332 (No Req.)	09/86	09/88
A-3,4,5	Steam Generator Tube Integrity	USI	NUREG-0844 (No Req.)	01/79	09/88
A-45	Shutdown Decay Heat Removal Requirements	USI	NUREG-1289 (No Req.)	02/81	09/88
<u>FY-89</u>					
51	Proposed Requirements for Improving Reliability of Open Cycle Service Water Systems	Medium	GL 89-13 (Req.)	06/83	08/89
82	Beyond Design Bases Accidents in Spent Fuel Pools	Medium	NUREG-1353 (No Req.)	12/07/83	04/89
99	RCS/RHR Suction Line Interlocks on PWRs	High	GL 88-17 (Req.)	08/85	11/88
101	BWR Water Level Redundancy	High	GL 89-11 (Req.)	05/06/85	06/89

TABLE 3 (Continued)
GENERIC SAFETY ISSUES RESOLVED BY FISCAL YEAR

Issue Number	Title	Priority	Resolution Product	Date Approved for Resolution	Date Resolved
<u>FY-89 (Cont.)</u>					
115	Enhancement of the Reliability of Westinghouse Solid State Protection System	High	NUREG-1341 (No Req.)	07/07/86	04/89
122.2	Initiating Feed-and-Bleed	High	No Req.	01/86	04/89
124	Auxiliary Feedwater System Reliability	NR	2 Plant-Specific Backfits (Req.)	02/86	01/89
125.I.3	SPDS Availability	NR	GL 89-06 (No. Req.)	05/06/88	04/89
134	Rule on Degree and Experience Requirements for Senior Operators	High	Policy Statement (No Req.)	01/86	08/89
A-17	Systems Interaction	USI	NUREG-1174 (No Req.)	01/79	08/89
A-40	Seismic Design Criteria	USI	SRP Revisions (Req.)	01/79	09/89
A-47	Safety Implications of Control Systems	USI	GL 89-19 (Req.)	02/81	08/89
A-48	Hydrogen Control Measures and Effects of Hydrogen Burns on Safety Equipment	USI	Rules (Req.)	02/81	04/89
HF1.1	Shift Staffing	High	Reg. Guide 1.114, Rev. 2 (Req.)	10/01/84	05/89

TABLE 3 (Continued)
GENERIC SAFETY ISSUES RESOLVED BY FISCAL YEAR

Issue Number	Title	Priority	Resolution Product	Date Approved for Resolution	Date Resolved
<u>FY-89 (Cont.)</u>					
HF4.1	Inspection Procedure for Upgraded Emergency Operating Procedures	High	IN 86-64 (No Req.)	10/01/84	10/88
I.F.1	Expand QA List	High	No Req.	NRR-OP FY83	01/89
II.C.4	Reliability Engineering	High	No Req.	NRR-OP FY83	10/88
II.E.6.1	Test Adequacy Study	Medium	GL 89-10 (Req.)	NRR-OP FY83	06/89
<u>FY-90</u>					
70	PORV and Block Valve Reliability	Medium	GL 90-06	05/14/84	06/90
75	Generic Implications of ATWS Events at the Salem Nuclear Power Plant	NR	Req.	10/19/83	05/90
84	CE PORVs	NR	SECY-90-232 (No Req.)	02/27/85	06/90
94	Additional Low-Temperature Overpressure Protection for LWRs	High	GL 90-06 (Req.)	07/23/85	06/90
103	Design for Probable Maximum Precipitation	NR	GL 89-22 (Req.)	09/04/85	11/89

TABLE 3 (Continued)
GENERIC SAFETY ISSUES RESOLVED BY FISCAL YEAR

<u>Issue Number</u>	<u>Title</u>	<u>Priority</u>	<u>Resolution Product</u>	<u>Date Approved for Resolution</u>	<u>Date Resolved</u>
<u>FY-90 (Cont.)</u>					
A-29	Nuclear Power Plant Design for the Reduction of Vulnerability to Industrial Sabotage	Medium	No Req.	NRR-OP FY83	10/89
C-8	Main Steam Line Isolation Valve Leakage Control Systems	High	No Req.	NRR-OP FY83	03/90
<u>FY-91</u>					
128	Electrical Power Reliability	High	GL 91-06, GL 91-11 (Req.)	11/28/86	09/91
130	Essential Service Water System Failures at Multiplant Sites	High	GL 91-13 (Req.)	03/10/87	09/91
135	Steam Generator and Steam Line Overfill	Medium	No Req.	05/27/86	03/91
II.J.4.1	Revise Deficiency Report Requirements	NR	Rule (Req.)	NRR-OP FY83	07/91
<u>FY-92</u>					
29	Bolting Degradation or Failure in Nuclear Power Plants	High	No Req.	NRR-OP FY-83	10/91

TABLE 3 (Continued)
GENERIC SAFETY ISSUES RESOLVED BY FISCAL YEAR

<u>Issue Number</u>	<u>Title</u>	<u>Priority</u>	<u>Resolution Product</u>	<u>Date Approved for Resolution</u>	<u>Date Resolved</u>
<u>FY-92 (Cont.)</u>					
73	Detached Thermal Sleeves	NR	NUREG/ CR-6010 No. Req.	08/20/91	09/92
79	Unanalyzed Reactor Vessel Thermal Stress During Natural Convection Cooldown	Medium	GL 92-02 No. Req.	NRR-OP FY-84	05/92
87	Failure of HPCI Steam Line Without Isolation	High	No Req.	09/26/85	12/91
113	Dynamic Qualification Testing of Large Bore Hydraulic Snubbers	High	No Req.	07/02/87	08/92
121	Hydrogen Control for Large, Dry PWR Containments	High	No Req.	09/26/85	03/92
151	Reliability of ATWS Recirculation Pump Trip in BWRs	Medium	No Req.	08/27/91	09/92
<u>FY-93</u>					
105	Interfacing Systems LOCA at LWRs	High	No Req.	06/11/85	06/93
120	On-Line Testability of Protection Systems	Medium	No Req.	11/23/90	03/93
142	Leakage Through Electrical Isolators	Medium	No Req.	06/20/90	03/93

TABLE 3 (Continued)
GENERIC SAFETY ISSUES RESOLVED BY FISCAL YEAR

Issue Number	Title	Priority	Resolution Product	Date Approved for Resolution	Date Resolved
<u>FY-93 (Cont.)</u>					
143	Availability of Chilled Water Systems and Room Cooling	High	No Req.	03/29/91	09/93
153	Loss of Essential Service Water in LWRs	High	No Req.	03/29/91	06/93
B-56	Diesel Reliability	High	Reg Guides: 1.9, Rev. 3; 1.160 (Req.)	NRR-OP FY83	06/93
HF4.4	Guidelines for Upgrading Other Procedures	High	No Req.	10/01/84	07/93
HF5.1	Local Control Stations	High	No Req.	10/01/84	06/93
HF5.2	Review Criteria for Human Factors Aspects of Advanced Controls and Instrumentation	High	No Req.	10/01/84	06/93
I.D.3	Safety System Status Monitoring	Medium	No Req.	NRR-OP FY83	09/93
<u>FY-94</u>					
106	Piping and Use of Highly Combustible Gases in Vital Areas	Medium	No Req.	11/03/87	11/93
I.D.5(3)	On-Line Reactor Surveillance Systems	NR	No Req.	NRR-OP FY83	11/93

TABLE 4
16 GSIs SCHEDULED FOR RESOLUTION (BY FISCAL YEAR)

PRIORITY	FY-94	FY-95	FY-96	FY-97	FY-98	FY-99	TBD	TOTAL
HIGH	II.H.2	23	15	-	-	-	165	4
MEDIUM	57 78	24 B-17 B-61	-	B-55	-	-	-	6
NR	83 145 155.1 B-64	-	-	-	-	-	166 168	6
TOTAL:	7	4	1	1	0	0	3	16

TABLE 4A
GENERIC SAFETY ISSUES SCHEDULED FOR RESOLUTION

Issue Number	Title	Lead Office/ Division/Branch	Priority	Date Approved for Resolution	Resolution Date at end of FY-93	Current Resolution Date
15	Radiation Effects on Reactor Vessel Supports	RES/DSIR/EIB	H	02/89	03/96	03/96
23	Reactor Coolant Pump Seal Failures	RES/DSIR/EIB	H	NRR-OP FY83	12/94	03/95
165	Spring-Actuated Safety and Relief Valve Reliability	RES/DSIR/EIB	H	11/26/93	NA	TBD
II.H.2	Obtain Technical Data on the Conditions Inside the TMI-2 Containment Structure	RES/DSR/AEB	H	NRR-OP FY83	TBD	02/94
24	Automatic Emergency Core Cooling System Switch to Recirculation	RES/DSIR/SAIB	M	07/23/91	08/94	01/95
57	Effects of Fire Protection System Actuation on Safety-Related Equipment	RES/DSIR/SAIB	M	06/08/88	12/93	02/94
78	Monitoring of Fatigue Transient Limits for Reactor Coolant System	RES/DSIR/EIB	M	07/10/92	TBD	04/94
B-17	Criteria for Safety-Related Operator Actions	RES/DSR/HFB	M	03/22/82	09/94	12/94
B-55	Improve Reliability of Target Rock Safety Relief Valves	RES/DSIR/EIB	M	NRR-OP FY83	TBD	08/97
B-61	Allowable ECCS Equipment Outage Periods	RES/DSR/PRAB	M	NRR-OP	12/94	05/95
83	Control Room Habitability	RES/DSIR/SAIB	NR	08/83	12/93	04/94

Table 4A (Continued)
GENERIC SAFETY ISSUES SCHEDULED FOR RESOLUTION

Issue Number	Title	Lead Office/ Division/Branch	Priority	Date Approved for Resolution	Resolution Date at end of FY-93	Current Resolution Date
145	Improve Surveillance and Startup Testing Programs	RES/DSIR/PRAB	NR	02/11/92	01/94	02/94
155.1	More Realistic Source Term Assumptions	RES/DSIR/SAIB	NR	02/26/92	01/94	07/94
166	Adequacy of Fatigue Life of Metal Components	NRR	NR	04/01/93	TBD	TBD
168	Environmental Qualification of Electrical Equipment	NRR	NR	04/01/93	TBD	TBD
B-64	Decommissioning of Nuclear Reactors	NMSS/LLWM/LLRB	NR	NRR-OP FY84	10/93	03/94

Total: 16

TABLE 5
NUMBER OF GENERIC ISSUES PRIORITIZED FROM FY-83 THROUGH THE 1ST QUARTER OF FY-94

<u>ISSUE TYPE</u>	<u>FY-83</u>	<u>FY-84</u>	<u>FY-85</u>	<u>FY-86</u>	<u>FY-87</u>	<u>FY-88</u>	<u>FY-89</u>	<u>FY-90</u>	<u>FY-91</u>	<u>FY-92</u>	<u>FY-93</u>	<u>FY-94</u>	<u>Total</u>
Issues Identified To Be Prioritized	56	19	54	45*	6	3	38	3	29	7	5	0	265
Issues Identified To Be Reprioritized	19	2	0	1	1	2	0	0	0	0	0	0	25
Total:	75	21	54	46	7	5	38	3	29	7	5	0	290
<hr/>													
<u>New Issues (Entered into GIMCS)</u>													
High	2	1	41	6	4	1	1	0	2	0	0	1	59
Medium	2	4	1	1	1	2	0	1	3	2	0	0	17
Nearly-Resolved	3	5	6	1	0	1	0	0	1	2	2	0	21
Subtotal:	7	10	48	8	5	4	1	1	6	4	2	1	97
Resolved	4	0	1	0	0	1	1	1	0	2	1	0	11
Low	1	4	0	2	1	0	0	2	0	4	3	0	17
Drop	1	4	4	6	13	9	2	8	5	24	6	0	82
RI/LI	0	0	4	6	0	2	33	1	1	3	0	0	50
Integrated	8	2	3	6	0	1	1	3	0	5	0	0	29
Total Issues Prioritized:	21	20	60	28	19	17	38	16	12	42	12	1	286
Remaining Issues To Be Prioritized or Reprioritized:	+54	+1	-6	+18	-12	-12	0	-13	+17	-35	-7	-1	4

* Extensive Revisions to Human Factors Issues Resulted in Priority Changes.

TABLE 6
GENERIC ISSUES PRIORITIZED AS OF 1ST QUARTER FY-94

Action Item/ Issue No.	Title	Identification Date	Date Prioritized	Current Priority
<u>FY-83</u>				
31	Natural Circulation Cooldown	09/82	07/83	S(I.C.1)
32	Flow Blockage in Essential Equipment Caused by Corbicula	09/82	05/83	S(51)
33	Correcting Atmospheric Dump Valve Opening Upon Loss of Integrated Control System Power	09/82	08/82	S(A-47)
39	Potential for Unacceptable Interaction Between the CRD System and Non-Essential Control Air System	09/82	07/82	S(25)
40	Safety Concerns Associated with Pipe Breaks in the BWR Scram System	09/82	07/83	NR
41	BWR Scram Discharge Volume Systems	09/82	07/83	RESOLVED
42	Combination Primary/Secondary System LOCA	09/82	04/83	S(18)
45	Inoperability of Instrumentation Due to Extreme Cold Weather	09/82	09/83	NR
46	Loss of 125 Volt DC Bus	09/82	02/83	S(76)
47	Loss of Off-Site Power	09/82	04/83	RESOLVED
50	Reactor Vessel Level Instrumentation in BWRs	09/82	07/83	NR
51	Proposed Requirements for Improving the Reliability of Open Cycle Service Water Systems	09/82	06/83	MEDIUM
52	SSW Flow Blockage by Blue Mussels	09/82	05/83	S(51)
56	Abnormal Transient Operating Guidelines as Applied to a Steam Generator Overfill Event	09/82	02/83	S(A-45/I.D.1)
58	Inadvertent Containment Flooding	09/82	08/83	DROP
64	Identification of Protection System Instrument Sensing Lines	10/82	02/83	RESOLVED
65	Probability of Core-Melt Due to Component Cooling Water System Failures	02/83	07/83	HIGH
77	Flooding of Safety Equipment Compartments by Back-Flow Through Floor Drains	06/83	09/83	HIGH
79	Unanalyzed Reactor Vessel Thermal Stress During Natural Convection Cooldown	06/83	07/83	MEDIUM
D-1	Advisability of a Seismic Scram	09/82	06/83	LOW
IV.E.2	Plan for Early Resolution of Safety Issues	09/82	06/83	RESOLVED

TABLE 6 (Continued)
GENERIC ISSUES PRIORITIZED AS OF 1ST QUARTER FY-94

Action Item/ Issue No.	Title	Identification Date	Date Prioritized	Current Priority
<u>FY-84</u>				
34	RCS Leak	09/82	02/84	DROP
35	Degradation of Internal Appurtenances in LWRs	09/82	02/84	LOW
36	Loss of Service Water	09/82	02/84	NR
43	Contamination of Instrument Air Lines	09/82	11/83	DROP
44	Failure of Saltwater Cooling System	09/82	10/83	S(43)
48	LCO for Class IE Vital Instrument Buses in Operating Reactors	09/82	10/83	NR
49	Interlocks and LCOs for Redundant Class IE Tie Breakers	09/82	07/84	MEDIUM
53	Consequences of a Postulated Flow Blockage Incident in a BWR	09/82	09/84	DROP
60	Lamellar Tearing of Reactor Systems Structural Supports	10/82	11/83	S(A-12)
61	SRV Line Break Inside the BWR Wetwell Airspace of Mark I and II Containments	10/82	11/83	MEDIUM
66	Steam Generator Requirements	06/83	11/83	NR
68	Postulated Loss of Auxiliary Feedwater System Resulting from Turbine-Driven Auxiliary Feedwater Pump Steam Supply Line Rupture	06/83	04/84	HIGH
69	Make-up Nozzle Cracking in B&W Plants	06/83	12/83	NR
70	PORV and Block Valve Reliability	06/83	05/84	MEDIUM
75	Generic Implications of ATWS Events at the Salem Nuclear Plant	06/83	10/83	NR
80	Pipe Break Effects on Control Rod Drive Hydraul- ic Lines in the Drywells of BWR Mark I and II Containments	06/83	01/84	LOW
82	Beyond Design Basis Accidents in Spent Fuel Pools	08/83	12/83	MEDIUM
90	Technical Specifications for Anticipatory Trips	02/84	08/84	LOW
92	Fuel Crumbling During LOCA	04/83	07/84	LOW
B-65	Iodine Spiking	09/82	06/84	DROP

TABLE 6 (Continued)
GENERIC ISSUES PRIORITIZED AS OF 1ST QUARTER FY-94

Action Item/ Issue No.	Title	Identification Date	Date Prioritized	Current Priority
<u>FY-85</u>				
37	Steam Generator Overfill and Combined Primary and Secondary Blowdown	09/82	05/85	S(A-47)
54	Valve Operator-Related Events Occurring During 1978, 1979, and 1980	09/82	06/85	S(II.E.6.1)
55	Failure of Class IE Safety-Related Switchgear Circuit Breakers	09/82	03/85	DROP
59	Technical Specification Requirements for Plant Shutdown	10/82	02/85	RI
67	Steam Generator Staff Actions	06/83	03/85	MEDIUM
81	Potential Safety Problems Associated With Locked Doors and Barriers in Nuclear Power Plants	11/83	10/84	DROP
83	Control Room Habitability	11/83	06/84	NR
84	CE PORVs	11/83	02/85	NR
85	Reliability of Vacuum Breakers Connected to Steam Discharge Lines Inside BWR Containments	11/83	07/85	DROP
86	NRC Pipe Cracking Review Group Study	12/83	10/84	NR
87	Failure of HPCI Steam Line Without Isolation	01/84	09/85	HIGH
91	Transamerica Delaval Emergency Diesel Generator Main Crankshaft Failure	03/84	07/85	NR
93	Steam Binding of Auxiliary Feedwater Pumps	07/84	10/84	HIGH
94	Additional Low Temperature Overpressure Protection For Light Water Reactors	08/84	07/85	HIGH
98	CRD Accumulator Check Valve Leakage	09/84	02/85	DROP
99	RCS/RHR Suction Line Interlocks on PWRs	09/84	08/85	HIGH
101	BWR Water Level Redundancy	09/84	05/85	HIGH
102	Human Error in Events Involving Wrong Unit or Wrong Train	09/84	02/85	S(HF-02)
103	Design For Probable Maximum Precipitation	10/84	09/85	NR
105	Interfacing Systems LOCA at LWRs	10/84	06/85	HIGH
108	BWR Suppression Pool Temperature Limits	12/84	02/85	RI(LOW)
119	Piping Review Committee Recommendations	07/85	09/85	RI(NR)
121	Hydrogen Control For Large Dry PWR Containments	08/85	09/85	HIGH

TABLE 6 (Continued)
GENERIC ISSUES PRIORITIZED AS OF 1ST QUARTER FY-94

Action Item/ Issue No.	Title	Identification Date	Date Prioritized	Current Priority
<u>FY-85 (Cont'd)</u>				
B-19	Thermal-Hydraulic Stability	02/83	01/85	NR
B-50	Post Operating Basis Earthquake Inspection	02/83	04/85	RI (LOW)
B-59	N-1 Loop Operation in BWRs and PWRs	02/83	06/85	RESOLVED
HF-01	Human Factor Program Plan (HFPP with 24 subtasks)	08/83	10/84	HIGH
HF-02	Maintenance and Surveillance Program Plan (MSPP with 10 subtasks)	04/84	03/85	HIGH
<u>FY-86</u>				
21	Vibration Qualification of Equipment	03/83	05/86	DROP
30	Potential Generator Missiles - Generator Rotor Retaining Rings	09/82	10/85	DROP
74	Reactor Coolant Activity Limits for Operating Reactors	06/83	05/86	DROP
97	PWR Reactor Cavity Uncontrolled Exposures	09/84	10/85	S(III.D.3.1)
111	Stress Corrosion Cracking of Pressure Boundary Ferritic Steels in Selected Environments	01/85	11/85	LI
112	Westinghouse RPS Surveillance Frequencies and Out-of-Service Times	01/85	10/85	RI(R)
114	Seismic-Induced Relay Chatter	03/85	06/86	S(A-46)
115	Reliability of Westinghouse Solid State Protection System	04/85	07/86	HIGH
122.1.a	Common Mode Failure of Isolation Valves in Closed Position	08/85	01/86	HIGH
122.1.b	Recovery of Auxiliary Feedwater	08/85	01/86	MEDIUM
122.1.c	Interruption of Auxiliary Feedwater Flow	08/85	01/86	HIGH
122.2	Initiating Feed-and-Bleed	08/85	01/86	HIGH
122.3	Physical Security System Constraints	08/85	01/86	LOW
124	Auxiliary Feedwater System Reliability	12/85	02/86	NR

TABLE 6 (Continued)
GENERIC ISSUES PRIORITIZED AS OF 1ST QUARTER FY-94

Action Item/ Issue No.	Title	Identification Date	Date Prioritized	Current Priority
<u>FY-86 (Cont'd)</u>				
125.I.2.a	PORV Reliability - Test Program	11/85	06/86	S(70)
125.I.2.b	PORV Reliability - Surveillance	11/85	06/86	S(70)
125.I.2.c	Auto Block Valve Closure	11/85	06/86	DROP
125.I.2.d	Equipment Qualification for Feed-and-Bleed Environment	11/85	06/86	S(A-45)
125.II.3	Review Steam/Feed Line Break Mitigation Systems for Single Failure	11/85	08/86	DROP
125.II.4	OTSG Dryout and Reflood Effects	11/85	09/86	DROP
125.II.7	Reevaluate Provisions to Automatically Isolate Feedwater from Steam Generator During Line Break	11/85	09/86	HIGH
125.II.9	Enhance Feed-and-Bleed Capability	11/85	08/86	S(A-45)
125.II.14	Remote Operation of Equipment Which Must Now be Operated Locally	11/85	08/86	LOW
133	Update Policy Statement on Nuclear Plant Staff Working Hours	07/86	07/86	LI
134	Rule on Degree and Experience Requirement	07/86	07/86	HIGH
C-4	Statistical Methods for ECCS Analysis	02/83	06/86	RI(R)
C-5	Decay Heat Update	02/83	06/86	RI(R)
C-6	LOCA Heat Sources	02/83	06/86	RI(R)
<u>FY-87</u>				
113	Dynamic Qualification Testing of Large Bore Hydraulic Snubbers	03/85	07/87	HIGH
125.I.1	Availability of the STA	11/85	07/87	DROP
125.I.4	Plant-Specific Simulator	11/85	02/87	DROP
125.I.7.b	Realistic Hands-On Training	11/85	03/87	DROP
125.I.8	Procedures and Staffing for Reporting to NRC Emergency Response Center	11/85	06/87	DROP

TABLE 6 (Continued)
GENERIC ISSUES PRIORITIZED AS OF 1ST QUARTER FY-94

Action Item/ Issue No.	Title	Identification Date	Date Prioritized	Current Priority
<u>FY-87 (Cont'd)</u>				
125.II.1.a	Two-Train AFW Reliability	11/85	10/86	DROP
125.II.1.b	Review Existing AFW Systems for Single Failure	11/85	10/86	HIGH
125.II.1.c	NUREG-0737 Reliability Improvements	11/85	10/86	DROP
125.II.1.d	AFW Steam and Feedwater Rupture Control System/ICS Interactions in B&W Plants	11/85	10/86	DROP
125.II.2	Adequacy of Existing Maintenance Requirements for Safety-Related Systems	11/85	06/87	DROP
125.II.5	Thermal-Hydraulic Effects of Loss and Restoration of Feedwater on Primary System Components	11/85	06/87	DROP
125.II.6	Reexamine PRA Estimates of Core Damage Risk from Loss of All Feedwater	11/85	03/87	DROP
125.II.8	Reassess Criteria for Feed-and-bleed Initiation	11/85	03/87	DROP
125.II.10	Hierarchy of Impromptu Operator Actions	11/85	02/87	DROP
125.II.12	Adequacy of Training Regarding PORV Operation	11/85	03/87	DROP
127	Testing and Maintenance of Manual Valves in Safety-Related Systems	05/86	06/87	LOW
128	Electrical Power Reliability	05/86	11/86	HIGH
130	Essential Service Water Pump Failures at Multiplant Sites	06/86	03/87	HIGH
135	Steam Generator and Steam Line Overfill	05/86	06/87	MEDIUM
<u>FY-88</u>				
43*	Reliability of Air Systems	04/87	12/87	HIGH
55*	Failure of Class IE Safety-Related Switchgear Circuit Breakers to Close on Demand	09/85	02/88	DROP
57	Effects of Fire Protection System Actuation on Safety-Related Equipment	09/82	06/88	MEDIUM

* Previous priority evaluation published in NUREG-0933.

TABLE 6 (Continued)
GENERIC ISSUES PRIORITIZED AS OF 1ST QUARTER FY-94

Action Item/ Issue No.	Title	Identification Date	Date Prioritized	Current Priority
<u>FY-88 (Cont'd)</u>				
62	Reactor Systems Bolting Applications	10/82	08/88	S(29)
88	Earthquakes and Emergency Planning	01/84	10/87	RESOLVED
104	Reduction of Boron Dilution Requirements	10/84	08/88	DROP
106	Piping and Use of Highly Combustible Gases in Vital Areas	10/84	11/87	MEDIUM
125.I.3	SPDS Availability	11/85	05/88	NR
125.I.6	Valve Torque Limit and Bypass Switch Settings	11/85	12/87	DROP
125.I.7A	Recover Failed Equipment	11/85	12/87	DROP
125.II.11	Recovery of Main Feedwater as Alternative to AFW	11/85	06/88	DROP
125.II.13	Operator Job Aids	11/85	03/83	DROP
126	Reliability of PWR Main Steam Safety Valves	03/86	03/88	LI
136	Storage and Use of Large Quantities of Cryogenic Combustibles on Site	09/86	03/88	LI
C-14	Storm Surge Model for Coastal Sites	02/83	05/88	LI(DROP)
III.D.1.1(2)	Review Information on Provisions for Leak Detection	12/82	09/88	DROP
III.D.1.1(3)	Develop Proposed System Acceptance Criteria	12/82	09/88	DROP
<u>FY-89</u>				
15*	Radiation Effects on Reactor Vessel Supports	09/88	02/89	HIGH
125.I.5	Safety Systems Tested in All Conditions Required by Design Basis Analysis	11/85	11/88	DROP
131	Potential Seismic Interaction Involving the Moveable In-Core Flux Mapping System Used in Westinghouse Plants	07/86	07/89	S(IPE)
139	Thinning of Carbon Steel Piping in LWRs	12/86	11/88	RESOLVED
B-31	Dam Failure Model	02/83	02/89	LI(DROP)
D-2	ECCS Capability for Future Plants	06/83	10/88	DROP

* Previous priority evaluation published in NUREG-0933.

TABLE 6 (Continued)
GENERIC ISSUES PRIORITIZED AS OF 1ST QUARTER FY-94

Action Item/ Issue No.	Title	Identification Date	Date Prioritized	Current Priority
<u>FY-90</u>				
63	Use of Equipment Not Classified as Essential to Safety in BWR Transient Analysis	10/82	02/90	DROP
71	Failure of Resin Demineralizer Systems and Their Effects on Nuclear Power Plant Safety	06/83	02/90	LOW
81*	Impact of Locked Doors and Barriers on Plant and Personnel Safety	12/86	02/90	DROP
95	Loss of Effective Volume for Containment Recirculation Spray	08/84	02/90	RESOLVED
96	RHR Suction Valve Testing	04/84	02/90	S(105)
107	Generic Implications of Main Transformer Failures	11/84	02/90	LOW
109	Reactor Vessel Closure Failure	12/84	02/90	DROP
116	Accident Management	04/85	09/90	S
117	Allowable Time for Diverse Simultaneous Equipment Outages	05/85	02/90	DROP
129	Valve Interlocks to Prevent Vessel Drainage During Shutdown Cooling	05/86	02/90	DROP
137	Refueling Cavity Seal Failure	10/86	05/90	DROP
140	Fission Product Removal Systems	03/87	02/90	DROP
141	LBLOCA With Consequential SGTR	04/87	05/90	DROP
142	Leakage Through Electrical Isolators	06/87	06/90	MEDIUM
B-29	Effectiveness of Ultimate Heat Sinks	02/83	08/90	LI(RESOLVED)
B-32	Ice Effects on Safety-Related Water Supplies	02/83	08/90	S(153)
<u>FY-91</u>				
24	Automatic Emergency Core Cooling System Switch to Recirculation	03/83	07/91	MEDIUM
38	Potential Recirculation System Failure as a Consequence of Ingestion of Containment Paint Flakes or Other Fine Debris	09/82	08/91	DROP

TABLE 6 (Continued)
GENERIC ISSUES PRIORITIZED AS OF 1ST QUARTER FY-94

Action Item/ Issue No.	Title	Identification Date	Date Prioritized	Current Priority
72	Control Rod Drive Guide Tube Support Pin Failures	06/83	10/90	DROP
73	Detached Thermal Sleeves	06/83	08/91	NR
100	Once-Through Steam Generator Level	09/84	09/91	DROP
120	On-line Testability of Protection Systems	08/85	11/90	MEDIUM
143	Availability of Chilled Water Systems and Room Cooling	10/87	03/91	HIGH
150	Overpressurization of Containment Penetrations	04/89	08/91	DROP
151	Reliability of Anticipated Transient Without Scram Recirculation Pump Trip in BWRs	04/89	08/91	MEDIUM
153	Loss of Essential Service Water in LWRs	05/90	03/91	HIGH
A-19	Digital Computer Protection System	02/83	11/90	LI
B-22	LWR Fuel	02/83	06/91	DROP
<u>FY-92</u>				
2	Failure of Protective Devices on Essential Equipment	05/83	07/92	DROP
76	Instrumentation and Control Power Interactions	06/83	04/92	DROP
78	Monitoring of Fatigue Transient Limits for Reactor Coolant System	06/83	07/92	MEDIUM
81*	Impact of Locked Doors and Barriers on Plant and Personnel Safety	08/91	04/92	LOW
89	Stiff Pipe Clamps	02/84	08/92	MEDIUM
110	Equipment Protection Devices on Engineered Safety Features	12/84	06/92	DROP
118	Tendon Anchorage Failure	07/85	01/92	RESOLVED

*Previous Priority Published in NUREG-0933

TABLE 6 (Continued)
GENERIC ISSUES PRIORITIZED AS OF 1ST QUARTER FY-94

Action Item/ Issue No.	Title	Identification Date	Date Prioritized	Current Priority
<u>FY-92 (Cont)</u>				
123	Deficiencies in the Regulations Governing DBA and Single Failure Criterion Suggested by the Davis- Besse Incident of June 9, 1985	11/85	12/91	DROP
132	RHR Pumps Inside Containment	07/86	03/92	DROP
138	Deinerting of BWRs With MARK I and II Containments During Power Operations Upon Discovery of Reactor Cooling System Leakage or a Train of a Safety System Inoperable	10/86	10/91	LOW
144	Scram Without a Turbine/Generator Trip	03/88	03/92	LOW
145	Improve Surveillance and Startup Testing Programs	09/88	02/92	NR
147	Fire-Induced Alternate Shutdown/Control Room Panel Interactions	04/89	08/92	LI
148	Smoke Control and Manual Fire-Fighting Effectiveness	04/89	08/92	
154	Adequacy of Emergency and Essential Lighting	09/90	01/92	LOW
155.1	More Realistic Source Term Assumptions	02/91	02/92	NR
155.2	Establish Licensing Requirements for Non-Operating Facilities	02/91	04/92	RI
155.4	Improve Criticality Calculations	02/91	08/92	DROP
155.5	More Realistic Severe Reactor Accident Scenario	02/91	06/92	DROP
155.6	Improve Decontamination Regulations	02/91	08/92	DROP
155.7	Improve Decommissioning Regulations	02/91	04/92	DROP
156.1.1	Settlement of Foundations and Buried Equipment	02/91	08/92	S(IPEEE)
156.1.2	Dam Integrity and Site Flooding	02/91	01/92	DROP
156.1.3	Site Hydrology and Ability to Withstand Floods	02/91	01/92	DROP
156.1.4	Industrial Hazards	02/91	03/92	DROP

TABLE 6 (Continued)
GENERIC ISSUES PRIORITIZED AS OF 1ST QUARTER FY-94

Action Item/ Issue No.	Title	Identification Date	Date Prioritized	Current Priority
<u>FY-92 (Cont)</u>				
156.1.5	Tornado Missiles	02/91	01/92	DROP
156.1.6	Turbine Missiles	02/91	10/91	DROP
156.2.1	Severe Weather Effects on Structures	02/91	01/92	DROP
156.2.2	Design Codes, Criteria, and Load Combinations	02/91	07/92	DROP
156.2.3	Containment Design and Inspection	02/91	05/92	DROP
156.2.4	Seismic Design of Structures, Systems, and Components	02/91	03/92	DROP
156.3.1.1	Shutdown Systems	02/91	03/92	DROP
156.3.1.2	Electrical Instrumentation and Control	02/91	03/92	DROP
156.3.2	Service and Cooling Water Systems	02/91	03/92	DROP
156.3.3	Ventilation Systems	02/91	09/92	DROP
156.3.4	Isolation of High and Low Pressure Systems	02/91	12/91	DROP
156.3.5	Automatic ECCS Switchover	02/91	11/91	S(24)
156.3.6.1	Emergency AC Power	02/91	02/92	S(B-56)
156.3.8	Shared Systems	02/91	06/92	DROP
156.4.1	RPS and ESFS Isolation	02/91	11/91	S(142)
156.4.2	Testing of the RPS and ESFS	02/91	03/92	S(120)
157	Containment Performance	10/91	02/92	RESOLVED
<u>FY-93</u>				
146	Support Flexibility of Equipment and Components	01/89	09/93	RESOLVED
149	Adequacy of Fire Barriers	04/89	10/92	LOW
152	Design Basis for Valves That Might Be Subjected to Significant Blowdown Loads	03/90	01/93	LOW
155.3	Improve Design Requirements for Nuclear Facilities	02/91	01/93	DROP
156.3.6.2	Emergency DC Power	02/91	03/93	LOW
159	Qualification of Safety-Related Pumps While Running on Minimum Flow	10/91	09/93	DROP

TABLE 6 (Continued)
GENERIC ISSUES PRIORITIZED AS OF 1ST QUARTER FY-94

Action Item/ Issue No.	Title	Identification Date	Date Prioritized	Current Priority
<u>FY-93(Cont.)</u>				
160	Spurious Actions of Instrumentation Upon Restoration of Power	10/91	09/93	DROP
161	Use of Non-Safety-Related Power Supplies in Safety-Related Circuits	10/91	03/93	DROP
162	Inadequate Technical Specifications for Shared Systems at Multiplant Sites When One Unit Is Shut Down	10/91	07/93	DROP
164	Neutron Fluence in Reactor Vessel	10/92	03/93	DROP
166	Adequacy of Fatigue Life of Metal Components	04/93	04/93	NR
168	Environmental Qualification of Electrical Equipment	04/93	04/93	NR
<u>FY-94</u>				
165	Spring-Actuated Safety and Relief Valve Reliability	10/92	11/93	HIGH

TABLE 7
GENERIC ISSUES TO BE PRIORITIZED AS OF 1ST QUARTER FY-94

Action Item/ Issue No.	Title	Identification Date	Current Schedule
156	<u>SEP Issues</u>	-	-
156.6.1	Pipe Break Effects on Systems and Components	02/91	03/94
158	Performance of Power-Operated Valves Under Design Basis Conditions	10/91	01/94
163	Multiple Steam Generator Tube Leakage	06/92	TBD
167	Combustible Gas Storage Facilities	06/93	TBD

TOTAL: 4

TABLE 8
GENERIC ISSUES TO BE REPRIORITIZED AS OF 1ST QUARTER FY-94

Action Item/ Issue No.	Title	Previous Priority	Identification Date	Current Schedule
NONE				

TABLE 9
REGULATORY IMPACT ISSUES SCHEDULED FOR RESOLUTION

Issue Number	Title	Lead Office/ Division/Branch	Priority	Date Approved for Resolution	Resolution Date at end of FY-92	Current Resolution Date
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NONE

TABLE 10
LICENSING ISSUES SCHEDULED FOR RESOLUTION

Issue Number	Title	Lead Office/ Division/ Branch	Resolution Date
67.5.1	Reassessment of Radiological Consequences	RES/DSIR/RPSI	03/94
147	Fire-Induced Alternate Shutdown/Control Room Panel Interactions	RES/DSIR/SAIB	01/94
148	Smoke Control and Manual Fire-Fighting Effectiveness	RES/DSIR/SAIB	06/94
I.D.5(5)	Disturbance Analysis Systems	RES/DSR/HFB	09/94
<u>TOTAL: 4</u>			

INDEX OF ISSUES
GENERIC ISSUE MANAGEMENT CONTROL SYSTEM

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15	RADIATION EFFECTS ON REACTOR VESSEL SUPPORTS	1
23	REACTOR COOLANT PUMP SEAL FAILURES	5
24	AUTOMATIC EMERGENCY CORE COOLING SWITCHOVER TO RECIRCULATION	9
57	EFFECTS OF FIRE PROT. SYS. ACTUATION ON SAFETY-RELATED EQUIP.	12
67.5.1	REASSESSMENT OF SGTR DESIGN BASIS	15
78	MONITORING OF FATIGUE TRANSIENT LIMITS FOR REACTOR COOLANT	18
83	CONTROL ROOM HABITABILITY	20
105	INTERFACING SYSTEMS LOCA AT LWRS	23
106	PIPING AND USE OF HIGHLY COMBUSTIBLE GASES IN VITAL AREAS	27
145	ACTIONS TO REDUCE COMMON CAUSE FAILURES	31
147	FIRE-INDUCED ALTERNATE SHUTDOWN/CONTROL ROOM PANEL INTERACTIONS	33
148	SMOKE CONTROL AND MANUAL FIRE-FIGHTING EFFECTIVENESS	35
155.1	MORE REALISTIC SOURCE TERM ASSUMPTIONS	37
165	SPRING-ACTUATED SAFETY AND RELIEF VALVE RELIABILITY	40
166	ADEQUACY OF FATIGUE LIFE OF METAL COMPONENTS	42
168	ENVIRONMENTAL QUALIFICATION OF ELECTRICAL EQUIPMENT	44
B-17	CRITERIA FOR SAFETY-RELATED OPERATOR ACTIONS	46
B-55	IMPROVE RELIABILITY OF TARGET ROCK SAFETY-RELIEF VALVES	48
B-56	DIESEL GENERATOR RELIABILITY	51
B-61	ALLOWABLE ECCS EQUIPMENT OUTAGE PERIODS	55
B-64	DECOMMISSIONING OF REACTORS	58
HF 4.4	GUIDELINES FOR UPGRADING OTHER PROCEDURES	60

RUN DATE : 02/03/94

INDEX OF ISSUES
GENERIC ISSUE MANAGEMENT CONTROL SYSTEM

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HF 5.1	LOCAL CONTROL STATIONS	62
HF 5.2	REV. CRIT. FOR HUMAN FACTORS ASPECTS OF ADV CONTROLS AND INSTRUMENT.	64
I.D.3	SAFETY SYSTEM STATUS MONITORING	66
I.D.5(3)	ON-LINE REACTOR SURVEILLANCE SYSTEMS	68
I.D.5(5)	DISTURBANCE ANALYSIS SYSTEMS	70
II.H.2	OBTAIN DATA ON CONDITIONS INSIDE TMI-2 CONTAINMENT	72

GENERIC ISSUE MANAGEMENT CONTROL SYSTEM

ISSUE NUMBER: 15 TITLE: RADIATION EFFECTS ON REACTOR VESSEL SUPPORTS
 IDENTIFICATION DATE: 06/80 PRIORITIZATION DATE: 02/89 TYPE: GSI PRIORITY: H
 TASK MANAGER: R. JOHNSON OFFICE/DIVISION/BRANCH: RES /DSIR/EIB ACTION LEVEL: ACTIVE STATUS:
 TAC NUMBERS: RESOLUTION DATE: 03/96
 WORK AUTHORIZATION: FEB. 8, 1989 MEMO FROM E. BECKJORD TO R.W. HOUSTON

FIN	CONTRACTOR	CONTRACT TITLE
A2256	ANL	SHIPPINGPORT SHIELD TANK TESTING (MEB)
A3841	BNL	TROJAN SUPPORT STABILITY ANALYSIS (ESGB)
B0415	ORNL	TROJAN SHIELD FLUX ATTENUATION (MEB)
B5848	MEA	MTR IRRADIATION OF STRUCTURAL STEELS (MEB)
B6021	LLNL	CONSEQUENCE EVALUATION OF EMBRITTLED RPV SUPPORTS (SSEB)
B6224	NIST	TROJAN CAVITY DOSIMETRY MEASUREMENT (MEB)
G1169	MPD NETWORK	NUCLEAR MATERIALS DATA RETRIEVAL FROM THE MPD NETWORK (EIB)
L1098	ORNL	HEAVY-SECTION STEEL IRRADIATION PROGRAM (MEB)
L1191	PAL ASSOCI	RPV SUPPORTS TECHNICAL ASSISTANCE (EMTB)
L1296	INEL	RADIATION EFFECTS ON REACTOR VESSEL SUPPORTS (EIB)
L1386	BNL	IRRADIATION ANALYSES (EIB)
L2552	ORNL	HFIR NEUTRON SPECTRUM MEASUREMENT (EIB)

WORK SCOPE

REVISED WRK SCOPE AS REPORTED FOR QUARTER 3, FY 1992:

1. QUANTITATIVELY EVALUATE THE HFIR NEUTRON FLUX OVER THE FULL ENERGY SPECTRUM.
2. REEVALUATE THE SIGNIFICANCE OF THE MEASURED AND REPORTED CHANGES IN MECHANICAL PROPERTIES OF THE HFIR
3. DETERMINE THE APPLICABILITY OF THE FINDINGS TO OPERATING REACTOR PRESSURE VESSEL SUPPORTS
4. RESOLVE GSI-15 WITH EITHER A GENERIC LETTER TO LICENSEES REQUESTING REASSESSMENT OF RPV SUPPORTS OR A CLOSEOUT MEMORANDUM IF THE SAFETY ASPECTS CAN BE ANNULLED.

STATUS

1. THE ORNL CONTRACT (FIN L2552) IS COMPLETE. THE FINAL LETTER REPORT WAS ISSUED AND A TOPICAL REPORT (NUREG/CR-6117) WAS SENT TO PUBLICATIONS.
2. A BASIS FOR RESOLUTION WAS PROPOSED; THE EXCESS EMBRITTLEMENT OF THE HFIR SURVEILLANCE SPECIMENS PROBABLY CAME FROM 20 YEARS OF GAMMA RADIATION EXPOSURE AT LOW TEMPERATURE. BECAUSE THE GAMMA FLUX AT RPV SUPPORTS IN OPERATING PLANTS IS MUCH LESS THAN IN THE HFIR VESSEL, EMBRITTLEMENT OF THE SUPPORTS CAN BE PREDICTED BASED ON NEUTRON DPA CALCULATIONS. THUS, THERE IS NO UNUSUAL SAFETY ASPECT TO BE ADDRESSED.
3. THE DOCUMENTS WHICH WILL PRESENT THE BASIS FOR RESOLUTION OF THE ISSUE ARE IN PREPARATION.

GENERIC ISSUE MANAGEMENT CONTROL SYSTEM

ISSUE NUMBER: 15

TITLE: RADIATION EFFECTS ON REACTOR VESSEL SUPPORTS

AFFECTED DOCUMENTS

10 CFR 50
STANDARD REVIEW PLAN SECTIONS 3.9 AND 5.4
ASME SECTION III
ASME SECTION XI
ASTM STANDARD E1035

PROBLEM/RESOLUTION

NO NEW PROBLEMS.

M I L E S T O N E S

	ORIGINAL DATE	CURRENT DATE	ACTUAL DATE
GS1 15 ASSIGNED TO RES/DSIR/EIB	--	--	02/89
DRAFT TAP TO DSIR DEPUTY DIRECTOR	05/89	--	05/89
DRAFT TAP APPROVED BY DSIR DIRECTOR AND FORWARDED TO OTHER RES DIVISION DIRECTORS AND NRR FOR COMMENT	06/89	--	07/89
INCORPORATE COMMENTS IN TAP	07/89	--	09/89
FINAL TAP APPROVED BY DSIR DIRECTOR	08/89	--	01/90
COMPLETE IRRADIATION DAMAGE CALC.	07/90	--	07/90
COMPLETE SUBTASK 3.1	09/90	--	09/90
COMPLETE SHIPPINGPORT NST REVIEW	09/90	--	10/90
TAP REV. 1 TO DSIR DEPUTY DIRECTOR	03/91	--	02/91
COMPLETE COST/BENEFIT ANAL. (INEL)	06/91	--	06/91
PROVIDE DRAFT SCREEN./EVAL. ANAL. (INEL)	06/91	--	05/91
COMPLETE HFIR DOSIMETRY (ORNL)	04/92	--	04/92
COMPLETE B/HE ANALYSIS (ORNL)	05/92	--	05/92
PROVIDE HFIR FLUX CALCULATIONS (ORNL)	07/91	--	08/92

AS OF FY94 QUARTER 1

GENERIC ISSUE MANAGEMENT CONTROL SYSTEM

ISSUE NUMBER: 15

TITLE: RADIATION EFFECTS ON REACTOR VESSEL SUPPORTS

M I L E S T O N E S	ORIGINAL DATE	CURRENT DATE	ACTUAL DATE
COMPLETE DOSIMETRY CONFIRMATION	10/92	--	02/93
IRRADIATE DOSIMETERS (L2552)	02/93	--	02/93
COMPLETE DOSIMETER MEASUREMENTS (L2552)	03/93	--	03/93
VERIFY HFIR FLUENCE (ORNL)	07/92	--	07/93
REPORT PEER-REVIEWED RESULTS (L2552)	06/93	--	08/93
PROPOSE SUPPORTS RADIATION MODEL	09/93	--	09/93
PROPOSE ISSUE RESOLUTION PATH	11/93	--	11/93
DRAFT PACKAGE TO DSIR DIRECTOR (REVIEW)	01/92	01/94	--
PROPOSED RESOLUTION PACKAGE TO OTHER OFFICES	02/92	02/94	--
PROPOSED RESOLUTION TO CRGR AND ACRS	07/93	05/94	--
PROPOSED RESOLUTION PACKAGE TO BE ISSUED FOR PUBLIC COMMENT (INCLUDES DRAFT GL)	08/92	08/94	--
END PUBLIC COMMENT PERIOD	11/92	11/94	--
FINAL RESOLUTION PACKAGE INCLUDING COMM INFO PAPER PREPARED BY EIB	03/92	02/95	--
FINAL RESOLUTION PACKAGE TO DSIR DIRECTOR (APPROVE)	03/93	03/95	--
FINAL RESOLUTION PACKAGE TO OTHER OFFICES FOR REVIEW	05/93	07/95	--
FINAL RESOLUTION PACKAGE TO CRGR AND ACRS	07/93	09/95	--
FINAL RESOLUTION PACKAGE TO NRR FOR ISSUANCE	10/93	11/95	--
NRR ISSUES GL	02/94	02/96	--

RUN DATE: 02/03/94

GENERIC ISSUE MANAGEMENT CONTROL SYSTEM

PAGE: 4

ISSUE NUMBER: 15

TITLE: RADIATION EFFECTS ON REACTOR VESSEL SUPPORTS

M I L E S T O N E S	ORIGINAL DATE	CURRENT DATE	ACTUAL DATE
----- ISSUE CLOSEOUT MEMO	03/93	03/96	--

AS OF FY94 QUARTER 1

GENERIC ISSUE MANAGEMENT CONTROL SYSTEM

ISSUE NUMBER: 23
 IDENTIFICATION DATE: 12/80C
 TASK MANAGER: K. SHAUKAT
 TAC NUMBERS:

TITLE: REACTOR COOLANT PUMP SEAL FAILURES
 PRIORITIZATION DATE: 04/83C
 OFFICE/DIVISION/BRANCH: RES /DSIR/EIB

TYPE: GSI
 ACTION LEVEL: ACTIVE

PRIORITY: H
 STATUS:
 RESOLUTION DATE: 03/95

WORK AUTHORIZATION: NRR FY-84 OPERATING PLAN, (APPENDIX G)

FIN	CONTRACTOR	CONTRACT TITLE
A3751	BNL	REACTOR COOLANT PUMP (RCP) SEAL INTEGRITY
A3771	BNL	ASSESS EFFECT OF MECH. MAINT. RCP SEAL
A3806	BNL	EVAL. ADEQ. OF RCP SEAL INSTR. & RESP. TO RCP
A6322	INEL	RCP SHAFT SEAL BEHAVIOR DURING ST. BLACKOUT
B2953	PNL	REACTOR COOL. PUMP SEAL FAIL. VAL. IMP. ANALYSIS
B3049	ETEC	LEAK RATE ANALYSIS OF WESTINGHOUSE RCP
D1760	SCIEN TECH	REG. ANAL. FOR GI-23, RCP SEAL FAILURES

WORK SCOPE

THE TAP FOR THIS ISSUE WAS REVISED TO INCORPORATE WORK NECESSARY TO COMPLETE GSI 85. REVIEW PWR RCP SEAL FAILURES CAUSED BY MECHANICAL FAILURES AND LOSS OF SEAL COOLING TO ASCERTAIN IF INCREASED NRC REQUIREMENTS ARE NECESSARY TO MINIMIZE SEAL FAILURES. EVALUATE THE EFFECTS OF SYSTEM DESIGN, MATERIALS, OPERATION, TESTING, AND MAINTENANCE ON SEAL FAILURE MECHANISMS. REVIEW THE RELIAB. OF SEAL COOLING SYSTEMS AND IDENTIFY METHODS TO MINIMIZE THE EFFECTS OF LOSS OF SEAL COOLING. PERFORM A RISK ASSESSMENT OF SEAL FAILURE FROM MAINTENANCE AND MECHANICAL EFFECTS. THIS PROGRAM WILL DETERMINE WHAT STEPS THE NRC

SHOULD TAKE, IF ANY, TO REDUCE RCP SEAL FAILURE TO AN ACCEPTABLE LEVEL COMMENSURATE WITH THE POTENTIAL SAFETY CONSEQUENCES OF SEAL FAILURE. THE PROGRAM CONSISTS OF THE FOLLOWING MAJOR TASKS:
 TASK 1 - DETERMINE ACTUAL SEAL FAILURE MECHANISMS
 TASK 2 - EVALUATE RCP SEAL COOLING FOR ALL MODES OF OPERATION FOR WHICH SEAL COOLING IS REQ'D USING THE RELIABILITY OF TYPICAL RCP SEAL COOLING SYSTEM DESIGNS.
 TASK 3 - REVIEW MECHANICAL AND MAINTENANCE-INDUCED FAILURE MECHANISMS.
 TASK 4- FEASIBILITY STUDY TO IDENTIFY MOST EFFECTIVE FIX.

STATUS

AN NRR GENERIC LETTER FOR INFORMATION ONLY WAS ISSUED MAY 2, 1991, TO INFCRM LICENSEES OF THE POTENTIAL IMPACT ON SBO ANALYSES. A FEDERAL REGISTER NOTICE WAS PUBLISHED APRIL 19, 1991, REQUESTING PUBLIC COMMENT ON DRAFT

REGULATORY GUIDE DG-1008 AND THE DRAFT REGULATORY ANALYSIS, NUREG-1401. THE STAFF DECIDED TO PROCEED WITH A RULEMAKING FOR OFF-NORMAL CONDITIONS, WHILE THE NORMAL CONDITIONS ARE BEING ADDRESSED BY A PROPOSED GENERIC LETTER.

GENERIC ISSUE MANAGEMENT CONTROL SYSTEM

ISSUE NUMBER: 23

TITLE: REACTOR COOLANT PUMP SEAL FAILURES

AFFECTED DOCUMENTS

(1) THE FOLLOWING REPORTS HAVE BEEN ISSUED:

NUREG/CR-4077. "REACTOR COOLANT PUMP SHAFT SEAL BEHAVIOR"

NUREG/CR-4294. "LEAK RATE ANALYSIS OF THE WESTINGHOUSE REACTOR COOLANT PUMP"

NUREG/CR-4821. "REACTOR COOLANT PUMP SHAFT SEAL STABILITY DURING STATION BLACKOUT"

NUREG/CR-4643. "EVALUATION OF CORE DAMAGE SEQUENCES INITIATED BY LOSS OF REACTOR COOLANT PUMP SEAL COOLING"

NUREG/CR-4400. "THE IMPACT OF MECHANICAL AND MAINTENANCE-INDUCED FAILURE OF MAIN REACTOR COOLANT PUMP SEALS ON PLANT SAFETY"

NUREG/CR-4544. "REACTOR COOLANT PUMP SEAL-RELATED INSTRUMENTATION AND OPERATOR RESPONSE"

NUREG/CR-4948. "TECHNICAL FINDINGS RELATED TO GI-23, REACTOR COOLANT PUMP SEAL FAILURES"

NUREG/CR-5167. "COST BENEFIT ANALYSIS FOR GI-23, REACTOR COOLANT PUMP SEAL FAILURE"

NUREG/CR-5918. "ANALYSIS OF PUBLIC COMMENTS ON GI-23, REACTOR COOLANT PUMP SEAL FAILURE" (IN PREPARATION)

(2) RULEMAKING RM-333

(3) REGULATORY GUIDE DG-1008

PROBLEM/RESOLUTION

NONE

M I L E S T O N E S	ORIGINAL DATE	CURRENT DATE	ACTUAL DATE
-----	-----	-----	-----
TAP ISSUED FOR COMMENT	04/83	--	03/83
TAP APPROVED BY DSI DIR	10/83	--	10/83
ISSUED REQUEST FOR PROPOSAL TO BNL FOR COMPONENT COOLING WATER PRA	11/83	--	11/83
TRANSFER RESOLUTION RESPONSIBILITY FROM DSI TO DE	06/84	--	06/84
TRANSFER RESOLUTION RESPONSIBILITY FROM DE TO DST/GIB	05/85	--	05/85

GENERIC ISSUE MANAGEMENT CONTROL SYSTEM

ISSUE NUMBER: 23

TITLE: REACTOR COOLANT PUMP SEAL FAILURES

M I L E S T O N E S	ORIGINAL DATE	CURRENT DATE	ACTUAL DATE
COMPLETE DOCUMENT OF BACKGROUND PAST SEAL FAILURES, INCLUDING LER SEARCH, REV OF INDUST. EXP & REV OF BWR SCR. (TASK 1)	05/85	--	12/85
EVAL RCP SEAL COOLING, INCL DETERM EFFECT OF LOSS COOLING, EVAL RELIAB OF COOLING SYS, AND REV RCP SEAL COOLING INSTR. (TASK 2)	12/84	--	08/86
EVALUATE MECHANICAL AND MAINTENANCE-INDUCED FAILURE MECHANISMS (TASK 3)	01/85	--	01/88
DRAFT SCIENTECH REPORTS TO IDENTIFY MOST EFFECTIVE FIX. (TASK 4)	03/85	--	01/89
REVIEW & FINALIZE SCIENTECH REPORTS	05/89	--	03/90
DTR PACKAGE COMPLETE	07/85	--	04/90
DIVISION DIRECTOR REVIEW COMPLETED	08/85	--	05/90
ACRS REVIEW OF DTR COMPLETE	11/85	--	09/90
CONCURRENCE BY OTHER OFFICES	09/85	--	10/90
DTR SENT TO CRGR	10/85	--	10/90
CRGR REVIEW OF DTR COMPLETE	11/85	--	03/91
FRN OF ISSUANCE OF PROPOSED RESOLUTION FOR PUBLIC COMMENT	12/85	--	04/91
DRAFT PERFORMANCE	07/91	--	08/91
PUBLIC COMMENT PERIOD ENDS	09/91	--	11/91
COMPLETE REVIEW AND COMPILATION OF PUBLIC COMMENTS	04/86	--	02/92
RECEIPT OF GUIDANCE FROM EDO ON RULEMAKING	05/92	--	11/92

GENERIC ISSUE MANAGEMENT CONTROL SYSTEM

ISSUE NUMBER: 23

TITLE: REACTOR COOLANT PUMP SEAL FAILURES

M I L E S T O N E S	ORIGINAL DATE	CURRENT DATE	ACTUAL DATE
PROPOSED APPROACH TO NRR FOR NORMAL OPERATION	11/92	--	12/92
PREPARE PROPOSED RULE	09/92	--	05/93
DIVISION DIRECTOR REVIEW OF PROPOSED RULE COMPLETED	10/92	--	06/93
CONCURRENCE BY OTHER OFFICES OF PROPOSED RULE	11/92	--	10/93
LETTER TO NRR FOR BWR SEALS UNDER SBO	05/93	--	10/93
ACRS AND CRGR REVIEW OF PROPOSED RULE	12/92	--	11/93
PROPOSED RULE TO EDO/COMMISSION	02/93	02/94	--
COMPLETE DRAFT RESOLUTION OF TECHNICAL COMMENTS AND PREPARE RESPONSES TO ALL COMMENTS	03/92	03/94	--
FRN OF ISSUANCE OF PROPOSED RULE FOR PUBLIC COMMENT	04/93	03/94	--
PUBLIC COMMENT PERIOD ENDS	06/93	05/94	--
RESOLVE PUBLIC COMMENTS	08/93	08/94	--
PREPARE FINAL RULE	10/93	09/94	--
DIVISION DIRECTOR REVIEW COMPLETED	11/93	10/94	--
CONCURRENCE BY OTHER OFFICES	12/93	11/94	--
ACRS AND CRGR REVIEW OF FTR COMPLETED	01/94	12/94	--
FTR SENT TO EDO/COMMISSION	03/94	01/95	--
FRN OF ISSUANCE OF FTR	04/94	03/95	--

RUN DATE: 02/03/94

GENERIC ISSUE MANAGEMENT CONTROL SYSTEM

PAGE: 9

ISSUE NUMBER: 24

TITLE: AUTOMATIC EMERGENCY CORE COOLING SWITCHOVER TO RECIRCULATION

IDENTIFICATION DATE: 03/83C

PRIORITIZATION DATE: 07/91C

TYPE: GSI

PRIORITY: M

TASK MANAGER: G. BURDICK

OFFICE/DIVISION/BRANCH: RES /DSIR/SMIB

ACTION LEVEL: ACTIVE

STATUS:

TAC NUMBERS:

RESOLUTION DATE: 01/95

WORK AUTHORIZATION: MEMORANDUM FROM E. BECKJORD TO W. MINNERS ON JULY 23, 1991

FIN
LI854

CONTRACTOR
SEA

CONTRACT TITLE

AUTOMATIC ECCS SWITCHOVER TO RECIRCULATION

WORK SCOPE

RESOLUTION OF THIS ISSUE WILL BE ACCOMPLISHED BY A REEVALUATION OF THE STUDY PERFORMED IN 1981, USING UP-TO-DATE HUMAN RELIABILITY AND AUTOMATED EQUIPMENT PRA ANALYSIS METHODS TO DETERMINE THE CORE DAMAGE FREQUENCY AND RISK CHANGES (AND UNCERTAINTY) THAT WOULD RESULT FROM CHANGEOVER TO AN AUTOMATIC SWITCHOVER CONTROL SYSTEM. THIS REEVALUATION WILL ALSO INCLUDE MAKING AN ESTIMATE OF WHAT IT

WOULD COST (WITH UNCERTAINTY) FOR A LICENSEE TO INSTALL SUCH AN AUTOMATIC SWITCHOVER CONTROL SYSTEM. THESE RESULTS WILL BE USED TO PROVIDE THE BASES FOR A STAFF RECOMMENDATION TO THE COMMISSION REGARDING WHETHER OR NOT PWR'S THAT CURRENTLY RELY ON A MANUAL SWITCHOVER SYSTEM SHOULD BE REQUIRED TO INSTALL AN AUTOMATIC SWITCHOVER SYSTEM.

STATUS

CONTRACT WORK UNDERWAY. FIRST LETTER REPORT (TASK 4.01, MODEL FOR MANUAL SWITCHOVER) SUBMITTED DURING LAST WEEK OF

DECEMBER 1992.

AFFECTED DOCUMENTS

POTENTIAL CHANGES TO SRP.

PROBLEM/RESOLUTION

WORK WAS DELAYED IN STARTING DUE TO NRC STAFF DISAPPROVAL OF HUMAN FACTORS CONSULTANT. WORK HAS RECENTLY (1/5/93) BEEN DELAYED BY NRC REQUEST FOR ADDITIONAL WORK REGARDING JUSTIFICATION FOR HUMAN FACTORS METHOD SELECTED BY REPLACEMENT HUMAN FACTORS CONSULTANT. SEA DRAFT REPORT WAS LATE. ADDITIONAL ANALYSES AND EXTENSIVE REVISION OF THE DRAFT DOCUMENT ARE REQUIRED. A SECOND DRAFT MUST BE SUBMITTED. ADDITIONAL DELAYS WERE CAUSED BY PROTRACTED DELIBERATION OF THE CONTRACTOR OVER THE REVISED SCOPE OF WORK.

AS OF FY94 QUARTER 1

GENERIC ISSUE MANAGEMENT CONTROL SYSTEM

ISSUE NUMBER: 24

TITLE: AUTOMATIC EMERGENCY CORE COOLING SWITCHOVER TO RECIRCULATION

M I L E S T O N E S	ORIGINAL DATE	CURRENT DATE	ACTUAL DATE
PRELIMINARY COMMUNICATION/NEGOTIATION WITH SEA	04/92	--	04/92
PREPARE DRAFT TASK ACTION PLAN	03/92	--	05/92
RFP TO SEA	04/92	--	04/92
ACCEPT SEA PROPOSAL (BEGIN WORK)	08/92	--	08/92
COMPLETE HF METHODS EVAL. (TASK 1)	10/92	--	12/92
APPLY TECHNIQUE (TASK 2)	10/92	--	03/93
BEST EST. COST OF CONVERSION (TASK 5)	01/93	--	01/93
DETERMINE CDF (TASK 3)	12/92	--	04/93
DETERMINE RELEASE (TASK 4)	12/92	--	04/93
DRAFT REPORT SUBMITTAL (TASK 6)	02/93	--	05/93
STAFF COMMENTS TO SEA	03/93	--	09/93
SUBMITTAL OF REDRAFT	11/93	04/94	--
STAFF COMMENTS TO SEA	12/93	05/94	--
SUBMITTAL OF CAMERA-READY COPY (TASK 8)	04/93	06/94	--
STAFF-PREPARED RESOLUTION DOCUMENT -- RES DIRECTOR CONCUR	06/93	07/94	--
STAFF-PREPARED RESOLUTION DOCUMENT -- ACRS AND CRGR REVIEW	08/93	08/94	--
STAFF-PREPARED RESOLUTION DOCUMENT -- PUBLIC COMMENTS INCORPORATED	12/93	10/94	--
STAFF-PREPARED RESOLUTION DOCUMENT -- EDO APPROVAL	01/94	11/94	--

RUN DATE: 02/03/94

GENERIC ISSUE MANAGEMENT CONTROL SYSTEM

PAGE: 11

ISSUE NUMBER: 24

TITLE: AUTOMATIC EMERGENCY CORE COOLING SWITCHOVER TO RECIRCULATION

M I L E S T O N E S	ORIGINAL DATE	CURRENT DATE	ACTUAL DATE
----- STAFF-PREPARED RESOLUTION DOCUMENT -- NRC APPROVED	03/94	01/95	--

AS OF FY94 QUARTER 1

GENERIC ISSUE MANAGEMENT CONTROL SYSTEM

ISSUE NUMBER: 57 TITLE: EFFECTS OF FIRE PROT. SYS. ACTUATION ON SAFETY-RELATED EQUIP.
 IDENTIFICATION DATE: 09/82C PRIORITIZATION DATE: 06/88C TYPE: GSI PRIORITY: M
 TASK MANAGER: R. WOODS OFFICE/DIVISION/BRANCH: RES /DSIR/SAIB ACTION LEVEL: ACTIVE STATUS:
 TAC NUMBERS: RESOLUTION DATE: 02/94
 WORK AUTHORIZATION: MEMO TO B. SHERON FROM E. BECKJORD DATED 6/8/88.

FIN	CONTRACTOR	CONTRACT TITLE
B6601	SNL	INADVERTENT ACTUATION OF FIRE PROTECTION SYSTEMS (GI-57), PRELIMINARY/SCOPING STUDY ONLY
L1220	INEL	GENERIC ISSUE 57, "EFFECTS OF FIRE PROTECTION SYSTEM ACTUATION ON SAFETY-RELATED EQUIPMENT"
L1334	SNL	GENERIC ISSUE 57, "EFFECTS OF FIRE PROTECTION SYSTEM ACTUATION ON SAFETY-RELATED EQUIPMENT"
L1592	NIST	GENERIC ISSUE 57, EFFECTS OF FIRE PROTECTION SYSTEM ACTUATION ON SAFETY RELATED EQUIPMENT.

WORK SCOPE

ACTUATIONS OF FIRE PROTECTION SYSTEMS (FPS'S) HAVE CAUSED DAMAGE TO PLANT EQUIPMENT WITH CONSIDERABLE FREQUENCY AND SAFETY IMPLICATIONS. A PROGRAM TO RESOLVE GI-57 WAS INITIATED IN FY 1989 INVOLVING THE EVALUATION OF FOUR PLANTS AND THE COLLECTION OF GENERIC PLANT DATA TO SUPPORT A GENERIC EVALUATION OF THE SAFETY CONCERNS OF GI-57. THESE EVALUATIONS INVOLVE REVIEWS OF FPS DESIGNS AND LAYOUTS,

PLANT WALKDOWNS, FIRE ZONE AND VITAL AREA ANALYSES, CUTSETS DEVELOPMENT, RISK ESTIMATES, AND VALUE-IMPACT ANALYSES OF POTENTIAL BACKFITS, INCLUDING UNCERTAINTY AND SENSITIVITY ANALYSES. A TASK TO DEVELOP AND APPLY A DECISIONMAKING UNDER UNCERTAINTY ANALYSIS METHOD WAS ADDED IN RESPONSE TO ACRS COMMENTS ON USE OF PRA'S.

STATUS

A CONTRACTOR'S FINAL REPORT FOR THE B&W PLANT EVALUATION WAS RECEIVED FOR PUBLICATION IN SEPTEMBER 1992. THE DRAFT FINAL GENERIC EVALUATION REPORT WAS RECEIVED IN AUGUST 1992 AND WAS PUBLISHED AS A SERIES OF NUREG/CR REPORTS IN DECEMBER 1992. PROPOSED RESOLUTION IS TO CONSIDER THE

PRINCIPAL CDF-CONTRIBUTING EVENTS AS PART OF THE IPEE AND RESOLVE ISSUE 57 WITH NO NEW REQUIREMENTS. FINAL PACKAGE WITH RESOLUTION ISSUED 09/30/93. INFORMATION NOTICE BY NRR IN CONCURRENCE CHAIN FOR ISSUANCE AS OF END OF 01/94.

GENERIC ISSUE MANAGEMENT CONTROL SYSTEM

ISSUE NUMBER: 57

TITLE: EFFECTS OF FIRE PROT. SYS. ACTUATION ON SAFETY-RELATED EQUIP.

AFFECTED DOCUMENTS

PROBLEM/RESOLUTION

SRP SECTION 9.5.1.

NONE

M I L E S T O N E S	ORIGINAL DATE	CURRENT DATE	ACTUAL DATE
DEVELOP PRA SCOPE OF WORK	10/88	--	10/88
COMPLETE RISK ANALYSIS	02/89	--	04/89
DEVELOP TAP	02/89	--	06/89
PRELIMINARY DISCUSSION OF DRAFT STATEMENT OF WORK WITH CONTRACTOR(S)	06/89	--	06/89
DEVELOP SOW AND WORK AUTHORIZATION	07/89	--	07/89
ISSUE CONTRACTS	08/89	--	08/89
EPRI DRAFT REPORT ON EFFECTS OF FIRE SUPPRESSANTS	09/89	--	11/89
LOADING OF PRA MODEL DATA FOR IP-2 EVALUATION (W PLANT)	11/89	--	11/89
DRAFT REPORT ON IP-2 EVALUATION (INEL)	05/90	--	05/90
DRAFT REPORT SNL/INEL ON SURRY/IP-2 EVALUATIONS	07/90	--	05/90
EQE SUBCONTRACT REPORT TO SNL	08/90	--	10/90
FINAL DRAFT CONTRACTOR REPORT TO RES FOR TWO WESTINGHOUSE PLANTS	01/91	--	01/91
DECISION ON B&W OR CE PLANT EVALUATION	01/91	--	01/91
DRAFT CONTRACTOR'S REPORT TO RES FOR GE PLANT	05/91	--	06/91
DRAFT SNL NUREG/CR FOR B&W PLANT	08/91	--	02/92

GENERIC ISSUE MANAGEMENT CONTROL SYSTEM

ISSUE NUMBER: 57

TITLE: EFFECTS OF FIRE PROT. SYS. ACTUATION ON SAFETY-RELATED EQUIP.

M I L E S T O N E S	ORIGINAL DATE	CURRENT DATE	ACTUAL DATE
DRAFT SNL NUREG/CR FOR GENERIC EVAL.	05/92	--	08/92
REGULATORY ANALYSIS COMPLETE	06/92	--	10/92
PROPOSED RESOLUTION TO DD	11/92	--	04/93
PACKAGE TO ACRS	02/92	--	04/93
COMMUNICATION TO NRR RE MOST VULNERABLE PLANT	07/93	--	07/93
COMPLETE ACRS REVIEW	07/91	--	08/93
ISSUE CLOSEOUT MEMO FROM RES DIRECTOR TO EDO	07/92	--	09/93
MEMO TO NRR RE MOST VULNERABLE PLANT	10/93	--	11/93
MEMO TO NRR CLOSING MOST VULNERABLE PLANT ISSUE (NO NEW ACTIONS REQUIRED)	01/94	01/94	--
ISSUE INFORMATION NOTICE	02/94	02/94	--

RUN DATE: 02/03/94

GENERIC ISSUE MANAGEMENT CONTROL SYSTEM

PAGE: 15

ISSUE NUMBER: 67.5.1 TITLE: REASSESSMENT OF SGTR DESIGN BASIS
IDENTIFICATION DATE: 06/83C PRIORITIZATION DATE: 03/85C TYPE: LI PRIORITY:
TASK MANAGER: J. HOPENFELD OFFICE/DIVISION/BRANCH: RES /DSIR/EIB ACTION LEVEL: ACTIVE STATUS: 5
TAC NUMBERS: NONE RESOLUTION DATE: 03/94
WORK AUTHORIZATION: MEMO TO R. BERNERO FROM H. DENTON DATED 3/25/85

FIN	CONTRACTOR	CONTRACT TITLE
A6879 B0832	INEL ORNL	RISK ASSOCIATED WITH A SPECTRUM OF SGTR ACCIDENTS STEAM GENERATOR TUBE RUPTURE

WORK SCOPE

REEVALUATE THE METHOD OF ESTIMATING OFFSITE DOSES FOLLOWING A STEAM GENERATOR TUBE RUPTURE. COMPLETE THE EXPERIMENTS ON IODINE PARTITION COEFFICIENTS. EVALUATE THE VALIDITY OF SRP IODINE SPIKE REQUIREMENTS IN LIGHT OF RECENT DATA USING BEST ESTIMATE MODELS. IDENTIFY MODIFICATIONS TO THE SRP. RESOLUTION OF GI 67.5.2 IS INCLUDED IN THE SCOPE OF THIS ISSUE.

STATUS

THE ORNL EXPERIMENTS TO DETERMINE THE IODINE PARTITION COEFFICIENT ARE COMPLETE. THE INEL EVALUATION OF PLANT IODINE SPIKING DATA HAS BEEN COMPLETED. INEL ANALYSIS OF SECONDARY SYSTEM PH FOLLOWING AN SGTR HAS BEEN COMPLETED. INEL EVALUATION OF PROPOSED MODEL MODIFICATIONS ON OFFSITE DOSE HAS BEEN COMPLETED. A DRAFT RESOLUTION PACKAGE HAS BEEN PREPARED AND IS UNDER MANAGEMENT REVIEW.

AFFECTED DOCUMENTS

SRP

PROBLEM/RESOLUTION

ACRS REQUESTED A REVIEW IN OCTOBER BUT CONFLICTS WITH GSI-23.

MILESTONES

PREPARE DRAFT TAP

ORIGINAL DATE	CURRENT DATE	ACTUAL DATE
06/87	--	06/87

AS OF FY94 QUARTER 1

GENERIC ISSUE MANAGEMENT CONTROL SYSTEM

ISSUE NUMBER: 67.5.1

TITLE: REASSESSMENT OF SGTR DESIGN BASIS

M I L E S T O N E S	ORIGINAL DATE	CURRENT DATE	ACTUAL DATE
REVISE DRAFT TAP TO INCLUDE PROBABILISTIC ASSESSMENT OF SGTR	10/87	--	12/87
MANAGEMENT APPROVAL OF REVISED TAP	11/87	--	12/87
COMPLETE WORK PLAN AND CONTRACT PROPOSAL	11/87	--	12/87
ISSUE CONTRACT TO INEL	01/88	--	03/88
DRAFT REPORT FROM INEL AND ORNL TO RES	06/88	--	07/88
FINAL REPORT FROM INEL	08/88	--	08/88
FINAL REPORT FROM ORNL	09/88	--	09/88
FINAL REPORT FROM INEL ON IODINE SPIKING	01/89	--	01/89
FINAL REPORT FROM INEL ON IODINE SPIKING DURING SGTR	07/89	--	09/89
DRAFT REGULATORY ANALYSIS PREPARED	09/88	--	11/89
DRAFT REG ANALYSIS SENT TO NRR FOR COMMENT	07/90	--	08/90
COMMENTS ON DRAFT REG ANALYSIS RECEIVED FROM NRR	01/91	--	04/91
DRAFT REPORT ON SECONDARY PH	03/91	--	11/91
FINAL REPORT ON SECONDARY PH	05/91	--	01/92
DTR PACKAGE FOR DEPUTY DIRECTOR COMMENT	10/88	--	01/92
DRAFT FINAL REPORT ON DOSE CALCULATIONS	07/92	--	10/92
CONTRACTOR PRESENTATION OF FINAL RESULTS	11/92	--	11/92
REVISE DTR PACKAGE TO DEPUTY DIRECTOR FOR COMMENT	07/92	--	11/92

GENERIC ISSUE MANAGEMENT CONTROL SYSTEM

ISSUE NUMBER: 67.5.1

TITLE: REASSESSMENT OF SGTR DESIGN BASIS

M I L E S T O N E S	ORIGINAL DATE	CURRENT DATE	ACTUAL DATE
TRANSMIT TO ACRS RESOLUTION WITH SRP REVISION RECOMMENDATIONS	06/93	--	08/93
ACRS PRESENTATION	08/93	--	01/94
RES MEMO TO NRR TRANSMITTING SRP REVISION RECOMMENDATIONS	09/93	02/94	--
RESOLUTION TO EDO	09/93	03/94	--

GENERIC ISSUE MANAGEMENT CONTROL SYSTEM

ISSUE NUMBER: 78

TITLE: MONITORING OF FATIGUE TRANSIENT LIMITS FOR REACTOR COOLANT

IDENTIFICATION DATE: 05/83C

PRIORITIZATION DATE: 07/92C

TYPE:

PRIORITY:

TASK MANAGER: S. DIAB

OFFICE/DIVISION/BRANCH: RES /DSIR/EIB

ACTION LEVEL: ACTIVE

STATUS: 4

TAC NUMBERS:

RESOLUTION DATE: 04/94

WORK AUTHORIZATION: 7/10/92 MEMO/BECKJORD TO MINNERS, "MONITORING OF FATIGUE..."
4/1/93 MEMO/BECKJORD TO SNIEZEK, "RESOLUTION OF FATIGUE..."

FIN

W6077

CONTRACTOR

ANL

CONTRACT TITLE

STATISTICAL ANALYSIS OF FATIGUE DATA

W O R K S C O P E

TO DEVELOP GUIDANCE ON POTENTIAL FATIGUE DEGRADATION OF
COMPONENT OF THE RCPB FOR ALL OPERATING PLANTS. THIS

GUIDANCE IS APPLICABLE TO THOSE REMAINING YEARS OF PLANT
OPERATION UNDER THE CURRENT 40-YEAR LICENSE.

S T A T U S

BECAUSE THE ORIGINAL DESIGN BASIS OF THE REACTOR COOLANT
PRESSURE BOUNDARY (RCPB) DID NOT ACCOUNT ADEQUATELY FOR THE
EFFECTS OF THE REACTOR COOLANT HARSH ENVIRONMENTS DURING
OPERATION ON THE RCPB FATIGUE RESISTANCE, GENERIC ISSUE 78
WAS ESTABLISHED TO ASSESS THE INCREASE IN RISK DUE TO OPERA-
TION UNDER THE HARSH ENVIRONMENT.

A DRAFT PRA PARAMETRIC STUDY WAS CONDUCTED TO ASSESS THE
RISK CONTRIBUTION OF VARIOUS COMPONENTS OF THE RCPB. THAT
STUDY WAS ISSUED FOR PEER REVIEW AND COMMENT ON NOVEMBER 20,
1992. THE DRAFT PRA STUDY TENTATIVELY IDENTIFIED SOME
HIGH RISK RCPB COMPONENTS. ALSO, THE DRAFT PRA STUDY
IDENTIFIED SOME AREAS WHERE THERE IS A LACK OF INFORMATION.
THESE AREAS ARE CURRENTLY UNDER STUDY BY ANL (CONTRACT
FIN-6077-3). THE ANL CONTRACT INCLUDES A STATISTICAL

ANALYSIS OF RELEVANT AND UP-TO-DATE FATIGUE DATA. WHEN THE
ANL STATISTICAL STUDY OF FATIGUE IS COMPLETED, FINAL
CONCLUSIONS CAN BE MADE ABOUT RISK SIGNIFICANT COMPONENTS
FROM THE FATIGUE STANDPOINT. ALSO, THE STUDY RESULTS MAY BE
USED IN VALUE-IMPACT ANALYSES, IF NECESSARY, TO SUPPORT ANY
POSSIBLE BACKFIT REQUIREMENTS BY THE STAFF.

GENERIC ISSUE 166 ENTITLED "MONITORING OF FATIGUE
TRANSIENT LIMITS FOR REACTOR COOLANT" WAS INITIATED BY AN
NRR MEMORANDUM FROM T. MURLEY TO J. SNIEZEK, DATED APRIL 1,
1993.

THE OUTCOME OF THE CONTRACTUAL STUDY BY ANL UNDER GI-78
WILL BE INTEGRATED IN THE NRR WORK SCOPE AS IT DEVELOPS.
THE PROGRESS AND THE DETAILS OF THE ONGOING ANL WORK WILL BE
MONITORED UNDER GI-78.

A F F E C T E D D O C U M E N T S

*** T O B E D E T E R M I N E D ***

P R O B L E M / R E S O L U T I O N

THERE HAS BEEN A THREE-MONTH SLIP IN THE SCHEDULE DUE TO
UNAVAILABILITY OF ANL PERSONNEL AT THE START OF THIS

GENERIC ISSUE MANAGEMENT CONTROL SYSTEM

ISSUE NUMBER: 78

TITLE: MONITORING OF FATIGUE TRANSIENT LIMITS FOR REACTOR COOLANT

AFFECTED DOCUMENTS

PROBLEM/RESOLUTION

PROGRAM, AND THE ADDITION OF MILESTONES TO HAVE A PRESENTATION OF PRELIMINARY RESULTS TO RES AND NRR PERSONNEL AND TO REVISE THE PRB ANALYSIS OF THE FATIGUE ISSUE. THE TECHNICAL EVALUATIONS BY THE CONTRACTOR ARE NOW NEARLY COMPLETE. NRR INDICATES THAT THE SCHEDULE DELAY IS ACCEPTABLE FOR THEIR NEEDS.

M I L E S T O N E S

	ORIGINAL DATE	CURRENT DATE	ACTUAL DATE
GI-78 ASSIGNED TO RES/DSIR/EIB	07/92	--	07/92
DRAFT PRA PARAMETRIC STUDY ISSUED FOR PEER REVIEW AND COMMENT TO OTHER NRC DIVISIONS	10/92	--	11/92
PREPARE DRAFT SOW FOR ANL CONTRACT FIN-6077-3	02/93	--	02/93
SOW APPROVED BY DSIR DIRECTOR	03/93	--	03/93
ISSUE SOW FOR ANL CONTRACT FIN-6077-3	03/93	--	03/93
ANL RECEIVES NRC/RES APPROVAL TO START WORK	05/93	--	06/93
ANL/RES MEETING TO DISCUSS STATISTICAL STUDIES	07/93	--	07/93
ANL PRESENTATION TO RES AND NRR	02/94	02/94	--
ANL COMPLETES DRAFT REPORT	08/93	02/94	--
ANL COMPLETES FINAL REPORT	09/93	03/94	--
REVISE PRA ANALYSIS OF FATIGUE ISSUE	03/94	03/94	--
CLOSEOUT MEMO TO NRR	04/94	04/94	--

RUN DATE: 02/03/94

PAGE: 20

GENERIC ISSUE MANAGEMENT CONTROL SYSTEM

ISSUE NUMBER: 83

TITLE: CONTROL ROOM HABITABILITY

IDENTIFICATION DATE: 11/83C

PRIORITIZATION DATE: 06/84C

TYPE: GSI

PRIORITY:

TASK MANAGER: C. FERRELL

OFFICE/DIVISION/BRANCH: RES /DSIR/SAIB

ACTION LEVEL: ACTIVE

STATUS: NR

TAC NUMBERS:

RESOLUTION DATE: 04/94

WORK AUTHORIZATION: MEMO FOR H. DENTON FROM W. DIRCKS, DATED 8/15/83.

FIN

CONTRACTOR

CONTRACT TITLE

A2328
A2334
B2929
D2096
L2560

ANL
ANL
PNL
SAIC
PNL

IN-PLANT REVIEWS ON CONTROL ROOM HABITABILITY
IN-PLANT REVIEWS ON CONTROL ROOM HABITABILITY
MATH. MODELS CONTROL ROOM HABITABILITY
CONTROL ROOM VULNERABILITIES
CONTROL ROOM HABITABILITY SYSTEM REVIEW MODELS

WORK SCOPE

ASSESS ADEQUACY OF HABITABILITY CRITERIA, NRR REVIEW, PLANT SYSTEMS AND COMPONENTS, AND TESTING PROCEDURES RELATIVE TO CONTROL ROOM HABITABILITY, INCLUDING THOSE AREAS OF ACRS CONCERN. A FINAL REPORT WILL BE PREPARED

SUMMARIZING RESULTS AND PROVIDING RECOMMENDATIONS. TWO TASKS (DEVELOP MODELS OF CHALLENGES TO HABITABILITY AND REVIEW PROCEDURES) HAVE BEEN ADDED.

STATUS

FINAL REPORT FROM SAIC ON "CONTROL ROOM HABITABILITY SYSTEM REVIEW MODELS" (NUREG/CR-5659) ISSUED IN DECEMBER 1990.
FINAL REPORT FROM PNL ON EXTRAN COMPUTER CODE [NUREG/CR-5659] ISSUED IN MARCH 1991. PNL CONTRACTOR REPORT ON TOXICITY LEVELS ISSUED JULY 1991 (NUREG/CR-5669).

NRR GENERIC LETTER IN CONCURRENCE. PNL HABITABILITY MODEL RECEIVED; BEING CHECKED. PNL CONTRACT EXTENDED UNTIL APRIL 1994 TO COMPLETE NUREG/CR AND PROVIDE CONSULTATION TO NRR.

AFFECTED DOCUMENTS

PROBLEM/RESOLUTION

SRP 6.4, 9.4 1, 12.3 6.5.1, 2.3, 2.2 1-2 2.3, 9.5.1, RGS 1.78, 1.95, 1.29, 1.52, 1.101, 1.140; NUREG-0737, ITFM III D.3.4, II 6.2, 10 CFR 50: 50.48, APPENDIX R TO 10 CFR 50; NUREG-0700; NUREG-0737, SUPPLEMENT 1.

PROPOSED GL HAS BEEN PLACED ON HOLD.

AS OF FY94 QUARTER 1

GENERIC ISSUE MANAGEMENT CONTROL SYSTEM

ISSUE NUMBER : 83

TITLE : CONTROL ROOM HABITABILITY

----- M I L E S T O N E S -----	ORIGINAL DATE	CURRENT DATE	ACTUAL DATE
WORKSHOP ON CRH	11/83	--	11/83
AWARD OF CONTRACT	02/84	--	01/84
VISIT TO INITIAL PLANT	03/84	--	05/84
REPORT ON CRH CRITERIA TO NRR DIRECTOR	06/84	--	06/84
COMPLETION OF CONTRACTOR VISITS TO OPERATING AND NTOL PLANTS	08/84	--	08/84
SOW WRITTEN FOR 12-PLANT SURVEY AND GENERIC STUDIES	11/84	--	10/84
RFP SENT TO POTENTIAL CONTRACTOR(S)	12/84	--	11/84
RECEIPT OF PROPOSAL(S)	02/85	--	03/85
INITIATE PROGRAM TO VISIT 12 ADDITIONAL OPERATING PLANTS	09/85	--	09/85
COMPLETE VISIT TO 11 OPERATING REACTORS	09/86	--	01/87
ISSUE TRANSFER MEMO TO RES	03/87	--	12/87
SOW SENT TO PNL FOR "MATHEMATICAL MODELS OF CONTROL ROOM HABIT- ABILITY SYSTEMS EFFECTIVENESS" (FIN B2929-8)	04/88	--	04/88
SOW SENT TO SAIC FOR "CONTROL ROOM VULNER- ABILITIES" FIN 020969	04/88	--	04/88
REQUEST FOR PROCUREMENT ACTION SENT TO CHANGE NRR CONTRACT WITH SAIC TO INCLUDE T/A IN MODELING CONTROL ROOM VULNERABILITIES (D2096)	05/88	--	05/88
RECEIPT OF PROPOSAL	06/88	--	06/88
DRAFT CODE FPF	10/88	--	11/88

GENERIC ISSUE MANAGEMENT CONTROL SYSTEM

ISSUE NUMBER: 83

TITLE: CONTROL ROOM HABITABILITY

M I L E S T O N E S	ORIGINAL DATE	CURRENT DATE	ACTUAL DATE
DRAFT CODE EXTRAN	10/88	--	12/88
DRAFT REPORT ON TOXIC LEVELS	11/88	--	01/89
PNL CONTRACT EXTENDED ON "NO-COST BASIS" UNTIL 03/90	10/89	--	10/89
ISSUE SAIC CONTRACT	03/89	--	10/89
REVISE TASK ACTION PLAN	10/89	--	03/90
RECOMMENDED REVISED DOSE ASSESSMENT MODELS	09/89	--	06/90
FINAL REPORTS ON MODELS STUDY	02/89	--	07/91
DRAFT CLOSEOUT MEMO	07/91	--	05/93
ISSUE MEMO TO ACRS	07/91	02/94	--
ACRS REVIEW	08/93	03/94	--
CLOSEOUT MEMO	09/93	04/94	--
PNL UPDATE COMPUTER CODE DOCUMENTATION	09/92	04/94	--

GENERIC ISSUE MANAGEMENT CONTROL SYSTEM

ISSUE NUMBER: 105 TITLE: INTERFACING SYSTEMS LOCA AT LWRS
 IDENTIFICATION DATE: 10/84C PRIORITIZATION DATE: 06/85C TYPE: GSI PRIORITY: H
 TASK MANAGER: G. BURDICK OFFICE/DIVISION/BRANCH: RES /DSIR/SAIB ACTION LEVEL: ACTIVE STATUS: 3B
 TAC NUMBERS: RESOLUTION DATE: 06/93C
 WORK AUTHORIZATION: MEMO FOR R. BERNERO FROM H. DENTON DATED 6/11/85.

FIN	CONTRACTOR	CONTRACT TITLE
A3829 B5699	BNL INEL	INTERFACING SYSTEMS LOCA INTERSYSTEM LOCA (GI-105)

WORK SCOPE

PERFORM A DETAILED IS-LOCA SPECIFIC PRA FOR 3 PWR PLANTS TO DETERMINE CORE-MELT FREQUENCIES AND RISK CONTRIBUTED BY FAILURE OF REACTOR COOLANT PRESSURE BOUNDARY AS CAUSED BY FAILURE OF PRESSURE ISOLATION VALVES (PIVS) AND RESULTING INTERFACING SYSTEMS LOCA. WHEN SUFFICIENT PROGRESS IS MADE ON PWR STUDIES, AND IN LIGHT OF INSIGHTS THUS GAINED, THE BWR WORK SCOPE WILL BE FORMULATED. DEVELOP NEW LICENSING CRITERIA IF NECESSARY FOR THE GENERIC RESOLUTION. IT HAS BEEN PROPOSED THAT PRESSURE ISOLATION VALVES, WHICH ARE TWO

VALVES IN SERIES AT EACH INTERFACE SEPARATING THE HIGHER PRESSURE RCS FROM ATTACHED LOW PRESSURE SYSTEMS, SHOULD BE PERIODICALLY LEAK-RATE TESTED OR VISUALLY INSPECTED. SOME OLDER PLANTS DO NOT PERFORM SUCH TESTS ON ALL PIV'S. STUDIES HAVE BEEN PERFORMED TO ATTEMPT TO DETERMINE IF IMPOSING SUCH PERIODIC TESTING ON ALL PIV'S IN ALL PLANTS IS JUSTIFIED. THESE STUDIES SUPPLEMENT THOSE ONGOING TO IDENTIFY SIGNIFICANT HARDWARE AND HUMAN CONTRIBUTIONS TO RISK OF IS-LOCA OUTSIDE CONTAINMENT FROM 3 PWR'S.

STATUS

THE RESOLUTION OF GI-105 WILL ENCOMPASS GI-98, "RHR SUCTION VALVE TESTING." THE PERTINENT MILESTONES FROM GENERIC ISSUE II E 6.1 HAVE BEEN TRANSFERRED TO GENERIC ISSUE 105 AND REVISED. NUREG/5102, "INTERFACING SYSTEM LOCA IN PWR'S," AND NUREG/5154, "INTERFACING SYSTEM LOCA IN BWR'S," ARE COMPLETE. ETEC HAS ALSO COMPLETED A STUDY OF THE EFFECTIVENESS OF LEAK-RATE TESTING AS AN INDICATION OF VALVE CONDITION. NUREG/CR-5604, "ASSESSMENT OF IS-LOCA RISK-METHODOLOGY AND APPLICATION TO A BABCOCK AND WILCOX NUCLEAR POWER PLANT," WAS PUBLISHED IN APRIL 1992 AS WERE

NUREG/CR-5744 AND NUREG/CR-5745 FOR WESTINGHOUSE AND CE PLANTS RESPECTIVELY. PUBLISHED IN MAY WAS NUREG/CR-5862, "SCREENING METHODS FOR DEVELOPING INTERNAL PRESSURE CAPACITIES IN SYSTEMS INTERFACING WITH NUCLEAR POWER PLANT REACTOR COOLANT SYSTEMS." RESOLUTION PACKAGE DELIVERED TO ACRS AND NRR. PACKAGE CONSISTS OF INFORMATION NOTICE, FINAL NUREG/CR, AND REGULATORY ANALYSIS. ADVANCED REACTOR SRP DRAFT SENT TO NRR. ACRS MEETING HELD, AND ACRS CONCURRENCE LETTER RECEIVED. RESOLUTION MEMORANDUM SENT, AND EDO CONCURRENCE RECEIVED. THE ISSUE IS RESOLVED.

GENERIC ISSUE MANAGEMENT CONTROL SYSTEM

ISSUE NUMBER: 105

TITLE: INTERFACING SYSTEMS LOCA AT LWRS

AFFECTED DOCUMENTS

NONE

PROBLEM/RESOLUTION

NONE

M I L E S T O N E S	ORIGINAL DATE	CURRENT DATE	ACTUAL DATE
(RES/EIB) REVIEW RES PROPOSALS FOR PRESSURE ISOLATION VALVE LEAK TESTS & OTHERS. PROVIDE RECS TO RES FOR REVS TO TEST SCOPE/LAB SEL.	05/84	--	05/84
(RES) INITIATE CONTRACT FOR PIV LEAK RATE TESTING (ETEC)	05/84	--	10/84
GSI 105 ASSIGNED TO RSB/DSI/NRR PRELIMINARY DISCUSSION OF WORK PLAN WITH BNL	06/85	--	06/85
BNL PRELIMINARY COST ESTIMATE OF WORK PLAN DEVELOP WORK PLAN AND CONTRACT PROPOSAL; DRAFT WORK PLAN TO DIRECTORS DSI, DE, DL, DHFS, AEOD	09/85	--	08/85
INCORPORATE COMMENTS INTO WORK PLAN	10/85	--	10/85
FINAL WORK PLAN APPROVED BY DSI DIRECTOR AND FORWARDED TO DST	10/85	--	11/85
ISSUE CONTRACT TO BNL	10/85	--	12/85
MODIFY CONTRACT	01/86	--	01/86
EXPAND CONTRACT TO INCLUDE PWRS	03/86	--	02/86
(RES/EIB) PREPARE GENERIC LETTER TO ALL LICENSEES ASKING FOR PROOF OF VERIFICATION OF RCPB BY TESTING PIV'S	07/86	--	07/86
(NRR) DIRECTOR NRR FORWARD 50.54(F) GENERIC LETTER FOR CRGR APPROVAL	08/86	--	02/87
(CRGR) REVIEW AND APPROVE 50.54(F) LTR TO ALL LICENSEES RE VERIF. OF PIV'S; (NRR/DSRO) ISSUE 10CFR50.54(F) LTR ON PIV VERIFICATION	09/86	--	03/87

GENERIC ISSUE MANAGEMENT CONTROL SYSTEM

ISSUE NUMBER: 105

TITLE: INTERFACING SYSTEMS LOCA AT LWRS

M I L E S T O N E S	ORIGINAL DATE	CURRENT DATE	ACTUAL DATE
(RES/EIB) REVIEW LIC RESP TO 10 CFR 50.54(F) ON PIV VERIF. DETERMINE PROCS. ARE ADEQUATE TO ASSURE INTEG OF RCTR COOLANT PRES BOUNDARY	01/87	--	10/87
FINAL DRAFT REPORTS RECEIVED FROM BNL INCLUDING VALUE/IMPACT ANALYSIS	07/86	--	02/88
PUBLISH NUREG REPORTS	02/89	--	02/89
REVISED TAP	04/89	--	02/90
NRR IS-LOCA AUDITS OF THREE PLANTS	04/90	--	08/90
COMPLETE B&W PLANT ANALYSIS	09/90	--	09/90
PUBLISH IMPEL COMPONENT FRAGILITY RESULTS	09/90	--	10/90
ACRS REVIEW B&W PLANT ANALYSIS COMPLETE WESTINGHOUSE ANALYSIS	11/90	--	12/90
COMPLETE CE PLANT ANALYSIS	12/90	--	06/91
EVALUATE IS-LOCA INSIDE CONTAINMENT EVALUATE IS-LOCA EXTERNAL EVENT INITIATORS	12/90	--	06/91
ENVIRONMENTAL QUALIFICATION ANALYSIS	08/91	--	08/91
BWR SITE VISIT	10/91	--	10/91
OGCB GUIDANCE REQUESTED	04/92	--	04/92
ADVANCED REACTOR INSIGHTS	04/92	--	05/92
NRR INFORMATION NOTICE	04/92	--	05/92
LEAD PM ASSIGNMENT REQUESTED	04/92	--	06/92
PWR AUX BLDG SURVEY	02/92	--	06/92
PWR ANALYSIS COST ESTIMATE	12/92	--	09/92

GENERIC ISSUE MANAGEMENT CONTROL SYSTEM

ISSUE NUMBER: 105

TITLE: INTERFACING SYSTEMS LOCA AT LWRS

M I L E S T O N E S	ORIGINAL DATE	CURRENT DATE	ACTUAL DATE
BWR ANALYSIS	03/92	--	10/92
DRAFT NUREG/CR AND PACKAGE TO ACRS	03/92	--	04/93
ACRS MEETINGS	10/88	--	05/93
FINAL NUREG/CR	03/92	--	06/93
RESOLUTION PACKAGE TO EDO	10/89	--	06/93
EDO CONCURRENCE	06/93	--	06/93
REG ANALYSIS PUBLISHED	07/93	--	07/93
PUBLISH IN	10/93	02/94	--

GENERIC ISSUE MANAGEMENT CONTROL SYSTEM

ISSUE NUMBER: 106 TITLE: PIPING AND USE OF HIGHLY COMBUSTIBLE GASES IN VITAL AREAS
 IDENTIFICATION DATE: 10/84C PRIORITIZATION DATE: 11/87C TYPE: GSI PRIORITY: M
 TASK MANAGER: R. WOODS OFFICE/DIVISION/BRANCH: RES /DSIR/SAIB ACTION LEVEL: ACTIVE STATUS:
 TAC NUMBERS: RESOLUTION DATE: 11/93C
 WORK AUTHORIZATION: MEMORANDUM FOR G. ARLOTTO FROM E. BECKJORD DATED 11/3/87

FIN	CONTRACTOR	CONTRACT TITLE
A6198	INEL	DISTRIBUTION OF HYDROGEN GAS IN VITAL AREAS

WORK SCOPE

SEARCH LERS AND INDUSTRIAL DATABASES FOR OPERATIONAL SAFETY EXPERIENCE INVOLVING COMBUSTIBLE GASES. EVALUATE INFORMATION TO ESTIMATE FREQUENCY AND SEVERITY OF POSSIBLE EVENTS. ASSESS EVENTS IN TERMS OF CURRENTLY ACCEPTABLE STAFF GUIDELINES FOR EQUIPMENT DESIGN AND HANDLING OF COMBUSTIBLE GASES AND REEVALUATE RISK ASSOCIATED WITH LACK OF PLANT CONFORMANCE TO CURRENTLY ACCEPTABLE GUIDELINES (E.G., INCORPORATION OF EXCESS FLOW CHECK VALVES OR EQUIVALENT PROTECTION AGAINST H2 PIPE BREAKS). EVALUATE NEED FOR ANY REVISION TO GUIDELINES AND PREPARE RESOLUTION PACKAGE WITH V/I ANALYSIS. THE SCOPE WAS EXPANDED TO INCLUDE: A) SITE VISITS TO BWR'S AND A SCOPING ANALYSIS TO DETERMINE THE NEED FOR MORE DETAILED RISK ANALYSES OF BWR'S,

B) EVALUATION OF THE RISK ASSOCIATED WITH HYDROGEN STORAGE FACILITIES USING INFORMATION FROM THE REGIONAL OFFICES REQUESTED BY NRR, AND C) A REVISED PRA APPROACH FOR PWR'S BASED ON A DETAILED EVALUATION OF INDIAN POINT 2 AND SENSITIVITY CALCULATIONS FOR OTHER PLANTS.
 H2 STORAGE FACILITIES REMOVED FROM SCOPE IN 04/92. REASON IS NEED FOR PEER REVIEW OF NEW DETONATION AND BUILDING RESPONSE CALCULATIONS WHICH DO NOT USE TNT EQUIVALENCY METHOD. NEW SEPARATION DISTANCE CRITERIA SIGNIFICANTLY LARGER THAN EPRI GUIDELINE VALUES FOR LARGER SUPPLIES. IMPACT OF NEW CALCULATIONS METHOD ON PLANTS NOT EVALUATED.

STATUS

MEETINGS HELD WITH A) ACRS SUBCOMMITTEE IN 07/92, B) ACRS FULL COMMITTEE IN 07/92, AND C) CRGR IN 09/92. ACRS RECOMMENDED THAT PROPOSED GENERIC LETTER BE SENT TO BOTH BWR'S AND PWR'S AND THAT PLANT EVALUATIONS INCLUDE WEAKNESSES OF FIRE BARRIERS IN EVALUATION OF DETONATION DAMAGE. CRGR CONCLUDED THAT RISK IS INSUFFICIENT TO JUSTIFY BURDEN OF GENERIC LETTER AND THAT SUBJECT COULD BE COVERED UNDER

IPEEE. REVISED LETTER FROM EDO TO ACRS PREPARED TO REFLECT THIS CONCLUSION. NEW GENERIC LETTER WAS SENT FOR INFORMATION ONLY TO BWR'S AND PWR'S REFERRING TO NEW INFORMATION DEVELOPED UNDER GSI-106 AND THE STAFF INTEREST IN THE COVERAGE OF RISK FROM HYDROGEN IN THE IPEEE. ISSUE RESOLVED AND CLOSED OUT.

GENERIC ISSUE MANAGEMENT CONTROL SYSTEM

ISSUE NUMBER: 106

TITLE: PIPING AND USE OF HIGHLY COMBUSTIBLE GASES IN VITAL AREAS

AFFECTED DOCUMENTS

SRP 9.5.1

PROBLEM/RESOLUTION

INFORMATION NOTICE HELD UP IN NRR DUE TO HIGHER PRIORITY WORK.

M I L E S T O N E S	ORIGINAL DATE	CURRENT DATE	ACTUAL DATE
DEVELOP TAP	03/88	--	09/88
PRELIMINARY DISCUSSIONS WITH CONTRACTOR	08/88	--	09/88
DRAFT SOW ISSUED TO CONTRACTOR	09/88	--	09/88
ISSUE CONTRACT	08/88	--	12/88
TAP APPROVED	08/88	--	01/89
COMPLETE SURVEY OF PLANT INFORMATION	02/89	--	02/89
COMPLETE REPORT ON SURVEY	03/89	--	03/89
COMPLETE REPORT ON RECOMENDATIONS FOR HYDROGEN SYSTEMS	05/89	--	03/89
ISSUE REVISED TAP	10/89	--	12/89
COMPLETE PRELIMINARY TER ON TASK 5 WORK ON TANK FARM HAZARD	11/89	--	11/89
COMPLETE DRAFT TR ON TASK 2 (LEAK FREQ AND POTENTIAL DAMAGE)	01/90	--	01/90
COMPLETE LETTER REPORT ON TASK 3 RESULTS OF CONSULTATION WITH INDUSTRY	01/90	--	01/90
COMPLETE FINAL TER ON TASK 5 WORK ON TANK FARM HAZARD	02/90	--	04/90
COMPLETE DRAFT TER ON TASK 6 ON PWR PRA	04/90	--	04/90

GENERIC ISSUE MANAGEMENT CONTROL SYSTEM

ISSUE NUMBER : 106

TITLE : PIPING AND USE OF HIGHLY COMBUSTIBLE GASES IN VITAL AREAS

M I L E S T O N E S	ORIGINAL DATE	CURRENT DATE	ACTUAL DATE
COMPLETE DRAFT TER ON TASK 1 (SURVEY OF BWR'S)	06/90	--	07/90
COMPLETE DRAFT NUREG/CR ON TASK 1 (SCOPING PRA FOR BWR'S)	02/91	--	04/91
COMPLETE FINAL DRAFT NUREG/CR ON PWR'S	03/91	--	03/91
KENNEDY DRAFT REPORT COMPLETE	04/91	--	04/91
KENNEDY REPORT ON EXPLOSION STANDARDS COMPLETE	05/91	--	05/91
COMPLETE CAMERA READY COPY NUREG/CR ON PWR'S	04/91	--	08/91
DRAFT REGULATORY ANALYSIS COMPLETED	01/90	--	09/91
COMPLETE CAMERA READY COPY OF NUREG/CR TASK 1 (BWR'S)	03/91	--	12/91
DTR PACKAGE COMPLETED	02/90	--	03/92
DTR PACKAGE TO ACRS	04/92	--	04/92
DTR PACKAGE TO NRR, AECD, OGC FOR COMMENTS-	06/92	--	06/92
ACRS REVIEW (SUBCOMMITTEE AND FULL COMMITTEE)	07/92	--	07/92
DTR PACKAGE TO DIRECTOR, RES	07/92	--	08/92
CRGR MEETING ON PROPOSED RESOLUTION	08/92	--	09/92
PREPARE GENERIC LETTER PKG WITH DRAFT REGULATORY ANALYSIS	10/92	--	11/92
GENERIC LETTER PACKAGE SENT TO NRR FOR ISSUANCE	10/92	--	12/92
REGULATORY ANALYSIS ISSUED AS NUREG	11/92	--	06/93

GENERIC ISSUE MANAGEMENT CONTROL SYSTEM

ISSUE NUMBER: 106

TITLE: PIPING AND USE OF HIGHLY COMBUSTIBLE GASES IN VITAL AREAS

M I L E S T O N E S	ORIGINAL DATE	CURRENT DATE	ACTUAL DATE
NRR ISSUES INFORMATION NOTICE	12/92	--	10/93
ISSUE CLOSEOUT MEMORANDUM	12/92	--	11/93

RUN DATE: 02/03/94

GENERIC ISSUE MANAGEMENT CONTROL SYSTEM

PAGE: 31

ISSUE NUMBER: 145

TITLE: ACTIONS TO REDUCE COMMON CAUSE FAILURES

IDENTIFICATION DATE: 07/85C

PRIORITIZATION DATE: 02/92C

TYPE: GSI

PRIORITY: M

TASK MANAGER: E. CHELLIAH

OFFICE/DIVISION/BRANCH: RES /DSIR/PRAB

ACTION LEVEL: ACTIVE

STATUS: NR

TAC NUMBERS:

RESOLUTION DATE: 02/94

WORK AUTHORIZATION: MEMORANDUM FROM E. S. BECKJORD TO W. MINNERS DATED
FEBRUARY 11, 1992

FIN
L2553

CONTRACTOR
BNL

CONTRACT TITLE

TECHNICAL SUPPORT TO GI-145

WORK SCOPE

THE ACTIONS REQUIRED TO RESOLVE THE ISSUE ARE MINIMAL. THE NUREG REPORT, SO AS TO BE READILY AVAILABLE TO LICENSEES.
SCOPE OF THE ISSUE IS ONLY TO COLLECT THE INFORMATION BNL COMPLETED ITS DRAFT FINDINGS (TASK 2).
AVAILABLE ON COMMON MODE FAILURES AND PUBLISH IT IN A

STATUS

BNL SUBMITTED A FINAL REPORT TO DSIR/PRAB. DSIR ALSO INITIATED A PREPARATION ORDER FOR CONTRACT CLOSE OUT

AFFECTED DOCUMENTS

POSSIBLE NUREG/CR.

PROBLEM/RESOLUTION

NONE.

MILESTONES

-----	ORIGINAL DATE	CURRENT DATE	ACTUAL DATE
COMPLETION OF GI PRIORITIZATION	02/92	--	02/92
COMPLETION OF TAP	04/92	--	04/92
ISSUE CONTRACT FUND	01/93	--	04/93

AS OF FY94 QUARTER 1

GENERIC ISSUE MANAGEMENT CONTROL SYSTEM

ISSUE NUMBER: 145

TITLE: ACTIONS TO REDUCE COMMON CAUSE FAILURES

M I L E S T O N E S	ORIGINAL DATE	CURRENT DATE	ACTUAL DATE
CONTRACTOR'S DRAFT FINDINGS	06/92	--	09/93
STAFF'S FINDINGS (DSIR)	10/92	--	11/93
CONTRACTOR'S FINAL FINDINGS	09/92	--	12/93
DECISION ON STAFF'S FINDINGS TO PUBLIC	11/92	02/94	--
CLOSEOUT ISSUE	01/93	02/94	--

RUN DATE: 02/02/94

GENERIC ISSUE MANAGEMENT CONTROL SYSTEM

PAGE: 33

ISSUE NUMBER: 147
IDENTIFICATION DATE: 06/89C
TASK MANAGER: J. CHEN
TAC NUMBERS:

TITLE: FIRE-INDUCED ALTERNATE SHUTDOWN/CONTROL ROOM PANEL INTERACTIONS
PRIORITIZATION DATE: 08/92C TYPE: LI PRIORITY:
OFFICE/DIVISION/BRANCH: NRR /DSIR/SAIB ACTION LEVEL: ACTIVE STATUS: 5
RESOLUTION DATE: 01/94

WORK AUTHORIZATION: MEMORANDUM FOR W. MINNERS FROM E. BECKJORD DATED AUGUST 26, 1992

FIN	CONTRACTOR	CONTRACT TITLE
NONE	NONE	NONE

WORK SCOPE

TO DEVELOP GUIDANCE FOR THE STAFF REVIEW OF LICENSEE IPEEE SHUTDOWN/CONTROL ROOM PANEL INTERACTIONS
SUBMITTALS IN THE AREA RELATED TO FIRE-INDUCED ALTERNATE

STATUS

TASK REASSIGNED TO SAIB IN FEBRUARY 1993.

AFFECTED DOCUMENTS

TO BE DETERMINED.

PROBLEM/RESOLUTION

NONE.

MILESTONES

MILESTONES	ORIGINAL DATE	CURRENT DATE	ACTUAL DATE
GI-147 ASSIGNED TO RES/DSIR/EIB	09/92	--	09/92
REASSIGNED TO SAIB	02/93	--	02/93
DRAFT STAFF REVIEW GUIDANCE ON FIRE-INDUCED ALTERNATE SHUTDOWN/CONTROL ROOM PANEL INTERACTION	08/93	--	09/93

AS OF FY94 QUARTER 1

RUN DATE: 02/03/94

GENERIC ISSUE MANAGEMENT CONTROL SYSTEM

PAGE: 34

ISSUE NUMBER: 147

TITLE: FIRE-INDUCED ALTERNATE SHUTDOWN/CONTROL ROOM PANEL INTERACTIONS

M I L E S T O N E S	ORIGINAL DATE	CURRENT DATE	ACTUAL DATE
----- STAFF IPEEE REVIEW GUIDANCE FOR GI-147	10/93	01/94	--

AS OF FY94 QUARTER 1

RUN DATE: 02/03/94

GENERIC ISSUE MANAGEMENT CONTROL SYSTEM

PAGE: 35

ISSUE NUMBER: 148 TITLE: SMOKE CONTROL AND MANUAL FIRE-FIGHTING EFFECTIVENESS
 IDENTIFICATION DATE: 04/89C PRIORITIZATION DATE: 08/92C TYPE: LI PRIORITY:
 TASK MANAGER: J. CHEN OFFICE/DIVISION/BRANCH: RES /DSIR/SAIB ACTION LEVEL: ACTIVE STATUS:
 TAC NUMBERS: RESOLUTION DATE: 06/94
 WORK AUTHORIZATION: MEMORANDUM FROM E. S. BECKJORD TO W. MINNERS DATED
 AUGUST 26, 1992

FIN CONTRACTOR ----- CONTRACT TITLE -----

WORK SCOPE

 TO DEVELOP STAFF GUIDANCE FOR THE REVIEW OF LICENSEES' MANUAL FIRE-FIGHTING EFFECTIVENESS.
 IPEEE SUBMITTALS RELATED TO THE EFFECTS OF SMOKE AND

STATUS

 TASK REASSIGNED TO SAIB/SECTION B IN FEBRUARY 1993.
 SCHEDULE CHANGED TO CONFORM TO IPEEE SCHEDULE.

AFFECTED DOCUMENTS

PROBLEM/RESOLUTION

TBD

NONE

MILESTONES

-----	ORIGINAL DATE	CURRENT DATE	ACTUAL DATE
GI-148 ASSIGNED TO RES/DSIR/RPSI	09/92	--	09/92
REASSIGNED TO SAIB	02/93	--	02/93
DRAFT STAFF REVIEW GUIDANCE RELATED TO SMOKE AND MANUAL FIRE-FIGHTING EFFECTIVENESS	09/93	04/94	--

AS OF FY94 QUARTER 1

RUN DATE: 02/03/94

GENERIC ISSUE MANAGEMENT CONTROL SYSTEM

PAGE: 36

ISSUE NUMBER: 148

TITLE: SMOKE CONTROL AND MANUAL FIRE-FIGHTING EFFECTIVENESS

M I L E S T O N E S	ORIGINAL DATE	CURRENT DATE	ACTUAL DATE
----- STAFF REVIEW GUIDANCE FOR GI-148	----- 11/93	----- 06/94	----- --

AS OF FY94 QUARTER 1

GENERIC ISSUE MANAGEMENT CONTROL SYSTEM

ISSUE NUMBER: 155.1
 IDENTIFICATION DATE: 03/79
 TASK MANAGER: L. SOFFER
 TAC NUMBERS:

TITLE: MORE REALISTIC SOURCE TERM ASSUMPTIONS
 PRIORITIZATION DATE: 02/92C
 OFFICE/DIVISION/BRANCH: RES /DSIR/SAIB
 TYPE: GSI
 ACTION LEVEL: ACTIVE

PRIORITY:
 STATUS: NR
 RESOLUTION DATE: 07/94

WORK AUTHORIZATION: MEMO FOR W. MINNERS FROM E. BECKJORD DATED 2/26/92

FIN	CONTRACTOR	CONTRACT TITLE
B0854	ORNL	IODINE CHEMICAL FORMS
L1535	BNL	RADIONUCLIDE RELEASE CHARACTER
L1611	INEL	TIMING ANALYSIS OF FUEL FAILURES
L2035	SNL	SOURCE TERM REMOVAL MECHANISMS
W6067	ORNL	CHEMICAL FORM OF SOURCE TERMS

W O R K S C O P E

DEVELOP A REVISED ACCIDENT SOURCE TERM TO REPLACE THAT GIVEN IN TID-14844, MAKING USE OF CURRENT RESEARCH INSIGHTS ON FISSION PRODUCT TIMING, COMPOSITION, MAGNITUDE,

AND IODINE CHEMISTRY. IMPLEMENT REVISED ACCIDENT SOURCE TERM, TOGETHER WITH SEVERE ACCIDENT INSIGHTS, IN A REVISION OF 10 CFR PART 50.

S T A T U S

FINAL REPORTS ON IODINE CHEMICAL FORM AND TIMING (NUREG/CR-5732 AND NUREG/CR-5787) HAVE BEEN ISSUED. DRAFT REPORT ON FISSION PRODUCT COMPOSITION AND MAGNITUDE (NUREG/CR-5747) HAS BEEN ISSUED. CONTRACTOR STUDIES ON FISSION PRODUCT REMOVAL MECHANISMS ARE CONTINUING. A COMMISSION PAPER (SECY-92-127) AND DRAFT PROPOSED NUREG REPORT CONTAINING A REVISED ACCIDENT SOURCE TERM HAVE BEEN ISSUED. A COMMISSION

MEETING WAS HELD ON APRIL 20, 1992. NUREG-1465 ISSUED JULY 28, 1992. BNL AND ORNL REVIEW OF PUBLIC COMMENTS RECEIVED APRIL 27, 1993. ORNL REPORT ON MELCOR RESULTS (NUREG/CR-5942) HAS BEEN ISSUED. FINAL SNL REPORT ON DF OF OVERLYING POOL (NUREG/CR-5901) ISSUED. FINAL BNL REPORT ON FISSION PRODUCT COMPOSITION AND MAGNITUDE (NUREG/CR-5747) ISSUED.

AFFECTED DOCUMENTS

PROBLEM/RESOLUTION

R.G. 1.3, 1.4, SRP 15.6.5
 R.G. 1.7, 1.25

DELAYS AT SNL IN COMPLETING NUREG/CR REPORTS.

GENERIC ISSUE MANAGEMENT CONTROL SYSTEM

ISSUE NUMBER: 155.1

TITLE: MORE REALISTIC SOURCE TERM ASSUMPTIONS

M I L E S T O N E S	ORIGINAL DATE	CURRENT DATE	ACTUAL DATE
IODINE/TID SCHEDULE TO COMMISSION	04/90	--	05/90
PRA BASED SOURCE TERM PAPER TO COMMISSION	05/90	--	05/90
FISSION PRODUCT TIMING PAPER TO COMMISSION	07/90	--	08/90
REVISED BNL DRAFT REPORT	07/90	--	08/90
TID REPORT SCHEDULE - IODINE CHEMICAL FORM REPORT ISSUED FOR COMMENT	12/90	--	07/91
PROPOSED REV. TO TID TO RES DIRECTOR	07/91	--	09/91
PROPOSED REVISION TO NRR/OGC	09/91	--	09/91
DRAFT TID TO ACRS	11/91	--	12/91
COMPLETE ACRS REVIEW	12/91	--	01/92
OFFICE CONCURRENCE ON TID	10/91	--	04/92
DRAFT BNL REPORT ISSUED FOR COMMENT	07/91	--	02/92
TID TO EDO	01/92	--	04/92
TID TO COMMISSION	02/92	--	04/92
FISSION PRODUCT TIMING REPORT ISSUED FOR COMMENT (INEL)	12/90	--	03/92
ISSUE REVISED TID FOR COMMENT	02/92	--	07/92
ISSUE ORNL IODINE REPORT AS FINAL	04/92	--	04/92
SNL REPORT ON POOL OF NUREG/CR ISSUED FOR COMMENT	08/92	--	09/92
ISSUE INEL REPORT FINAL	08/92	--	09/92
END COMMENT PERIOD ON DRAFT SOURCE TERM NUREG	08/92	--	10/92

GENERIC ISSUE MANAGEMENT CONTROL SYSTEM

ISSUE NUMBER: 155.1

TITLE: MORE REALISTIC SOURCE TERM ASSUMPTIONS

M I L E S T O N E S	ORIGINAL DATE	CURRENT DATE	ACTUAL DATE
----- RESPONSE TO SRM ON IMPACT OF INCREASED AMOUNT OF RADIATION (WITS 9200124) -- DRAFT COMMISSION PAPER AVAILABLE	09/92	--	11/92
-- RES CONCURRENCE	09/92	--	12/92
-- COMMISSION PAPER TO EDO AND OFFICE CONCURRENCE	10/92	--	01/93
-- COMMISSION PAPER TO COMMISSION	11/92	--	02/93
SNL SPRAY NUREG/CR ISSUED FINAL	11/92	--	06/93
ORNL NUREG/CR ON MELCOR ISSUED	09/93	--	09/93
SNL REPORT ON POOL OF NUREG/CR ISSUED FINAL	11/92	--	11/93
ISSUE BNL REPORT FINAL	06/92	--	11/93
SNL NUREG/CR ON MELCOR ISSUED	11/93	01/94	--
SNL SUPP. POOL NUREG/CR ISSUED FINAL	03/93	02/94	--
SNL AEROSOL NUREG/CR ISSUED FINAL	03/93	05/94	--
COMPLETE REVISED DRAFT SOURCE TERM NUREG (FOR USE WITH PART 50 REVISION)	11/92	06/94	--
TRANSMIT CLOSEOUT MEMO TO ACRS	11/92	06/94	--
ISSUE CLOSEOUT MEMO	12/92	07/94	--

RUN DATE: 02/03/94

GENERIC ISSUE MANAGEMENT CONTROL SYSTEM

PAGE: 40

ISSUE NUMBER: 165

TITLE: SPRING-ACTUATED SAFETY AND RELIEF VALVE RELIABILITY

IDENTIFICATION DATE: 10/92C

PRIORITIZATION DATE: 11/93C

TYPE: GSI

PRIORITY: H

TASK MANAGER: C. HRABAL

OFFICE/DIVISION/BRANCH: RES /DSIR/EIB

ACTION LEVEL: ACTIVE

STATUS:

TAC NUMBERS:

RESOLUTION DATE: _____

WORK AUTHORIZATION: NOVEMBER 26, 1993, MEMORANDUM FROM E. BECKJORD TO W. MINNERS

FIN

TBD

CONTRACTOR

TO BE DETERM

CONTRACT TITLE

TO BE DETERMINED

W O R K S C O P E

1. SELECT OR LOAD PRA'S FOR ANALYSIS IN SARA VERSIONS 4.0 OR 5.0
2. DETERMINE WHAT SYSTEMS HAVE SPRING-ACTUATED RELIEF VALVES AND OTHER IMPORTANT INFORMATION BY REVIEWING P&IDS FROM THE SELECTED PRAS.
3. PERFORM ANALYSIS TO DETERMINE THE EXTENT OF VALVE FAILURE REQUIRED TO FAIL THE SYSTEM IN WHICH THEY OPERATE.
4. REVIEW LERS TO MORE ACCURATELY DETERMINE THE FAILURE PROBABILITY FOR SPRING-ACTUATED RELIEF VALVES.

5. CALCULATE THE CHANGE IN CORE DAMAGE FREQUENCY FROM INCLUDING RELIEF VALVES IN THE PRAS.
6. PERFORM COST/BENEFIT ANALYSES TO DETERMINE THE BEST FIX FOR THE PROBLEM AND WHICH VALVES SHOULD BE TESTED.
7. PREPARE REGULATORY ANALYSIS.
8. RESOLVE GSI-165 WITH EITHER THE IMPLEMENTATION OF THE BEST FIX OR A CLOSEOUT MEMO IF THE COST/BENEFIT IS NEGLIGIBLE.

S T A T U S

INITIAL INPUT FOR GIMCS, INCLUDING MILESTONES THROUGH APPROVAL OF FINAL TAP, WAS PROVIDED TO R. EMRIT ON

JANUARY 5, 1994.

AFFECTED DOCUMENTS

ASME OM CODE

PROBLEM/RESOLUTION

*** NONE ***

GENERIC ISSUE MANAGEMENT CONTROL SYSTEM

ISSUE NUMBER: 165

TITLE: SPRING-ACTUATED SAFETY AND RELIEF VALVE RELIABILITY

----- M I L E S T O N E S -----	ORIGINAL DATE	CURRENT DATE	ACTUAL DATE
GSI 165 ASSIGNED TO RES/DSIR/EIB	11/93	--	11/93
DRAFT TAP TO DSIR DEPUTY DIRECTOR	04/94	04/94	--
DRAFT TAP APPROVED BY DSIR DIRECTOR AND FORWARDED TO OTHER RES DIVISION DIRECTORS, AEOO, AND NRR FOR COMMENT	06/94	06/94	--
INCORPORATE COMMENTS IN TAP	08/94	08/94	--
FINAL TAP APPROVED BY DSIR DIRECTOR	11/94	11/94	--
BALANCE OF MILESTONES TBD	--	--	--

RUN DATE: 02/03/94

GENERIC ISSUE MANAGEMENT CONTROL SYSTEM

PAGE: 42

ISSUE NUMBER: 168

TITLE: ADEQUACY OF FATIGUE LIFE OF METAL COMPONENTS

IDENTIFICATION DATE: 04/93C

PRIORITIZATION DATE: 06/93C

TYPE: GSI

PRIORITY:

TASK MANAGER: J. FAIR

OFFICE/DIVISION/BRANCH: NRR /DE /EMEB

ACTION LEVEL: ACTIVE

STATUS: NR

TAC NUMBERS: M86344

RESOLUTION DATE: _____

WORK AUTHORIZATION: APRIL 1, 1993, MEMORANDUM FROM T. MURLEY TO J. SNIEZEK

FIN

CONTRACTOR

CONTRACT TITLE

WORK SCOPE

EVALUATE FATIGUE ADEQUACY OF OPERATING PLANT COMPONENTS.

STATUS

ACTION PLAN APPROVED. NUMARC AND NRC HELD PUBLIC MEETING ON SEPTEMBER 17, 1993. STAFF MET WITH ASME PRESSURE VESSEL

RESEARCH COUNCIL (PVRC) ON OCTOBER 12, 1993. CONTRACT NEGOTIATIONS ARE IN PROGRESS.

AFFECTED DOCUMENTS

NONE

PROBLEM/RESOLUTION

NONE

MILESTONES

	ORIGINAL DATE	CURRENT DATE	ACTUAL DATE
INFORM COMMISSION (PERIODIC, 6 MONTHS)	02/94	02/94	--
MEET WITH INDUSTRY (PERIODIC)	09/93	--	09/93
PROGRAMMATIC REVIEW	12/93	--	12/93
RISK ASSESSMENT	03/94	03/94	--

AS OF FY94 QUARTER 1

GENERIC ISSUE MANAGEMENT CONTROL SYSTEM

ISSUE NUMBER: 166

TITLE: ADEQUACY OF FATIGUE LIFE OF METAL COMPONENTS

M I L E S T O N E S	ORIGINAL DATE	CURRENT DATE	ACTUAL DATE
DATA COLLECTION AND ANALYSIS	05/94	05/94	--
TECHNICAL ISSUES	05/94	05/94	--
OPTIONS FOR RESOLUTION	06/94	06/94	--

RUN DATE: 02/03/94

GENERIC ISSUE MANAGEMENT CONTROL SYSTEM

PAGE: 44

ISSUE NUMBER: 168
IDENTIFICATION DATE: 04/93C
TASK MANAGER: G. HUBBARD
TAC NUMBERS: M85648

TITLE: ENVIRONMENTAL QUALIFICATION OF ELECTRICAL EQUIPMENT
PRIORITIZATION DATE: 06/93C TYPE: GSI
OFFICE/DIVISION/BRANCH: NRR / /SPLB ACTION LEVEL: ACTIVE

PRIORITY:
STATUS: NR
RESOLUTION DATE: _____

WORK AUTHORIZATION: APRIL 1, 1993, MEMORANDUM FROM T. MURLEY TO J. SNIEZEK

FIN	CONTRACTOR	CONTRACT TITLE
A2336 E2097	ANL SCIEN TECH	RISK IMPACT OF EQ EQ FOR OPERATING REACTORS

WORK SCOPE

- | | |
|---|---|
| 1. TO GATHER, REVIEW, AND EVALUATE OPERATING EXPERIENCE DATA AND EQUIPMENT REPLACEMENT SCHEDULES FOR NUCLEAR POWER PLANTS TO PROVIDE INSIGHT AS TO WHERE NRC SHOULD FOCUS ITS | RESOURCES IN THE PERFORMANCE OF EQ AGING REVIEWS.
2. TO PERFORM A DETAILED ASSESSMENT OF THE RISK ASSOCIATED WITH EQ ISSUES. |
|---|---|

STATUS

THE PROGRAMMATIC REVIEW IS CONTINUING. SPLB HAS PERFORMED TWO SITE VISITS. A CONTRACTOR FOR SPLB HAS REVIEWED OPERATING EXPERIENCE AND PROVIDED A DRAFT REPORT ON EQ-RELATED EVENTS ON 12/07/93. A CONTRACTOR FOR SPSB PROVIDED A DRAFT REPORT IN DEC. 1993 STATING THAT THERE IS	NO RELIABILITY DATA FOR EQ. SPLB AND RES WILL DEVELOP AVAILABLE EQ INFORMATION INTO A DATABASE WHICH COULD PROVIDE SOME RELIABILITY INSIGHT INTO EQ EQUIPMENT. RES/DE HELD A WORKSHOP ON 11/15-16/93 TO GATHER INFORMATION ON PREAGING, EQ TESTING, OPERATING EXPERIENCE, AND CONDITION MONITORING.
---	---

AFFECTED DOCUMENTS

PROBLEM/RESOLUTION

ISSUED IN 92-81 AND 93-33.

NONE

MILESTONES

INFORM COMMISSION

ORIGINAL DATE

05/93

CURRENT DATE

--

ACTUAL DATE

05/93

AS OF FY94 QUARTER 1

GENERIC ISSUE MANAGEMENT CONTROL SYSTEM

ISSUE NUMBER: 168

TITLE: ENVIRONMENTAL QUALIFICATION OF ELECTRICAL EQUIPMENT

M I L E S T O N E S	ORIGINAL DATE	CURRENT DATE	ACTUAL DATE
MEET WITH INDUSTRY	--	--	--
PROGRAMMATIC REVIEW	06/94	06/94	--
DATA COLLECTION AND ANALYSIS	08/94	08/94	--
RISK ASSESSMENT	10/94	10/94	--
TECHNICAL ISSUES	10/94	10/94	--
OPTIONS FOR RESOLUTION	--	--	--
IMPLEMENTATION	--	--	--

RUN DATE: 02/03/94

GENERIC ISSUE MANAGEMENT CONTROL SYSTEM

PAGE: 46

ISSUE NUMBER: B-17

TITLE: CRITERIA FOR SAFETY-RELATED OPERATOR ACTIONS

IDENTIFICATION DATE: 06/78C

PRIORITIZATION DATE: 11/83C

TYPE: GSI

PRIORITY: M

TASK MANAGER: N. GROSSMAN

OFFICE/DIVISION/BRANCH: RES /DSR /HFB

ACTION LEVEL: ACTIVE

STATUS:

TAC NUMBERS:

RESOLUTION DATE: 12/94

WORK AUTHORIZATION: NRR FY-84 OPERATING PLAN (APPENDIX G)

FIN	CONTRACTOR	CONTRACT TITLE
A0465	LLNL	INTEGRATION OF HUMAN PERFORMANCE FACTORS FULLY INTO THE RELIABILITY EVALUATION PROCESS (TASKS 5 AND 6)

WORK SCOPE

THE WORK SCOPE INCLUDES: (1) DEVELOP TIME CRITERION FOR SAFETY RELATED OPERATOR ACTIONS; (2) DETERMINE WHETHER OR NOT AUTOMATIC ACTUATION WILL BE REQUIRED; AND (3) DEVELOP GUIDANCE AND REVIEW CRITERIA FOR LICENSEE PROPOSALS TO USE

MANUAL OPERATOR ACTION OR AUTOMATIC ACTION. THE SCOPE OF THIS GENERIC ISSUE COVERS DESIGN BASIS ACCIDENTS OR EVENTS CRITERIA FOR SAFETY RELATED OPERATOR ACTIONS DURING SEVERE ACCIDENTS WILL BE COVERED UNDER THE SEVERE ACCIDENT PROGRAM.

STATUS

THIS ISSUE WAS TRANSFERRED FROM NRR TO RES WITH THE 4/87 NRC REORGANIZATION (4/16/87 MEMO FROM W. RUSSELL TO T. SPEIS). NO WORK HAD BEEN COMPLETED ON THE ISSUE PRIOR TO TRANSFER. DURING 1983 AND 1984, RES WORKED ON THIS ISSUE. DURING THAT TIME DATA WERE COLLECTED BY RES CONCERNING OPERATOR RESPONSE TIMES. ANSI/ANS 58.8 WAS PUBLISHED IN SEPTEMBER 1984. THE GENERIC ISSUE WAS LATER TRANSFERRED TO NRR. NO

WORK WAS COMPLETED BY NRR ON THIS ISSUE. WORK HAS BEGUN WHICH SHOULD LEAD TO RESOLUTION OF THIS GSI BY ENDORSING ANSI/ANS 58.8. THIS STANDARD IS BEING REVISED BASED ON NEW INDUSTRY DATA. LLNL BEGAN WORK TO MANAGE THE TECHNICAL EFFORT INVOLVING EPRI AND A SUBCONTRACTOR TO EPRI IN MARCH 1989.

AFFECTED DOCUMENTS

PROBLEM/RESOLUTION

TO BE PROVIDED LATER

SCHEDULE HAS SLIPPED DUE TO SLIPPAGE IN ANS SCHEDULE FOR APPROVAL OF STANDARD ANSI/ANS 58.8. ALSO, SINCE NRC ENDORSEMENT WILL BE IN THE FORM OF A REGULATORY GUIDE, TIME IS NEEDED TO GO THROUGH THE REGULATORY GUIDE APPROVAL PROCESS.

GENERIC ISSUE MANAGEMENT CONTRL SYSTEM

ISSUE NUMBER: B-17

TITLE: CRITERIA FOR SAFETY-RELATED OPERATOR ACTIONS

M I L E S T O N E S	ORIGINAL DATE	CURRENT DATE	ACTUAL DATE
MEET WITH ANSI/ANS 58.8	07/88	--	06/89
DEVELOP TAP	08/87	--	06/89
NUPPSCO SUBMITS DRAFT OF 58.8	04/92	--	01/92
COMPLETE DRAFT OF 58.8	08/92	--	08/92
NUPPSCO BALLOTS DRAFT STD 58.8	10/92	--	03/93
NUPPSCO ISSUES FINAL 58.8 (SCHEDULE DEPENDS UPON ISSUANCE OF ANS 58.8 REVISION BY END OF 10/93.)	08/93	03/94	--
ISSUE DRAFT REG. GUIDE FOR 90-DAY COMMENT PERIOD	12/93	03/94	--
ACRS REVIEW	04/93	05/94	--
CRGR REVIEW	04/93	05/94	--
REVISE REG. GUIDE TO REFLECT PUBLIC COMMENTS	05/94	07/94	--
ACRS REVIEW	07/94	09/94	--
CRGR REVIEW	07/94	09/94	--
ISSUE FINAL REG. GUIDE	09/94	12/94	--
CLOSE OUT ISSUE	09/94	12/94	--

GENERIC ISSUE MANAGEMENT CONTROL SYSTEM

ISSUE NUMBER: B-55 TITLE: IMPROVE RELIABILITY OF TARGET ROCK SAFETY-RELIEF VALVES
 IDENTIFICATION DATE: 06/78C PRIORITIZATION DATE: 11/83C TYPE: GSI PRIORITY: M
 TASK MANAGER: F CHERNY OFFICE/DIVISION/BRANCH: RES /DSIR/EIB ACTION LEVEL: ACTIVE STATUS:
 TAC NUMBERS: RESOLUTION DATE: 08/97
 WORK AUTHORIZATION: NRR FY-84 OPERATING PLAN, (APPENDIX G)

FIN	CONTRACTOR	CONTRACT TITLE
NONE	NONE	NONE

WORK SCOPE

REVIEW BWR UTILITY APPROACHES TO INCREASING SRV RELIABILITY TO OBTAIN SUFFICIENT DATA BASE TO SUPPORT THE CONCLUSION THAT THIS ISSUE HAS BEEN RESOLVED. THE UTILITY APPROACHES MAY INCLUDE: (1) DEMONSTRATING SUFFICIENT RELIABILITY OF 3-STAGE VALVES BY STRICT

CONFORMANCE TO NSSS VENDOR INSTRUCTIONS, (2) DEMONSTRATING SUFFICIENT RELIABILITY OF 2-STAGE VALVES AFTER DESIGN AND/OR SYSTEM MODIFICATIONS, AND (3) USING DIFFERENT VALVES AND DEMONSTRATING THEIR RELIABILITY.

STATUS

WHILE INITIAL PERFORMANCE WITH THE PH 13-8 MO REPLACEMENT DISCS INDICATED AN IMPROVEMENT, ADDITIONAL PERFORMANCE DATA MADE IT APPARENT THE NEW DISCS ARE NOT PROVIDING THE IMPROVED PERFORMANCE THAT WAS ORIGINALLY EXPECTED. THE BWROG CONCLUDED THE PH 13-8 MO REPLACEMENT DISCS ARE NOT A VIABLE SOLUTION FOR ELIMINATING SETPOINT DRIFT AND RECOMMENDED THEY NO LONGER BE INSTALLED TO RESOLVE THIS PROBLEM. THE BWROG IS CURRENTLY WORKING ON THE DESIGN OF CATALYSTS (OPTION 1) FOR THE TARGET ROCK 2-STAGE SRV'S. INSTALLATION OF REDESIGNED SRV'S WITH CATALYSTS IS SCHEDULED FOR THE FALL OF 1993. THEY ARE ALSO WORKING ON A PARALLEL CONTINGENCY OPTION 2, I.E., THE USE OF PRESSURE SWITCHES SIMILAR TO THOSE USED ON BWR/S PLANTS. THE BWROG HAS MODIFIED THE SCHEDULE WITH THE CATALYST DESIGN FOR THE TARGET ROCK 2-STAGE SRV'S. THE STAFF MET WITH THE BWROG ON AUGUST 11, 1992, AND DISCUSSED DETAILS OF THE CATALYST DESIGN (OPTION 1) AND ALSO THE PROGRESS ON THE PARALLEL CONTINGENCY OPTION 2. AN INDEPENDENT DESIGN REVIEW HAS BEEN COMPLETED IN THE THIRD QUARTER OF FY '92 FOR OPTION 1. LABORATORY TESTING IS COMPLETE AND DEMONSTRATES THAT THE ALLOY COATING

IS VERY EFFECTIVE IN REDUCING OXYGEN CONCENTRATION AT THE VALVE DISK/SEAT INTERFACE. PILOT VALVE CYCLING TESTS THAT DEMONSTRATE CATALYST ADHERENCE TO THE TREATED SURFACES HAVE ALSO BEEN SUCCESSFULLY COMPLETED. A DESIGN SPECIFICATION FOR THE ALLOY COATING AND THE TREATED SURFACES HAS BEEN SUBMITTED TO THE BWROG FOR APPROVAL. THE PILOT VALVE CONTACT SURFACES ARE A STELLITE TO STELLITE MATERIAL WITH THE PLATINUM-STEELITE ALLOY COATING APPLIED TO THE PILOT VALVE DISK AND THE STABILIZER DISK. HOWEVER, AT THE FEBRUARY 1993 BWROG MEETING, IT WAS DECIDED TO APPLY THE ALLOY COATING TO THE PILOT VALVE DISK ONLY AND NOT TO THE STABILIZER DISK. IT WAS ALSO DECIDED TO GIVE EQUAL STATUS TO BOTH THE CATALYST OPTION AND THE PRESSURE SWITCH OPTION. THIS IS IN APPARENT RECOGNITION OF THE FACT THAT SOME OF THE BWROG MAY BE LOSING INTEREST IN THE CATALYST OPTION DUE TO THE LONG LEAD TIME TO OBTAIN THE CATALYTIC COATED VALVE PARTS. A LICENSING TOPICAL REPORT ON THE PRESSURE SWITCH OPTION AND A DETAILED LETTER STATUS FROM THE BWROG HAD BEEN SCHEDULED FOR SUBMITTAL TO THE NRC IN APRIL 1993.

GENERIC ISSUE MANAGEMENT CONTROL SYSTEM

ISSUE NUMBER: B-55

TITLE: IMPROVE RELIABILITY OF TARGET ROCK SAFETY-RELIEF VALVES

S T A T U S

SINCE MOST OF THE NRC STAFF EFFORT RELATED TO THE BALANCE OF THE GENERIC RESOLUTION OF THIS GENERIC ISSUE IS EXPECTED TO BE UNDERTAKEN WITHIN NRR, AND SINCE ANY PLANT MODIFICATIONS RESULTING FROM THE RESOLUTION OF THIS ISSUE ARE EXPECTED TO BE EITHER VOLUNTARY OR JUSTIFIED ON A "COMPLIANCE" BASIS, RES HAS FORMALLY REQUESTED THAT THE ISSUE BE TRANSFERRED TO NRR FOR THE BALANCE OF THE GENERIC RESOLUTION PERIOD AND SUBSEQUENT PLANT SPECIFIC IMPLEMENTATION. A TRANSFER MEMORANDUM WAS TRANSMITTED TO NRR ON AUGUST 3, 1993.

IN A MEMORANDUM DATED OCTOBER 13, 1993, NRP DECLINED TO ACCEPT THE TRANSFER OF RESPONSIBILITY FOR MANAGING THE BALANCE OF THE RESOLUTION OF THE GSI. RES WILL CONTINUE THE GSI MANAGEMENT, BUT ONLY AT A MINIMAL LEVEL DUE TO RESOURCE CONSTRAINTS. THE EFFORT WILL CONSIST OF MAINTAINING GIMCS AND BEING THE FORMAL BWROG CONTACT FOR THE GSI.

BY TELEPHONE ON JANUARY 10, 1994, THE BWROG ADVISED THAT THE PRESSURE SWITCH TOPICAL REPORT WOULD BE SUBMITTED TO NRC IN FEBRUARY OF 1994.

AFFECTED DOCUMENTS

NONE

PROBLEM/RESOLUTION

NONE

M I L E S T O N E S

	ORIGINAL DATE	CURRENT DATE	ACTUAL DATE
MEETING WITH BWR OWNERS SETPOINT DRIFT COMMITTEE	12/83	--	11/83
BWR OWNERS REPORT SUBMITTED	07/84	--	06/84
BOSTON EDISON REPORT SUBMITTED	--	--	08/84
COMPLETION OF NRP EVALUATION OF OWNERS GROUP AND BOSTON EDISON REPORTS	10/84	--	10/84
COMMUNICATION TO OG RE: NRC COMMENTS/CONCERNS	--	--	03/85
OG PRESENTATION TO THE STAFF	10/85	--	10/85
STAFF RECEIVES OWNERS' PROGRAM FOR ELIMINATING SET POINT DRIFT	10/85	--	10/85
OWNERS BEGIN TO INSTALL NEW PILOT DISKS TO TEST IN PLANT FOR ONE FUEL CYCLE	01/86	--	01/86

GENERIC ISSUE MANAGEMENT CONTROL SYSTEM

ISSUE NUMBER: B-55

TITLE: IMPROVE RELIABILITY OF TARGET ROCK SAFETY-RELIEF VALVES

M I L E S T O N E S	ORIGINAL DATE	CURRENT DATE	ACTUAL DATE
ISSUE STATUS REPORT ON INDUSTRY EFFORTS	12/87	--	01/88
ISSUE PROGRESS REPORT ON INDUSTRY EFFORTS	11/88	--	11/88
BWROG APPROVAL FOR FUNDING CATALYST/PRESSURE SWITCH PROGRAM	06/90	--	06/90
BWROG SELECT/DESIGN CATALYST	03/91	--	02/91
NRC STAFF ISSUE STATUS REPORT ON INDUSTRY PROGRESS	06/91	--	08/91
BWROG COMPLETES FINALIZING DESIGN AND LABORATORY TESTING	04/92	--	07/92
NRC STAFF ISSUE STATUS REPORT ON INDUSTRY PROGRESS	03/92	--	08/92
REASSESS ISSUE	03/93	--	05/93
REASSIGN TO NRR	08/93	--	08/93
NRR RESPONSE TO REASSIGNMENT	10/93	--	10/93
BWROG SUBMIT TOPICAL REPORT TO NRC DESCRIBING PRESSURE SWITCH OPTION	12/91	02/94	--
OWNERS BEGIN TO INSTALL CATALYSTS TO TEST IN PLANTS FOR FIRST FUEL CYCLE	06/94	06/94	--
NRR ISSUE APPROVAL FOR PRESSURE SWITCH OPTION	02/95	02/95	--
ISSUE CLOSEOUT MEMORANDUM	08/97	08/97	--

RUN DATE: 02/03/94

GENERIC ISSUE MANAGEMENT CONTROL SYSTEM

PAGE: 51

ISSUE NUMBER: B-56 TITLE: DIESEL GENERATOR RELIABILITY
IDENTIFICATION DATE: 06/78C PRIORITIZATION DATE: 11/83C TYPE: GSI PRIORITY: H
TASK MANAGER: A. SERKIZ OFFICE/DIVISION/BRANCH: RES /DSIR/EIB ACTION LEVEL: INACTIVE STATUS: 3A
TAC NUMBERS: RESOLUTION DATE: 06/93C
WORK AUTHORIZATION: NRR FY-83 OPERATING PLAN (APPENDIX G).

FIN	CONTRACTOR	CONTRACT TITLE
A1806 B2937	SNL PNL	GUIDELINES FOR EMERG. DIESEL GENERATOR (EDG) RELIABILITY PROGRAMS AGING CONSIDERATIONS-DIESEL GENERATOR RELIABILITY PROGRAM

WORK SCOPE

THE WORK SCOPE OF B-56 IS CLOSELY COORDINATED WITH THE RESOLUTION OF USI A-44, STATION BLACKOUT RESOLUTION OF USI A-44 INCLUDES, AMONG OTHER THINGS, THE NEED FOR A DIESEL RELIABILITY PROGRAM TO MAINTAIN THE RELIABILITY LEVEL OF EMERGENCY DIESEL GENERATORS (EDG) AT OR ABOVE RELIABILITY

LEVELS SELECTED FOR RESPONSE TO THE SBO RULE (10CFR50, SECTION 50.63).
THE CONCLUDING EFFORTS ON B-56 WILL BE DIRECTED AT CARRYING OUT ACTIONS RELATED TO THE SRM IN RESPONSE TO SECY-93-044.

STATUS

THE COMMISSION DIRECTED THE STAFF (SRM COMJC-91-001/001-A DATED JUNE 26, 1991) TO REVISE THE SBO RULE TO ADDRESS EDG RELIABILITY THROUGH A RESULTS-ORIENTED APPROACH UTILIZING THE FAILURE TO START AND LOAD-RUN TRIGGERS OUTLINED IN NUMARC'S INITIATIVE 5A. A REVISED RULE AND ACCOMPANYING REGULATORY GUIDE (RG 1.9, REV. 3) WAS ISSUED FOR COMMENT ON APRIL 21, 1992.

NUMARC AND INDUSTRY RESPONDEES OPPOSED THE NEED FOR THIS RULE CITING CURRENTLY INDUSTRY-WIDE EDG RELIABILITY LEVELS OF 99% AND RECOMMENDED THAT THE PLANNED MAINTENANCE RULE GUIDANCE BE USED TO MAINTAIN CURRENT LEVELS OF EDG RELIABILITY.

THE STAFF EVALUATED COMMENTS RECEIVED AND SUBMITTED A RECOMMENDATION TO THE COMMISSION THAT THE PROPOSED RULE AMENDMENT BE WITHDRAWN (SECY-93-044). IN PLACE OF THE PROPOSED RULE, THE STAFF RECOMMENDED THAT GUIDANCE FOR MONITORING EDG PERFORMANCE AGAINST TARGET LEVELS SELECTED FOR THE SBO ANALYSES BE INCORPORATED INTO THE REGULATORY GUIDE AND GUIDANCE BEING DEVELOPED BY NUMARC FOR IMPLEMENTA-

TION OF THE MAINTENANCE RULE (10 CFR 50.65). THE COMMISSION APPROVED THE STAFF'S RECOMMENDATIONS ON 03/25/93. THE MAINTENANCE RULE REGULATORY GUIDE WAS ISSUED JUNE 1993.

THE NRC HAS WITHDRAWN THE RULE AMENDMENT (58 FR 37884) TO 10 CFR 50.63 PROPOSED IN APRIL 1992 WHICH WOULD HAVE REQUIRED LICENSEES TO TEST AND MONITOR EMERGENCY DIESEL GENERATORS AGAINST SPECIFIED CRITERIA (58 FR 37884). IN ADDITION, REVISION 3 TO REGULATORY GUIDE 1.9, "SELECTION, DESIGN, QUALIFICATION AND TESTING OF EMERGENCY DIESEL GENERATOR UNITS USED AS CLASS 1E ONSITE ELECTRIC POWER SYSTEMS AT NUCLEAR POWER PLANTS," JULY 1993, WAS ISSUED. THESE TWO ACTIONS CONSTITUTE RESOLUTION OF GENERIC SAFETY ISSUE B-56, "DIESEL GENERATOR RELIABILITY."

THIS REVISION TO REGULATORY GUIDE 1.9 INTEGRATES INTO A SINGULAR REGULATORY GUIDE PERTINENT GUIDANCE PREVIOUSLY ADDRESSED IN REVISION 2 OF REGULATORY GUIDE 1.9, REVISION 1 OF REGULATORY GUIDE 1.108, AND GENERIC LETTER 84-15. THE GUIDANCE FOR THE MONITORING AND MAINTAINING OF EMERGENCY

AS OF FY94 QUARTER 1

GENERIC ISSUE MANAGEMENT CONTROL SYSTEM

ISSUE NUMBER: B-56

TITLE: DIESEL GENERATOR RELIABILITY

STATUS

DIESEL PERFORMANCE AGAINST VALUES SELECTED BY LICENSEES IN THEIR STATION BLACKOUT COPING ANALYSES HAS BEEN INCORPORATED INTO REGULATORY GUIDE 1.160, "MONITORING

THE EFFECTIVENESS OF MAINTENANCE AT NUCLEAR POWER PLANTS," JUNE 1993.

AFFECTED DOCUMENTS

REG GUIDE 1.9 WILL BE REVISED TO CONFORM TO SRM SECY-93-044.

PROBLEM/RESOLUTION

NONE.

M I L E S T O N E S

	ORIGINAL DATE	CURRENT DATE	ACTUAL DATE
DTR IDENTIFIED; ISSUED BY DSI DIRECTOR FOR STAFF COMMENT	02/83	--	02/83
PACKAGE TO NRR DIRECTOR	04/83	--	03/83
PACKAGE TO CRGR (29); CRGR REVIEW COMPLETE	05/83	--	07/83
SCOPE OF ISSUE MODIFIED TO COORDINATE TECHNICAL RESOLUTION OF B-56 WITH USI A-44	09/83	--	09/83
SCOPE OF ISSUE MODIFIED TO CONSIDER NUMBER OF COLD FAST START SURVEILLANCE TESTS	11/83	--	11/83
COORDINATED GSI B-56 AND USI A-44 CRGR PACKAGE SENT TO NRR DIRECTOR	02/84	--	01/84
REVISED PACKAGE TO CRGR - ACTUAL DATE 03/84; CRGR REVIEW COMPLETE	05/84	--	05/84
ISSUED 50.54(F) GL 84-15 TO LICENSEES	06/84	--	07/84
RECEIVED PUBLIC COMMENTS	09/84	--	11/84
ISSUED T/A CONTRACT TO BNL	06/85	--	06/85

GENERIC ISSUE MANAGEMENT CONTROL SYSTEM

ISSUE NUMBER: B-56

TITLE: DIESEL GENERATOR RELIABILITY

M I L E S T O N E S	ORIGINAL DATE	CURRENT DATE	ACTUAL DATE
PUBLISH CONTRACTOR REPORT ON LICENSEE RESPONSE TO 50.54(F)	04/86	--	04/86
REVISE SRPS AND REG. GUIDES (NRR)	03/88	--	05/88
DEVELOP DIESEL GENERATOR RELIABILITY PROGRAM GUIDELINES	10/86	--	04/88
CRGR PACKAGE TO RES DIRECTOR	01/87	--	07/88
PACKAGE TO CRGR/ACRS (53)	03/87	--	08/88
CRGR REVIEW COMPLETE	05/87	--	09/88
ACRS REVIEW COMPLETE; ISSUE FOR PUBLIC COMMENTS	06/87	--	11/88
PUBLISH NUREG/CR-5078, VOL 2	11/88	--	01/89
RECEIVE PUBLIC COMMENTS	08/87	--	03/89
FTR TO RES DIRECTOR; FTR TO CRGR/ACRS	09/87	--	08/89
RECEIVE NUMARC APPENDIX D SUBMITTAL REVISE CRGR PACKAGE	04/90	--	05/90
NRR & OGC CONCURRENCES	06/90	--	06/90
CRGR/ACRS MEETINGS (ACRS MEETING HELD IN AUGUST OF 1990)	07/90	--	07/90
COMMISSION PAPER TO EDO	10/90	--	10/90
RECEIVE B-56 SRM	07/91	--	06/91
REVISE SBO RULE; R.G. 1.9, REV. 3; PREPARE DRAFT FRN	07/91	--	11/91
PACKAGE TO ACRS	10/91	--	11/91

GENERIC ISSUE MANAGEMENT CONTROL SYSTEM

ISSUE NUMBER: B-56

TITLE: DIESEL GENERATOR RELIABILITY

M I L E S T O N E S	ORIGINAL DATE	CURRENT DATE	ACTUAL DATE
PACKAGE TO CRGR	08/91	--	11/91
CRGR AND ACRS MEETINGS	09/91	--	11/91
ISSUE LETTER TO NUMARC ON ACCELERATED TESTING	10/91	--	11/91
SUBMIT DRAFT PACKAGE TO EDO	10/91	--	01/92
PACKAGE TO COMMISSION (SECY-92-025)	01/92	--	01/92
COMMISSION RESPONSE	12/91	--	03/92
ISSUE PACKAGE FOR COMMENT	12/91	--	04/92
END 75-DAY PUBLIC COMMENT PERIOD	02/92	--	07/92
REVISE PACKAGE AND SUBMIT TO RES DIRECTOR	03/92	--	11/92
FORWARD B-56 PACKAGE TO ACRS	11/92	--	11/92
ACRS REVIEW	12/92	--	12/92
CRGR REVIEW	12/92	--	01/93
FORWARD RECOMMENDATION TO EDO	01/93	--	01/93
COMMISSION REVIEW	02/93	--	03/93
ISSUE MAINTENANCE RULE REG. GUIDE (WITH EDO GUIDANCE)	06/93	--	06/93
CLOSEOUT MEMO TO EDO	06/93	--	06/93
WITHDRAW SRO RULE AMENDMENT	08/93	--	07/93

GENERIC ISSUE MANAGEMENT CONTROL SYSTEM

ISSUE NUMBER: B-61 TITLE: ALLOWABLE ECCS EQUIPMENT OUTAGE PERIODS
 IDENTIFICATION DATE: 06/78C PRIORITIZATION DATE: 11/83C TYPE: GSI PRIORITY: M
 TASK MANAGER: A. BUSLIK OFFICE/DIVISION/BRANCH: RES /DSR /PRAB ACTION LEVEL: ACTIVE STATUS:
 TAC NUMBERS: RESOLUTION DATE: 05/95
 WORK AUTHORIZATION: NRR FY-84 OPERATING PLAN (APPENDIX G)

FIN	CONTRACTOR	CONTRACT TITLE
NONE	NONE	NONE

WORK SCOPE

IN GENERAL, PLANTS WILL BE CLASSIFIED SUCH THAT IN EACH CLASS SIMILAR ECCS OUTAGE UNAVAILABILITIES WILL HAVE SIMILAR IMPACTS ON RISK. THE EFFECT OF A GIVEN ECCS AVAILABILITY ON RISK WILL BE DETERMINED FOR EACH CLASS OF PLANT, AS WELL AS THE IMPACTS OF ANY CHANGES IN ALLOWABLE ECCS OUTAGE PERIOD. CONSIDERATION

OF SUPPORT SYSTEM OUTAGES MUST BE INCLUDED. DETAILS OF THE WORK SCOPE ARE CONTAINED IN THE DRAFT ACTION PLAN. BECAUSE CUMULATIVE OUTAGE TIMES ON COMPONENTS OTHER THAN ECCS COMPONENTS APPEAR IMPORTANT, THE WORK SCOPE HAS BEEN EXPANDED TO INCLUDE ALL SAFETY-RELATED COMPONENTS.

STATUS

TASK I HAS BEEN COMPLETED.

AFFECTED DOCUMENTS

TECHNICAL SPECIFICATION CHANGES ARE POSSIBLE. REVISED INSPECTION GUIDANCE FOR VOLUNTARY ENTRY INTO LCO'S.

PROBLEM/RESOLUTION

THE MAJOR PROBLEM ASSOCIATED WITH THE RESOLUTION OF B-61 IS AN ANALYSIS OF THE COSTS AND BENEFITS OF THE IMPOSITION OF A CUMULATIVE OUTAGE REQUIREMENT. BOTH THE FRENCH AND GERMANS USE AN APPROACH TO CUMULATIVE OUTAGE REQUIREMENT, APPLYING THE CUMULATIVE OUTAGE REQUIREMENT ONLY TO SCHEDULED MAINTENANCE. A STUDY OF THE NEED FOR CONTROL OF CUMULATIVE OUTAGE IS BEING PERFORMED IN-HOUSE. THE WORK TAKES INTO ACCOUNT THE MAINTENANCE RULE, AND THE NUMARC GUIDANCE FOR THE IMPLEMENTATION OF THE MAINTENANCE RULE, AS WELL AS CURRENT INSPECTION GUIDANCE ON VOLUNTARY ENTRY INTO LIMITED CONDITIONS OF OPERATION ACTION STATEMENTS TO PERFORM PREVENTIVE MAINTENANCE. AN INITIAL RECOMMENDATION FOR STAFF

GENERIC ISSUE MANAGEMENT CONTROL SYSTEM

ISSUE NUMBER: B-61

TITLE: ALLOWABLE ECCS EQUIPMENT OUTAGE PERIODS

AFFECTED DOCUMENTS

PROBLEM/RESOLUTION

COMMENTS WAS COMPLETED IN SEPTEMBER 1992. THIS INCLUDED A PROPOSED DRAFT TECHNICAL RESOLUTION. COMMENTS WERE RECEIVED ON THIS PROPOSED DTR FROM IMMEDIATE MANAGEMENT IN AUGUST 1993. THE DRAFT TECHNICAL RESOLUTION IS BEING REWRITTEN TO INCLUDE MORE SENSITIVITY STUDIES, TO IMPROVE THE UNCERTAINTY ANALYSIS, AND TO PROVIDE, IF POSSIBLE, A BETTER BASIS FOR THE BENEFIT OF THE MAINTENANCE RULE AND INSPECTION GUIDANCE. THE CURRENT VERSION OF THE DRAFT TECHNICAL RESOLUTION WOULD MODIFY THE INSPECTION GUIDANCE ON VOLUNTARY ENTRY INTO LCOS, BUT NOT IMPOSE A CUMULATIVE OUTAGE REQUIREMENT ON SCHEDULED MAINTENANCE, AS IS DONE IN FRANCE. IT IS CLEAR THAT A CUMULATIVE OUTAGE REQUIREMENT ON UNSCHEDULED MAINTENANCE WILL NOT BE IMPOSED.

M I L E S T O N E S

	ORIGINAL DATE	CURRENT DATE	ACTUAL DATE
DRAFT TAP ISSUED FOR DD APPROVAL	01/84	--	01/84
COMPLETE COORDINATION OF TECHNICAL RESOLUTION WITH RES	04/84	--	08/84
TAP APPROVED BY DIVISION DIRECTOR	05/84	--	07/84
TASK I-METHODOLOGY REVIEW AND DEVELOPMENT COMPLETED	11/84	--	04/85
TASK II-APPLY TO A PWR OR BWR	02/85	--	02/92
DEVELOP AN INITIAL RECOMMENDATION FOR STAFF COMMENTS	08/87	--	09/92
APPLY METHODOLOGY TO A SELECTION OF PLANTS	08/85	--	09/92
PREPARE DTR	04/86	03/94	--
PREPARE CRGR PACKAGE	06/86	04/94	--
REVIEW OF CRGR PACKAGE BY DIVISION DIRECTORS	07/86	05/94	--

GENERIC ISSUE MANAGEMENT CONTROL SYSTEM

ISSUE NUMBER: B-61

TITLE: ALLOWABLE ECCS EQUIPMENT OUTAGE PERIODS

M I L E S T O N E S	ORIGINAL DATE	CURRENT DATE	ACTUAL DATE
RES DIRECTOR REVIEW COMPLETE/ PACKAGE TO CRGR	08/86	06/94	--
CRGR REVIEW AND EDO APPROVAL COMPLETED	09/86	07/94	--
ISSUANCE OF FRN FOR PUBLIC COMMENT	11/86	08/94	--
SIXTY-DAY COMMENT PERIOD ENDS	01/87	10/94	--
DIVISION REVIEW OF PUBLIC COMMENTS COMPLETED, AND FINAL CRGR PACKAGE COMPLETED	02/87	12/94	--
RES REVIEW OF CRGR PACKAGE COMPLETED (DIVISION REVIEW AND RES DIRECTOR REVIEW)	04/87	02/95	--
ACRS REVIEW	09/93	03/95	--
CRGR REVIEW AND EDO APPROVAL COMPLETED	05/87	04/95	--
CLOSE OUT ISSUE	09/93	05/95	--

RUN DATE: 02/03/94

GENERIC ISSUE MANAGEMENT CONTROL SYSTEM

PAGE: 58

ISSUE NUMBER: B-64 TITLE: DECOMMISSIONING OF REACTORS
 IDENTIFICATION DATE: 06/78C PRIORITIZATION DATE: 11/83C TYPE: GSI PRIORITY:
 TASK MANAGER: L. BELL OFFICE/DIVISION/BRANCH: NMSS/LLWM/LLRB ACTION LEVEL: ACTIVE STATUS: NR
 TAC NUMBERS RESOLUTION DATE: 03/94
 WORK AUTHORIZATION: NRR FY-84 OPERATING PLAN, [APPENDIX G].

FIN	CONTRACTOR	CONTRACT TITLE
B2342	PNL	DECONTAMINATION AND DECOMMISSIONING REVIEW

WORK SCOPE

DEVELOP A GENERAL DECOMMISSIONING POLICY, PREPARE THE ATTENDENT CHANGES IN REGULATIONS, DEVELOP THE DETAILED INFORMATION NEEDED FOR USE IN LICENSING DECISIONS FOR

DECOMMISSIONING, AND ESTABLISH GUIDANCE FOR FACILITATION OF DECOMMISSIONING.

STATUS

FINAL RULE CHANGES WERE AFFIRMED BY THE COMMISSION ON MAY 12, 1988 AND PUBLISHED IN THE FR ON JUNE 27, 1988. NRC DECOMMISSIONING RESPONSIBILITIES WERE TRANSFERRED

TO NMSS. NMSS IS PREPARING THE SRP. THE SRP HAS BEEN SENT FOR OFFICE CONCURRENCE PRIOR TO SUBMITTAL TO CRGR. MEMO TO BE ISSUED TO TRACK SRP ISSUANCE IN SRP TRACKING SYSTEM.

AFFECTED DOCUMENTS

- (1) 10 CFR PARTS 50 AND 51 AMENDMENTS,
- (2) REGULATORY GUIDE 1.86 REVISION,
- (3) NEW REGULATORY GUIDES ON FINANCIAL ASSURANCE, TERMINATION SURVEYS AND FORMAT AND CONTENT FOR NUCLEAR REACTOR DECOMMISSIONING PLANS,
- (4) NEW STANDARD REVIEW PLAN FOR DECOMMISSIONING REVIEW.

PROBLEM/RESOLUTION

NONE.

MILESTONES

	ORIGINAL DATE	CURRENT DATE	ACTUAL DATE
CRGR REVIEW COMPLETED ON DECOMMISSIONING RULE {29}	08/84	--	08/84

AS OF FY94 QUARTER 1

GENERIC ISSUE MANAGEMENT CONTROL SYSTEM

ISSUE NUMBER: B-64

TITLE: DECOMMISSIONING OF REACTORS

M I L E S T O N E S	ORIGINAL DATE	CURRENT DATE	ACTUAL DATE
EDO APPROVAL OF DECOMMISSIONING - SECY - 34-354	09/84	--	09/84
COMMISSION APPROVAL OF SECY-84-354	12/84	--	12/84
ISSUANCE OF DRAFT RULE CHANGE FOR PUBLIC COMMENT	02/85	--	02/85
FORMAT AND CONTENT FOR DECOMMISSIONING	02/84	--	06/85
EXTENDED PUBLIC COMMENT PERIOD ON RULE COMPLETE	07/85	--	07/85
ISSUE TRANSFER MEMO TO RES (SRP)	01/86	--	01/86
REVISE DRAFT RULE INCORPORATING PUBLIC COMMENTS PREPARE FOR RES MANAGEMENT REVIEW	02/86	--	11/86
REVISE DRAFT RULES/PUBLIC COMMENTS REVIEWED AND APPROVED BY CRGR (53)	08/86	--	08/87
FTR TO EDO FOR TRANSMITTAL TO COMMISSION	10/86	--	12/87
FRM OF FINAL RULE	01/87	--	06/88
ISSUE MEMO TO TRANSFER TO SRP TRACKING SYSTEM	10/93	03/94	--

GENERIC ISSUE MANAGEMENT CONTROL SYSTEM

ISSUE NUMBER: HF 4.4
 IDENTIFICATION DATE: 08/83C
 TASK MANAGER: J. WACHTEL
 TAC NUMBERS:

TITLE: GUIDELINES FOR UPGRADING OTHER PROCEDURES
 PRIORITIZATION DATE: 10/84C
 OFFICE/DIVISION/BRANCH: RES /DSR /HFB

TYPE: GSI
 ACTION LEVEL: ACTIVE

PRIORITY: H
 STATUS:
 RESOLUTION DATE: 07/93C

WORK AUTHORIZATION: MEMO FOR H. THOMPSON FROM H. DENTON DATED 10/1/84,
 "HUMAN FACTORS PROGRAM PLAN"

FIN	CONTRACTOR	CONTRACT TITLE
B2360	PNL	DEVELOP GUIDANCE AND REGULATORY ACTION FOR OPS & AOPS
B2935	PNL	VALVE IMPACT ASSESSMENT OF PLANT OPERATING PROCEDURES
B2966	PNL	DEVELOP GUIDANCE & REGULATORY ACTION FOR OPS & AOPS
I2005	PNL	DEVELOP GUIDANCE & REGULATORY ACTION FOR OPS & AOPS
L1827	PNL	GUIDANCE FOR REVIEW OF PROGRAMS TO UPGRADE PROCEDURE AT NUCLEAR POWER PLANTS

WORK SCOPE

RECOMMEND IMPROVEMENTS IN NUCLEAR POWER PLANT OPERATING PROCEDURES AND DEVELOP AN IMPLEMENTATION PLAN FOR UPGRADING OPERATING PROCEDURES. NRC WILL DETERMINE THE AMOUNT OF UPGRADE EFFORT

NECESSARY FOR EACH TYPE OF PROCEDURE BASED ON A CONTRACTOR INTERIM REPORT AND INDUSTRY INPUTS WITH RESPECT TO COST AND BENEFITS.

STATUS

THIS ISSUE WAS TRANSFERRED FROM NRR TO RES WITH THE 4/12/87 NRC REORGANIZATION (4/10/87 MEMO FROM W. RUSSELL TO T. SPEIS). CONTRACTOR EFFORTS ON FIN B2360/PNL AND FIN B2966/PNL WERE COMPLETED AT THAT TIME. NUREG/CR'S-3968 AND -4613 WERE PUBLISHED IN FEBRUARY 1987 AS A RESULT OF WORK UNDER THESE FINs. REVIEW OF NUREG/CR'S-3968 AND -4613 PRIOR TO THE NRC REORGANIZATION INDICATED THE NEED FOR A REGULATORY ANALYSIS OF THIS ISSUE. NRR AWARDED AN ADDITIONAL CONTRACT TO PNL IN JUNE 1987 TO CONDUCT THAT ANALYSIS UNDER FIN I-2005. RESPONSIBILITY FOR THAT CONTRACT WAS TRANSFERRED TO RES BY NOVEMBER 16, 1987 MEMORANDUM FROM L. RUTH TO M. KALTMAN. A NEW RES TASK

MANAGER WAS ASSIGNED, AND THE SCHEDULE, BUDGET, AND SCOPE WERE REVISED IN JULY 1988. NUREG/CR-5458 WAS PUBLISHED IN DECEMBER 1989. AGREEMENT REACHED WITH NRR TO DEVELOP A PROCEDURES UPGRADE "GOOD PRACTICES" GUIDE. WORK TO BE INITIATED FEBRUARY 1991. STATEMENT OF WORK FOR "GOOD PRACTICES" PROJECT ISSUED MAY 1991. EFFORT EXPANDED BEYOND NORMAL AND ABNORMAL PROCEDURES, PER NRR, TO INCLUDE PROCEDURES DURING NONPOWER OPERATIONS. AT A MEETING WITH J. SNIJEK IN NOVEMBER 1991, THE FOCUS OF THIS PROJECT WAS CHANGED TO EMPHASIZE NONPOWER OPERATIONS AND MINIMIZE THE ATTENTION PAID TO NORMAL AND ABNORMAL PROCEDURES.

GENERIC ISSUE MANAGEMENT CONTROL SYSTEM

ISSUE NUMBER : HF 4.4

TITLE : GUIDELINES FOR UPGRADING OTHER PROCEDURES

AFFECTED DOCUMENTS

 LITERATURE REVIEW DOCUMENTING PROCEDURES PRACTICES AT
 NUCLEAR POWER PLANTS.

PROBLEM/RESOLUTION

 ALTHOUGH FINAL NUREG/CR WAS PUBLISHED IN DECEMBER 1989,
 REQUESTS FROM OTHER OFFICES RESULTED IN MAINTAINING THIS
 ISSUE OPEN PENDING A RESOLUTION OF WHETHER ADDITIONAL EFFORT
 IS NECESSARY UNDER THIS ISSUE. AS A RESULT OF NOVEMBER
 1991 MEETING WITH J. SNIEZEK, THE CONTRACT WAS REVISED
 TO ADDRESS ONLY THOSE PROCEDURES RELATED TO SHUTDOWN AND LOW
 POWER OPERATIONS. A RECENT NRR USER NEED LETTER (3/8/93
 MEMO FROM T. MURLEY) HAS WITHDRAWN THE NEED FOR ANY OF THE
 REMAINING WORK AND ACCORDINGLY THIS ISSUE WAS CLOSED
 OUT BASED UPON THE WORK DONE TO DATE.

M I L E S T O N E S

-----	ORIGINAL DATE	CURRENT DATE	ACTUAL DATE
PUBLISH FINAL REPORT ON OPERATING PROCEDURES PROJECT AS NUREG/CR	12/85	--	02/87
PUBLISH FINAL REPORT ON PROCEDURES INTERFACE PROJECT AS NUREG/CR	05/86	--	02/87
CONDUCT REGULATORY ANALYSIS FOR OPERATING PROCEDURES, ABNORMAL OPERATING PROCEDURES	02/87	--	07/89
DEVELOP PLAN FOR REGULATORY GUIDANCE	06/89	--	09/89
PREPARE CLOSEDOUT MEMO	04/93	--	04/93
CLOSE OUT ISSUE	06/89	--	07/93
CLOSE OUT CONTRACT BY DOCUMENTING LITERATURE SEARCH ON PROCEDURES FOR LOW POWER AND SHUTDOWN OPERATIONS AT NUCLEAR POWER PLANTS	08/92	03/94	--

GENERIC ISSUE MANAGEMENT CONTROL SYSTEM

ISSUE NUMBER: HF 5.1 TITLE: LOCAL CONTROL STATIONS
 IDENTIFICATION DATE: 08/83C PRIORITIZATION DATE: 10/84C TYPE: GSI PRIORITY: H
 TASK MANAGER: J. WACHTEL OFFICE/DIVISION/BRANCH: RES /DSR /HFB ACTION LEVEL: ACTIVE STATUS: 3B
 TAC NUMBERS: RESOLUTION DATE: 06/93C
 WORK AUTHORIZATION: MEMO TO H. THOMPSON FROM H. DENTON DATED 10/1/84.
 " HUMAN FACTORS PROGRAM PLAN"

FIN	CONTRACTOR	CONTRACT TITLE
A3972	BNL	LOCAL CONTROL STATIONS

WORK SCOPE

THE FIRST GOAL OF THIS WORK IS TO DETERMINE THE RISK TO PUBLIC HEALTH AND SAFETY RESULTING FROM HUMAN ERROR AT LOCAL CONTROL STATIONS WITHIN A NUCLEAR POWER PLANT AND THE COST OF POTENTIAL MODIFICATIONS. THE METHOD TO DETERMINE RISK IS THROUGH THE USE OF PROBABILISTIC RISK ANALYSIS (PRA). FOR PLANNING PURPOSES, WE ASSUME THE RESULTS OF THE VALUE/IMPACT ANALYSIS WILL REQUIRE ADDITIONAL WORK TO

RESOLVE THE LOCAL CONTROL STATIONS ISSUE. THE OBJECTIVE OF THIS WORK IS TO DEVELOP A REGULATORY GUIDE ON LOCAL CONTROL STATIONS AND REVIEW GUIDELINES FOR THE EVALUATION OF LOCAL CONTROL STATIONS. AS A RESULT OF SENIOR MANAGEMENT REVIEW OF THIS PROJECT ON JUNE 2, 1992, WORK SCOPE WAS REDUCED TO ELIMINATE DEVELOPMENT OF REGULATORY GUIDANCE AND REVIEW GUIDELINES.

STATUS

BY LETTER DATED DECEMBER 29, 1987, THE SOW WAS FORWARDED TO BNL. BY LETTER DATED MARCH 25, 1988 NRC AUTHORIZED BNL TO BEGIN PROGRAM. BNL BEGAN WORK IN MAY, 1988. A DRAFT VALUE/IMPACT REPORT WAS SUBMITTED BY BNL IN

FEBRUARY 1989. A REVISED SOW WAS DEVELOPED AND WORK BEGAN APRIL 1990. WORK TERMINATED (SEE MEMORANDUM FROM E. BECKJORD TO EDG ON JUNE 25, 1993).

AFFECTED DOCUMENTS

NUREG/CR-5572, GOOD PRACTICES FOR THE DESIGN OF LOCAL CONTROL STATIONS.

PROBLEM/RESOLUTION

NONE.

GENERIC ISSUE MANAGEMENT CONTROL SYSTEM

ISSUE NUMBER: HF 5 1

TITLE: LOCAL CONTROL STATIONS

M I L E S T O N E S	ORIGINAL DATE	CURRENT DATE	ACTUAL DATE
BEGIN VALUE/IMPACT STUDY OF LOCAL CONTROL STATIONS	02/88	--	05/88
COMPLETE PRELIMINARY VALUE/IMPACT STUDY	07/88	--	02/89
NRC COMPLETE REVIEW OF STUDY RESULTS	09/88	--	02/89
COMPLETE PLANT-SPECIFIC VALUE-IMPACT STUDY FOR EXPANDED LCS DEFINITION	09/90	--	04/91
PREPARE CLOSE OUT MEMO	05/93	--	05/93
CLOSE OUT MEMO TO EDO	12/92	--	06/93
CONTRACTOR TO PREPARE FINAL REPORT ON GOOD PRACTICES FOR DESIGN OF LOCAL CONTROL STATIONS	12/92	03/94	--

GENERIC ISSUE MANAGEMENT CONTROL SYSTEM

ISSUE NUMBER: HF 5.2
 IDENTIFICATION DATE: 08/83C
 TASK MANAGER: J. WACHTEL
 TAC NUMBERS:

TITLE: REV. CRIT. FOR HUMAN FACTORS ASPECTS OF ADV CONTROLS AND INSTRUMENT.
 PRIORITIZATION DATE: 10/84C
 OFFICE/DIVISION/BRANCH: RES /DSR /HFB
 TYPE: GSI
 ACTION LEVEL: ACTIVE
 PRIORITY: H
 STATUS: 3B
 RESOLUTION DATE: 06/93C

WORK AUTHORIZATION: MEMO TO H. THOMPSON FROM H. DENTON DATED 10/1/84.
 "HUMAN FACTORS PROGRAM PLAN"

FIN	CONTRACTOR	CONTRACT TITLE
A3967	BNL	ANNUNCIATORS

RISK SCOPE

THIS ISSUE COMBINES THE ISSUES FROM THE HUMAN FACTORS PROGRAM PLAN OF SECTION 1.5.2
 (A) COMPUTER AND COMPUTER DISPLAYS
 (B) EVALUATION OF OPERATOR AID SYSTEMS
 (C) ANNUNCIATORS
 (D) SAFETY STATUS INDICATION AND APPLICATIONS OF AUTOMATION AND ARTIFICIAL INTELLIGENCE
 A MANAGEMENT REVIEW OF THESE ISSUES RESULTED IN A DECISION TO RETAIN ONLY THE ANNUNCIATOR ISSUE AS A GENERIC ISSUE. THIS DECISION ALSO CONCLUDED THAT THE REMAINING ISSUES BE REASSIGNED AS RESEARCH ISSUES. A RESEARCH PROJECT ENTITLED "REVIEW CRITERIA FOR HUMAN FACTORS ASPECTS OF ADVANCED CONTROL AND INSTRUMENTATION," FIN B0852, ADDRESSES THESE ISSUES.
 THE GOAL OF THIS WORK IS TO DETERMINE THE RISK TO PUBLIC HEALTH AND SAFETY RESULTING FROM HUMAN ERROR IN THE

USE OF INFORMATION FROM CONTROL BOARD ANNUNCIATORS. THE RESULTS FROM A PRA ESTABLISHED THE MAGNITUDE OF THE RISK INVOLVED AND THE POTENTIAL REDUCTION IN RISK EXPECTED. A VALUE/IMPACT ANALYSIS WAS COMPLETED AND ESTIMATED THE COST OF POTENTIAL MODIFICATIONS TO ANNUNCIATORS.
 THE RESULTS OF THE VALUE/IMPACT ANALYSIS REQUIRE THAT ADDITIONAL EXPERIMENTAL WORK ON ANNUNCIATOR UPGRADES BE CONDUCTED ON A TTC SIMULATOR TO RESOLVE THE ANNUNCIATOR ISSUE. IN EVALUATING ALARM REDUCTION TECHNIQUES, WE ALSO PLAN TO USE THE RESEARCH RESULTS ON ADVANCED ALARM SYSTEMS CONDUCTED AT THE MALDEN PROJECT. THE RESULTS FROM THIS EFFORT WILL DEVELOP GUIDELINES ON COMPUTER-BASED ANNUNCIATOR SYSTEMS. THESE GUIDELINES WILL SERVE AS THE BASIS FOR A NUREG ON ANNUNCIATORS. NRR WILL INCORPORATE RESULTS OF THIS WORK IN THEIR MODIFICATIONS TO THE STANDARD REVIEW PLAN.

STATUS

BY LETTER DATED DECEMBER 29, 1987, THE SOW FOR THE VALUE-IMPACT STUDY WAS FORWARDED TO BNL. PROJECT AND BUDGET PROPOSAL FOR NRC WORK (NRC FORM 189) RECEIVED MARCH 7, 1988. BNL BEGAN WORK IN APRIL 1988. VALUE/IMPACT STUDY WAS

COMPLETED IN NOVEMBER 1988. ALARM REDUCTION STUDY BEGAN OCTOBER 1990. WORK TERMINATED. (SEE MEMORANDUM FROM E. BECKJORD TO THE EDO ON JUNE 29, 1993.)

GENERIC ISSUE MANAGEMENT CONTROL SYSTEM

ISSUE NUMBER: HF 5.2

TITLE: REV. CRIT. FOR HUMAN FACTORS ASPECTS OF ADV CONTROLS AND INSTRUMENT

AFFECTED DOCUMENTS

NONE

PROBLEM/RESOLUTION

NONE

M I L E S T O N E S

	ORIGINAL DATE	CURRENT DATE	ACTUAL DATE
BEGIN VALUE/IMPACT STUDY OF ANNUNCIATORS	02/88	--	04/88
COMPLETE VALUE/IMPACT STUDY	07/88	--	10/88
NRC COMPLETES REVIEW OF STUDY RESULTS	09/88	--	11/88
ISSUE SOW FOR ALARM REDUCTION EVALUATION	11/88	--	01/89
BEGIN ALARM REDUCTION STUDY	01/89	--	03/89
PREPARE CLOSEOUT PACKAGE	04/93	--	04/93
CLOSE OUT MEMO TO EDO	06/93	--	06/93
COMPLETE ALARM REDUCTION STUDY	11/89	03/94	--

RUN DATE: 02/03/94

GENERIC ISSUE MANAGEMENT CONTROL SYSTEM

PAGE 66

ISSUE NUMBER: I.D.3 TITLE: SAFETY SYSTEM STATUS MONITORING
 IDENTIFICATION DATE: 05/80C PRIORITIZATION DATE: 11/83C TYPE: GSI PRIORITY: M
 TASK MANAGER: S. DIAB OFFICE/DIVISION/BRANCH: RES /DSIR/EIB ACTION LEVEL: INACTIVE STATUS:
 TAC NUMBERS: RESOLUTION DATE: 09/93C
 WORK AUTHORIZATION: TMI ACTION PLAN, NUREG-0660

FIN	CONTRACTOR	CONTRACT TITLE
B2360	PNL	VERIFICATION, SYSTEM STATUS, AUTOMATIC VS. MANUAL

WORK SCOPE

PNL HAS REVIEWED THE EFFECTIVENESS OF SYSTEMS AND PROCEDURES RELATED TO SAFETY SYSTEM STATUS MONITORING AND HAS RECOMMENDED A REGULATORY POSITION ON THIS ISSUE. PNL FINDINGS INDICATE THAT A REVISION OF R.G. 1.47 IS

NECESSARY AND FURTHER THAT THIS NEW REVISION SHOULD BE APPLIED TO ALL PLANTS. BASED ON THE PNL RECOMMENDATIONS, DHFS TRANSFERRED RESPONSIBILITY FOR THIS ISSUE TO RES (REVISE R.G. 1.47).

STATUS

ALL TECHNICAL REPORTS ON THIS ISSUE ARE COMPLETE. THE PROJECT FINAL REPORT INCLUDES GUIDELINES FOR IMPROVING SYSTEM STATUS MONITORING AS WELL AS THE FINAL RECOMMENDATION OF PNL ON A REGULATORY POSITION. THEIR RECOMMENDATION IS TO REVISE R.G. 1.47 AND APPLY IT TO ALL PLANTS. STAFF PROPOSES TO APPLY NEW REQUIREMENTS FOR FUTURE PLANTS ONLY. COMPLIANCE BY OL'S WILL BE VOLUNTARY.

A DRAFT OF THE PROPOSED REVISION 1 TO R.G. 1.47 WAS DISTRIBUTED IN JULY, 87 FOR STAFF REVIEW. A MEETING WAS HELD ON SEPTEMBER 23, 1987 TO RESOLVE NRR (HUMAN FACTORS

ASSESSMENT BRANCH) PERTAINING TO PROPOSED REVISION 1. STAFF CONSENSUS IS HOWEVER NOT REACHED. RESPONSIBILITY HAS BEEN TRANSFERRED TO RES/EIB IN FEBRUARY 1992.

A LETTER REPORT TO REDEFINE THE ISSUE WAS COMPLETED JANUARY 1993. ON THE BASIS OF THE LETTER REPORT, A QUICK LOOK VALUE/IMPACT ANALYSIS HAS BEEN PREPARED TO DETERMINE IF THE ISSUE CAN BE CLOSED OUT. ON THAT BASIS, A DRAFT TECHNICAL RESOLUTION PACKAGE HAS BEEN PREPARED. SUBSEQUENTLY, THE FINAL RESOLUTION PACKAGE WAS ISSUED WITH NO LICENSEE ACTION REQUIRED.

AFFECTED DOCUMENTS

REVISION TO R.G. 1.47

PROBLEM/RESOLUTION

NONE

GENERIC ISSUE MANAGEMENT CONTROL SYSTEM

ISSUE NUMBER: I.D.3

TITLE: SAFETY SYSTEM STATUS MONITORING

M I L E S T O N E S	ORIGINAL DATE	CURRENT DATE	ACTUAL DATE
STUDY BACKGROUND OF THE ISSUE AND EVALUATE THE NEED FOR REVISION TO EXISTING GUIDANCE PROVIDED IN R.G. 1.47	05/92	--	05/92
PREPARE TASK ACTION PLAN	06/92	--	06/92
TAP APPROVED BY DSIR DIRECTOR	07/92	--	08/92
LETTER REPORT TO REDEFINE ISSUE IN LIGHT OF CURRENT STATUS OF SSSM	01/93	--	01/93
PRELIMINARY VALUE IMPACT	05/93	--	03/93
NRR REVIEW OF RESOLUTION PACKAGE	03/94	--	07/93
TR PACKAGE TO DIR/DSIR	05/93	--	07/93
PACKAGE TO ACRS	05/94	--	07/93
ACRS REVIEW COMPLETE	07/94	--	07/93
ISSUE CLOSEOUT MEMO; DIR, RES TO EDO	12/94	--	08/93
CONCURRENCE BY EDO	09/93	--	09/93

RUN DATE: 02/03/94

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GENERIC ISSUE MANAGEMENT CONTROL SYSTEM

ISSUE NUMBER: I D.5(3) TITLE: ON-LINE REACTOR SURVEILLANCE SYSTEMS
 IDENTIFICATION DATE: 05/80C PRIORITIZATION DATE: 11/83C TYPE: GSI PRIORITY:
 TASK MANAGER: M. VAGINS OFFICE/DIVISION/BRANCH: RES /DE /EMEB ACTION LEVEL: ACTIVE STATUS: NR
 TAG NUMBERS: NONE RESOLUTION DATE: 11/93C
 WORK AUTHORIZATION: TMI ACTION PLAN, NUREG-0660

FIN	CONTRACTOR	CONTRACT TITLE
B0828	ORNL	CONTINUOUS ON-LINE REACTOR NOISE SYSTEM EVALUATIONS

W O R K S C O P E

THE OBJECTIVE OF THIS REPORT IS TO PERFORM RESEARCH TO DETERMINE THE FEASIBILITY OF DETECTING AND DIAGNOSING NUCLEAR POWER PLANT OPERATING ANOMALIES USING A CONTINUOUS

ON-LINE NOISE SURVEILLANCE SYSTEM AND TO PERFORM DEMONSTRATIONS OF SUCH A SYSTEM IN AN NRC-LICENSED COMMERCIAL BWR PLANT.

S T A T U S

A DEMONSTRATION OF SYSTEM FEASIBILITY AT AN OPERATING PWR (SEQUOYAH UNIT 2) HAS BEEN COMPLETED. THE SURVEILLANCE INSTRUMENT SYSTEM WAS INSTALLED AT THE PEACH BOTTOM UNIT 2. DUE TO THE LONG OUTAGE AT PEACH BOTTOM (24 MONTHS), THE EXPERIMENT WAS TERMINATED AND THE EQUIPMENT REMOVED AND PLACED IN STORAGE AT ORNL. IT IS BELIEVED THAT SUFFICIENT DATA IS AVAILABLE FROM

THE ON-LINE SURVEILLANCE MEASUREMENT TESTS AT THE SEQUOYAH UNIT 2 PWR IN 1983/84 PLUS AVAILABLE NOISE DATA FROM MEASUREMENTS AT THE BROWNS FERRY BWR TO DEVELOP AN ACCEPTABLE EVALUATION OF THE ON-LINE SURVEILLANCE SYSTEM. A RIL SUMMARIZING THE RESEARCH RESULTS WAS PREPARED.

AFFECTED DOCUMENTS

PROBLEM/RESOLUTION

NONE

A RIL WAS PREPARED SUMMARIZING THE CONCLUSIONS AND RECOMMENDATIONS FROM THE RESEARCH ON THIS ACTION ITEM. RES MEMO SEND TO NRR PRIOR TO CLOSEOUT.

M I L E S T O N E S

INSTALL SYSTEM AT A PWR	ORIGINAL DATE	CURRENT DATE	ACTUAL DATE
	--	--	09/80

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GENERIC ISSUE MANAGEMENT CONTROL SYSTEM

ISSUE NUMBER: I.D.5(3)

TITLE: ON-LINE REACTOR SURVEILLANCE SYSTEMS

M I L E S T O N E S	ORIGINAL DATE	CURRENT DATE	ACTUAL DATE
COMPLETE PWR DATA ACQUISITION	06/84	--	01/84
INSTALL SYSTEM AT A BWR	04/84	--	06/85
REMOVAL OF SYSTEM FROM PEACH BOTTOM	12/88	--	04/89
ISSUE RIL TO NRR WITH CONCLUSIONS AND RECOMMENDATIONS	09/88	--	04/93
ISSUE CLOSEOUT MEMO TO EDO	12/88	--	11/93

GENERIC ISSUE MANAGEMENT CONTROL SYSTEM

ISSUE NUMBER: I.D.5(5)

TITLE: DISTURBANCE ANALYSIS SYSTEMS

M I L E S T O N E S	ORIGINAL DATE	CURRENT DATE	ACTUAL DATE
FINAL REPORT FROM CONTRACTOR ON RESULTS OF SURVEY ON HUMAN FACTORS OF ADVANCED CONTROLS/ INSTRUMENTATION	06/89	--	05/90
CLOSE OUT ISSUE	03/91	09/94	--

GENERIC ISSUE MANAGEMENT CONTROL SYSTEM

ISSUE NUMBER: II.H.2 TITLE: OBTAIN DATA ON CONDITIONS INSIDE TMI-2 CONTAINMENT
 IDENTIFICATION DATE: 05/80C PRIORITIZATION DATE: 1/1/83C TYPE: GSI PRIORITY: H
 TASK MANAGER: A. RUBIN OFFICE/DIVISION/BRANCH: RES /DSR /AEB ACTION LEVEL: ACTIVE STATUS:
 TAC NUMBERS: RESOLUTION DATE: 02/94
 WORK AUTHORIZATION: TMI ACTION PLAN, NUREG-0660

FIN	CONTRACTOR	CONTRACT TITLE
A2220	ANL	TMI-2 CORE EXAMINATION
A6899	EG&G	TMI-2 LOWER HEAD COMPANION SAMPLE EXAM & ANALYSIS
D2072	MPR	TEST SPECIMENS FROM DAMAGED TMI-2 REACTOR BOTTOM HEAD

W O R K S C O P E

NRC HAS COOPERATED WITH DOE AND INDUSTRY ON RESEARCH AND DATA-GATHERING PROGRAMS WITHIN THE TMI-2 CONTAINMENT. THESE PROGRAMS CONCENTRATE ON RETRIEVING DATA WHICH CHARACTERIZE THE PROGRESSION OF THE ACCIDENT AND THE RESULTING RADIOLOGICAL SOURCE TERM. PRIOR TO CY88, NRC PRIMARILY PROVIDED TECHNICAL EXPERTISE AND HOT-CELL EXAMINA-

TION OF CORE MATERIALS. THE PROGRAMS WERE GENERALLY CARRIED OUT WITH FUNDS FROM DOE AND OTHERS. EARLY IN CY88 NRC INITIATED ITS OWN COOPERATIVE PROGRAM TO EXAMINE THE DAMAGED BOTTOM HEAD. PRESSURE VESSEL WALL SPECIMENS, INSTRUMENT TUBE PENETRATIONS, GUIDE TUBE SPECIMENS, AND BOTTOM HEAD ONCE MOLTEN DEBRIS SPECIMENS ARE TO BE REMOVED AND ANALYZED.

S T A T U S

PRIOR EFFORTS CONCENTRATED ON TMI-2 CORE EXAMINATION BOTH ONSITE AND OFFSITE. CORE EXAMINATIONS INDICATE A LARGE FLOW OF MOLTEN MATERIAL (MORE THAN 15 TONS) INTO THE LOWER PLENUM AFTER THE ACCIDENT HAD BEEN IN PROGRESS FOR ABOUT 225 MINUTES. DOE HAD NOT BUDGETED FOR THE ADDITIONAL RESEARCH TO EXAMINE THE EFFECTS OF THIS MOLTEN MATERIAL ON THE VESSEL BOTTOM AND VESSEL COMPONENTS. THE CHAIRMAN INDICATED THAT HE BELIEVED THIS EXAMINATION TO BE VERY IMPORTANT. NRC ASKED DOE TO PERFORM THE EXAMINATION BUT DOE DECLINED. NRC INITIATED A COOPERATIVE PROGRAM JANUARY 15, 1988. EPRI AND 10 OECD-NEA COUNTRIES PARTICIPATED. THE SAMPLING EFFORT WAS COMPLETED ON MARCH 1, 1990. FIFTEEN PRESSURE VESSEL WALL SAMPLES, 14 INSTRUMENT TUBE PENETRATIONS, AND 2 GUIDE TUBE SPECIMENS WERE OBTAINED. TWO

NATIONAL LABORATORIES ARE BEING UTILIZED TO EXAMINE AND ANALYZE THE SPECIMENS. IN APRIL 1991, THE MANAGEMENT BOARD OF THE JOINT OECD TMI VESSEL INSPECTION PROGRAM VOTED TO EXTEND THE PROGRAM FOR A PERIOD OF 18 MONTHS BEYOND THE ORIGINAL PROGRAM. THE EXTENDED PROGRAM INCLUDES DETAILED TESTING AND EXAMINATION OF ALL THE VESSEL, NOZZLE, AND GUIDE TUBE SAMPLES EXTRACTED FROM TMI-2. A SCOPING ANALYSIS OF POTENTIAL REACTOR VESSEL FAILURE MODES AND MARGIN TO FAILURE WILL BE PERFORMED. THIS PROJECT WAS COMPLETED IN OCTOBER 1993 WITH THE ISSUANCE OF THE FINAL TMI-2 VESSEL INVESTIGATION PROJECT REPORTS. A PUBLIC MEETING WAS HELD IN BOSTON, MA, FROM OCTOBER 20-22, 1993, TO PRESENT THE RESULTS AND CONCLUSIONS OF THE PROJECT.

GENERIC ISSUE MANAGEMENT CONTROL SYSTEM

ISSUE NUMBER: II H.2 TITLE: OBTAIN DATA ON CONDITIONS INSIDE TMI-2 CONTAINMENT

PROBLEM/RESOLUTION

NONE

AFFECTED DOCUMENTS

1. "REMOVAL OF TEST SPECIMENS FROM THE TMI-2 REACTOR VESSEL BOTTOM HEAD. PROJECT SUMMARY - PHASE 4 STATUS REPORT MPR-1195. MPR ASSOCIATES, INC. OCTOBER 1, 1990.
2. G. E. KORTH. "METALLOGRAPHIC AND HARDNESS EXAMINATIONS OF TMI-2 LOWER PRESSURE VESSEL HEAD SAMPLES." TMIV(92)EG01, OECD-NEA-TMI-2 VESSEL INVESTIGATION PROJECT, JANUARY 1992.
3. D. W. AKERS, ET AL. "COMPANION SAMPLE EXAMINATIONS, TMIV(92)EG10, OECD-NEA-TMI-2 VESSEL INVESTIGATION PROJECT, JULY 1992.
4. L. A. NEIMARK, ET AL. "TMI-2 INSTRUMENT NOZZLE EXAMINATIONS AT ARGONNE NATIONAL LABORATORY." TMIV(93)AL01, OECD-NEA-TMI-2 VESSEL INVESTIGATION PROJECT, FEBRUARY 1993.
5. D. R. DIERCKS AND L. A. NEIMARK. "RESULTS OF MECHANICAL TESTS AND SUPPLEMENTARY MICROSTRUCTURAL EXAMINATIONS OF THE TMI-2 LOWER HEAD SAMPLES." TMIV(93)AL02, OECD-NEA-TMI-2 VESSEL INVESTIGATION PROJECT, JUNE 1993.
6. L. A. STICKLER, ET AL. "CALCULATIONS TO ESTIMATE THE MARGIN TO FAILURE IN THE TMI-2 VESSEL." TMIV(93)EG01, OECD-NEA-TMI-2 VESSEL INVESTIGATION PROJECT, OCTOBER 1993.
7. J. R. WOLF, ET AL. "TMI-2 VESSEL INVESTIGATION PROJECT INTEGRATION REPORT." TMIV(93)EG10, OECD-NEA-TMI-2 VESSEL INVESTIGATION PROJECT, OCTOBER 1993.

M I L E S T O N E S

REVISED CORE EXAMINATION PLAN (DOC RESPONSIBILITY)

HEAD REMOVAL

PLENUM REMOVAL

BEGIN DEFUELING

REVISE CORE EXAMINATION PLAN

CORE DRILLING

ORIGINAL DATE	CURRENT DATE	ACTUAL DATE
04/83	--	10/84
07/83	--	07/84
07/85	--	05/85
10/85	--	12/85
12/85	--	02/86
07/86	--	03/87

GENERIC ISSUE MANAGEMENT CONTROL SYSTEM

ISSUE NUMBER : II.H.2

TITLE : OBTAIN DATA ON CONDITIONS INSIDE TMI-2 CONTAINMENT

M I L E S T O N E S	ORIGINAL DATE	CURRENT DATE	ACTUAL DATE
LOWER HEAD SAMPLE REMOVAL	12/88	--	03/90
COMPLETE DEFUELING	06/87	--	05/90
SAMPLE EVALUATION AND ANALYSIS REPORT	06/90	--	10/93
SUMMARY ANALYSIS	12/90	--	10/93
CLOSEOUT MEMO TO EDO	02/94	02/94	--