



UNITED STATES
NUCLEAR REGULATORY COMMISSION

REGION II
101 MARIETTA ST., N.W., SUITE 3100
ATLANTA, GEORGIA 30303

November 23, 1982

Report No. 50-416/82-75

Licensee: Mississippi Power and Light Company
Jackson, MS 39205

Facility Name: Grand Gulf

Docket No. 50-416

License No. NPF-13

Inspection at Grand Gulf site near Port Gibson, Mississippi

Inspector: *T. R. Collins* 11/23/82
T. R. Collins Date Signed

Approved by: *K. P. Barr* 11/23/82
K. P. Barr, Section Chief Date Signed
Technical Engineering Inspection Branch
Division of Engineering and Technical Programs

SUMMARY

Inspection on November 8-9, 1982

Areas Inspected

This routine, unannounced inspection involved 16 inspector-hours on site in the areas of solid, liquid and gaseous radwaste pre-operational tests, solid radwaste processing system, radiation protection program, and licensee event reports.

Results

Of the four areas inspected, no violations or deviations were identified.

REPORT DETAILS

1. Persons Contacted

Licensee Employees

- *C. K. McCoy, Plant Manager
- R. A. Ambrosino, Assistant Plant Manager
- R. R. Weedon, Chemistry and Radiation Control Superintendent
- R. Brown, Chemistry Supervisor
- T. G. Lee, Staff Health Physicist
- D. H. Wells, Health Physics Supervisor
- *J. Roberts, Start-up Manager
- *C. C. Hayes, Plant Quality Superintendent
- *J. D. Bailey, Plant Quality

Other Organizations

NUMANCO, Inc.

NRC Resident Inspector

A. G. Wagner

D. Scott

*Attended exit interview

2. Exit Interview

The inspection scope and findings were summarized on November 9, 1982, with those persons indicated in paragraph 1 above.

3. Licensee Action on Previous Enforcement Matters

Not inspected.

4. Unresolved Items

Unresolved items were not identified during this inspection.

5. Inspector Identified Items

- a. (Open) IFI (81-26-02), Ventilation in Hot Machine Shop. A design change request (DCR) issued by the licensee for Nuclear Plant Engineering (NPE) to evaluate and modify the ventilation system in the hot machine shop has not been completed. This item will be reviewed during the next routine inspection.

- b. (Open) IFI (81-26-03), Ventilation and Contamination Control in the Decontamination Room. A design change request (DCR) issued by the licensee for NPE to evaluate and modify the ventilation system in the decontamination room has not been completed. This item will be reviewed during the next routine inspection.
- c. (Open) IFI (81-26-04), Face velocity checks for fume hoods and sample hoods. This item is currently incomplete and will be reviewed during the next routine inspection.
- d. (Open) IFI (82-21-04), Burial allocation at Chem-Nuclear Systems, Inc., (CNSI). The licensee is still negotiating a burial allocation with CNSI. Presently, the licensee has a "first come first serve" allocation. The status of this item will be reviewed during the next routine inspection.
- e. (Open) IFI (82-33-02), Post Accident Sampling System Effluent Monitor. The licensee has issued a Design Change Package (DCP) (number 82/493) which is to be completed in August of 1982. However, at this time, the DCP has not yet been completed. This item will be reviewed during the next inspection.
- f. (Open) IFI (82-33-03), Adequate shielding on liquid sample lines of post accident sampling system. The inspector was informed by a licensee representative that this item was sent to Bechtel Corporation for review and evaluation. The licensee reviewed Bechtel's written evaluation and disagreed with their findings. The licensee concludes that additional shielding is necessary. This item will be reviewed during the next inspection.
- g. (Open) VIO (82-58-01), Routine radiation and contamination surveys. The inspector discussed the requirements of radiation protection procedures with a licensee management representative and was informed that a complete review of all procedures was being performed to determine the adequate level of direction that these procedures provide. The inspector informed licensee management that these procedures would be reviewed after the review process has been completed.
- h. (Closed) VIO (82-58-02), Failure to perform a 10 CFR 50.59 Safety Evaluation. The licensee has instructed all personnel who are responsible for controlling modifications to permanent plant equipment that these changes must have a 10 CFR 50.59 Safety Evaluation completed prior to performing the modification. In addition, all Maintenance Work Orders (MWOs) are being screened for potential design changes prior to issuance for work. The inspector concluded that these corrective actions appeared to be adequate and had no further questions.

- i. (Closed) VIO (82-58-03), Plant Safety Review Committee (PSRC) inadequate reviewed of Chem-Nuclear Solidification System prior to operation. On August 4, 1982, the PSRC reviewed and approved the 10 CFR 50.59 Safety Evaluation for the Chem-Nuclear Solidification System. In addition, the licensee is reviewing the overall requirements of 10 CFR 50.59 and will revise its procedures, if necessary, to ensure full compliance of 10 CFR 50.59. The inspector concluded these corrective actions appeared to be adequate and had no further questions.
- j. (Closed) IFI (82-53-02), Adequate sample line flow for the Liquid Radwaste Effluent Discharge Monitor. The inspector discussed this with a licensee representative and determined that Procedure No. 04-S-01-G17/4, Liquid Radwaste Discharge System had been revised to require that verification of the sample line flow rate to the liquid radwaste discharge monitor to be ≥ 6 gpm, prior to and during discharge. In addition, this verification is also required to be documented on the liquid radwaste discharge permit. The inspector concluded this appeared to be adequate and had no further questions.

6. Licensee Event Reports (LERs)

- a. (Closed) Licensee Event Report (LER) 82-055/03. Discharge Basin flow trip channel inoperable. The discharge basin low flow trip setting of the liquid radioactive waste effluent valve was altered from August 28, 1982 until September 8, 1982, in order to discharge non-radioactive, glycol-contaminated water without requiring the large dilution flow rate from the circulating water system blowdown line. The licensee procedurally controlled this temporary alteration and a 10 CFR 50.59 Safety Evaluation and a Environmental Evaluation was performed. As required by Technical Specification 3.3.7.11, Action Statement 111, discharge basin effluent flow was sampled every four (4) hours and no detectable radioactivity was discharged. The trip was returned to operable status at the end of the discharge. The inspector concluded that the proper state agencies were notified of this discharge and no deviations or deficiencies were identified.
- b. (Closed) Licensee Event Report (LER) 82-082/03. Control Room Ventilation Radiation Monitors out of tolerances on Channels C and D. On September 25, 1982, Routine Surveillance Procedure 06-OP-1D17-M-0007 revealed Channels C and D of the Control Room Ventilation Radiation Monitors outside the tolerances of the HiHi trip setpoint required by Technical Specification 3.3.7.1-1.6. Channels C and D were immediately recalibrated. Also, on September 25, 1982, the above Surveillance Procedure was repeated and Channels C and D were found to be satisfactory. During this event redundant Channels A and B were operable. The inspector concluded that the immediate corrective action was adequate and had no further questions.

7. Liquid, Solid, Gaseous Radwaste Systems, and Radiation Monitoring Systems Pre-Operational Test Programs

The inspector discussed the pre-operational test program with licensee representative and was informed that the status of the Liquid, Solid, Gaseous Radwaste Systems and the Radiation Monitoring Systems pre-op test were still in progress. However, completion of these systems were expected in the near future, except for the solid radwaste processing system. The overall completion status of the pre-operational test program is approximately 40% complete.

The licensee is experiencing problems with the radwaste solidification system (G18) and due to the magnitude, an accurate predication of the test completion date cannot be made. A licensee management representative stated that pre-op test of this system may not be completed. Instead, the licensee is considering the use of a mobile radwaste solidification system to process generated liquid waste. The inspector informed licensee management that if the G18 Solid Radwaste Processing System is not considered for normal operation and a mobile radwaste solidification system is decided upon, a 10 CFR 50.59 Safety Evaluation must be completed prior to the operation of the facility. Plant management acknowledged the inspector concerns. The inspector informed licensee management that this would remain as an inspector followup item pending their decision of the use of the G18 System. (IFI) 50-416/82-75-01.

8. Radiation Protection Procedures

The licensee has reclassified the majority of all Health Physics Procedures (Administrative, Radiation Protection Procedures and Instructions) as "non-safety related". This change will take place upon the next scheduled revision. The inspector informed licensee management representatives that Station Administrative Procedures, either "non-safety related" or "safety related" are required to be reviewed by the Plant Safety Review Committee (PSRC), as described in Technical Specification 6.5.1.6.a. Licensee management representatives acknowledged the inspectors concerns and stated that upon their final evaluation, affected Administrative Radiation Protection Procedures would remain as a required review by the PSRC prior to implementation and any changes thereto. The inspector informed licensee manager that this would remain as an inspector followup item pending their final evaluation of the Administrative Radiation Protection Procedures (IFI) 50-416/82-75-02.

9. Posting, Labeling and Control

The inspector toured the Fuel Handling Building and the Containment Building to verify proper posting of Radiation Controlled Areas, Radioactive Materials Areas and Radiation Areas. No violations or deviations were observed.

10. Posting of Notices to Workers

10 CFR 19.11 requires, in part, that each licensee post "current" copies of Form NRC-3, Notice to Employees, in a sufficient number of places to permit individuals engaged in licensed activities to observe them on the way to or from any licensed activity location. The inspector observed the posting of notices required by 10 CFR 19.11. No violations were identified.