Docket No. 52-002

Mr. C. B. Brinkman, Acting Director Nuclear Systems Licensing Combustion Engineering, Inc. 1000 Prospect Hill Road Windsor, Connecticut 06095-0500

Dear Mr. Brinkman:

SUBJECT: COMMENTS ON TECHNICAL SPECIFICATIONS (TS) FOR SYSTEM 80+

The Instrumentation & Controls Branch and the Civil Engineering & Geosciences Branch of the Nuclear Regulatory Commission (NRC) have reviewed the System 80+ TS, and the comments are shown on the markup copy of the affected TS in the enclosure. Provide responses to these comments no later than two weeks from your receipt of this letter to allow us to maintain our review schedule.

This affects nine or fewer respondents, and therefore, is not subject to review by the Office of Management and Budget under P.L. 96-511.

Sincerely,

(Original signed by)

Kristine M. Shembarger, Project Manager Standardization Project Directorate Associate Directorate for Advanced Reactors and License Renewal Office of Nuclear Reactor Regulation

Enclosure: As stated

cc w/enclosure: See next page

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ABB-Combustion Engineering, Inc.

cc: Mr. C. B. Brinkman, Acting Director Nuclear Systems Licensing ABB-Combustion Engineering, Inc. 1000 Prospect Hill Road Windsor, Connecticut 06095-0500

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# ADDITIONAL NOTES CONCERNING SYSTEM 80+ TECH SPECS

- Bases for 3.3.2 should explain that TRCBs are closed for channel functional tests.
- 2) Define TRIP Leg used on pages 3.3-19 and 3.3-26.
- 3) Explain in Bases reason for Conditions E&F in 3.3.5.
- 4) APS is 2/2 Change 3.3.8 to no channel in bypass, and 72 hours to repair (consistent with BWRs).
- 5) Table headers should be all caps; and make headers consistent with use of footnote modifiers.

- 3.3 INSTRUMENTATION
- 3.3.1 Reactor Protective System (RPS) Instrumentation Operating

LCO 3.3.1 Four RPS TRIP CHANNELS and operating bypass removal CHANNELS for each Function in Table 3.3.1-1 shall be OPERABLE.

APPLICABILITY: According to Table 3.3.1-1.

- Separate Condition entry is allowed for each RPS Function.
- 2. Alf a CHANNEL is placed in bypass, continued operation with the CHANNEL in the bypassed condition for the Completion Time specified by Required Action A.2 or C.2.2 shall be reviewed in accordance with Specification [5.5.1.2.e].

#### ACTIONS

	CONDITION		REQUIRED ACTION	COMPLETION TIME
Α.	One or more Functions with one sutomatic RPS TRIP CHANNEL inoperable.	A.1	Place CHANNEL in bypass or trip.	1 hour
		A.2	Restore CHANNEL to OPERABLE status.	Prior to entering MODE 2 following next MODE 5 entry
			LCO 3.0.4 is not applicable.	
В.	One or more Functions with two sutematic RPS TRIP CHANNELS inoperable.	B.1	Place one CHANNEL in bypass and the other in trip.	1 hour

# ACTIONS (Continued)

CONDITION	REQUIRED ACTION	COMPLETION TIME
C. One or more Functions with one automatic operating bypass removal CHANNEL inoperable.	OR  C.2.1 Place affected automatic TRIP CHANNEL in bypass or trip.  AND  C.2.2 Restore operating bypass removal CHANNEL and associated automatic TRIP CHANNEL to OPERABLE status	1 hour  Once per 12 hours thereafter  1 hour  Prior to entering MODE 2 following next MODE 5 entry
	AND  C.2.2 Restore operating bypass removal CHANNEL and associated suitomatic	Prior to entering MODE 2 following next MODE 5
One or more Functions with two automatic operating bypass removal CHANNELS inoperable.	D.1 Verify operating bypasses are not in effect.	1 bour AND
+10-12-10-15	OR	once per 12 hours thereafter

# ACTIONS (Continued)

	CONDITION		REQUIRED ACTION	COMPLETION TIME
D.	(Continued)	D.2 Place one affected automatic trip CHANNEL in bypass and place the other in trip.		1 hour
	Deleted			
E.	Required Action and associated Completion Time not met.	E.1	Be in Mode 3.	6 hours

# SURVEILLANCE REQUIREMENTS

		SURVEILLANCE	FREQUENCY
SR	3.3.1.1	Perform & CHANNEL CHECK of each RPS instrument CHANNEL	12 hours
SR	3.3.1.2	Not required to be performed until 12 hours after THERMAL POWER ≥ 70% RTP.	
	more to	Verify total Reactor Coolant System (RCS) flow rate as indicated by each Co is less than or equal to the RCS total flow rate.	12 hours
	Move to Bases	If necessary, adjust the CPC addressable constant flow coefficients such that each CPC indicated flow is less than or equal to the RCS flow rate.	
SR	3.3.1.3	Check the CPC autorestart count is less than three:	12 hours
SR	3.3.1.4	Verify PPS cabinet temperatures a e within limits o	12 hours

# SURVEILLANCE REQUIREMENTS (Continued)

-	SURVEILLANCE	FREQUENCY	
SR 3.3.1.5	<ol> <li>Not required to be performed until 12 hours after THERMAL POWER ≥ 20% RTP.</li> <li>The daily calibration may be suspended during PHYSICS TESTS, provided the calibration is performed upon reaching each major test power plateau and prior to proceeding to the next major test power plateau.</li> </ol>	Jefine in Bis	
	Perform calibration Theat balance only) and adjust the linear Power Level signals and the CPC addressable constant multipliers to make the CPC ΔT at power and CPC nuclear power calculations agree with the calorimetric, if the absolute difference is ≥ [2] %. Then adjust excore nuclear power to agree with CPC nuclear power if the absolute difference is ≥ [2%].	[92] days	
SR 3.3.1.6	Not required to be performed until 12 hours after THERM AL POWER ≥ 70% RTP.		
	Verify total RCS flow rate indicated by each CPC is less than or equal to the RCS flow determined by calcrimetric calculations.	31 days	
SR 3.3.1.7	Not required to be performed until 12 hours after THERMAL POWER ≥ 15% RTP.		
0	Verify linear power SUBCHANNEL gains of the excore detectors are consistent with the values used to establish the shape annealing matrix elements in the CPCs.	31 days	

# SURVEILLANCE REQUIREMENTS (Continued)

	SURVEILLANCE	FREQUENCY
SF. 3.3.1.8	NOTES  1. The CPC CHANNEL FUNCTIONAL TEST shall include verification that the correct values of addressable constants are installed in each OPERABLE CPC.  2. Not required to be performed for logarithmic power level - H CHANNELS until 2 hours after reducing THERMAL POWER below 1E-4% RTP and only if reactor trip circuit breakers (RTCBs) are closed.	ish
- DX	Perform CHANNEL FUNCTIONAL TEST on each CHANNEL	92 days
SR 3.3.1.9	Neutron detectors are excluded from CHANNEL CALIBRATION.	
	Perform CHANNEL CALIBRATION of the power range neutron flux CHANNELS.	92 days
[SR 3.3.1.10	Perform CHANNEL CALIBRATION on A/D Reference Sources	[12] months]
SR 3.3.1.11	Neutron detectors are excluded from CHANNEL CALIBRATION.	
	Perform CHANNEL CALIBRATION on each CHANNEL, including operating bypass removal functions.	[18 months]
SR 3.3.1.12	Perform & CHANNEL FUNCTIONAL TEST on each CPC	[18 months]
SR 3.3.1.13	Using the incore detectors, determine the shape annealing matrix elements to be used by the CPCs.	Once after each refueling prior to exceeding 70% RTP
SR 3.3.1.14	Perform a CHANNEL FUNCTIONAL TEST on each automatic operating bypass removal function.	Once within 92 days prior to each reactor startup

# SURVEILLANCE REQUIREMENTS (Continued)

The state of the s	FREQUENCY	
SR 3.3.1.15	Neutron detectors are excluded.  Verify RPS RESPONSE TIME is within limits.	[18] months on a STAGGERED TEST BASIS

#### Table 3.3.1-1 (Sheet 1 of 3)

### REACTOR PROTECTIVE SYSTEM INSTRUMENTATION - OPERATING

FURECT I ON	APPLICABLE MODES OR OTHER SPECIFIED COMDITION	SURVETILLANCE MEQUIREMENTS	ALLOWARLE VALUE
1. Variable Overpower - High	1,2	SR 3.3.1.1 SR 3.3.1.4 SR 3.3.1.5 SR 3.3.1.7 SR 3.3.1.8 SR 3.3.1.9 [SR 3.3.1.90] SR 3.3.1.11	\$ [112.7]% RTP
2. Logarithmic Power Level - High <sup>(e)</sup> Cr.	2	SR 3.3.1.1 SR 3.3.1.4 SR 3.5.1.8 [SR 3.3.1.10] SR 3.3.1.11 SR 3.3.1.16 SR 3.3.1.15	\$ [0.018]% RYP
3. Pressurizer Pressure - High	1,2	SR 3.3.1.1 SR 3.3.1.4 SR 3.3.1.8 [SR 3.3.1.10] SR 3.3.1.11 SR 3.3.1.15	≤ [2370 psia]
4. Pressurizer Pressure - Low	٧,2	SR 3.3.1.1 SR 3.3.1.4 SR 3.3.1.8 [SK 3.3.1.10] SR 3.3.1.11 SR 3.3.1.14 SR 3.3.1.15	Trip ≥ [1825 psia Irip Operating Bypass Removal > [500 psia] Step ≤ [400 psia] Floor ≥ [300 psia
5. Containment Pressure - High	1,2	SR 3.3.1.1 SR 3.5.1.4 SR 3.5.1.8 (SR 3.3.1.10) SR 3.3.1.11 SR 3.3.1.15	≤ (2.7) psig
5. Steam Generator #1 Pressure - Low <sup>(b)</sup>	1,2	SR 3.3.1.1 SR 3.3.1.6 SR 3.3.1.8 [SR 3.3.1.10] SR 3.3.1.11 SR 3.3.1.15	≥ (843 psia)
7. Steam Generator #2 Pressure -	1,2	SR 3.3.1.1 SR 3.3.1.4 SR 3.3.1.8 (SR 3.3.1.10) SR 3.3.1.11 SR 3.3.1.15	≥ [843 psia]

(Continued)

- (a) Trip may be bypassed when THERMAL POWER is > [1E-4]% RTP. Operating bypass shall be automatically removed when THERMAL POWER is \$ [1E-4]% RTP. Trip may be manually bypassed during physics testing pursuant to LCO 3.4.17, "RCS Loops Test Exceptions."
- (b) The Steam Generator Pressure Low trip setpoint way be manually decreased as steam generator pressure is reduced, provided the margin between steam generator pressure and the setpoint is maintained at ≤ 200 psi. The setpoint shall be increased automatically as steam generator pressure is increased.

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Amendment U - 12/31/93

\* STS - Footnote b- "When any RTCB is closed"
\* what about Pressurizer pressure operations bypess?

### Table 3.3.1-1 (Sheet 2 of 3)

# REACTOR PROTECTIVE SYSTEM INSTERNEMENTATION - OPERATING

FLANCYTON	APPLICABLE MODES OR OTHER SPECIFIED CONDITION	SURVEILLANCE REGUIRENENTS	ALLOWABLE VALUE
8. Steam Generator #1 Level - Low <sup>(c)</sup>	1,2	SR 3.3.1.1 SR 3.3.1.4 SR 3.3.1.8 [SR 3.3.1.10] SR 3.3.1.11 SR 3.3.1.11	≥ [44.2]%
9. Steam Generator #2 Leve! - Low <sup>(c)</sup>	1,2	SR 3.3.1.1 SR 3.3.1.4 SR 3.3.1.8 [SR 3.3.1.10] SR 3.3.1.11 SR 3.3.1.15	≥ (44.2)%
10. Steam Generator #1 Level - High	1,2	SR 3.3.1.1 SR 3.3.1.4 SR 3.3.1.8 [SR 3.3.1.10] SR 3.3.1.11 SR 3.3.1.15	≤ [90.8]%
11. Steam Generator #2 Level — High	1,2	SR 3.3.1.1 SR 3.3.1.4 SR 3.3.1.8 (SR 3.3.1.10) SR 3.3.1.11 SR 3.3.1.15	≤ (90.8)%
12. Reactor Coolant Flow - Low <sup>(6)</sup>	1,2	SR 3.3.1.1 SR 3.3.1.4 SR 3.3.1.8 [SR 3.3.1.10] SR 3.3.1.11 SR 3.3.1.14 SR 3.3.1.15	Rate: 5 [*] psi/sec. Floor: 2 [*] psid Step: [*] psi

(Continued)

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explain rate limiting in

\* Value to be determined by system detail design.

SYSTEM 80+

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<sup>(</sup>c) The Steam Generator Level-Low trip setpoint varies with reactor power and is rate limited with a preset low power value.

<sup>(</sup>d) The Reactor Coolant Flow-Low trip setpoint varies with reactor power and is rate limited with a preset low power level.

Table 3.3.1-1 (Sheet 3 of 3)

#### REACTOR PROTECTIVE SYSTEM INSTRUMENTATION - OPERATING

FUNCT I ON	APPLICABLE MODES OR OTHER SPECIFIED COMBITION	SURVETLLANCE REQUIREMENTS	ALLOWABLE VALUE
13. Local Power Density - High <sup>(b)</sup>	1,2	SR 3.3.1.1 SR 3.3.1.2 SR 3.3.1.3 SR 3.3.1.6 SR 3.3.1.6 SR 3.3.1.6 SR 3.3.1.7 SR 3.3.1.8 SR 3.3.1.9 [SR 3.3.1.10] SR 3.3.1.11 SR 3.3.1.12 SR 3.3.1.12 SR 3.3.1.12 SR 3.3.1.15	≤ [21.0] kW/ft
14. Departure From Nucleate Boiling Ratio (DNBR) - Low <sup>(b)</sup>	1,2	SR 3.3.1.1 SR 3.3.1.2 SR 3.3.1.3 SR 3.3.1.6 SR 3.3.1.6 SR 3.3.1.6 SR 3.3.1.7 SR 3.3.1.0 SR 3.3.1.10 SR 3.3.1.11 SR 3.3.1.11 SR 3.3.1.12 SR 3.3.1.12 SR 3.3.1.13 SR 3.3.1.15	≥ [1.24]

(b) Trip may be bypassed when THERMAL POWER IS < [1E-4]% RTP. Bypass shall be automatically removed when THERMAL POWER is 2 [1E-4]% RTP. During testing pursuant to LCO 3.4.17, trip way be bypassed below [5%] RTP. Bypass shall be automatically removed when THERMAL POWER is > [5%] RTP.

We, here but not etsewhere in other places:

3.3 INSTRUMENTATION

3.3.2 Reactor Protective System (RPS) Instrumentation - Shutdown

LCO 3.3.2 Four RPS TRIP CHANNELS and operating bypass removal CHANNELS for each Function in Table 3.3.2-1 shall be OPERABLE.

APPLICABILITY: According to Table 3.3.2-1

- Separate Condition entry is allowed for each RPS Function.
- 2. If a CHANNEL is placed in bypass, continued operation with the CHANNEL in the bypassed condition for the Completion Time specified by Required Action A.2 or C.2.2 shall be reviewed in accordance with Specification [5.5.1.2.e].

#### ACTIONS

No.	CONDITION	REQUIRED ACTION	COMPLETION TIME
Α.	One or more Functions with one automatic RPS TRIP CHANNEL inoperable.	utomatic RPS TRIP	
В.	One or more functions with two automatic RPS TRIP CHANNELS inoperable.	B.1 Place one CHANNEL in bypass and place the other in trip.	

## ACTIONS (Continued)

	CONDITION	RI	EQUIRED ACTION	COMPLETION TIME
			Applies only to Functions 1, 4, 5, and 6 in Table 3.3.2-1	
C.	One or more functions with one automatic operating bypass removal	C.1	Disable operating bypass CHANNEL. FUNDION	1 hour
	CHANNELS inoperable.	OR		
	The training	C.2.1	Place affected automatic TRIP CHANNEL in bypass or trip.	1 hour
		AND		
		C.2.2 Function	Restore operating bypass removal CHANNEL and associated automatic TRIP CHANNEL to OPERABLE status.	Prior to entering MODE 2 following next MODE 5 entry
			NOTES—	
			<ol> <li>LCO 3.0.4 is not applicable.</li> </ol>	
			<ol> <li>Applies only to functions 1, 4,</li> <li>5 and 6 in Table 3.3.2-1.</li> </ol>	
D.	One or more functions with two automatic operating bypass removal CHANNELS inoperable.	D.1	Disable operating bypass - CHANNELS - Tunction 3.	1 hour
		D.2	Place one affected automatic TRIP CHANNEL in bypass and place the other in trip.	1 hour
E.	Required Action and associated Completion Time not met.	E.1	Open all RTCBs.	1 bour

#### SURVEILLANCE REQUIREMENTS

---NOTE----

Refer to Table 3.3.2-1 to determine which SR shall be performed for each RPS function.

-	ŞURVEILLANCE	FREQUENCY
SR 3.3.2.1	Perform & CHANNEL CHECK of each RPS instrument	12 hours
SR 3.3.2.2	Check the CPC autorestart count is less than three.	12 hours
SR 3.3.2.3	1. The CPC CHANNEL FUNCTIONAL TEST shall include verification that the correct values of addressable constants are installed in each OPERABLE CPC.  2. Not required to be performed for logarithmic power level channels until 2 hours after reducing THERMAL POWER below [1E-4%] RTP and only if reactor trip circuit breakers (RTCBs) are closed.  Perform a CHANNEL FUNCTIONAL TEST on each CHANNEL.	92 days
SR 3.3.2.4	Verify RPS cabinet temperatures are within limits.	12 hours
SR 3.3.2.5	Perform a CHANNEL FUNCTIONAL TEST on each automatic operating bypass removal function.	92 days
[SR 3.3.2.6	Perform CHANNEL CALIBRATION on A/D Reference Sources.	[12] months]

## SURVEILLANCE REQUIREMENTS (Continued)

	SURVEILLANCE	
SR 3.3.2.7	Neutron detectors are excluded from CHANNEL CALIBRATION.	
	Perform's CHANNEL CALIBRATION on each TRIP CHANNEL including operating bypass removal function.	[18] months
SR 3.3.2.8	Neutron detectors are excluded.	
	Verify RPS RESPONSE TIME is within limits.	[18] months on a STAGGERED TEST BASIS

TABLE 3.3.2-1

REACTOR PROTECTION INSTRUMENTATION - SHUTDOWN

Function	Applicable Modes or Other Specified Conditions	Surveillance Requirements	Attownble Value
1. Logarithmic Power Level - High (*)	footnete?	SR 3.3.2.1 SR 3.3.2.3 SR 3.3.2.4 SR 3.3.2.5 SR 3.3.2.6 SR 3.3.2.7 SR 3.3.2.7	≤ [0.018] % RTP
2. Steam Generator Pressure #1 - Low <sup>(c)</sup>	Z (a)	SR 3.3.2.1 SR 3.3.2.3 SR 3.3.2.4 SR 3.3.2.6 SR 3.3.2.7 SR 3.3.2.7	≥ [843 psia]
<ol> <li>Steam Generator Pressure #2 - Low<sup>(c)</sup></li> </ol>	3(4)	SR 3.3.2.1 SR 3.3.2.3 SR 3.5.2.4 SR 3.5.2.6 SR 3.3.2.6 SR 3.3.2.7	≥ [843 psia]
Reactor Coolant Flow - Low(*)  Reses needs KESSAR reference	3(1), 4(1), 5(1)	SR 3.3.2.1 SR 3.3.2.3 SR 3.3.2.4 SR 3.3.2.6 SR 3.3.2.7 SR 3.3.2.7	Rate: [*] psi/sec floor: [*] psid Step: [*] psi
5. Local Power Density - High <sup>(e)</sup>	3(0), 4(0), (5(0)	SR 3.3.2.1 SR 3.3.2.2 SR 3.3.2.3 SR 3.3.2.5 SR 3.3.2.5 SR 3.3.2.6 SR 3.3.2.7 SR 3.3.2.8	s [21.0] kw/ft
6. Departure from Nucleate Boiling Ratio - Low <sup>(*)</sup>	3(0), 6(0), 5(0)	SR 3.3.2.1 SR 3.3.2.2 SR 3.3.2.3 SR 3.3.2.4 SR 3.3.2.5 SR 3.3.2.5 SR 3.3.2.5	≥ [1.24]

(8) With any Reactor Trip Circuit Breakers (RYCBs) closed and any Control Element Assembly capable of being withdrawn.

Trip may be bypassed when THERMAL POWER is > [1E-4] RTP. Operating bypass shall be automatically removed when THERMAL POWER is < [1E-4]% RTP. Trip may be manually bypassed during physics testing pursuant to LCO [3.4.17] "RCS Loops - Test Exceptions".

(c) The Steam Generator Pressure - Low trip setpoint may be manually decreased as steam generator pressure is reduced, provided the margin between steam generator pressure and the setpoint is maintained at ≤ 200 psi. The setpoint shall be increased automatically as steam generator pressure is increased.

(d) The Reactor Coolant Flow - Low trip setpoints may be manually adjusted when THERMAL POWER is < 10-5.

(e) Trip may be bypassed when THERMAL POWER is < [1E-4]% RTP. Operating bypass shall be automatically removed when THERMAL POWER is ≥ [1E-4]% RTP. During testing pursuant to LCO 3.4.17, trip may be bypassed below [5%] RTP. Operating bypass shall be automatically removed when THERMAL POWER is > [5%] RTP.

Value to be determined by system detail design.

SYSTEM 80+

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3.3.3

3.3 INSTRUMENTATION

3.3.3 Control Element Assembly Calculators (CEACs)

LCO 3.3.3 Two CEACs shall be OPERABLE.

APPLICABILITY: Modes 1 and 2

#### ACTIONS

CONDITION		REQUIRED ACTION	COMPLETION TIME	
Α.	One CEAC inoperable.	A.1 Perform SR 3.1.5.1.  AND	Once per 4 hours	
		A.2 Restore inoperable CEAC to OPERABLE status.	7 days	
B.	Required Action and associated Completion Time of Condition A not met.	B.1 Ensure the departure from nucleate boiling ratio requirement of LCO 3.2.4, "Departure from Nucleate Boiling Ratio (DNBR)," is met and the Reactor Power Cutback System is disabled.	4 hours	
	Both CEACs inoperable.	AND		

CEAC₅ 3.3.3

# ACTIONS (Continued)

CONDITION	REQUIRED ACTION	COMPLETION TIME
B. (Continued)	B.2 Verify all control element assembly (CEA) groups are fully withdrawn and maintained fully withdrawn, except during Surveillance testing pursuant to SR 3.1.5.3 and SR 3.1.5.4 [or for control, when CEA group #6 may be inserted to a maximum of 127.5 inches].	4 bours
	AND	
	B.3 Ensure the "RSPT/CEAC Inoperable" addressable constant in each core protection calculator (CPC) is set to indicate that both CEACs are inoperable.	4 hours
	AND	
	B.4 Verify the CEA Drive Mechanism Control System is placed in "STANDBY" and maintained in "STANDBY," except during CEA motion permitted by Required Action B.2.	4 hours
	AND	
	B.5 Pe.form SR 3.1.5.1.	Once per 4 hours

CEAC<sub>8</sub> 3.3.3

# ACTIONS (Continued)

CONDITION		REQUIRED ACTION	COMPLETION TIME
c.	Required Action and associated Completion Time of Condition 8 not met.	C.1 Be in MODE 3.	6 hours

# SURVEILLANCE REQUIREMENTS

	SURVEILLANCE	FREQUENCY
SR 3.3.3.1	Perform CHANNEL CHECK.	12 bours
SR 3.3.3.2	Check the CEAC autorestart count is less than three.	12 hours
SR 3.3.3.3	Verify CEAC cabinet temperatures are within limits.	12 hours
SR 3.3.3.4	Perform CHANNEL FUNCTIONAL TEST.	92 days
SR 3.3.3.5	Perform & CHANNEL CALIBRATION.	92 days
SR 3.3.3.6	Perform & CHANNEL FUNCTIONAL TEST.	[18] months





RPS Logic and Trip Initiation 3.3.4

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3.3 INSTRUMENTATION

3.3.4 Reactor Protective System (RPS) Logic and Trip Initiation

LCO 3.3.4 Four RPS LOGIC CHANNELS, four CHANNELS of Reactor Trip Circuit Breakers (RTCBs), and four manual TRIP CHANNELS shall be OPERABLE.

APPLICABILITY: MODES 1 and 2,

MODES 3, 4, and 5, with any RTCBs closed and any control element assemblies capable of

being withdrawn.

#### ACTIONS

romana,	CONDITION	REQUIRED ACTION	COMPLETION TIME
	RTCBs associated with one inoperable CHANNEL may be closed for up to 1 hour for the performance of an RPS CHANNEL FUNCTIONAL TEST.		
Α.	One CHANNEL of RTCBs, one MANUAL TRIP CHANNEL, or ONE RPS LOGIC CHANNEL inoperable in MODE 1 or 2.	A.1 Open the affected RTCBs.	1 hour

RPS Logic and Trip Initiation 3.3.4

## ACTIONS (Continued)

	CONDITION	REQUIRED ACTION	COMPLETION TIME
	RTCBs associated with one inoperable CHANNEL may be closed for up to 1 hour for the performance of an RPS CHANNEL FUNCTIONAL TEST.		
В.	One CHANNEL of RTCBs, one MANUAL TRIP CHANNEL, or one RPS LOGIC CHANNEL inoperable in MODE 3, 4, or 5.	B.1 Open all RTCBs.	48 hours
c.	Two CHANNELS of RTCBS, two MANUAL TRIP CHANNELS, or one RPS LOGIC CHANNEL affecting the same TRIP LEG inoperable.	C.1 Open the affected RTCBs.	Immediately

(Continued)

Define TRIP LEG

RPS Logic and Trip Initiation 3.3.4

## ACTIONS (Continued)

	CONDITION	REQUIRED ACTION	COMPLETION TIME	
D.	Required Action and Associated Completion Time of Condition A or C not met.	D.1 Be in MODE 3.  AND	6 hours	
	OR	D.2 Open all RTCBs.	6 hours	
	One or more Functions with more than one RPS LOGIC CHANNEL, MANUAL TRIP CHANNEL, or RTCB CHANNEL inoperable for reasons other than Condition A or C.			

# SURVEILLANCE REQUIREMENTS

	FREQUENCY	
SR 3.3.4.1	Perform a CHANNEL FUNCTIONAL TEST on each RPS LOGIC CHANNEL and RTCB CHANNEL.	[92] days
SR 3.3.4.2	Perform a CHANNEL FUNCTIONAL TEST, including separate verification of the undervoltage and shunt trips, on each RTCB.	[18] months
SR 3.3.4.3	Perform & TRIP TEST on each set of RPS MANUAL TRIP CHANNELS.	Once within 7 days prior to each reactor startup







ESFAS Instrumentation 3.3.5

3.3 INSTRUMENTATION

3.3.5 Engineered Safety Features Actuation System (ESFAS) Instrumentation

LCO 3.3.5 Four ESFAS TRIP CHANNELS and operating bypass CHANNELS for each Function in Table 3.3.5-1 shall be OPERABLE.

APPLICABILITY: According to Table 3.3.5-1.

#### ACTIONS

Separate Condition entry is allowed for each ESFAS Function.

2. If a CHANNEL is placed in bypass, continued operation with the CHANNEL in the bypassed condition for the Completion Time specified by Required Action A.2 or C.2.2 shall be reviewed in accordance with Specification [5.5.1.2.e].

	CONDITION		REQUIRED ACTION	COMPLETION TIME
A. One or more Functions with one automatic ESFAS TRIP CHANNEL inoperable.		A.1	Place CHANNEL in bypass or trip.	1 hour
		A.2	Restore CHANNEL to OPERABLE status.	Prior to entering MODE 2 following next MODE 5 entry.
			LCO 3.0.4 is not applicable.	
В.	One or more Functions with two automatic ESFAS TRIP CHANNELS inoperable.	B.1	TRIP Place one CHANNEL in bypass and the other in trip.	1 hour

ESFAS Instrumentation 3.3.5

# ACTIONS (Continued)

	CONDITION	REQUIRED ACTION	COMPLETION TIME
c.	One or more Functions with one automatic operating bypass removal CHANNEL (U.) chick inoperable.	C.1 Verify operating bypass is not in  offect.  OR	1 bour
		C.2.1 Place affected automatic TRIP CHANNEL in bypass or trip.	Once per 12 hours thereafter
		C.2.2 Restore operating bypass removal CHANNEL and associated automatic TRIP CHANNEL to OPERABLE status.	Prior to entering MODE 2 following next MODE 5 entry.
>.	One or more Functions with two automatic operating bypass removal CHANNELS Tundies inoperable.	D.1 Verify operating bypasses are not in effect.	1 hour  AND  Once per 12 hours
		D.2 Place one affected automatic TRIP CHANNEL in bypass and place the other in trip.	thereafter  1 hour

ESFAS Instrumentation 3.3.5

## ACTIONS (Continued)

	CONDITION	REQUIRED ACTION	COMPLETION TIME
		Does not apply to Functions 1, 2, 3, and  4 of Table 3, 3.5 1  Applies only to Functions 5 16	of Table 3.3.5-1
E.	Required Action and associated Completion Time not met.	E.1 Be in MODE 3.	6 hours
		E.2 Be in MODE 4.	[12] hours
		Docs not apply to Functions 5 and 6 of 7	000 +
F.	Required Action and associated Completion Time not met.	F.1 Be in MODE 3	6 hours
		F.2 Be in MODE 5	36 hours

Explain in Buses

ESFAS Instrumentation 3.3.5

# SURVEILLANCE REQUIREMENTS

	SURVEILLANCE	FREQUENCY
SR 3.3.5.1	Perform CHANNEL CHECK of each ESFAS CHANNEL	12 hours
SR 3.3.5.2	Perform & CHANNEL FUNCTIONAL TEST of each ESFAS CHANNEL, including operating bypass removal functions:	92 days
SR 3.3.5.3	Perform & CHANNEL CALIBRATION of each ESFAS CHANNEL, including operating bypass removal functions.	[18] months
SR 3.3.5.4	Verify ESFAS RESPONSE TIME is within limits.	[18] months on a STAGGERED TEST BASIS
SR 3.3.5.5	Perform CHANNEL FUNCTIONAL TEST on each automatic operating bypass removal CHANNEL.	Once within 92 days prior to each reactor startup

ESFAS Instrumentation 3.3.5

Table 3.3.5-1
ENGINEERED SAFETY FEATURES ACTUATION SYSTEM INSTRUMENTATION

FUNCTION	APPLICABLE MODES OR OTHER SPECIFIED COMDITIONS	ALLOWABLE VALUE
1. Safety Injection Actuation Signal  a. Containment Pressure - High b. Pressurizer Pressure - Low(*)	1,2,3,4	<pre></pre>
Containment Spray Actuation     Signal     a. Containment Pressure - High     High	1,2,3,4	≤ [8.5 psig]
3. Containment Isolation Actuation Signal a. Containment Pressure - High b. Pressurizer Pressure - Low <sup>(*)</sup>	1,2,3,4 1,2,3,4	S [2.7] paig Trip 2 [1825 paid]; Trip Operating Bypass Removal 2 [500 paid) Step 5 [400 paid) Floor 2 [300 paid)
<ul> <li>Main Steam Isolation Signal</li> <li>a. Steam Generator Prossure -         Low<sup>(b)</sup></li> <li>b. Containment Pressure - High</li> <li>c. Steam Generator Level - High</li> </ul>	1, 2 <sup>(c)</sup> , 3 <sup>(c)</sup> , 4 <sup>(c)</sup> 1, 2 <sup>(c)</sup> , 3 <sup>(c)</sup> , 4 <sup>(c)</sup> 1, 2 <sup>(c)</sup> , 3 <sup>(c)</sup> , 4 <sup>(c)</sup>	≥ [843 psia] ≤ [3.7] psig ≤ [90.8% NR]
5. Emergency Feedwater Actuation Signal SG #1 (EFAS-1) a. Steam Generator Level - Low b. Steam Generator Level - Nigh	1,2,3 1,2,3	≥ [23.4% MR] ≤ [53.4% MR]
S. Emergency feedwater Actuation Signal SG #2 (EFAS-2) a. Steam Generator Level - Low b. Steam Generator Level - High	1,2,3 1,2,3	≥ (23.4% WR) ≤ (53.4% WR)

- (a) The setpoint may be decreased to a minimum value of [300] psia, as pressurizer pressure is reduced, provided the margin between pressurizer pressure and the setpoint is maintained ≤ [400] psia. Trips may be bypassed when pressurizer pressure is < [400] psia. Bypass shall be automatically removed when pressurizer pressure is ≥ [500] psia. The setpoint shall be automatically increased to the normal setpoint as pressurizer pressure is increased.</p>
- (b) The setpoint may be decreased as steam pressure is reduced, provided the margin between steam pressure and the setpoint is maintained ≤ [200] psig. The setpoint shall be automatically increased to the normal setpoint as steam pressure is increased.
- (c) The Main Steam Isolation Signal (MSIS) Function (Steam Generator Pressure Low, Containment Pressure -High, and Steam Generator Level - High signals is not required to be OPERABLE when all associated valves isolated by the MSIS Function are closed and [deactivated].

SYSTEM 80+

3.3-25

16.3 Tech Spec

ESFAS Logic and Manual Initiation 3.3.6

3.3

INSTRUMENTATION

3.3.6

Engineered Safety Features Actuation System (ESFAS) Logic and Manual Initiation

define or drunge to LC

Four ESFAS LOGIC CHANNELS, four ESFAS MANUAL INITIATION CHANNELS, two divisions of ACTUATION LOGIC and COMPONENT CONTROL LOGIC for CIAS and MSIS, four divisions of ACTUATION LOGIC and COMPONENT CONTROL LOGIC for SIAS, CSAS, EFAS-1, and EFAS-2, and one DIVERSE MANUAL ESF ACTUATION CHANNEL Shall be

operable.

APPLICABILITY: According to Table 3.3.6-1

for diverse monual ESF attention interior to

ACTIONS

Separate Condition entry is allowed for each Function.

	CONDITION		REQUIRED ACTION	COMPLETION TIME
Α.	One or more Functions with one LOGIC CHANNEL or MANUAL INITIATION CHANNEL inoperable.	A.1	Place the affected TRIP LEG in each division in trip.	1 hour
B. E	One DIVERSE MANUAL ESF ACTUATION CHANNEL inoperable.	₽.1 E	Declare associated ESF Function inoperable.	1 hour
K.	One or more Functions with two LOGIC CHANNELS or MANUAL INITIATION CHANNELS affecting the same TRIP LEG inoperable.	E.1 B	Place the affected TRIP LEG in each division in trip.	Immediately

ESFAS Logic and Manual Initiation 3.3.6

# ACTIONS (Continued)

CONDITION			REQUIRED ACTION	COMPLETION TIME	
D.	One or more Functions with one or more divisions of ACTUATION LOGIC or COMPONENT CONTROL LOGIC inoperable.	D.1	Declare the associated ESF Function inoperable.	Immediately	
É.	One or more Functions with two or more LOGIC CHANNELS or MANUAL INITIATION CHANNELS affecting both TRIP LEGS in the associated Function inoperable.	Æ.1	Declare the associated ESF Function inoperable.	Immediately	
	3		of Table 3.3.6-1.		
F.	Required Action and associated Completion Time not met.	F.1	Be in MODE 3.	6 hours	
		F.2	Be in MODE 4.	[12] hours	
			NOTE————————————————————————————————————	7	
G.	Required Action and associated Completion Time not met.	G.1	Be in '10DE 3.	6 hours	
		G.2	Be in MODE 5.	36 hours	

ESFAS Logic and Manual Initiation

## SURVEILLANCE REQUIREMENTS

		SURVEILLANCE	FREQUENCY
	SR 3.3.6.1	Testing of Actuation Logic shall include the verification of the proper operation of each initiation circuit.	
X		Perform & CHANNEL FUNCTIONAL TEST on each ESFAS LOGIC CHANNEL and on each ESFAS division of ACTUATION LOGIC and COMPONENT CONTROL LOGIC.	[92] days
#	SR 3.3.6.2	Perform a selective group test on each division of ACTUATION LOGIC and COMPONENT CONTROL LOGIC to verify the OPERABILITY of each selective group.	[18] months
*	SR 3.3.6.3	Perform & CHANNEL FUNCTIONAL TEST on each ESFAS MANUAL INITIATION CHANNEL and on the DIVERSE MANUAL ESF ACTUATION CHANNEL.	[18] months

TRIP LEG

ESFAS Logic and Manual Initiation 3.3.6

Table 3.3.6-1

ENGINEERED SAFETY FEATURES ACTUATION SYSTEM LOGIC AND MARGAL TRIP

FUNCTION	APPLICABLE MODES
1. Safety Injection Actuation Signal (SIAS)  a. LOGIC CHANNEL b. ACTUATION LOGIC c. COMPONENT CONTROL LOGIC d. MANUAL INITIATION CHANNEL	1,2,3,4 1,2,3,4 1,2,3,4 1,2,3,4
2. Containment Spray Actuation Signal (CSAS)	OFFICE STATE OF THE PROPERTY O
a. LOGIC CHANNEL b. ACTUATION LOGIC c. COMPONENT CONTROL LOGIC d. MANUAL INITIATION CHANNEL	1,2,3,4 1,2,3,4 1,2,3,4 1,2,3,4
3. Containment Isolation Actuation Signal (CIAS)	
6. LOGIC CHANNEL b. ACTUATION LOGIC c. COMPONENT CONTROL LOGIC d. MANUAL INITIATION CHANNEL	1,2,3,4 1,2,3,4 1,2,3,4 1,2,3,4
4. Main Steam Isolation Signal (MSIS)	
#. LOGIC CHANNEL b. ACTUATION LOGIC c. COMPONENT CONTROL LOGIC d. MANUAL INITIATION CHANNEL	1,2,3,4 1,2,3,4 1,2,3,4 1,2,3,4
5. Emergency Feedwater Actuation Signal SG #1 (EFAS-1)	
a. LOGIC CHANNEL b. ACTUATION LOGIC c. COMPONENT CONTROL LOGIC d. MANUAL INITIATION CHANNEL	1,2,3 1,2,3 1,2,3 1,2,3
6. Emergency Feedwater Actuation Signal SG #2 (EFAS-2)	
a. LOGIC CHANNEL b. ACTUATION LOGIC c. COMPONENT CONTROL LOGIC d. MANUAL INITIATION CHANNEL	1,2,3 1,2,3 1,2,3 1,2,3
7. Diverse Manual ESF Actuation Interface to ESF Components.	
8. DIVERSE MANUAL ESF ACTUATION CHANNEL	1,2,3,4

DG - 1.GVS 3.3.7

#### 3.3 INSTRUMENTATION

# 3.3.7 Diesel Generator (DG) - Loss of Voltage Start (LOVS)

LCO 3.3.7

Three CHANNELS of Loss of Voltage Function and three CHANNELS of Degraded Voltage Function auto-initiation instrumentation per DG shall be OPERABLE.

APPLICABILITY: | MODES 1, and 2, 3, and 4. | When associated DG is required to be OPERABLE by LCO 3.8.2, "AC Sources - Shutdown."

## ACTIONS

Separate Condition entry is allowed for each Function.

	CONDITION	REQUIRED ACTION	COMPLETION TIME
		LCO 3.0.4 is not applicable.	
A.	One or more Functions with one CHANNEL inoperable.	A.1 Place CHANNEL in trip.	1 hour
B.	One or more Functions with two CHANNELS inoperable.	B.1 Place one CHANNEL in trip  AND CRAABLE STATE  B.2 Enter applicable Conditions and Required Actions for the associated DG made inoperable by DG - LOVS instrumentation.	Immediately   here
C.	Required Action and associated Completion Time of Condition A not met.	C.1 Enter applicable Conditions and Required Actions for the associated DG made inoperable by DG - LOVS instrumentation.	Immediately



DG - LOVS 3.3.7

# SURVEILLANCE REQUIREMENTS

	SURVEILLANCE	FREQUENCY
[SR 3.3.7.1	Perform CHANNEL CHECK.	12 hours]
SR 3.3.7.2	Perform CHANNEL FUNCTIONAL TEST.	[92] days
SR 3.3.7.3	Perform CHANNEL CALIBRATION with setpoint Allowable Values as follows:	[18] months
	a. Degraded Voltage Function $\geq$ [*] V and $\leq$ [*] V	
	Time delay: $\geq$ [*] seconds and $\leq$ [*] seconds at [*] V; and	
	b. Loss of Voltage Function $\geq$ [*] V and $\leq$ [*] V	
	Time delay: ≥ [*] seconds and ≤ [*] seconds at [*] V.	

<sup>\*</sup> Value to be determined by system detail design.

APS 3.3.8

3.3

INSTRUMENTATION

3.3.8

Alternate Protection System (APS)

LCO 3.3.8

Two elternate protection System CHANNELS for each function in Table 3.3.8-1 shall be OPERABLE.

AFOLICABILITY: MODES 1 and 2 .

ACTIONS

ACTI	ONS	Ly	
-	CONDITION	REQUIRED ACTION COMPLETION TIME	23
Α.	One or more CHANNELS inoperable.	A.1 Place CHANNEL in bypass. 1 hour  AND  A.2 Restore all CHANNELS to OPERABLE status. [31 days]	"XL BYN
В.	Required Action and associated completion time of Condition A not met.	B.1 Be in MODE 3. 6 hours	

# SURVEILLANCE REQUIREMENTS

	SUKVEILLANCE		
SR 3.3.8.1	Perform CHANNEL CHECK on each APS instrument CHANNEL.	12 hours	
SR 3.3.8.2	Perform a CHANNEL FUNCTIONAL TEST on each CHANNEL.	92 days	
SR 3.3.8.3	Perform & CHANNEL CALIBRATION on each CHANNEL.	[18] months	

APS 3.3.8

## TABLE 3.3.8-1

ALTERNATE PROJECTION SYSTEM

	Function	Applicable Modes	Adlowable Value
1.	Pressurizer Pressure - Reactor Trip	1,2	≥ [2420 psia]
2.	Steam Generator 1 Level - AFAS	1,2	≤ [23.4]%
3.	Steam Generator 2 Level - AFAS	1,2	≤ [23.4]%
4.	CEDMCS Bus Under Voltage - Turbine Trip	1,2	≤ [*]

<sup>\*</sup> Value to be determined by system detail design.

CRIFS 3.3.9

3.3 INSTRUMENTATION

3.3.9 Control Room Intake/Filtration Signal (CRIFS)

LCO 3.3.9 Two CRIFS divisions shall be OPERABLE.

APPLICABILITY: MODES 1, 2, 3, 4, 5, and 6 during CORE ALTERATIONS; during movement of irradiated fuel assemblies.

#### ACTIONS

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CONDITION		REQUIRED ACTION	COMPLETION TIME	
A. See CRIFS division inoperable in MODE : 3, or 4.	1, 2, A.1	Close bypass dampers on control room filtration units and start filtration units.	1 hour	
	AND			
gramman and a management of the second	A.2	Restore division to OPERABLE status	7 days	
B. Two CRIFS divisions inoperable in MODE 1  3, 4 or Required Action assorted Completion T Condition A not met.	on and	Close bypass dampers on control room filtration units and start the filtration units.	1 hour	
	B.2 AND	Be in MODE 3	6 hours	
	B.2	Be in MODE 5	36 hours	
C. One CRIFS division inoperable in MODE 5 during CORE ALTERATIONS or du movement of irradiated assemblies.	iring	Close bypass dampers on the control room filtration units and start filtration units.	1 hour	
	C.2	Restore division to OPERABLE status	7 days	

CRIFS 3.3.9

#### SURVEILLANCE REQUIREMENTS (Continued)

	SURVEILLANCE	FREQUENCY
SR 3.3.9.3	Surveillance of a CTUATION LOGIC shall include the verification of the proper operation of each subgroup.      Subgroups associated with plant equipment that cannot be	6.5
1	operated during plant operation are required to be tested during each MODE 5 entry exceeding 24 hours unless tested within the previous 6 months.	20,
1	Perform a selective group test on each CRIFS OUTPUT DIVISION which includes ACTUATION LOGIC to verify the OPERABILITY of each selective group.	[18] months
SR 3.3.9.4	Perform a CHANNEL CALIBRATION on required CRIFS radiation monitor CHANNELS.	[18] months

1

CRIFS CHANNEL not defined in Beses

CRIFS 3.3.9

#### ACTIONS (Continued)

CONDITION	REQUIRED ACTION	COMPLETION TIME
D. Two CRIFS divisions inoperable in MODE 5 or 6, during CORE  ALTERATIONS or during movement of irradiated fuel assemblies.	D.1 Close bypass dampers on control room filtration units and start the filtration units.  OR	Immediately
	D.2 Suspend movement of irradiated fuel assemblies.	Immediately
	AND	
	D.3 Suspend positive reactivity additions.	Immediately
	AND	
	D.4 Suspend CORE ALTERATIONS.	Immediately

## SURVEILLANCE REQUIREMENTS

*****	SURVEILLANCE	FREQUENCY
SR 3.3.9.1	Perform a CHANNEL CHECK on each control room radiation monitor CHANNEL.	12 hours
SR 3.3.9.2	Perform & CHANNEL FUNCTIONAL TEST on each CRIFS LOGIC CHANNEL.	[92] days
	Verify CRIFS high radiation setpoint [Allowable Value] is ≤ [] cpm above normal background	



## CESSAR DESIGN CERTIFICATION

Containment Bypass Instrumentation 455,768

3.3

INSTRUMENTATION



Containment Bypass Instrumentation Steam Generator Tube Rupture (SGTR)



LCO 3.3.10 The Containment Bypass Instrumentation SGTR in Table 3.3.10-1 shall be OPERABLE.

APPLICABILITY: According to Table 3.3.10-1

#### **ACTIONS**

	CONDITION		REQUIRED ACTION	COMPLETION TIME
Α.	One required CHANNEL inoperable.	A.1	Restore required CHANNEL to OPERABLE status	30 days
B.	Required Action and associated Completion Time of Condition	B.1	Be in MODE 3	6 hours
	A not met or two Functions With two required CHANNELS	AND		
	inoperable.	B.2	Be in MODE 5	36 hours

## SURVEILLANCE REQUIREMENTS

	FREQUENCY	
SR 3.3.10.1	R 3.3.10.1 Perform & CHANNEL CHECK on each instrument CHANNEL	
SR 3.3.10.2	Perform a CHANNEL FUNCTIONAL TEST on each instrument CHANNEL.	92 days
SR 3.3.10.3	Perform a CHANNEL CALIBRATION on each instrument CHANNEL.	[18] months



Containment Bypass Instrumentation 5GTP 3.3.10

TABLE 3.3.10-1

## CONTAINMENT BYPASS INSTRUMENTATION (STEAM GENERATOR TUBE RUPTURE)

*************		FUNCTION	APPLICABLE MODE OR OTHER SPECIFIED CONDITION	REQUIRED NUMBER OF CHANNELS
1.	SG 1	Tube Rupture Identification		
	ă.,	SG1 Blowdown/Steam Line Radiation Monitors	1,2,3,4	1
	ь.	Main Steam Line 1 N-16 Monitor	1 (a)	1
2.	SG 2	Tube Rupture Identification		The thirt was a second
	a.	SG2 Blowdown/Steam Line Radiation Monitors	1,2,3,4	1
	b.	Main Steam Line 2 N-16 Monitor	1 (a)	1

[(a) When ≥ 25 % RTP]

PAMI 3.3.11

3.3 INSTRUMENTATION

3.3.11 Post Accident Monitoring Instrumentation (PAMI)

LCO 3.3.11 The PAMI for each Function in Table 3.3.11-1 shall be OPERABLE.

APPLICABILITY: MODES 1, 2, and 3.

#### ACTIONS

LCO 3.0.4 not applicable.

- and the second production
- 2. Separate Condition Entry is allowed for each Function.

	CONDITION		REQUIRED ACTION	COMPLETION TIME
Α.	One or more Functions with one required CHANNEL inoperable. MEASUREMENT	A.1	Restore required CHANNEL to OPERABLE status.	30 days
В.	Required Action and associated Completion Time of Condition A not met.	B.1	Initiate action in accordance with Specification 5.9.2.c.	Immediately
C.	One or more Functions with two required CHANNELS inoperable. MEASURE INC.	C.1	Restore one CHANNEL to OPERABLE status.	7 days
D.	Required Action and associated Completion Time of Condition C not met.	D.1	Enter the Condition referenced in Table 3.3.11-1 for the CHANNEL.	Immediately

PAMI 3.3.11

#### ACTIONS (Continued)

	CONDITION		REQUIRED ACTION	COMPLETION TIME
E.	As required by Required Action D.1 and referenced in Table 3.3.11-1.	E.1	Be in MODE 3.	6 hours
		E.2	Be in MODE 4.	12 hours
F.	As required by Required Action D.1 and referenced in Table 3.3.11-1.	F.1	Initiate action in accordance with Specification 5.9.2.C.	Immediately

#### SURVEILLANCE REQUIREMENTS

These SRs apply to each PAMI Function in Table 3.3.11-1.

	FREQUENCY	
SR 3.3.11.1	Perform CHANNEL CHECK for each required  Instrumentation CHANNEL that is normally energized.	31 days
SR 3.3.11.2	Perform CHANNEL CALIBRATION.	[18] months

PAMI 3.3.11

Table 3.3.11-1
POST ACCIDENT MONITORING INSTRUMENTATION

	FUNCTION	REGUIRED CHAMBELS	CONDITIONS REFERENCED FROM REGUIRED ACTION E.1
1.	Neutron Flux Power Level	2	E
2.	Reactor Coolant Outlet Temperature (T-Hot) Wide Range (WR)	2 per locp	E
3.	Reactor Coolant Inlet Temperature (T-Cold) WR	2 per loop	E
4.	Reactor Coolant System Pressure (WR)	2	E
5.	Reactor Vessel Coolant Level	2	[F]
6.	Reactor Cavity Level	2	E
7.	Containment Pressure (WR) (WR)	5	E (E)
8.	Containment Isolation Valve Position	1 per valve(a)	E
9.	Containment Area Radiation	2	[F]
10.	Containment Hydrogen Concentration	2	E
11.	Pressurizer Water Level	2	E
12.	Steam Generator (SG) Water Level (WR)	2 per steam generator	E
13.	Emergency Feedwater Storage Tank Level	2 Per tank	E
14.	Core Exit Temperature - Quadrant [1]	[2")]	E
15.	Core Exit Temperature - Quadrant [2]	(2 <sup>(b)</sup> )	E
16.	Core Exit Temperature - Quadrant [3]	[2(*)]	E
17.	Core Exit Temperature - Quadrant [4]	[20)]	E
18.	Steam Generator Pressure	2	ε
19.	Degree of Subcooling	2(c)	E
20.	Primary Coolant (T-Hot) Radiation Level	2	E

<sup>(</sup>a) Not required for isolation valves whose associated penetration is isolated by at least one closed and deactivated automatic valve, closed manual valve, blind flange, or check valve with flow through the valve secured.

(b) A CHANNEL consists of two or more core exit thermocouples. Tradial enthalpy distribution

(c) A CHANNEL consists of one or more Core Exit Temperature, Reactor Coolant Inlet Temperature (T-Cold) Wide Range, Reactor Coolant Dutlet Temperature (T-Hot) wide range, and Pressurizer Pressure (High Range, Mid Range, and Low Range).



3.3

INSTRUMENTATION

3.3.12

Remote Shutdown Instrumentation and Controls

LCO 3.3.12 The Remote Shutdown Instrumentation and Control Functions in Table 3.3.12-1 shall be OPERABLE.

APPLICABILITY: MODES 1, 2 and 3.

ACTIONS

1. LCO 3.0.4 is not applicable.

-NOTE-

_	CONDITION		REQUIRED ACTION	COMPLETION TIME
A.	One division with one or more Required Functions inoperable.	A.1	Restore required division to OPERABLE status.	92 days
В.	Two divisions with one or more required Functions inoperable.	B.1	Restore one division to OPERABLE status.	31 days
C.	Required Action and associated Completion Time not met.	C.1	Be in MODE 3.	6 hours
		C.2	Be in MODE 4.	[12] hours

### TABLE 3.3.12-1

## (Sheet 1 of 4)

REMOTE	SHUTDOWN	INSTRUMENTATION	AND CONTROLS
--------	----------	-----------------	--------------

FUNCTION (INSTRUMENT OR CONTROL PARAMETER)	REQUIRED NUMBER OF DIVISION
Instrumentation and Controls for Hot Str	andby (MODE 3)
NSSS Instrumentation	
1. Neutron Logarithmic Power	2
2. Hot Leg Temperature	å 1 per loop
3. Cold Leg Temperature	1 per loop
Pressurizer Pressure	XZ
5. Pressurizer Level	2
5. Pressurizer RCGV Position	1 per valve
7. Steam Generator (SG) No. 1 Pressure	2
3. Steam Generator No. 1 Level	2
9. Steam Generator No. 2 Pressure	2
10. Steam Generator No. 2 Level	2
11. CVCS Charging Flow	1
12. CVCS Charging Pressure	1
13. Boric Acid Storage Tank Level	1
14. In-Containment Refueling Water Storage Tank (IRWST) Level	2
15. SIS Pump Discharge Flow	2(*)
16. SIS Pump Discharge Header Pressure	2(a)
17. EFW Pump Discharge Pressure (SG No. 1)	Z <sup>(b)</sup>
18. EFW Pump Discharge Pressure (SG No. 2)	2(p)
19. EFW Pump Suction Pressure and Low Pressure Alarm (SG No. 1)	2 <sup>(b)</sup>

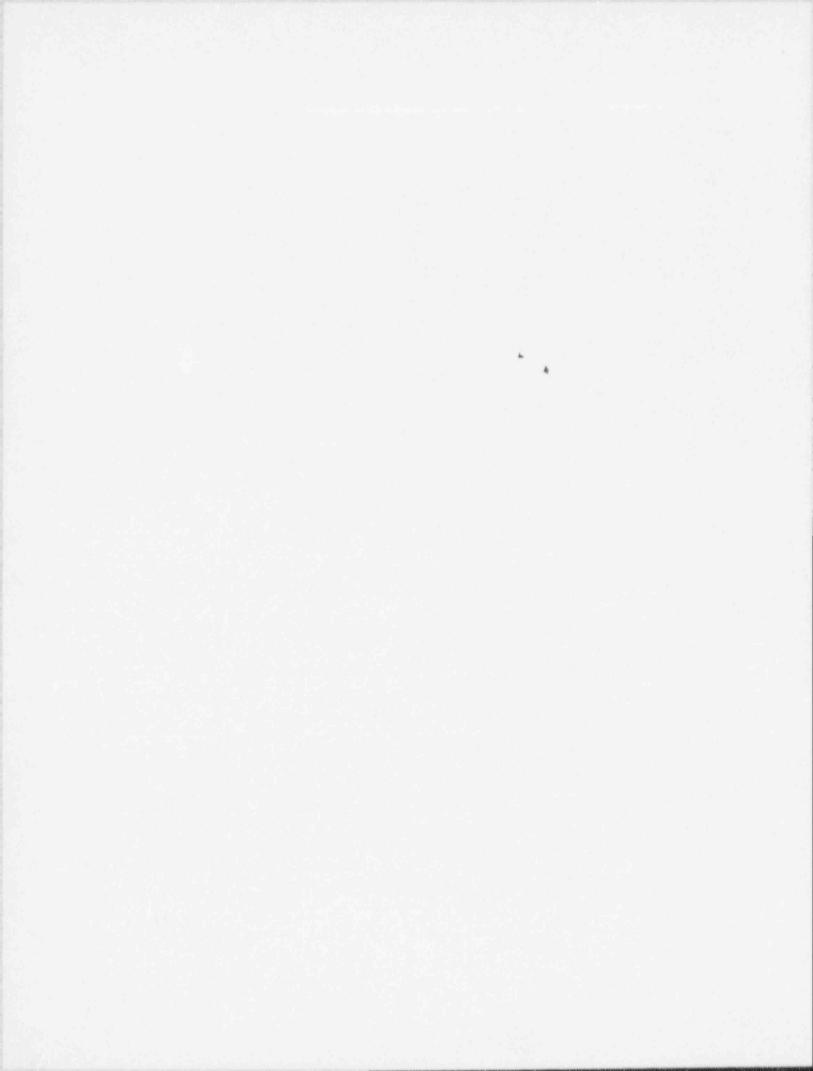


TABLE 3.3.12-1 (Continued)

RSIC 3.3.12

FUNCTION (INSTRUMENT OR CONTROL PARAMETER)	REQUIRED NUMBER OF DIVISIONS
Instrumentation and Controls for Hot Sta	ndby (MODE_3)
NSSS Instrumentation (Cont'd)	
33. Almospheric Steam Dump Valve and ADV Block Valvey (SG No. 2)	2
34. Pressurizer Auxiliary Spray Valve Controls	1
35. Pressurizer Reactor Gas Vent (RCGV) Valve Controls	1 per valve
36. Charging Pump Controls	1
37. Letdown Isolation Valve Controls	l per valve
38. Reactor Coolant Pump Seal Bleedoff Valve Controls	1 per valve
39. SIS Pump Controls	2(*)

40. SIS Header Valve Controls

42. MSIS Actuation Switches

41. Manual Reactor Trip Switches

43. EFW Pump Controls (SG No. 1)

44. EFW Pump Controls (SG No. 2)

45. EFW Isolation Valves (SG No. 1)

46. EFW Isolation Valves (SG No. 2)

47. EFW Flow Control Valves (SG No. 1)

2(\*)

 $I^{(d)}$ 

2(b)

200)

2(0)

2(6)

2(6)

## SURVEILLANCE REQUIREMENTS

	SURVEILLANCE	FREQUENCY
SR 3.3.12.1	Perform CHANNEL CHECK for each required instrumentation CHANNEL that is normally energized.	31 days
SR 3.3.12.2	Verify each required indicator, control circuit, and transfer switch is capable of performing the intended function.	[18] months
SR 3.3.12.3	NOTE— Neutron detectors are excluded from the CHANNEL CALIBRATION.  Perform CHANNEL CALIBRATION for each required	[18] months

#### TABLE 3.3.12-1

#### (Sheet 1 of 3)

## REMOTE SHUTDOWN INSTRUMENTATION AND CONTROLS

FUNCTION (INSTRUMENT OR CONTROL PARAMETER)	REQUIRED NUMBER OF DIVISIONS
Instrumentation and Controls for Hot Sta	ndby (MODE 3)
NSSS Instrumentation	
Neutron Logarithmic Power	2
2. Hot Leg Temperature	1 per loop
3 Cold Leg Temperature	1 per loop
Pressurizer Pressure	2
5. Pressurizer Level	2
6. Pressurizer RCGV Position	1 per valve
7. Steam Generator (SG) No. 1 Pressure	2
8. Steam Generator No. 1 Level	2
9. Steam Generator No. 2 Pressure	2
10. Steam Generator No. 2 Level	2
11. In-Containment Refueling Water Storage Tank (IRWST) Level	2
12. SIS Pump Discharge Flow	2 <sup>(a)</sup>
13. SIS Pump Discharge Header Pressure	2 <sup>(a)</sup>
14. EFW Pump Discharge Pressure (SG No. 1)	2 <sup>(b)</sup>
15. EFW Pump Discharge Pressure (SG No. 2)	2(6)
16. EFW Pump Suction Pressure and Low Pressure Alarm (SG No. 1)	2 <sup>(b)</sup>
17. EFW Pump Suction Pressure and Low Pressure Alarm (SG No. 2)	2 <sup>(b)</sup>
18. EFW Steam Motive Power Instrumentation (SG No. 1)	1(b)(c)
19. EFW Steam Motive Power Instrumentation (SG No. 2)	1 <sup>(b)(c)</sup>
20. EFW Pump Flow (SG No. 1)	2 <sup>(b)</sup>
21. EFW Pump Flow (SG No. 2)	2 <sup>(b)</sup>
22. EFW Pump Recirculation Flow (SG No. 1)	2 <sup>(b)</sup>

## TARLE 3.3.12-1

(Sheet 2 of 3)

## REMOTE SHUTDOWN INSTRUMENTATION AND CONTROLS

FUNCTION (INSTRUMENT OR CONTROL PARAMETER)	REQUIRED NUMBER OF DIVISIONS
Instrumentation and Controls for Hot St	andby (MODE 3)
NSSS Instrumentation (Cont'd)	
23. EFW Pump Recirculation Flow (SG No. 2)	2 <sup>(b)</sup>
24. EFW Storage Tank Level and Low Alarm	1 per tank
BOP Instrumentation	
25. Ultimate Heat Sink Status Indication	1
26. Emergency Diesel Generator (DG) Status Indication	1 per DG
NSSS Controls	
27. Reactor Coolant Pump (RCF) Trip Pushbution	1 per RCP
28. Backup Heater Control	1
29. Atmospheric Steam Dump Valve (ADV) and ADV Block Valve Controls (SG No. 1)	2
30. Atmospheric Steam Dump Valve and ADV Block Valve Controls (SG No. 2)	2
31. Pressurizer Reactor Gas Vent (RCGV) Valve Controls	1 per valve
32. Reactor Coolant Pump Seal Bleedoff Valve Controls	1 per valve
33. SIS Pump Controls	2 <sup>(a)</sup>
34. SIS Header Valve Controls	2 <sup>(a)</sup>
35. Manual Reactor Trip Switches	1 <sup>(d)</sup>
36. MSIS Actuation Switches	1
37. EFW Pump Controls (SG No. 1)	2 <sup>(b)</sup>
38. EFW Pump Controls (SG No. 2)	2(%)
39. EFW Isolation Valves (SG No. 1)	2(%)
40. EFW Isolation Valves (SG No. 2)	2 <sup>(b)</sup>

#### TABLE 3.3.12-1

#### (Sheet 3 of 3)

#### REMOTE SHUTDOWN INSTRUMENTATION AND CONTROLS

FUNCTION (INSTRUMENT OR CONTROL PARAMETER)	REQUIRED NUMBER OF DIVISIONS			
Instrumentation and Controls for Hot Standby (MODE 3)				
NSSS Instrumentation (Cont'd)	è			
41. EFW Flow Control Valves (SG No. 1)	2 <sup>(b)</sup>			
42. EFW Flow Control Valves (SG No. 2)	2(6)			
43. EFW Steam Motive Power Controls (SG No. 1)	1(p)(e)			
44. EFW Steam Motive Power Controls (SG No. 2)	1(p)(e)			
BOP Controls				
45. Ultimate Heat Sink Controls	1			

- (a) SIS Train No. 3 for Division I, SIS Train No. 4 for Division II
- (b) Steam Driven Pump Instrumentation and Controls for Division I, Motor Driven Pump Instrumentation and Controls for Division II
- (c) Includes Steam-Driven Pump Turbine Inlet Pressure, Steam-Driven Pump Turbine Speed, Turbine Trip and Throttle (Stop) Valves Open/Close Position and Close Position Alarm, to Division I Steam Motive Power, No Instrumentation for Division II Motive Power.
- (d) A division consists of two Manual Reactor Trip Switches in opposite TRIP LEGS to meet the selective two-out-of-four logic for a reactor trip.
- (e) Includes EFW Steam Supply Bypass Valves, EFW Steam Supply Isolation Valves, EFW Turbine Trip and Throttle (Stop) Valves Trip/Reset Control, and EFW Turbine Speed Control for Division I, No Steam Motive Power Controls for Division II.
- (f) SCS Train No. 1 for Division I, SCS Train No. 2 for Division II

Logarithmic Power Monitoring Channels
3.3.13

#### 3.3 INSTRUMENTATION

#### 3.3.13 Logarithmic Power Monitoring CHANNELS

LCO 3.3.13 Two CHANNELS of logarithmic power level monitoring instrumentation shall be OPERABLE.

APPLICABILITY: MODES 3, 4, and 5 with the reactor trip circuit breakers open or Control Element Assembly (CEA) Drive System not capable of CEA withdrawal.

#### ACTIONS

CONDITION			REQUIRED ACTION	COMPLETION TIME	
A.	One or more required CHANNEL(S) inoperable.	A.1	Suspend all operations involving positive reactivity additions.	Immediately	
		ANI	2		
		A.2	Perform SDM verification in accordance with SR 3.1.1.1, if	4 hours	
			$T_{\text{evg}} > 200^{\circ}\text{F. or SR } 3.1.2.1, \text{ if } T_{\text{evg}} \leq 200^{\circ}\text{F.}$	AND	
				Once per 12 hours thereafter	

# CESSAR DESIGN CERTIFICATION

Logarithmic Power Monitoring Channels
3.3.13

## SURVEILLANCE REQUIREMENTS

	SURVEILLANCE	FREQUENCY
SR 3.3.13.1	Perform CHANNEL CHECK.	12 hours
SR 3.3.13.2	Perform CHANNEL FUNCTIONAL TEST.	[92] days
SR 3.3.13.3	NOTE -	
	Neutron detectors are excluded from CHANNEL CALIBRATION.	
	Perform CHANNEL CA LIBRATION.	
		[18] months

Previsors Tech Spec

Containment (Atmospheric and Dual)

Reviewed by ECGB

SURVEILLANCE REQUIREMENTS

	FREQUENCY	
SR 3.6.1.1	Perform required visual examinations and leakage rate testing except for containment air lock testing, in accordance with 10 CFR 50, Appendix J, as modified by approved exemptions.  The maximum allowable leakage rate, L, is []% of containment air weight per day at the calculated peak containment pressure, Pa.	In accordance with 10 CFR 50, Appendix J, as modified by approved exemptions
SR 3.6.1.2	Verify containment structural integrity in accordance with the Containment Tendon Surveillance Program.	In accordance with the Containment Tendon Surveillance Program

Protection requirements le monty structural entegraty of start containment, particularly in the anchorage in it the trans of the steel containment.