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Dave Morey Vice President Parley Project

Southern Nuclear Operating Company

the southern electric system

February 23, 1994

Docket No. 50-348

10 CFR 50.90

U. S. Nuclear Regulatory Commission ATTN: Document Control Desk Washington, D. C. 20555

Joseph M. Farley Nuclear Plant - Unit 1
Technical Specification Changes Associated With
Steam Generator Tube Support Plate Interim Repair Criteria

Gentlemen:

By letter dated February 26, 1991, Alabama Power Company submitted a steam generator tube support plate alternate plugging criteria. As a result of delays in approving the full voltage alternate repair criteria, a reduced voltage interim plugging criteria (IPC) was approved for Unit 1. The Unit 1 criteria were approved for one cycle of use on October 8, 1992. The one cycle interim plugging criteria is only valid until the next refueling outage currently scheduled for March 1994.

By letter dated December 9, 1993, Southern Nuclear submitted a proposed amendment to the Unit 1 Technical Specifications to allow implementation of a 2 volt IPC. During public meetings held on February 8-9, 1994, the NRC Staff provided comments on the proposed 2 volt IPC.

In response to the NRC Staff comments and subsequent discussions with the Staff, Southern Nuclear will take the following actions in support of our 2 volt IPC request:

- Southern Nuclear will continue to report predicted steam line break leakage prior to restart from the outage. Following completion of the outage, Southern Nuclear will provide a more detailed assessment of the effectiveness of the IPC methodology similar to that provided following the Unit 2 ninth refueling outage. Included in this detailed assessment will be a calculated probability of tube rupture in the event of a steam line break.
- Prior to using smaller bobbin probes in the implementation of the IPC, additional
 evaluation of the smaller probes will be performed. This evaluation will be discussed
 with the NRC Staff prior to its implementation.
- 3. Southern Nuclear will reduce the upper bobbin voltage limit at which tubes inspected using the rotating pancake coil probe can be left in service if degradation is not detected to the level previously approved in earlier IPCs. Attachment 1 contains a revised Technical Specification page reducing the voltage. This page should be used as a replacement page for page 3/4 4-12a included in the December 9, 1993, submittal.

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9403090209 940223 PDR ADDCK 05000348 P PDR 4 Southern Nuclear will continue to inform the NRC, prior to restart from the refueling outage, of any unexpected inspection findings relative to the assumed characteristics of the flaws at tube support plates. This includes any detectable circumferential indications or detectable indications extending outside the tube support plate thickness. Any applicable safety evaluations for unexpected findings will be provided to the NRC.

Attachment 2 provides the updated voltage repair criteria.

Following a review of the 10CFR50.92 Significant Hazards Consideration evaluation contained in the December 9, 1993, submittal, it was determined that the evaluation remains valid for these changes. Consequently, a revised Significant Hazards Consideration evaluation is not required.

A copy of these proposed changes is being sent to Dr. D. E. Williamson, the Alabama State Designee, in accordance with 10 CFR 50.91(b)(1).

If there are any questions, please advise.

Respectfully submitted.

SOUTHERN NUCLEAR OPERATING COMPANY

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Attachments

Mr. S. D. Ebneter

Mr. B. L. Siegel

Mr. T. M. Ross

Dr. D. E. Williamson

SWORN TO AND SUBSCRIBED BEFORE ME

THIS 33M DAY OF February 1994

Datha Payle Down

Notary Public

My Commission Expires: 700.1, 1997