



UNITED STATES
NUCLEAR REGULATORY COMMISSION
ADVISORY COMMITTEE ON REACTOR SAFEGUARDS
WASHINGTON, D. C. 20555

SL-0374

PDR 10/11/90

September 21, 1990

The Honorable Kenneth M. Carr
Chairman
U.S. Nuclear Regulatory Commission
Washington, D.C. 20555

Dear Chairman Carr:

SUBJECT: SUMMARY REPORT - THREE HUNDRED SIXTY FIFTH MEETING
OF THE ADVISORY COMMITTEE ON REACTOR SAFEGUARDS,
SEPTEMBER 6-7, 1990

During its 365th meeting, September 6-7, 1990, the Advisory Committee on Reactor Safeguards discussed several matters and completed the reports and memoranda noted below. A summary of the activities of the Committee during this meeting is given below.

Reports to the Commission

- Implementation of the Safety Goal Policy (Report to Chairman Carr, dated September 11, 1990).
- Yankee Rowe Reactor Pressure Vessel Integrity (Report to Chairman Carr, dated September 12, 1990).
- Reevaluation of the SALP Program (Report to Chairman Carr, dated September 12, 1990).

Memoranda to the EDO

- Proposed Priority Rankings of Generic Issues: Sixth Group (Memorandum from C. Michelson, ACRS Chairman, for J. M. Taylor, EDO, dated September 11, 1990).
- Proposed Resolution of Generic Issue 23, "Reactor Coolant Pump Seal Failures" (Memorandum from R. F. Fraley, ACRS, for J. M. Taylor, EDO, dated September 11, 1990).

Mr. Fraley has informed Mr. Taylor of the Committee's decision that, although the Committee believes that the proposed resolution for Generic Issue 23 is not well developed, it has no objection to issuing it for public comment. The Committee desires a timely opportunity to review the proposed final resolution of this generic issue.

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Highlights of Matters Considered by the Committee• Implementation of the Safety Goal Policy

The Committee members discussed the Commission's guidance to the staff that is included in the June 15, 1990 Staff Requirements Memorandum for implementing the NRC Safety Goal Policy and provided a report to the Commission applauding its efforts in providing such a guidance. The members noted that they look forward to future interactions as the policy implementation develops.

• Yankee Rowe Nuclear Power Station

The Committee heard a Subcommittee report and discussed the potential safety significance of the Yankee Rowe reactor pressure vessel embrittlement due to neutron irradiation and its impact on the continued operation of this plant.

The Committee provided a report to the Commission on this matter, including comments and recommendations.

• Reevaluation of the SALP Program

The Committee discussed the staff's reevaluation of the SALP program, the contents of the Staff Requirements Memorandum dated August 10, 1990, related to SECY-90-189, "Reevaluation of the Systematic Assessment of Licensee Performance Program," and a September 4, 1990 letter from NUMARC on the subject of SALP and regulatory impact.

The Committee provided a report to the Commission on this matter. The Committee plans to meet with the staff in order to evaluate the staff's proposed regulatory reforms based on the Regulatory Impact Survey, including any reforms to the SALP program that may go beyond SECY-90-189.

• Reactor Operating Experience

The Committee heard presentations by and held discussions with representatives of the NRC staff regarding a request from certain Westinghouse plant owners for a license amendment to allow relaxation of the requirements for surveillance test frequency of turbine stop valves, governor valves, and the intercept valves associated with the turbine overspeed protection, and the related staff actions.

This was an information iter. - the Committee took no action.

- Restart of Nine Mile Point Unit 1

The Committee heard presentations by and held discussions with representatives of the NRC staff regarding the efforts taken by the staff and the licensee in restarting Nine Mile Point Unit 1 that has been shut down since December 1987 because of several safety-related problems and management ineffectiveness in identifying and solving the problems. The staff presented the results of the review conducted by the Restart Assessment Panel, Integrated Assessment Team, and Readiness Assessment Team on the corrective actions taken by the licensee.

This was an information item - the Committee took no action.

- Application of the Federal Advisory Committee Act (FACA) Requirements

The Committee members met with representatives of the NRC Office of General Counsel (OGC) and discussed OGC's interpretation of the application of the FACA requirements to ACRS Subcommittee and Subgroup activities.

There was no explicit decision to pursue this matter further. The instructions contained in the June 25, 1990 memorandum from Mr. Fitzgerald, OGC, for Mr. Fraley, ACRS, will be used as a guide for conducting ACRS Subcommittee and Subgroup activities.

- Appointment of New ACRS Members

The Committee members reviewed the qualifications of candidates for appointment to the ACRS and decided to continue the discussion of this matter during the October 4-6, 1990 ACRS meeting.

- Legal Services for the ACRS

The Committee members discussed a proposed response to your August 1, 1990 letter regarding the ACRS use of part time consultants and decided to continue the discussion of this matter during the October 4-6, 1990 ACRS meeting.

Subcommittee Meetings

Since the last summary report of ACRS activities, the following Subcommittee meetings have been held:

- Joint Thermal Hydraulic Phenomena and Decay Heat Removal Systems, August 28-30, 1990

The Subcommittees discussed the following:

- Details of the modifications made to the RELAP-5 MOD-2 Code as specified in the MOD-3 version.
- Use of feed and bleed for decay heat removal in PWRs.
- Proposed resolution of Generic Issue 23, "Reactor Coolant Pump Seal Failures."

- Materials and Metallurgy, September 5, 1990

The Subcommittee discussed the potential safety significance of the Yankee Rowe reactor pressure vessel embrittlement due to neutron irradiation and its impact on the continued operation of this plant.

- Advanced Pressurized Water Reactors, September 20, 1990

The Subcommittee reviewed the staff's Safety Evaluation Report for the Westinghouse SP/90 standard plant design.

- Advanced Pressurized Water Reactors, September 21, 1990

The Subcommittee continued its review of the CE System 80+ design and related safety issues.

Future Activities

The Committee agreed to the following tentative schedule for the 366th, October 4-6, 1990 ACRS meeting:

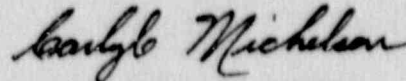
- Reactor Operating Experience (Open) - Briefing by representatives of the NRC staff regarding reactor operating experience and events, including the status of activities regarding the impact of non-condensable gases on operation of safety-related systems and components.
- Quality Assurance (Open) - Briefing by representatives of the NRC staff on the status of activities regarding performance-based quality assurance at nuclear facilities and the related proposed revision of the NRC Standard Review Plan.
- International Activities (Open/Closed) - Briefing regarding recent visit and meetings of the US-USSR Joint Coordinating Committee on Civilian Nuclear Reactor Safety and visit of U.S. representatives to the Loviisa Nuclear Power Plant in Finland.

- License Renewal Standard Review Plan (Open) - Review and comment on proposed revision to the NRC Standard Review Plan and associated Regulatory Guide regarding guidance for license renewal reviews of nuclear power plants. Representatives of the NRC staff and industry, as appropriate will participate in the discussion.
- Severe Accident Risk Assessment for Five U.S. Nuclear Power Plants (NUREG-1150) (Open) - The members will hear and discuss the report of the ACRS joint subcommittee meeting on Severe Accidents, Extreme External Phenomena, and Probabilistic Risk Assessment (fires and seismic events) as they relate to NUREG-1150 evaluations. The members will discuss comments and recommendations regarding the nature and use of NUREG-1150 and severe accident risk assessment in the regulatory process. Representatives of the NRC staff will participate as appropriate.
- Reactor Safety Research (Open) - The members will discuss the scope and content of the annual ACRS report to Congress on the NRC Safety Research Program.
- ACRS Subcommittee Activities (Open) - The members will hear reports and discuss activities of cognizant ACRS subcommittee meetings regarding safety-related and regulatory activities including the status of the NRC staff review of standardized nuclear plants such as the Combustion Engineering CESSAR Systems 80+ nuclear power plant, and use of bleed and feed cooling in nuclear power plants. Representatives of the NRC staff will participate as appropriate.
- Westinghouse Standard Plant Design (SP/90) (Open) - Review and report on the proposed preliminary design for this standardized nuclear power plant. Representatives of the NRC staff and the Westinghouse Electric Corporation will participate as appropriate.
- ACRS Procedures and Practices (Open) - The members will discuss ACRS use of part-time consultants in support of ACRS activities. Administrative matters such as a proposed update and revision of the ACRS ByLaws will be discussed, as appropriate.
- Advanced Reactor Designs (Open) - Briefing by representatives of the NRC staff regarding the nature and scope of activities being devoted to review and evaluation of advanced reactor designs.

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- Appointment of ACRS Members (Closed) - The members will discuss the qualifications of candidates proposed for appointment to the Committee.
- Regulatory Impact Survey (Open) - Briefing by representatives of the NRC staff regarding the status of staff actions to resolve the comments received during the Regulatory Impact Survey.

Sincerely,



Carlyle Michelson
Chairman