

# NORTHEAST UTILITIES



The Connecticut Light And Power Company  
Western Massachusetts Electric Company  
Holyoke Water Power Company  
Northeast Utilities Service Company  
Northeast Nuclear Energy Company

General Offices · Selden Street, Berlin Connecticut

P.O. BOX 270  
HARTFORD, CONNECTICUT 06414-0270  
(203)665-3000

Re: 10CFR50.73(a)(2)(i)(B)  
October 4, 1990  
MP-90-1082

U.S. Nuclear Regulatory Commission  
Document Control Desk  
Washington, D.C. 20555

Reference: Facility Operating License No. DPR-65  
Docket No. 50-336  
Licensee Event Report 90-013-00

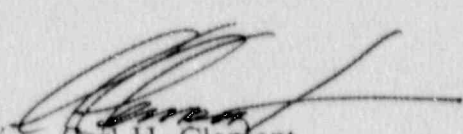
Gentlemen:

This letter forwards Licensee Event Report 90-013-00 required to be submitted within thirty (30) days pursuant to paragraph 50.73(a)(2)(i)(B).

Very truly yours,

NORTHEAST NUCLEAR ENERGY COMPANY

FOR: Stephen E. Scace  
Director, Millstone Station

BY:   
Carl H. Clement  
Millstone Unit 3 Director

SES/ERF:mo

Attachment: LER 90-013-00

cc: T. T. Martin, Region I Administrator  
W. J. Raymond, Senior Resident Inspector, Millstone Unit Nos. 1, 2 and 3  
G. S. Vissing, NRC Project Manager, Millstone Unit No. 2

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LICENSEE EVENT REPORT (LER)

Estimated burden per response to comply with this information collection request: 50.0 hrs. Forward comments regarding burden estimate to the Records and Reports Management Branch (p-530), U. S. Nuclear Regulatory Commission, Washington, DC 20555, and to the Paperwork Reduction Project (3150-0104), Office of Management and Budget, Washington, DC 20503.

FACILITY NAME (1) Millstone Nuclear Power Station Unit 2		DOCKET NUMBER (2) 0   5   0   0   0   3   3   6   1	PAGE (3) OF 0   3
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TITLE (4)  
Failure to Record Surveillance Required Data

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)												
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES												
0	9	0	9	0	1	3	0	0	1	0	0	4	9	0	0	5	0	0	0	0	0

OPERATING MODE (9) 1	THIS REPORT IS BEING SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR § (Check one or more of the following) (11)										
POWER LEVEL (10) 1   0   0	20.402(b)		20.402(c)		50.73(a)(2)(iv)		79.71(b)				
	20.405(a)(1)(i)		50.36(c)(1)		50.73(a)(2)(v)		79.71(c)				
	20.405(a)(1)(ii)		50.36(c)(2)		50.73(a)(2)(vii)		OTHER (Specify in Abstract below and in Text, NRC Form 366A)				
	20.405(a)(1)(iii)	X	50.73(a)(2)(i)		50.73(a)(2)(viii)(A)						
	20.405(a)(1)(iv)		50.73(a)(2)(ii)		50.73(a)(2)(viii)(B)						
	20.405(a)(1)(v)		50.73(a)(2)(iii)		50.73(a)(2)(ix)						
	20.405(a)(1)(vi)		50.73(a)(2)(iv)								

LICENSEE CONTACT FOR THIS LER (12)										
NAME Edwin R. Foster, Ext. 4416							TELEPHONE NUMBER AREA CODE: 2   0   3   4   4   7   -   1   7   9   1			

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC
D	I	M							

SUPPLEMENTAL REPORT EXPECTED (14)							EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR			
<input checked="" type="checkbox"/> YES (If yes, complete EXPECTED SUBMISSION DATE)								0	1	0	1	9	1

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

On September 5, 1990 at 1400 hours, while the plant was at 100% power, Mode 1, an internal audit identified two dates that Technical Specification required surveillance procedure 2602B was missed. To preclude an over-pressure condition in the steam generators, this surveillance calls for recording steam generator coolant temperatures at least once per hour when pressures in the steam generator are greater than 200 psig and Tave is less than 200 degrees. An investigation has since shown additional dates that these temperatures were not recorded, or failed to be identified as not meeting the acceptance criteria.

The root cause was procedure inadequacy and operator inattention to detail. Insufficient procedural guidance was provided to complete the surveillance.

Corrective action included a procedure change to preclude use of the plant process computer to collect data. Changes are being effected to modify the computer program, to assist the operator in recording the required data. Additionally, all procedures that could place the plant in the conditions under which Surveillance 4.7.2.1 would be required, are being changed to reference the this surveillance. Due to an out of specification condition, a determination of steam generator acceptability for continued operation is ongoing.

LICENSEE EVENT REPORT (LER)  
TEXT CONTINUATION

Estimated burden per response to comply with this information collection request: 60.0 hrs. Forward comments regarding burden estimate to the Records and Reports Management Branch (p-530), U.S. Nuclear Regulatory Commission, Washington, DC 20555, and to the Paperwork Reduction Project (3150-0104), Office of Management and Budget, Washington, DC 20503.

FACILITY NAME (1)  Millstone Nuclear Power Station Unit 2	DOCKET NUMBER (2)  0   5   0   0   0   3   3   6   9   0   -	LER NUMBER (6)		PAGE (3)	
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	
		0   1   3	-	0   0	0   2 OF 0   3

TEXT (If more space is required, use additional NRC Form 366A (6)) (17)

I. Description of Event

On September 5, 1990 at 1400 hours while the plant was at 100% power, Mode 1, an internal audit noted that, in November 1989 and June of 1990, Surveillance 2602B was missed. The surveillance requires that steam generator temperature be recorded hourly when steam generator pressures are greater than 200 psig and Tave is less than 200 degrees F. A review of Heat-up Cooldown Data Sheets since 1985, when the current computer program was put into service, has shown that this surveillance has been missed on a total of eight (8) separate dates. And, on one (1) additional date, the completed surveillance failed to identify an out of specification condition, not meeting the acceptance criteria. The dates involved were as follows:

- October 25, 1985
- December 9, 1985
- February 11 and 12, 1989
- November 18, 19, 20, and 21, 1989
- June 10, 1990

This event required no immediate operator response, nor the actuation of any safety system.

II. Cause of Event

The root cause of the missed surveillances was procedure inadequacy, in that the procedure did not give the operator sufficient guidance to complete the surveillance. The procedure lead the operator to believe that the computer program could be used to satisfy all acceptance criteria, when in fact, it only satisfied heatup/cooldown rate criteria. The acceptance criteria for steam generator temperature was not satisfied because the program lacked the necessary temperature points on the printout.

The root cause of the failure to identify an out of specification condition, on one occasion when the appropriate data was recorded, was operator inattention to detail.

III. Analysis of Event

This report is submitted pursuant to the requirements of paragraph 50.73(a)(2)(i), any operation or condition prohibited by the plant's Technical Specifications. There were no apparent safety consequences resulting from these occurrences, since the lowest temperature recorded was 66 degrees F. Although below the 70 degrees specified, this is well above the steam generator Reference Temperature for Nil Ductility Transition (RT-NDT) of 50 degrees F. As noted in the basis for this technical specification, the limitations of 70 degrees F and 200 psig are based on a steam generator RT-NDT of 50 degrees F and are sufficient to prevent brittle fracture. A preliminary review, based on the above, indicates that brittle fracture probably did not occur. An detailed review will be conducted to confirm this. Additionally, the steam generators have operated and undergone several internal inspections, since the missed surveillances or the out of specification condition existed. No evidence of brittle fracture has been identified. Based this and the preliminary review, it is not likely the plant was in an unsafe condition, nor that there was any risk to the public, during these occurrences.



LICENSEE EVENT REPORT (LER)  
TEXT CONTINUATION

Estimated burden per response to comply with this information collection request: 50.0 hrs. Forward comments regarding burden estimate to the Records and Reports Management Branch (p-530), U.S. Nuclear Regulatory Commission, Washington, DC 20555, and to the Paperwork Reduction Project (3150-0114), Office of Management and Budget, Washington, DC 20503.

FACILITY NAME (1)  Millstone Nuclear Power Station Unit 2	DOCKET NUMBER (2)  0 5 0 0 0 3 3 6 9 0	LER NUMBER (6)			PAGE (3)		
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			
			0 1 3	0 0	0 3	OF	0 3

TEXT (If more space is required, use additional NRC Form 366A's) (17)

IV. Corrective Action

The surveillance procedure has been changed to prevent the operator from using the computer printout option until a modified computer program, that incorporates the required temperature points, can be implemented. Additionally, a procedure review is to be completed to ensure that procedures, other than the Heatup/Cooldown procedure, requiring pressure to be raised above 200 psig, with Tave less than 200 degrees, refer the operator to procedure SP2602B. Currently the computer program has been modified to incorporate the required temperature points in the printout, however, because additional heatup/cooldown enhancements are in progress, the program is not ready for implementation. Until these changes are complete the operators will continue to manually log the temperatures as required by this surveillance. A detailed engineering review, to confirm no damage was done to the steam generator that experienced out of specification temperatures, will be completed prior to heat-up at the end of the current refuel outage.

An update LER will report the results of the above engineering review, as well as the completion of all procedure changes and the implementation of the modified computer program, by January 1, 1991.

V. Additional Information

There were no failed components.

There were no similar previous events.

EHS Component Code for Steam Generator: SG