

INSERVICE INSPECTION SUMMARY REPORT
FOR
MAINE YANKEE ATOMIC POWER COMPANY
AUGUSTA, MAINE

GENERATING PLANT: Maine Yankee Atomic Power Plant
Wiscasset, Maine

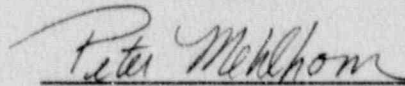
UNIT DESIGNATION NUMBER: Reactor Vessel 20865

COMMERCIAL OPERATING DATE: 12/29/72

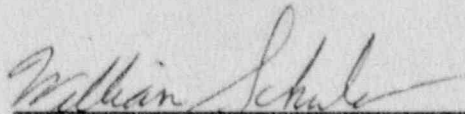
EXAMINATION DATES FROM 01/12/89 to 07/02/90 IN
THE SECOND TEN YEAR INTERVAL.

COMPLETION

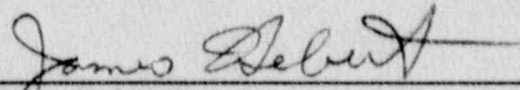
DATE: 07/02/90



Peter Mehlhorn
ISI Coordinator



William Schubert
Performance Engineering Supervisor



James Hebert
Manager, Plant Engineering

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PREFACE

This report summarizes the preservice and inservice inspections and pressure tests that were performed at Maine Yankee Atomic Power Plant between January 12, 1989 and July 2, 1990 that fall within the scope of Section XI of the ASME Boiler and Pressure Vessel Code and the Maine Yankee Inservice Inspection program. The end of the third inspection period of the second ten year inspection interval is August 19, 1992. This completes approximately 50% of the examination for the third period. The 10 year Reactor Vessel examinations have been completed and are included in this report.

The Authorized Nuclear Inservice Inspector on-site to witness the inspections was Mr. William Nicholas of Factory Mutual Engineering.

FORM NIS-1 OWNERS' DATA REPORT FOR INSERVICE INSPECTIONS

As required by the Provisions of the ASME Code Rules

1. Owner Maine Yankee Atomic Power Company, Augusta, Maine
(Name and Address of Owner)
2. Plant Maine Yankee Atomic Power Plant, Wiscasset, Maine
(Name and Address of Owner)
3. Plant Unit 1 4. Owner Certificate of Authorization (if required) DPR-36
5. Commercial Service Date 12/29/72 6. National Board for Unit Reactor Vessel 20865
7. Components Inspected

Component or Appurtenance	Manufacturer or Installer	Manufacturer or Installer Serial No.	State or Province No.	National Board No.
Reactor	C.E.	67206	NA	20865
St. Gen. #1	C.E.	67501	NA	20919
St. Gen. #2	C.E.	67502	NA	20920
St. Gen. #3	C.E.	67503	NA	20921
Pressurizer	C.E.	67601	NA	20858
Piping	Stone & Webster	NA	NA	NA

Note: Supplemental sheets in form of lists, sketches, or drawings may be used provided (1) size is 8 1/2 in. x 11 in., (2) information in Items 1 through 6 on this data report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

This form (E00029) may be obtained from the Order Dept, ASME, 345 E. 47th St., New York, N.Y. 1001

FORM NIS-1 (back)

- 8. Examination Dates 01/12/89 to 07/02/90 9. Inspection Interval from 12/29/82 to 12/29/92.
- 10. Abstract of Examinations. Include a list of examinations and a statement concerning status of work required for current interval. See pages 4 through 3 and Preface.
- 11. Abstract of Conditions Noted See pages 9 through 13.
- 12. Abstract of Corrective Measures Recommended and Taken See pages 9 through 13.

We certify that the statements made in this report are correct and the examinations and corrective measures taken conform to the rules of the ASME Code, Section XI.

Date 9-13 1990 Signed Maine Yankee By [Signature]
Owner

Certificate of Authorization No. (if applicable) N/A Expiration Date N/A

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and/or the State or Province of MAINE and employed by Fidelity Mutual Assoc. of New York NY have inspected the components described in this Owners' Data Report during the period 01/12/89 to 07/02/90, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owners' Data Report in accordance with the requirements of the ASME Code, XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measure described in this Owners' Data Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any person injury or property damage or a loss of any kind arising from or connected with this inspection.

Date 9-13 1990

[Signature] Commissions Me 446
Inspector's Signature National Board, State, Province and No.

1989-1990
INSERVICE INSPECTIONS

1. OWNER: Maine Yankee Atomic Power Company, Augusta, Maine
 2. PLANT: Maine Yankee Atomic Power Plant, Wiscasset, Maine
 3. PLANT UNIT: 1 4. OWNER CERTIFICATE OF AUTHORIZATION: DPR-36
 5. COMMERCIAL SERVICE DATE: 12/29/72
 6. NATIONAL BOARD NUMBER FOR UNIT: Reactor Vessel 20865

10. ABSTRACT OF EXAMINATIONS:

<u>CATEGORY</u>	<u>ITEM</u>	<u>NUMBER EXAMINED</u>	<u>METHOD</u>
B-A	B1.11	1-Upper Shell-to-Int. Shell Weld	UT
	B1.12	1-Longitudinal Weld @ 90° Lower Shell	UT
	B1.21	1-Lower Head Circumferential Weld	UT
	B1.22	1-Lower Head Meridional Weld	UT
	B1.30	1-Flange-to-Upper Shell Weld -50% from Flange Face	UT UT
	B1.40	-50% Closure Head-to-Flange Weld	UT, MT
B-B	B2.11	1-Upper Shell-to-Upper Head Weld	UT
	B2.12	2-Upper Shell Longitudinal Welds (12 inches of each)	UT
B-D	B3.90	6-Nozzle-to-Vessel Welds	UT
	B3.100	6-Nozzle Inside Radius Sections	UT
B-E	B4.20	Heater Penetrations	VT-2
B-F	B5.21	1-3", Nozzle-to-Safe End Weld	PT
	B5.50	1-33 1/2", Dissimilar Metal Pipe Weld	UT*
		1-33 1/2", Dissimilar Metal Pipe Weld	PT
		1-14", Dissimilar Metal Pipe Weld	UT*
		1-12", Dissimilar Metal Pipe Weld	UT*, PT
	*NOTE:	UT from carbon steel side only	
	B5.51	2-3", Dissimilar Metal Pipe Welds	PT
		1-2", Dissimilar Metal Pipe Welds	PT
B-G-1	B6.30	27-Reactor Vessel Closure Head Studs (removed)	UT, MT
B-G-2	B7.30	120-Primary Manway Bolting, Steam Generators 1, 2, & 3	MT, VT-1
	B7.50	1-Pipe Flange Bolting	VT-1
	B7.70	7-Body-to-Bonnet Valve Bolting (in place)	VT-1
		3-Body-to-Bonnet Valve Bolting (removed)	MT, VT-1

1989-1990
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10. ABSTRACT OF EXAMINATIONS:

<u>CATEGORY</u>	<u>ITEM</u>	<u>NUMBER EXAMINED</u>	<u>METHOD</u>	
B-J	B9.11	5-33 1/2", Circumferential Pipe Welds	UT, MT	
		1-14", Circumferential Pipe Weld	UT, PT	
		2-10" Circumferential Pipe Welds	UT, PT	
		1-8", Circumferential Pipe Weld	RT, PT	
		2-4", Circumferential Pipe Welds	UT, PT	
		B9.21	5-3", Circumferential Pipe Welds	PT
		B9.31	1-8", Branch Pipe Connection Weld	RT, PT
		B9.32	1-2", Branch Pipe Connection Weld	PT
		B9.40	11-2", Socket Welds	PT
		B-K-1	B10.10	5-Integral Attachments
B-M-1	B12.31	1-Valve Body Weld	RT	
B-M-2	B12.40	2-Valve Body internal	VT-3	
B-N-1	B13.10	Reactor Vessel	VT-3	
B-N-3	B13.30	Reactor Vessel Internals	VT-3	

COMMITTED REACTOR VESSEL THERMAL SHIELD INSPECTION

- References: 1. Maine Yankee letter MN-84-89 to the NRC dated May 17, 1984.
2. Maine Yankee letter MN-88-37 to the NRC dated April 5, 1988.

Maine Yankee conducted an underwater visual examination of the Maine Yankee reactor vessel thermal shield during the 1990 refueling outage as committed in the above references. The inspection included all of the twenty-six (26) positioning pins and nine (9) vertical support lugs as well as a general inspection of the thermal shield.

The thermal shield and its principle support structure components, including the three top replacement positioning pins installed in 1984, appeared to be undamaged and in satisfactory condition. The staking mechanism used to resecure the positioning pins that were tightened in 1984 failed. This allowed a number of the previously tightened lower positioning pins to back out. All of the lower positioning pins tightened in 1984 were retightened and restaked with an enhanced staking pin design.

1989-1990
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10. ABSTRACT OF EXAMINATIONS:

<u>CATEGORY</u>	<u>ITEM</u>	<u>NUMBER EXAMINED</u>	<u>METHOD</u>
B-P	B15.00	Reactor Coolant System Leak Test	VT-2
	B15.71	1-2" Valve Weld	VT-2
B-Q	B16.00	Steam Generator #3 Tubing as required by Maine Yankee Tech. Spec. 4.10	ECT
		-514 Tubes, First Sample	
		-1029 Tubes, Second Sample	

ADDITIONAL STEAM GENERATOR EDDY CURRENT EXAMS

Maine Yankee experienced some activity in secondary side samples just prior to the refueling outage. Therefore, during the outage each steam generator's tube bundle was leak tested, which indicated 3 suspect leaking tubes. As a result of this, Maine Yankee examined 100% of all accessible tube ends (34,047). The extent of exam was from approximately 2 inches above the tube sheet to 2 inches below the top of the tube sheet. All circumferential indications were also inspected using ultrasonic techniques. See pages 11 and 12 for results and corrective measures.

F-A,B,&C	F-1,2,3&4	Safety Class 1 Supports	
		32 Supports	VT-3
		8 Supports	VT-3&4
		2 Additional exams as required by IWF-2430.*	VT-3
		1 Additional exam as required by IWF-2430.*	VT-3&4
		*See Page 13 for details.	

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10. ABSTRACT OF EXAMINATIONS:

<u>CATEGORY</u>	<u>ITEM</u>	<u>NUMBER EXAMINED</u>	<u>METHOD</u>
C-A	C1.10	2-Shell Circumferential Welds Steam Generator #1	UT
C-B	C2.21	1-Nozzle-to-Shell Weld Steam Generator #2	UT, MT
		2-Nozzle-to-Shell Welds, Residual Heat Removal Heat Exchanger	UT, PT
	C2.22	1-Nozzle Inside Radius Section Steam Generator #2	UT
C-C	C3.10	2-Integral Attachment Welds, Steam Generator #2	MT
C-F	C5.11	3-14", Circumferential Pipe Welds	PT
		5-12", Circumferential Pipe Welds	PT
		14-10", Circumferential Pipe Welds	PT
		8-6", Circumferential Pipe Welds	PT
		1-6", Circumferential Pipe Welds	MT
	C5.21	2-14", Circumferential Pipe Welds	RT, MT
		5-14", Circumferential Pipe Welds	UT, MT
		1-10", Circumferential Pipe Weld	UT, PT
C-H	C7.00	Hydrostatic Pressure Tests	VT-2
	a.	Charging System	
	b.	High Pressure Safety Injection, 2 Hydros	
	c.	Containment Spray, 2 Hydros	
	d.	Nitrogen System	
	e.	Steam Generator Blowdown, 2 Hydros	
	f.	Residual Heat Removal	
	g.	Primary Component Cooling, 4 Hydros	
	h.	Main Steam	
	i.	Low Pressure Safety Injection	
	j.	Primary Water	

1989-1990
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10. ABSTRACT OF EXAMINATIONS:

<u>CATEGORY</u>	<u>ITEM</u>	<u>NUMBER EXAMINED</u>	<u>METHOD</u>
F-A,B,&C	F-1,2,3&4	Safety Class 2 Supports	
		25 Supports	VT-3
		8 Supports	VT-3&4
		1 Additional exam as required by IWF-2430.*	VT-3
		3 Additional exams as required by IWF-2430.*	VT-3&4

*See Page 13 for details.

1989-1990
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11. ABSTRACT OF CONDITIONS NOTED:
12. ABSTRACT OF CORRECTIVE MEASURES RECOMMENDED AND TAKEN:

<u>CATEGORY</u>	<u>CONDITION</u>				<u>CORRECTIVE MEASURE</u>
B-Q	Steam Generator #3 Tubing				
	<u>Row</u>	<u>Line</u>	<u>% TWD</u>	<u>LOCATION</u>	
Defective Tubes	23	36	62	6.3" Above Cold Tube Sheet	Plugged Tube
	35	36	67	6.4" Above Cold Tube Sheet	Plugged Tube
	37	36	65	6.2" Above Cold Tube Sheet	Plugged Tube
	41	36	47	6.2" Above Cold Tube Sheet	Plugged Tube
					Additional exams were conducted per Maine Yankee Tech Spec. 4.10.
Degraded Tubes	1	118	31	0.7" Above Hot Tube Sheet	Additional exams not required for less than 40% through wall degradation (TWD).
	23	22	38	4.1" Above Hot Tube Sheet	
	24	37	24	5.6" Above Cold Tube Sheet	
	24	37	21	4.6" Above Cold Tube Sheet	
	30	113	21	6.1" Above Hot Tube Sheet	
	32	29	30	7.1" Above Hot Tube Sheet	
	32	103	23	9.7" Above Hot Tube Sheet	
	33	32	31	11.8" Above Hot Tube Sheet	
	35	48	22	11.3" Above Cold Tube Sheet	
	36	37	37	6.5" Above Cold Tube Sheet	
	39	36	34	6.2" Above Cold Tube Sheet	
	45	38	38	7.0" Above Cold Tube Sheet	
	47	50	36	12.2" Above Cold Tube Sheet	
	48	51	30	12.0" Above Cold Tube Sheet	
	48	51	38	11.7" Above Cold Tube Sheet	

1989-1990
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11. ABSTRACT OF CONDITIONS NOTED:
12. ABSTRACT OF CORRECTIVE MEASURES RECOMMENDED AND TAKEN:

CATEGORY	CONDITION			CORRECTIVE MEASURE
	Row	Line	% TWD	
Degraded Tuber (Cont'd)	49	26	27	1.0" Above Hot Tube Sheet
	53	88	30	11.7" Above Cold Tube Sheet
	83	48	30	1.1" Above Hot Tube Sheet
	91	60	22	1.1" Above Hot Tube Sheet
	92	57	32	0.9" Above Hot Tube Sheet
Imperfect tubes	7	106	<20	@ Third Vertical Support
	8	103	<20	@ Third Vertical Support
	20	41	<20	0.6" Above Hot Tube Sheet
	21	22	<20	3.9" Above Hot Tube Sheet
	21	40	<20	0.6" Above Hot Tube Sheet
	22	37	<20	5.9" Above Cold Tube Sheet
	25	22	<20	2.0" Above Hot Tube Sheet
	32	115	<20	5.2" Above Cold Tube Sheet
	38	37	<20	6.2" Above Cold Tube Sheet
	40	37	<20	6.1" Above Cold Tube Sheet
	43	38	<20	6.9" Above First Cold Leg Support
	43	118	<20	0.7" Above Hot Tube Sheet
	44	37	<20	6.7" Above Cold Tube Sheet
	47	26	<20	1.5" Above Hot Tube Sheet
	52	105	<20	12.2" Above Hot Tube Sheet
	53	28	<20	@ Diagonal Brace Hot Leg
	62	81	<20	5.1" Above Second Cold Leg Support
68	71	<20	9.8" Above Cold Tube Sheet	
91	64	<20	1.0" Above Hot Tube Sheet	
92	59	<20	0.9" Above Hot Tube Sheet	

1989-1990
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 6. NATIONAL BOARD NUMBER FOR UNIT: Reactor Vessel 20865
 11. ABSTRACT OF CONDITIONS NOTED:
 12. ABSTRACT OF CORRECTIVE MEASURES RECOMMENDED AND TAKEN:

ADDITIONAL STEAM GENERATOR EXAMINATION RESULTS:

<u>CATEGORY</u>	<u>CONDITION</u>	<u>CORRECTIVE MEASURE</u>
STEAM GENERATOR #1		
<u>Row</u>	<u>Line</u>	<u>% T/W</u>
		<u>LOCATION</u>
8	95	56
		0.9" Above Hot Tube Sheet
		Plugged Tube
20	135	Crack
		@ Hot Tube Sheet
		Plugged & Staked
45	122	Crack
		0.2" Above Hot Tube Sheet
		Plugged & Staked
52	65	Crack
		@ Hot Tube Sheet
		Plugged & Staked
69	122	Crack
		@ Hot Tube Sheet
		Plugged & Staked
75	26	Crack
		@ Hot Tube Sheet
		Plugged & Staked
75	34	Crack
		@ Hot Tube Sheet
		Plugged & Removed
75	118	Crack
		0.1" Above Hot Tube Sheet
		Plugged & Staked
78	115	Crack
		0.1" Above Hot Tube Sheet
		Plugged & Staked
80	111	Crack
		@ Hot Tube Sheet
		Plugged & Staked
82	33	Crack
		0.1" Above Hot Tube Sheet
		Plugged & Staked
82	111	Crack
		0.1" Above Hot Tube Sheet
		Plugged & Staked
82	113	Crack
		0.1" Above Hot Tube Sheet
		Plugged & Staked
85		Crack
		@ Hot Tube Sheet
		Plugged & Staked
89	46	Crack
		@ Hot Tube Sheet
		Plugged & Staked
91	38	Crack
		0.1" Above Hot Tube Sheet
		Plugged & Staked
96	51	Crack
		@ Hot Tube Sheet
		Plugged & Staked
97	100	Crack
		@ Hot Tube Sheet
		Plugged & Staked
101	82	Crack
		0.1" Above Hot Tube Sheet
		Plugged & Staked
101	86	Crack
		0.2" Above Hot Tube Sheet
		Plugged & Staked
103	86	Crack
		0.1" Above Hot Tube Sheet
		Plugged & Staked
108	69	Crack
		0.1" Above Hot Tube Sheet
		Plugged & Staked
108	73	Crack
		0.2" Above Hot Tube Sheet
		Plugged & Staked

1989-1990
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11. ABSTRACT OF CONDITIONS NOTED:
12. ABSTRACT OF CORRECTIVE MEASURES RECOMMENDED AND TAKEN:

ADDITIONAL STEAM GENERATOR EXAMINATION RESULTS (Cont'd)

<u>CATEGORY</u>	<u>CONDITION</u>			<u>CORRECTIVE MEASURE</u>
	STEAM GENERATOR #1 (Cont'd)			
	<u>Row</u>	<u>Line</u>	<u>% TWD</u>	<u>LOCATION</u>
	108	77	Crack	0.1" Above Hot Tube Sheet Plugged & Staked
	108	79	Crack	0.2" Above Hot Tube Sheet Plugged & Staked
	STEAM GENERATOR #2			
	35	126	Crack	0.2" Above Hot Tube Sheet Plugged & Staked
	40	107	51	0.6" Above Hot Tube Sheet Plugged
	42	103	Crack	0.1" Above Hot Tube Sheet Plugged & Staked
	42	107	Crack	0.1" Above Hot Tube Sheet Plugged & Staked
	STEAM GENERATOR #3			
	14	31	Crack	@ Second Hot Leg Support Plugged
	33	28	Crack	0.1" Above Cold Tube Sheet Plugged & Staked
	47	38	Dent	0.4" Above First Hot Leg Support Plugged
	49	96	Crack	@ Hot Tube Sheet Plugged & Staked
	54	97	57	0.1" Above Hot Tube Sheet Plugged
	55	50	47	0.3" Above Hot Tube Sheet Plugged
	60	97	Crack	0.1" Above Hot Tube Sheet Plugged & Staked
	67	46	Crack	@ Hot Tube Sheet Plugged & Staked
	112	71	Crack	@ Hot Tube Sheet Plugged & Staked
	100	37	Blocked	----- Plugged

1989-1990
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<u>CATEGORY</u>	<u>CONDITION</u>	<u>CORRECTIVE MEASURE</u>
	<u>SAFETY CLASS 1 SUPPORTS</u>	
F-A, B, & C	One support for a 2" loop drain header was found with a loose bolt on the pipe clamp.	The support was repaired and reinspected. Three (3) additional supports were examined in accordance with IWF-2430.
	<u>SAFETY CLASS 2 SUPPORTS</u>	
F-A, B, & C	One support on a 10" residual heat removal line was found with a loose bolt on the pipe clamp.	The support was repaired and reinspected. Four (4) additional supports were examined in accordance with IWF-2430.

**FORM NIS-2 OWNER'S REPORT OF REPAIR OR REPLACEMENT
As Required by the Provisions of ASME Code Section XI**

1. Owner Maine Yankee Atomic Power Company Date 1990 Refueling
(Name)
Augusta, Maine Sheet 1 of 14
(Address)
2. Plant Maine Yankee Atomic Power Plant Unit Reactor Vessel 20865
(Name)
Wiscasset, Maine
(Address)
3. Work Performed by See pages 15 through 27. See pages 15 through 27.
(Name) Repair Organization P.O. No., Job No., etc.
4. Identification of System See pages 15 through 27.
5. (a) Applicable Construction Code B31.1 1977 Edition, _____ Addenda, Code Class _____
(b) Applicable Edition of Section XI Utilized for Repairs or Replacements - 1980, W80 Addenda, Code Class _____
6. Identification of Components Repaired or Replaced, and Replacement Components.

Name of Components	Name of Mfr.	Mfrs. Ser. No.	Nat'l Bd. No.	CRN No.	Other Identification	Year Built	Repaired or Replacement	ASME Code Stamped (yes or no)
See pages	15-27							

7. Description of Work See pages 15 through 27.
8. Tests Conducted: Hydrostatic Pneumatic Nominal Operating Pressure
Other Pressure _____ psi Test Temp. _____ °F.
9. Remarks _____
(Applicable Manufacturer's Data Reports to be attached)

CERTIFICATE OF COMPLIANCE

We certify that the statements made in this report are correct and this repair & replacement conforms to Section XI of the ASME Code.

Signed Walter M. McKim ISI COORDINATOR 9-13 90
(Owner or Owner's Designee) Title (Date)

CERTIFICATE OF INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of Maine, employed by Factory of Mutual Eng. have inspected the items described in this Report on 1-2, 1990 and state that to the best of my knowledge and belief, this repair or replacement has been constructed in accordance with Section XI of the ASME Code. By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the repair or replacement described in this Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date 4-13-90 William S. McKim Commissions Me 446
(Inspector) (State or Province, Nat'l Bd)

Note: Supplemental sheets in form of lists, sketches, or drawings may be used provided (1) size is 8 1/2 in. x 11 in., (2) information in items 1 through 4 on this data report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

1989 - 1990
INSERVICE INSPECTIONS

1. OWNER: Maine Yankee Atomic Power Company, DATE: 01/20/89
Augusta, Maine SHEET: 2 of 14

2. PLANT: Maine Yankee Atomic Power Plant, UNIT: Reactor Vessel 20865
Wiscasset, Maine

Job No. and Title: DR/RO 6685-88; Charging Pump Oil Cooler Pipe Repair

System: Primary Component Cooling (PCC)

Safety Class: 3

Work Performed by: Maine Yankee

Description: A 1 inch union was replaced due to excessive leakage.

Section XI Preservice NDE: None required. New welds were inspected per the construction code and Yankee Specifications.

Pressure Test: The new welds were hydrostatically tested to at least 1.10 times the design pressure of 150 psi at a temperature greater than 40°F.

1989 - 1990
INSERVICE INSPECTIONS

1. OWNER: Maine Yankee Atomic Power Company, DATE: 10/11/89
Augusta, Maine SHEET: 3 of 14

2. PLANT: Maine Yankee Atomic Power Plant, UNIT: Reactor Vessel 20865
Wiscasset, Maine

Job No. and Title: DR/RO 2216-89; EFW Recirc Line Repair

System: Emergency Feedwater (EFW)

Safety Class: 3

Work Performed by: Cianbro Corporation, Pittsfield, Maine

Description: Weld repairs were made to degraded EFW underground piping. The leaks resulted from external pitting at gaps in the protective covering on the piping.

Section XI Preservice NDE: None required. New welds were inspected per the construction code and Yankee Specifications.

Pressure Test: The new welds were hydrostatically tested to at least 1.10 times the safety valve setting of 75 psi at a temperature greater than 40°F.

1989 - 1990
INSERVICE INSPECTIONS

1. OWNER: Maine Yankee Atomic Power Company, DATE: 1990 Refueling
Augusta, Maine SHEET: 4 of 14

2. PLANT: Maine Yankee Atomic Power Plant, UNIT: Reactor Vessel 20865
Wiscasset, Maine

Job No. and Title: DR/RO 1962-90; Replacement of Valve SIA-340

System: Safety Inj (SIA)

Safety Class: 1

Work Performed by: Anchor Darling Valve Co., Williamsport, PA

Description: A 2 inch valve was replaced due to excessive seat leakage.

Section XI Preservice NDE: Liquid penetrant exams were performed on the new welds as well as inspections per the construction code and Yankee General Specifications.

Pressure Test: The new welds were hydrostatically tested to at least 1.25 times the design pressure of 2485 psi at a temperature greater than 40°F.

1989 - 1990
INSERVICE INSPECTIONS

1. OWNER: Maine Yankee Atomic Power Company, DATE: 1990 Refueling
Augusta, Maine SHEET: 5 of 14

2. PLANT: Maine Yankee Atomic Power Plant, UNIT: Reactor Vessel 20865
Wiscasset, Maine

Job No. and Title: DR/RO 1954-90; Replacement of Valve MS-69

System: Main Steam (MS)

Safety Class: 2

Work Performed by: Anchor Darling Valve Co., Williamsport, PA

Description: One 2 inch valve was replaced because it was difficult to stroke.

Section XI Preservice NDE: None required. New welds were inspected per the construction code and Yankee General Specifications.

Pressure Test: The downstream side of the valve was hydrostatically tested to at least 1.25 times the design pressure of 950 psi at a temperature greater than 40°F.

The upstream side of the valve received an inservice leak test in lieu of a hydrostatic test because the subject weld could not be isolated from the Steam Generator. See IWA-5214 (d), IWA-2240 and Code Case N 416.

1989 - 1990
INSERVICE INSPECTIONS

1. OWNER: Maine Yankee Atomic Power Company, DATE: 1990 Refueling
Augusta, Maine SHEET: 6 of 14
2. PLANT: Maine Yankee Atomic Power Plant, UNIT: Reactor Vessel 20865
Wiscasset, Maine

Job No. and Title: DR/RO 3599-89; Replacement of Valve CS-54

System: Containment Spray (CS)

Safety Class: 2

Work Performed by: Anchor Darling Valve Co., Williamsport, PA

Description: One 2 inch valve was replaced due to excessive seat leakage.

Section XI Preservice NDE: None required. New welds were inspected per the construction code and Yankee General Specification..

Pressure Test: The new welds were pneumatically tested to at least 1.25 times the design pressure of 70 psi at a temperature greater than 40°F.

1989 - 1990
INSERVICE INSPECTIONS

1. OWNER: Maine Yankee Atomic Power Company, DATE: 1990 Refueling
Augusta, Maine SHEET: 7 of 14
2. PLANT: Maine Yankee Atomic Power Plant, UNIT: Reactor Vessel 20865
Wiscasset, Maine

Job No. and Title: DR/RO 6979-86; Replacement of Valve BD-148

System: Blowdown (BD)

Safety Class: 2

Work Performed by: Anchor Darling Valve Co., Williamsport, PA

Description: One 1 1/2 inch valve was replaced due to excessive seat leakage.

Section XI Preservice NDE: None required. New welds were inspected per the construction code and Yankee General Specifications.

Pressure Test: The new welds were hydrostatically tested to at least 1.25 times the design pressure of 950 psi at a temperature greater than 40°F.

1989 - 1990
INSERVICE INSPECTIONS

1. OWNER: Maine Yankee Atomic Power Company, DATE: 1990 Refueling
 Augusta, Maine SHEET: 8 of 14

2. PLANT: Maine Yankee Atomic Power Plant, UNIT: Reactor Vessel 20865
 Wiscasset, Maine

Job No. and Title: DR/RO 3597-89; Replacement of Valve PW-80.

System: Primary Water (PW)

Safety Class: 2 and NNS (see IWA 1300 [f])

Work Performed by: Anchor Darling Valve Co., Williamsport, PA

Description: A 2 inch valve was replaced due to excessive seat leakage.

Section XI Preservice NDE: None required. New welds were inspected per the construction code and Yankee General Specifications.

Pressure Test: The new welds were pneumatically tested to at least 1.10 times the design pressure of 150 psi at a temperature greater than 40°F.

1989 - 1990
INSERVICE INSPECTIONS

1. OWNER: Maine Yankee Atomic Power Company, DATE: 1990 Refueling
Augusta, Maine SHEET: 9 of 14

2. PLANT: Maine Yankee Atomic Power Plant, UNIT: Reactor Vessel 20865
Wiscasset, Maine

Job No. and Title: EDCR 89-53-4; Appendix J - Modification to the Nitrogen System

System: Nitrogen (N)

Safety Class: 2 and NNS (see IWA-1300 [f])

Work Performed by: Cianbro Corporation, Pittsfield, ME

Description: A 2 inch check valve (N-29) was replaced with softseat check valve due to repetitive seat leakage problems.

Section XI Preservice NDE: None required. New welds were inspected per the construction code and Yankee General Specifications.

Pressure Test: The new welds were pneumatically tested to at least 1.10 times the design pressure of 250 psi at a temperature greater than 40°F.

1989 - 1990
INSERVICE INSPECTIONS

1. OWNER: Maine Yankee Atomic Power Company, DATE: 1990 Refueling
Augusta, Maine SHEET: 10 of 14
2. PLANT: Maine Yankee Atomic Power Plant, UNIT: Reactor Vessel 20865
Wiscasset, Maine

Job No. and Title: DR/RO 1428-90; Cleaning of AC-1A Tubes

System: Primary Component Cooling (PCC)

Safety Class: 3

Work Performed by: Maine Yankee

Description: In order to access the tubes of AC-1A, a PCC line had to be cut. The pipe was subsequently re-welded.

Section XI Preservice NDE: None required. New welds were inspected per the construction code and Yankee General Specifications.

Pressure Test: The new welds were hydrostatically tested to at least 1.10 times the design pressure of 150 psig at a temperature greater than 40°F.

1989 - 1990
INSERVICE INSPECTIONS

1. OWNER: Maine Yankee Atomic Power Company, DATE: 1990 Refueling
Augusta, Maine SHEFT: 11 of 14

2. PLANT: Maine Yankee Atomic Power Plant, UNIT: Reactor Vessel 20865
Wiscasset, Maine

Job No. and Title: DR/RO 2626-90; Cleaning of AC-1B tubes

System: Secondary Component Cooling (SCC)

Safety Class: 3

Work Performed by: Maine Yankee

Description: In order to access the tubes of AC-1B, a SCC line had to be cut. The pipe was subsequently re-welded.

Section XI Preservice NDE: None required. New welds were inspected per the construction code and Yankee General Specifications.

Pressure Test: The new welds were hydrostatically tested to at least 1.10 times the design pressure of 150 psi at a temperature greater than 40°F.

1989 - 1990
INSERVICE INSPECTIONS

1. OWNER: Maine Yankee Atomic Power Company, DATE: 1990 Refueling
Augusta, Maine SHEET: 12 of 14

2. PLANT: Maine Yankee Atomic Power Plant, UNIT: Reactor Vessel 20865
Wiscasset, Maine

Job No. and Title: DR/RO 2209-90; Replacement of Valve DG-14.

System: Diesel Generator Starting Air (DG)

Safety Class: 3

Work Performed by: Maine Yankee

Description: One 1 1/2 inch valve was replaced because it was difficult to stroke.

Section XI Preservice NDE: None required. New welds were inspected per the construction code and Yankee General Specifications.

Pressure Test: The new welds were pneumatically tested to at least 1.25 times the safety valve setting of 220 psi at a temperature greater than 40°F.

1989 - 1990
INSERVICE INSPECTIONS

1. OWNER: Maine Yankee Atomic Power Company, DATE: 1990 Refueling
Augusta, Maine SHEET: 13 of 14

2. PLANT: Maine Yankee Atomic Power Plant, UNIT: Reactor Vessel 20865
Wiscasset, Maine

Job No. and Title: EDCR 89-53-2; Appendix J - Primary Vent System Modification -
Penetration No. 24

System: Primary Vent (PV)

Safety Class: 3

Work Performed by: Cianbro Corporation, Pittsfield, ME

Description: A 2 inch valve (PV-24) was installed to permit testing of the ~~Building~~ Building in accordance with 10CFR50, Appendix J.

Section XI Preservice NDE: None required. New welds were inspected per the construction code and Yankee General Specifications.

Pressure Test: The upstream side of the valve was pneumatically tested to at least 1.25 times the design pressure of 150 psi at a temperature greater than 40°F.

The downstream side of the valve is open to the atmosphere, therefore, flow through the valve was verified; see IWD-5223 (d).

1989 - 1990
INSERVICE INSPECTIONS

1. OWNER: Maine Yankee Atomic Power Company, DATE: 1990 Refueling
Augusta, Maine SHEET: 14 of 14
2. PLANT: Maine Yankee Atomic Power Plant, UNIT: Reactor Vessel 20865
Wiscasset, Maine

Job No. and Title: EDCR 89-53-1; Appendix J - Primary Drain System Modification - Penetration No. 39.

System: Quench Tank (PR)

Safety Class: 3

Work Performed by: Cianbro Corporation, Pittsfield, ME

Description: A 2 inch valve (PR-93) was installed to permit testing of the Containment Building in accordance with 10CFR50, Appendix J.

Section XI Preservice NDE: None required. New welds were inspected per the construction code and Yankee General Specifications.

Pressure Test: The upstream side of the valve was pneumatically tested to at least 1.25 times the design pressure of 150 psi at a temperature greater than 40°F.

The downstream side of the valve received an inservice leak test in lieu of a hydrostatic test because the weld could not be isolated from a storage tank and the inservice pressure is greater than the tank's static pressure. See IWA-2240.