In Reply Refer To: Docket: 50-382/90-14

Entergy Operations, Inc.
ATTN: Ross P. Barkhurst, Vice President
Operations, Waterford
P.O. Box B
Fillona, Louisiana 70066

Gentlemen:

Thank you for your letter of August 20, 1990, in response to our letter and Notice of Violation dated July 19, 1990. We have reviewed your reply and find it responsive to the concerns raised in our Notice of Violation. We will review the implementation of your corrective actions during a future inspection to determine that full compliance has been achieved and will be maintained.

Sincerely,

Division of Reactor Projects

cc:

Entergy Operations, Inc.

ATTN: Donald C. Hintz, Executive Vice

President & Chief Operating Officer

P.O. Box 31995

Jackson, Mississippi 39286

Entergy Operations, Inc.

ATTN: Gerald W. Muench, Vice President

Operations Support

P.O. Box 31995

Jackson, Mississippi 39286

Wise, Carter, Child & Caraway ATTN: Robert B. McGehee, Esq.

P.O. Box 651

Jackson, Mississippi 39205

RIV: TOBEL C:TPS

MEMurphy/ WCSeidle 9/7/90 9/7/90

C:TPS W D:DRS WCSeidle LJCall

D:DRS fann Locallan 1/0/90 De DRP SJCollins 9/10/90 Entergy Operations, Inc.
ATTN: J. R. McGaha, Jr., General
Manager Plant Operations
P.O. Box B
Killona, Louisiana 70066

Entergy Operations, Inc.
ATTN: J. G. Dewease, Senior Vice
President, Planning & Assurance
P.O. Box 31995
Jackson, Mississippi 39286-1995

Entergy Operations, Inc.
ATTN: L. W. Laughlin, Site
Licensing Support Supervisor
P.O. Box B
Killona, Louisiana 70066

Monroe & Leman ATTN: W. Malcolm Stevenson, Esq. 201 St. Charles Avenue, Suite 3300 New Orleans, Louisiana 70170-3300

Shaw, Pittman, Potts & Trowbridge ATTN: Mr. E. Blake 2300 N Street, NW Washington, D.C. 20037

Chairman Louisiana Public Service Commission One American Place, Suite 1630 Baton Rouge, Louisiana 70825-1697

Entergy Operations, Inc. ATTN: R. F. Burski, Director Nuclear Safety 317 Baronne Street New Orleans, Louisiana 70112

Department of Environmental Quality ATTN: William H. Spell, Administrator Radiation Protection Division P.O. Box 14690 Baton Rouge, Louisiana 70898

President, Parish Council St. Charles Parish Hahnville, Louisiana 70057 Entergy Operations, Inc.

Mr. William A. Cross Bethesda Licensing Office 3 Metro Center Suite 610 Bethesda, Maryland 20814

U.S. Nuclear Regulatory Commission ATTN: Resident Inspector P.O. Box 822 Killona, Louisiana 70066

U.S. Nuclear Regulatory Commission ATTN: Regional Administrator, Region IV 611 Ryan Plaza Drive, Suite 1000 Arlington, Texas 76011

bcc to DMB (IEO1)

bcc distrib. by RIV:

R. D. Martin
Section Chief (DRP/A)
DRSS-FRPS
Project Engineer (DRP/A)
RIV File
M. E. Murphy
A. Singh
D. Wigginton, NRR Project Manager (MS: 13-D-18)
Lisa Shea, RM/ALF



Raymond F. Burski

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August 20, 1990

U.S. Nuclear Regulatory Commission ATTN: Document Control Desk Washington, D.C. 20555

Subject:

Waterford 3 SES Docket No. 50-382 License No. NPF-38

NRC Inspection Report 90-14 Reply to Notice of Violation

Gentlemen:

In accordance with 10CFR2.201, Entergy Operations, Inc. hereby submits in Attachment 1 the response to the violation identified in Appendix A of the subject Inspection Report.

If you have any questions concerning this response, please contact L.W. Laughlin at (504) 739-6726.

Very truly yours,

R. F Bush

RFB/BRL/ssf Attachment

Messrs. R.D. Martin, NRC Region IV

D.L. Wigginton, NRC-NRR

E.L. Blake

W.M. Stevenson

R.B. McGehee

NRC Resident Inspectors Office

IC-90-254 (4p)

ATTACHMENT 1

ENTERGY OPERATIONS, INC. RESPONSE TO THE VIOLATION IDENTIFIED IN APPENDIX A OF INSPECTION REPORT 90-14

VIOLATION NO. 9014-01

Failure to Comply with Technical Specification Requirements while Performing the Local Leak Rate Tests for Containment Penetrations

Paragraph 4.6.1.2.d of Waterford Steam Electric Station, Unit 3 (W3) Technical Specifications requires that Types B and C local leak rate tests be conducted with the test pressure (Pa) at 44 psig.

Contrary to the above, the licensee conducted Types B and C local leak rate tests involving a number of containment penetrations, during the last three refueling outages, using a test method which allowed test pressure to drop below 44 psig during the test performance.

This is a Severity Level IV violation.

RESPONSE

4. 1 ...

(1) Reason for the Violation

Entergy Operations, Inc. admits this violation and as stated in Licensee Event Report number LER-90-007-00, believes that the root cause was an inad quate test procedure governing the Local Leak Rate Testing (LLRT) of containment isolation boundaries as required by the Waterford 3 Technical Specifications.

The NRC conducted an inspection during the period of June 25-29, 1990, which included a review of W3 containment local leak rate testing data. During this review, it was noted that 31 containment penetrations were local leak rate tested at test pressures less than the test pressure specified by W3 Technical Specifications.

W3 Technical Specification 4.6.1.2 requires that containment leakage rates shall be determined in accordance with the criteria specified in Appendix J of 10 CFR Part 50, and that Type B and C tests shall be conducted with gas at a pressure at Pa, 44 psig. Pa is the calculated peak containment internal pressure related to the leakage associated with the design basis accident. Type B and C tests detect local leaks and measure the leakage across pressure containing boundaries. Type B test penetrations are containment boundaries other than valves and type C test penetrations are containment isolation valves.

The containment electrical penetrations were tested in accordance with W3 Surveillance Procedure OP-903-114, Local Leak Rate Testing (LLRT), which utilizes the pressure decay method. The pressure decay method involves pressurizing the test volume and measures the change in pressure over a period of time to calculate the leakage rate. Section IWV-3423e of ASME Section XI allows the use of a correction factor, to correct for the decay in pressure, and normalize test results to the required test pressure. However, Procedure OP-903-114 did not include provisions for the use of a correction factor when the test pressure dropped below 44 psig. In those tests where pressure decay resulted in a test pressure less than 44 psig, the results (local leak rate) were less conservative than they would have been if a correction factor had been used.

(2) Corrective Steps That Have Been Taken and the Results Achieved

Immediate corrective actions included a review of LLRT activities conducted during the last three refueling outages. A total of 37 penetrations (31 penetrations in Refuel III were identified in the inspection, in addition to 3 penetrations each from both Refuel I and Refuel II) were identified as having been tested with test pressures less than 44 psig, without the appropriate correction factor applied to the test results. A correction factor was applied for each of the deficient penetration tests using a maximum correction factor based on paragraph IWV-3423(e) of ASME Section XI. The calculations resulted in leakage rate increases of 6.7 sccm (standard cubic centimeters per minute) for Refuel I, 2.3 sccm for Refuel II, and 24.7 sccm for Refuel Respective combined total leakage rates were calculated to be 36,275 sccm for Refuel I, 102,733 sccm for Refuel II, and 51,188 sccm for Refuel III. The combined total leakage rates for each refuel's testing were below the W3 Technical Specification 3.6.1.2 allowable of 630,697 sccm. Additionally, the adjusted leakage rate for each individual penetration was found to be acceptable and below given limits. Potentially Reportable Event number 90-33 and Licensee Event Report number LER-90-007-00 were issued in response to this concern.

In addition, a review of LLRT procedures other than OP-903-114 was conducted in an effort to identify additional deficiencies similar to those discussed above. This review did not identify any other deficiencies concerning the use of the pressure decay method for LLRT. A procedural update for OP-903-114 has been scheduled as a result of this review and the subject inspection. Prior to the procedural update, LLRT activities shall be administratively controlled to require the use of a test method other than pressure decay.

(3) Corrective Steps Which Will Be Taken to Avoid Further Violations

Corrective actions planned, but not yet complete are as follows:

- 1) Revise OP-903-114 to include precautions and limitations stating the minimum allowable test pressure of 44 psig as stated in Tech Spec 4.6.1.2.d.
- 2) Revise OP-903-114 to include a correction factor that normalizes reduced pressure test leakrates to 44 psig. Limit the use of the correction factor to "Information Only" tests where excessive leakage is the cause of reduced pressure. An alternate test method shall be used to validate any reduced pressure test.

It is believed that the revision of this procedure, coupled with the review of similar test procedures, will prevent the pot stial for future violations in this area.

(4) Date When Full Compliance Will Be Achieved

The procedures discussed above in Section 3 of this response will be revised by September 30, 1990, at which time Waterford 3 will be in full compliance.