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Quantitative Analysis of Potential Performance Improvements for the Dry PWR Containment

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ABSTRACT

This report calculates the risk benefit associated with potential performance improvements for the large dry pressurized water reactor (PWR) containment. The analysis is based on the June 1989 draft NUREG-1150 results for the Zion commercial nuclear reactor. Simplified containment event trees and the large accident progression event trees from draft NUREG-1150 are used to evaluate the effects of potential improvements on the response of the Zion containment to dominant severe accident sequences. Source terms are generated parametrically using the ZISOR code and offsite consequences are calculated with the MELCOR Accident Consequence Code System (MACCS). These results give point estimates of the risk reduction associated with each containment improvement identified by Brookhaven National Laboratory in their draft Issues Characterization Report.

FIN No. A6890—Quantitative Analysis of Potential Performance Improvements for the
Dry Pressurized Water Reactor Containment

EXECUTIVE SUMMARY

This report provides a quantitative analysis of the risk reduction potential associated with the containment performance improvements identified in the draft Issues Characterization Report for the dry PWR containment. These improvements are as follows: (1) enhanced reactor depressurization to mitigate direct containment heating (DCH), (2) addition of a cavity flooding system to ensure that the reactor cavity is flooded at the time of vessel breach, (3) improvements in the hydrogen control system, (4) containment venting, and (5) modifications to reduce the frequency of the interfacing systems loss-of-coolant accident (LOCA). The last two improvements [items (4) and (5)] are not evaluated in this report.

The quantitative analysis in this report relied exclusively upon the June 1989 draft NUREG-1150 analysis of the Zion plant. Therefore, the findings in this report are necessarily specific to the Zion plant. These results should not be applied to other dry PWR containments without further analysis, with due consideration given to plant-specific and site-specific features that can affect the results.

Simplified containment event trees were used wherever possible to analyze the containment response. They were derived from the large accident progression

event trees used for the June 1989 Zion draft NUREG-1150 analysis. In some cases, dependencies among questions in the accident progression event trees made the use of simplified event trees impracticable. In these cases, the draft NUREG-1150 computer codes were used to analyze the improvements, with the Zion accident progression event trees used as input files.

Table ES-1 presents a summary of the calculated results of the risk benefit analysis.

Table ES-2 shows the relative contribution of the various modes of containment failure to two of the off-site risk measures in the base case, the 50- and 1000-mile population doses. These contributions are a weighted average of the contributions from each plant damage state group.

The following conclusions can be drawn from the analysis performed for this report. The above caveat about the plant-specific nature of the results should be kept in mind.

The benefits to risk of intentional operator depressurization cannot be judged conclusively. Vessel depressurization appears to have both positive and negative effects. First, vessel breach may be prevented in some

Table ES-1. Composite annual risk results

	Mean Early Fatalities	Mean Latent Fatalities	Mean 50-Mile Dose (Person-Rem)	Mean 1000-Mile Dose (Person-Rem)	Mean Offsite Costs (\$)
Base case	3.31E-05	1.39E-02	26.3	84.6	5.99E+04
Depressurization via PORVs	3.18E-05	1.38E-02	26.2	83.9	5.93E+04
Full depressurization #1 (no HPME)	4.12E-05	1.39E-02	26.3	84.4	5.98E+04
Full depressurization #2 (no HPME) ^a	1.60E-05	1.09E-02	18.7	65.2	5.23E+04
Cavity flooding	3.12E-05	1.28E-02	24.0	76.5	5.58E+04
H ₂ control	3.31E-05	1.39E-02	26.3	84.6	5.99E+04

a. This case includes the use of a point estimate α mode failure probability of 8.0E-04.

Table ES-2. Containment failure mode contribution to offsite dose (base case)

Containment Failure Mode	Contribution to 50-Mile Dose	Contribution to 1000-Mile Dose
	(%)	(%)
DCH	2.1	1.7
α	71.5	75.0
Bypass	26.1	23.1
Late overpressure	ϵ	ϵ
Basemat melt-through	ϵ	ϵ

ϵ = negligible

sequences by intentional depressurization, because depressurization may allow injection from available low pressure systems. This is the case, for example, in sequences where AC power is available. Depressurization also eliminates temperature-induced steam generator tube ruptures, which bypass the containment. However, depressurization reduces but does not eliminate DCH failures for Zion, because the capacity of the pressurizer pilot-operated relief valves (PORVs) has not been shown to be sufficient to fully depressurize the reactor (<200 psig). Finally, if the conditional probabilities of α mode failure (containment failure as the result of an in-vessel steam explosion) developed for draft NUREG-1150 are used, then the benefits of depressurization may be offset by an increased probability of early containment failure, because NUREG-1150 has judged α mode failure to be more likely at low reactor coolant system (RCS) pressures.

The addition of a cavity flooding system yields a slight reduction in risk but may increase the probability of DCH failure in some sequences. The effect of a flooded cavity on the threat from DCH is not known. The discussion in the draft Issues Characterization Report for the dry PWR containment appears to indicate that the effect may be plant-specific, enhancing the threat at some plants while mitigating it at others. At any rate, *the risk reduction is not significant for Zion.* Flooding the cavity does increase the conditional probability of an ex-vessel steam explosion; however, because ex-vessel steam explosions are an insignificant threat to containment integrity at Zion, this effect does not increase offsite risk.

Improvements in the hydrogen control systems are of no benefit in terms of risk. This result is very specific to Zion. Other plants, particularly those with smaller sub-atmospheric containments, might realize a more significant risk reduction from hydrogen control improvements.

Improvements to reduce the frequency of containment bypass sequences would provide the greatest tangible risk reduction benefit. Containment bypass sequences, both the so-called interfacing systems LOCA and steam generator tube rupture (SGTR), contribute very significantly to the annual offsite risk at Zion. "Front-end" improvements to reduce the incidence of bypass initiators have not been analyzed. However, any reductions in bypass frequency would provide a corresponding reduction in all risk measures. In addition, a preliminary report from Sandia National Laboratory (SNL) mentions two possible mitigative strategies, based on a plant visit to Surry. The first of these strategies is to ensure that the break location in the interfacing systems LOCA is submerged. The second is a suggestion to reflood the steam generators in the case of SGTR, to ensure that the release from the RCS is scrubbed through a volume of water. Neither of these suggestions has been evaluated in this report, because the means to do so were unavailable. A preliminary evaluation of these improvements for Surry by SNL found a significant reduction in the early and latent fatality risk (no doses or offsite costs were calculated). Both may have the potential to generically reduce the bypass sequence risk. However, the efficacy and cost-effectiveness of these strategies is best determined on a plant-specific basis.

Gradual overpressurization by noncondensable gases (including steam) is not a threat to containment integrity for Zion. Containment failure by eventual overpressurization (time scale of one or more days) was predicted only in the APET runs made for the LOCA plant damage state group. Even in these cases, the conditional probability of eventual overpressurization was very small. Again, this result is Zion-specific. A relatively high probability of containment failure attributable to basemat melt-through (BMT) was found; however, these failures are negligible contributors to offsite risk.

FOREWORD

SECY-88-47, dated May 25, 1988, presented the NRC staff's program plan to evaluate generic severe accident containment vulnerabilities via the Containment Performance Improvement (CPI) program. This effort was predicated on the assumption that there are generic severe accident challenges for each light water reactor (LWR) containment type that should be addressed to determine whether additional regulatory guidance or requirements concerning needed containment features are warranted, and to confirm the adequacy of the existing Commission policy. The bases for the assumption that such assessments were needed included the uncertainty in the ability of some LWR containments to successfully survive some severe accident challenges, as indicated in draft NUREG-1150. All LWR containment types have been assessed beginning with the boiling water reactors (BWRs) with Mark I containments. This effort was closely integrated with the Individual Plant Examination (IPE) program and is intended to focus on resolution of hardware and procedural issues related to generic containment challenges.

This report documents the results of NRC-sponsored research related to severe accident challenges and potential enhancements that could improve containment performance. The purpose of this report is to provide pressurized water reactor (PWR) dry containment owners with information they may find useful in their IPE. No requirements are contained in this report; it is provided for information only.

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ACRONYMS

AFWS	Auxiliary feedwater system	LWR	Light water reactor
APET	Accident progression event tree	MACCS	MELCOR Accident Consequence Code System
ATWS	Anticipated transient without scram	PDS	Plant damage state
BMT	Basemat melt-through	PORV	Pilot-operated relief valve
BNL	Brockhaven National Laboratory	PWR	Pressurized water reactor
BWR	Boiling water reactor	RCP	Reactor coolant pump
CCI	Core-concrete interaction	RCS	Reactor coolant system
CPI	Containment performance improvement	S2	Medium LOCA
DCH	Direct containment heating	SAIC	Science Applications International Corporation
EDG	Emergency diesel generator	SBO	Station blackout
EVSE	Ex-vessel steam explosion	SCET	Simplified containment event tree
HPME	High pressure melt ejection	SGTR	Steam generator tube rupture
LHS	Latin hypercube sampling	SNL	Sandia National Laboratory
LOCA	Loss-of-coolant accident	STCP	Source Term Code Package
LPIS	Low pressure injection system		

QUANTITATIVE ANALYSIS OF POTENTIAL PERFORMANCE IMPROVEMENTS FOR THE DRY PRESSURIZED WATER REACTOR CONTAINMENT

1. INTRODUCTION

In SECY-88-147, dated May 25, 1988, the NRC staff presented to the Commission its program plan to evaluate generic severe accident containment vulnerabilities in the Containment Performance Improvement (CPI) program. This effort is predicated on the presumption that there are generic severe accident challenges to each light water reactor (LWR) containment type that should be assessed to determine whether additional regulatory guidance or requirements concerning needed containment features are warranted, and to confirm the adequacy of the existing Commission policy. The bases for the presumption that such assessments are needed include the uncertainty in the ability of LWR containments to successfully survive some severe accident challenges, as indicated by draft NUREG-1150.¹

This report focuses on dominant severe accident challenges, as identified by the most recent NUREG-1150 research, which can conceivably threaten dry PWR containment integrity. Potential improvements from the draft Issues Characterization Report² are evaluated for their ability to arrest core damage, prevent or delay containment failure during postulated severe accidents, or mitigate the offsite health consequences of a fission product release. Accordingly, a risk analysis has to be performed to correlate containment challenges, resulting consequences, sequence frequencies, and potential improvement benefits. Potential improvements and benefits are considered for each containment challenge.

A quantitative risk analysis is presented to correlate severe accident sequence frequencies, containment failure mode probabilities, and the magnitude of the offsite consequences. As seen in Equation 1.1, the risk from operation of a nuclear power plant stems from all these factors:

$$\text{RISK}_k = \sum_{i=1}^m \sum_{j=1}^n \left[\text{FREQ}_i \cdot \text{CRMP}_{ij} \right. \\ \left. \cdot \text{CONS}_k (\text{FP}_{ij}) \right] \quad (1.1)$$

where,

RISK_k = the risk associated with consequence measure k

FREQ_i = the frequency of accident sequence i

CRMP_{ij} = the conditional probability of containment release mode j , given accident sequence i

FP_{ij} = fission product source term for containment release mode j of accident sequence i

CONS_k = mean magnitude of consequence k , given fission product source term (FP_{ij}) for release mode j and sequence i .

Consequently, all factors affecting plant risk should be considered in a program to improve containment performance.

Because of time and budget constraints, this report analyzes only PWR containments that operate at atmospheric pressure, with Zion being chosen as the reference plant. Subatmospheric containments such as Surry are not specifically analyzed in this report, although some of the conclusions reached may also be applicable to this type of PWR containment as well. The fact that Zion was used as the reference plant for the atmospheric PWR containment should also be stressed. As a result of this choice, some of the results may be specific to Zion; applicability to other sites would have to be verified on a plant-specific basis.

The analyses in this report are based on the June 1989 draft NUREG-1150 analysis of the Zion plant.^{1,3,4} The methodology used is explained in detail in Section 2. Briefly, simplified containment event trees (SCETs), each consisting of 10-15 top events, were developed from the large 72-question accident progression event trees (APET) used to analyze the Zion containment response for draft NUREG-1150. A base case SCET was constructed for each of the dominant Zion plant damage states: (1) loss-of-coolant

accidents (LOCAs), (2) station blackout (SBO), (3) transients (including anticipated transients without scram), and (4) containment bypass. Each of these plant damage states is defined in draft NUREG/CR-4550 for Zion.³ The SCETs were utilized to the greatest extent possible in analyzing the effects of the potential improvements on containment response. The details of how this was done are provided in Section 2.

To reduce the number of source term calculations, the end states of the SCETs were grouped into accident progression bins in accordance with the binning scheme presented in draft NUREG/CR-4551 for Zion.⁴ This scheme groups together end states that have similar characteristics of containment failure mode, type of reactor vessel breach, amount of core-concrete interaction (CCI), etc. The details of the scheme used to bin the SCET end states are presented in Section 2.

For each accident progression bin produced by grouping the SCET end states, a source term must be calculated so that offsite consequences can be determined. As in the draft NUREG-1150 analysis of Zion, the ZISOR parametric source term generation code was used to do this.⁴ For each accident progression bin, ZISOR parametrically calculates important characteristics of the containment release. Examples of these characteristics are the time and duration of the release, the release fractions of the various nuclide groups, and the energy of the release. In all of the cases in this report, *point estimates* of the source terms were obtained using ZISOR. This is an important technicality that will be discussed in more detail in later sections.

The source terms generated with the ZISOR code are input to the MELCOR Accident Consequence Code System (MACCS), along with site data from the Zion draft NUREG-1150 MACCS deck. The

MACCS code generates conditional offsite consequences for each set of source terms input from ZISOR, generally for each accident progression bin. These conditional consequences are the last input needed to calculate risk using Equation 1.1. For this report, five risk measures are reported: (1) the mean number of early (acute) fatalities per reactor-year of operation, (2) the mean number of latent cancer fatalities per reactor-year of operation, (3) the mean dose (in person-rem per reactor-year) within 50 miles of the plant, (4) the mean dose (in person-rem per reactor-year) over the entire 1000-mile MACCS calculational grid, and (5) the mean offsite costs (\$ per reactor-year).

The authors would like to emphasize that *point estimates* of risk were calculated for this report. There is an uncertainty range associated with each of these numbers, either because of stochastic variations or lack of knowledge. These uncertainties have not been fully evaluated for this report, since a full uncertainty analysis was beyond the specified scope of work. In some instances, sensitivity cases have been run in an attempt to provide an estimate of uncertainty. However, the reader should keep this limitation in mind when using any of the risk estimates in this report.

A word of caution should likewise be added about the number of significant figures associated with the values in this report. In most cases, the codes used in the analysis provide two or more significant figures. The authors generally have reported all values using two or three significant figures. However, this does not signify high confidence in these values to this level of precision. On the contrary, our level of knowledge is limited to, at most, one significant figure. The important point is that the reader should not take the values in this report literally to the second or third place after the decimal, because the values are not really accurately known to that level. As an example, a reported value of 0.167 should probably be interpreted as 0.2.

2. METHODOLOGY

This section describes in some detail the methodology used to construct and quantify the SCETs. It also describes the process used to bin the SCET end states into accident progression bins, the use of the ZISOR code to generate source terms, and the use of the MACCS code to calculate consequences. Readers interested primarily in the results of the analysis rather than the details of the analysis itself may omit this section.

2.1 Construction of SCETs

The EVNTRE event progression analysis code⁵ provided the essential tool for developing SCETs from the large APETs used for Reference 4. The Zion APET consists of 72 questions or top events, with most questions having multiple branches (see Appendix A for a listing of a representative Zion APET). Because the APET is so large and complex, it cannot be graphically represented; it exists only as a computer input file. The APET's complexity makes it very useful for detailed evaluations of containment response but the analysis is similarly complicated and the results can be difficult to interpret without highly detailed knowledge of the analysis. One of the goals of this analysis was to condense the information contained in the APET into a form that can be graphically displayed so that individual paths through the event tree can be visually traced out. Thus, there was a need to construct SCETs.

Rather than the 72 questions in the Zion APET, the SCETs each have at most 10-20 questions. In addition, all branching in the SCETs is binary. Tertiary and higher order branching was eliminated in order to simplify the paths through the tree to the greatest possible extent.

2.1.1 Branching Structure Determination.

The skeletal structure of each SCET was developed by using the sorting feature in the EVNTRE code. This feature allows the user to select summary questions displaying the relevant accident progression phenomenology from the APET and sort the output of the APET on these questions. The result of this sorting is a greatly simplified event tree, whose top events are simply the summary questions arranged according to the order specified in the sort. The sorted output file from EVNTRE provides the branching structure for the SCET. To make this concept more concrete, the base case SCET for the LOCA plant damage state (PDS) is developed in this section as an example.

The full 72-question APET for the Zion LOCA PDS is listed in Appendix A for reference. The first step in constructing the SCET is to select the summary questions from the APET that will represent the top events in the SCET. For the LOCA PDS, the following questions were chosen (the question numbers refer to the APET listed in Appendix A).

1. CI - #11: Event CI questions whether there is pre-existing containment leakage.
2. VB - #23: Event VB questions whether reactor vessel breach occurs.
3. E-SPRY - #24: Event E-SPRY questions whether containment sprays are available prior to vessel breach.
4. FLD - #31: Event FLD questions whether the reactor cavity is flooded or dry at the time of vessel breach.
5. HPME - #35: Event HPME questions whether high pressure melt ejection occurs at the time of vessel breach.
6. EVSE - #40: Event EVSE questions whether a significant ex-vessel steam explosion occurs following vessel breach.
7. E-CF - #42: Event E-CF questions whether containment failure occurs at, or shortly after, vessel breach.
8. SPRY - #43: Event SPRY questions the availability of containment sprays following vessel breach.
9. CCI - #50,55: Event CCI questions whether there is either a prompt or delayed core-concrete interaction (CCI) following vessel breach.
10. L-OP - #64: Event L-OP questions whether late overpressure containment failure occurs.
11. BMT - #68: Event BMT questions whether basemat melt-through occurs.

To obtain the sorted output file that contains the branch structure information for the SCET, the EVNTRE code is run using the LOCA APET as the tree input file, with a binning input file constructed to

sort the output on the above 11 questions. The EVNTRE code initially had to be run in the point estimate mode (mode 2), because the post-processors needed to evaluate output from the sampling mode were not available at the beginning of this project. This can be a fairly severe limitation, because the results of

a point estimate run can be drastically different from results obtained in the sampling mode. However, this limitation was overcome to a degree, as will be explained shortly. The binning input file and the sorted output from the point estimate run for the LOCA PDS will be presented first.

Zion LOCA Input binning File

		CI	VB	E-SPRY	FLD	HPME	EVSE	E-CF	SPRY
		CCI	OP	BMT					
2	2	CI	CI						\$ Pre-existing cont. leakage
1	1	11							
		1							
		CI							
1	2	11							
		/1							
		No-CI							
2	2	VB	No-VB						\$ Vessel breach
1	1	23							
		/1							
		VB							
1	2	23							
		1							
		No-VB							
2	2	E-SPRY	noE-SPRY						\$ Early sprays
1	1	24							
		1							
		E-SPRY							
1	2	24							
		/1							
		noE-SPRY							
2	2	FLD	Dry						\$ Amount of water in cavity
1	1	31							
		1							
		Wet							
1	2	31							
		/1							
		Dry							
2	2	HPME	No-HPME						\$ High pressure melt ejection
1	1	35							
		1							
		PrEj							
1	2	35							
		/1							
		nPrEj							
2	2	EVSE	nEVSE						\$ Ex-vessel steam explosion
1	1	40							
		1							
		EVSE							
1	2	40							
		/1							
		nEVSE							

Zion LOCA Input Binning File (continued)

2	2	E-CF	noE-CF									\$ Early Cont. Failure
1	1	42										
		/4										
		E-CF										
1	2	42										
		4										
		noE-CF										
2	2	SPRY	noSPRY									\$ Sprays after vessel breach
1	1	43										
		1										
		SPRY										
1	2	43										
		/1										
		noSPRY										
2	2	CCI	noCCI									\$ Core-concrete interaction
2	1	50										
		1	+									
		PrmCCI										
2	2	50										
		/1										
		noPrmptCCI										
2	2	L-OP	noL-OP									\$ Late overpressure cont. failure
1	1	64										
		/5										
		OP-CCI										
1	2	64										
		5										
		noOP-CCI										
2	2	BMT	noBMT									\$ Basemat melt-through
1	1	68										
		/3										
		BMT										
1	2	68										
		3										
		noBMT										
1												
11		1	2	3	4	5	6	7	8	9	10	11
		CI	VB	E-SPRY	FLD	HPME	EVSE	E-CF	SPRY			
		CCI	L-OP	BMT								

The SCET branching structure is obtained from this sorted output by tracing lines through the output, starting at the lower left hand corner, with branches at each point where the conditional probability changes. The first several branches for the LOCA SCET have been outlined in the sorted output above to illustrate this procedure. The split fraction at each branch is calculated by dividing the sequence fraction at the branch in question by the fraction at the preceding point in the SCET. For larger SCETs with more top events upon which to sort the APET output, this procedure quickly becomes unwieldy to perform by hand. To automate the construction of the SCETs from the sorted output, use was made of an interface code currently under development by Science Applications International Corporation (SAIC) called ET-LOAD.⁶ This code reads the sorted output file from EVNTRE and constructs the SCET using SAIC's ETA-II event tree code.⁷ The branching structure and split fractions are determined automatically by ET-LOAD. Note that the sorted output indicates that early containment sprays are always available in the LOCA PDS. Therefore, in order to further simplify the event tree, the column headed by event E-SPRY was deleted.

There is a further limitation in that the EVNTRE code is set up to sort upon at most ten questions. This limitation was overcome by making a modification to the EVNTRE source code to allow use of up to 20 sort parameters. However, this same limitation also exists in the version of ET-LOAD obtained from SAIC. Because the source code for ET-LOAD was unavailable, the allowed number of sort parameters could not be increased. Therefore, for larger SCETs, the branching structure and split fractions for the last few questions had to be input by hand.

2.1.2 Split Fraction Evaluation. As mentioned above, the point estimate split fractions are calculated by dividing the sequence fraction at the branch in question by the fraction at the preceding point in the SCET. However, because these split fractions were obtained from a *point estimate* evaluation of the APET, the SCET with these split fractions may not always

accurately model the APET results. An example of this type of problem is seen in the conditional probability of early containment failure attributable to direct containment heating (DCH). The point estimate evaluation of the APET does not predict any failures of this type. On the other hand, Reference 4 indicates that the conditional probability of DCH failure, given the occurrence of the LOCA PDS, is $\sim 1.7 \times 10^{-3}$. Because the post-processors needed to extract a composite sorted output from an APET evaluated in the sampling mode were not initially available, a method was developed to reduce the impact of having to sort the APET in the point estimate mode. The key to this method is the use of the EVNTRE frequency output file generated from a sampling mode evaluation of the APET. This file, as discussed in Reference 5, provides the realized split fractions for each question in the APET. A particular split fraction in this file is actually the conditional probability of taking that particular branch of the APET. By referring to the appropriate question in the APET, the conditional probability of DCH failure, vessel breach, basemat melt-through (BMT), and other parameters of interest can be determined. The frequency output file for the base case LOCA PDS is shown in Appendix B.

Figure 2.1 shows the LOCA SCET obtained from the point estimate run. Similarly, Figure 2.2 shows the final base case LOCA SCET obtained by modifying the SCET in Figure 2.1 to obtain agreement with the sampled frequency output.

2.1.3 Binning of SCET End States. The next step in the analysis is to map the end states of the SCET into the set of accident progression bins defined in Reference 4. This mapping is required in using the ZISOR code to generate source terms for the consequence calculations. Binning the SCET end states is a two-step process. First, an EVNTRE input file must be created to describe the SCET. This file is analogous to the APET file listed in Appendix A but it describes the SCET instead of the 72-question APET. The file that describes the LOCA SCET is listed as follows:

Zion Simplified Containment Event Tree (SCET)

10			
NQ			
1	1.000		
	LOCA		
1	Is there pre-existing containment leakage?		
2	CL	ncCL	
1	1	2	
	0.005	0.995	

Zion Simplified Containment Event Tree (SCET) (continued)

2	Does the reactor pressure vessel fail?				
2	VB	noVB			
1	1	2			
	0.9999832	0.0000168			
3	Is the reactor cavity flooded?				
2	FLD	noFLD			
2	1	2			
2					
1	2				
	1				
	VB				
	0.9943	0.0057			
	Otherwise				
	0.879	0.121			
4	Does HPME occur at vessel failure?				
2	HPME	noHPME			
2	1	2			
2					
2	2	3			
	1	* 1			
	VB	FLD			
	5.80E-02	9.42E-01			
	Otherwise				
	0.000	1.000			
5	Does a large ex-vessel steam explosion occur?				
2	EVSE	noEVSE			
2	1	2			
2					
3	2	3	4		
	1	* 1	* 2		
	VB	FLD	noHPME		
	4.95E-01	5.05E-01			
	Otherwise				
	0.000	1.000			
6	Does early containment failure occur?				
2	E-CF	noE-CF			
2	1	2			
4					
5	1	2	3	4	5
	2	* 1	* 1	* 1	* 2
	noCL	VB	FLD	HPME	noEVSE
	2.50E-02	9.75E-01			
4	2	3	4	5	
	1	* 1	* 2	* 2	
	VB	FLD	noHPME	noEVSE	
	1.50E-02	9.85E-01			
2	2	3			
	1	* 2			
	VB	noFLD			
	8.00E-03	9.92E-01			
	Otherwise				
	0.000	1.000			

Zion Simplified Containment Event Tree (SCET) (continued)

7 Sprays after vessel breach?

2	SPRY	noSPRY
2	1	2
2		
2	4	6
	2	* 1
	noHPME	E-CF
	0.000	1.000
	Otherwise	
	1.000	0.000

8 Does core-concrete interaction occur?

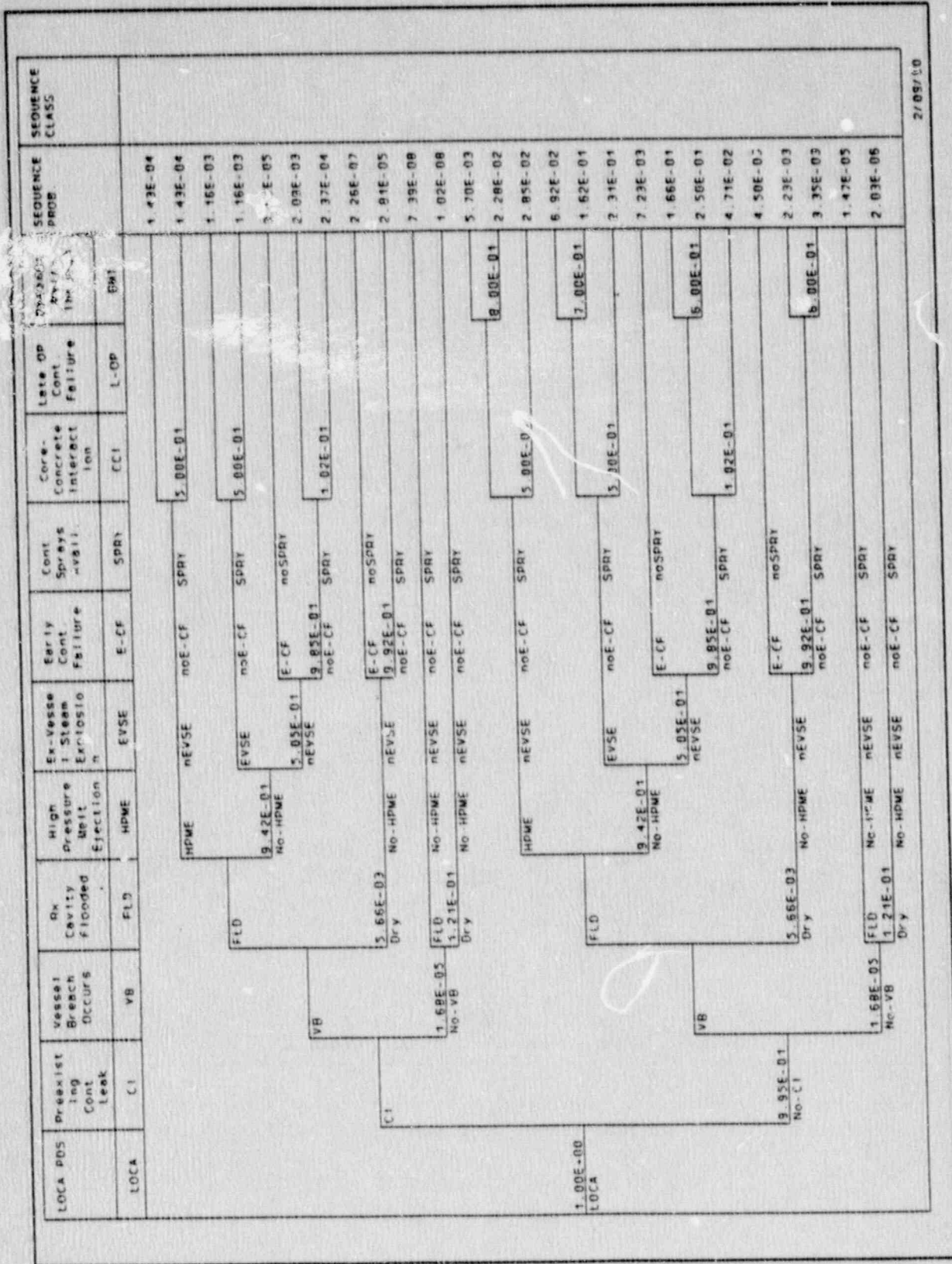
2	CCI	noCCI			
2	1	2			
5					
1	4				
	1				
	HPME				
	5.00E-01	5.00E-01			
1	5				
	1				
	EVSE				
	5.00E-01	5.00E-01			
5	2	3	4	5	6
	1	* 1	* 2	* 2	* 2
	VB	FLD	noHPME	noEVSE	noE-CF
	8.98E-01	1.02E-01			
1	2				
	2				
	noVB				
	0.000	1.000			
	Otherwise				
	1.000	0.000			

9 Does late containment overpressure failure occur?

2	L-OP	noL-OP			
2	1	2			
	1	6	8		
	2	* 2	* 1		
	noCL	noE-C	CCI		
	9.99E-04	9.991E-01			
	Otherwise				
	1.000				

Does melt-through occur?

	T	noBMT					
	1	2					
	1	2	4	6	8	9	
	2	* 1	* 1	* 2	* 1	* 2	
	noCL	VB	HPME	noE-CF	CCI	noL-OP	
	2.0E-01	8.0E-01					
7	1	2	4	5	6	8	9
	2	* 1	* 2	* 1	* 2	* 1	* 2
	noCL	VB	noHPME	EVSE	noE-CF	CCI	noL-OP
	3.0E-01	7.0E-01					
7	1	2	4	5	6	8	9
	2	* 1	2	* 2	* 2	* 1	* 2
	noCL	VB	noHPME	noEVSE	noE-CF	CCI	noL-OP
	4.0E-01	6.0E-01					
	Otherwise						
	0.000	1.000					



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Figure 2.1. Point estimate LOCA SCET.

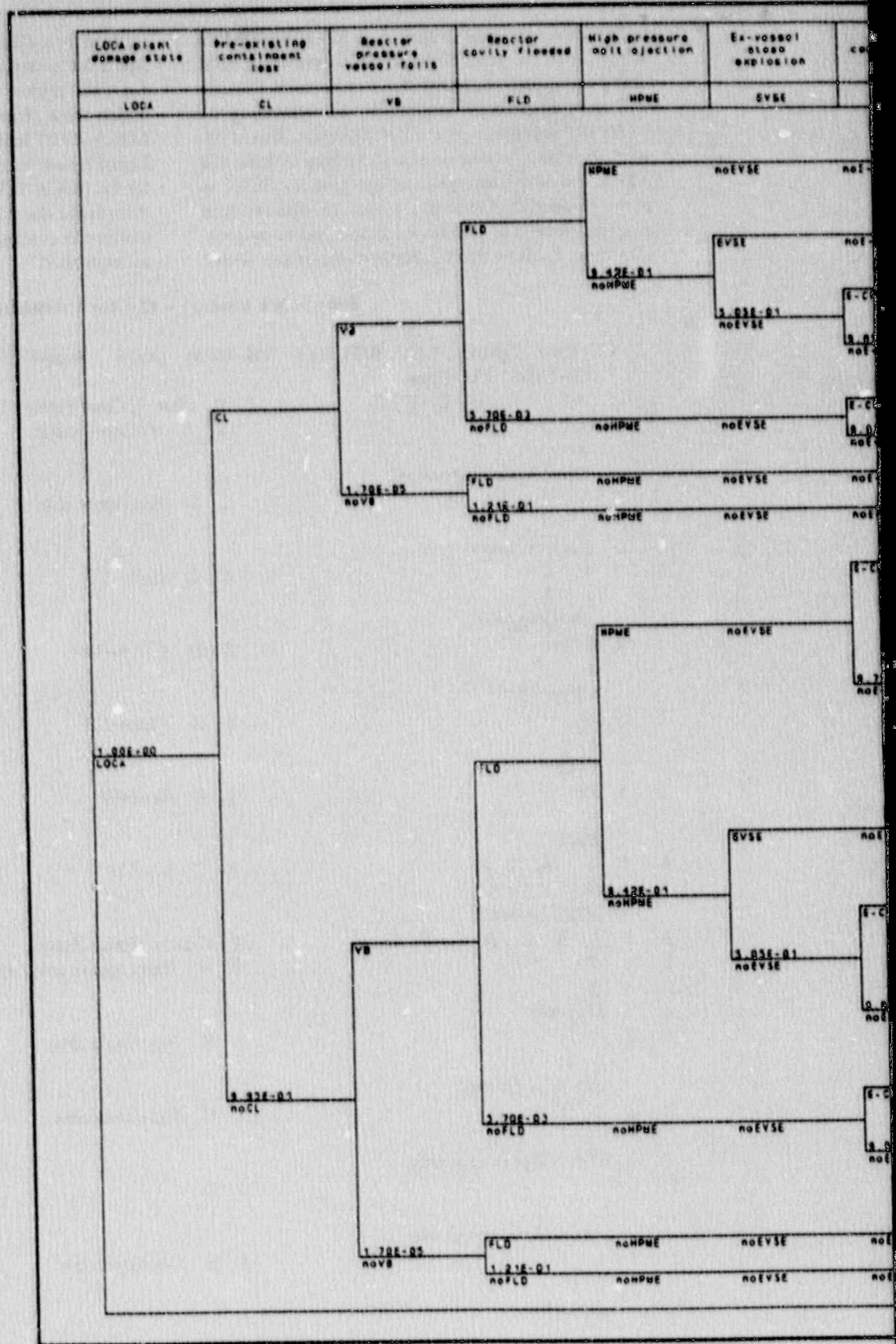


Figure 2.2. Base case LOCA SCET.

Primary containment failure	Sprays after vessel breach	Core-concrete interaction	Loss of pressure containment failure	Basemat melt-through	SEQUENCE PROB	SEQUENCE CLASS	SEQUENCE DESCRIPTION
E-CF	SPRY	CCI	L-OP	BM?			
		CCI	NOL-OP	NOBMT	1.44E-04	ECF-CL-D	EDDCACBACBB
	SPRY	5.00E-01 NOCCI	NOL-OP	NOBMT	1.44E-04	ECF-CL-D	EDCCACBACBB
		CCI	NOL-OP	NOBMT	1.16E-03	ECF-CL	EDDDBCBDCBB
	SPRY	5.00E-01 NOCCI	NOL-OP	NOBMT	1.16E-03	ECF-CL	EDCDBCBDCBB
	NOSPRY	CCI	NOL-OP	NOBMT	3.55E-05	ECF-CL-A	EDFDDCBDBBB
		CCI	NOL-OP	NOBMT	2.09E-03	ECF-CL	EDDBCBACBB
	SPRY	5.00E-01 NOCCI	NOL-OP	NOBMT	2.30E-04	ECF-CL	EDCDBCBDCBB
	NOSPRY	CCI	NOL-OP	NOBMT	2.20E-07	ECF-CL-A	EDADCCBDBBB
		CCI	NOL-OP	NOBMT	2.03E-05	ECF-CL	EDADCBACBB
	SPRY	NOCCI	NOL-OP	NOBMT	7.47E-06	ECF-CL	EDCAFCDDBCB
	SPRY	NOCCI	NOL-OP	NOBMT	1.03E-09	ECF-CL	EDCAFCDDBCB
		CCI	NOL-OP	NOBMT	7.17E-04	ECF-D	EDDCACBACBB
	SPRY	5.00E-01 NOCCI	NOL-OP	NOBMT	7.17E-04	ECF-D	EDCCACBACBB
		L-OP	NOBMT		2.52E-05	LCF	EDDCACBACBB
		CCI	5.00E-01 NOL-OP	BM?	5.59E-03	BM?	EDDCACBACBB
	SPRY	5.00E-01 NOCCI	NOL-OP	NOBMT	2.24E-02	NCF	EDDCACBACBB
		L-OP	NOBMT		2.09E-04	LCF	EDDDBCBDCBB
		CCI	5.00E-01 NOL-OP	BM?	5.91E-02	BM?	EDDDBCBDCBB
	SPRY	5.00E-01 NOCCI	NOL-OP	NOBMT	1.61E-31	NCF	EDDDBCBDCBB
	NOSPRY	CCI	NOL-OP	NOBMT	2.31E-01	NCF	EDCDBCBDCBB
		CCI	NOL-OP	NOBMT	7.06E-03	ECF-A	EDFDDCBDBBB
		L-OP	NOBMT		3.73E-04	LCF	EDDDBCBDCBB
		CCI	5.00E-01 NOL-OP	BM?	1.64E-01	BM?	EDDDBCBDCBB
	SPRY	5.00E-01 NOCCI	NOL-OP	NOBMT	2.50E-01	NCF	EDDDBCBDCBB
		L-OP	NOBMT		4.73E-02	NCF	EDCDBCBDCBB
	NOSPRY	CCI	NOL-OP	NOBMT	4.54E-05	ECF-A	EDADCCBDBBB
		L-OP	NOBMT		5.06E-04	LCF	EDADCBACBB
	SPRY	CCI	5.00E-01 NOL-OP	BM?	2.25E-03	BM?	EDADCBACBB
		L-OP	NOBMT		3.37E-03	NCF	EDADCBACBB
	SPRY	NOCCI	NOL-OP	NOBMT	1.49E-05	NCF	EDCAFCDDBCB
	SPRY	NOCCI	NOL-OP	NOBMT	2.05E-08	NCF	EDCAFCDDBCB

2/09/90

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The second step is to construct a binning file that assigns each end state of the SCET to one of the accident progression bins defined in Reference 4. This file will serve as the input binning file for evaluating the SCET file above using the EVNTRE code. This is exactly analogous to what was done initially with the full APET. The only difference is that now the SCET is being evaluated. Reference 4 uses 12-dimensional character vectors to identify the Zion accident progression bins. Each of the 12 components of the vector

represents a characteristic of the sequence that may affect the source term (see Table 2.4-1 in Reference 4 for a description of these characters). The file used to assign these character vectors to the end states of the LOCA APET in Reference 4 is shown in Appendix C. Listed below is the analogous binning file developed for the LOCA SCET. The question numbers in this file refer to the questions in the SCET, with the questions numbered consecutively from left to right, starting with question 1.

Zion SCET Binning - 12 Characteristics

12	CF-Time	Sprays	CCI	RCS-Pres	VB-Mode	SGTR	Amt-CCI	Zr-Ox	HPME	CF-Size	
	RCS-Hole	CD-Time									
7	7	A B C D E F G									\$ Char. 1, Cont. Failure Time
2	1	1 1									\$ A Not Applicable
		1 /1									
		Event V, not submerged									
2	2	1 1									\$ B Not Applicable
		1 /1									
		Event V, submerged									
2	3	1 6									\$ C Early-CF
		1 2									
		CF before VB									
1	4	6									\$ D CF-at-VB
		1									
		CF at VB									
1	5	9									\$ E VLate-CF
		1									
		L-OP									
1	6	10									\$ F Final-CF
		1									
		BMT									
4	7	1 6 9 10									\$ G No-CF
		2 *2 *2 *2									
		SGTR or NoCF									
8	8	A B C D E F G H									\$ Char. 2, Spray Status
1	1	7									\$ A Early sprays only; sprays fail at vessel breach.
		2									
		ESp only									
2	2	1 1									\$ B Not Applicable
		1 /1									
		ESp & ImSp only									
2	3	1 1									\$ C Not Applicable
		1 /i									
		ESp, ImSp, & LSp only									
1	4	7									\$ D
		1									
		Sprays always available									
2	5	1 1									\$ E Not Applicable
		1 /1									
		LSp only									

Zion SCET Bin/aug - 12 Characteristics (continued)

2	6	1	1						\$ F	Not Applicable
		1	/1							
		LSp & VLSp only								
2	7	1	1						\$ G	Not Applicable
		1	/1							
		VLSp only								
2	8	1	1						\$ H	Not Applicable
		1	/1							
		No sprays								
6	6	A	B	C	D	E	F		\$	Char. 3, CCI
2	1	3	8						\$	A Prmpt CCI in dry cavity.
		2	1							
		PrmptDry								
2	2	1	1						\$ B	Not Applicable
		1	/1							
		Prmpt CCI in shallow pool								
1	3	8							\$	C
		2								
		No CCI								
3	4	3	7	8					\$	D
		1	1	1						
		Prmpt CCI in wet cavity								
2	5	1	1						\$ E	Not Applicable
		1	/1							
		Short delay CCI-cavity not replenished								
3	6	3	7	8					\$ F	CCI occurs after a long delay; cavity not replenished.
		1	2	1						
		LDlyd-Dry								
4	4	A	B	C	D				\$	Char. 4, RCS Pressure before VB
1	1	2							\$	A
		2								
		System setpoint pressure								
2	2	1	1						\$ B	Not Applicable
		1	/1							
		High Pressure								
2	3	2	4						\$	C
		1	1							
		Intermediate								
2	4	2	4						\$	D
		1	2							
		Low pressure								
6	6	A	B	C	D	E	F		\$	Char. 5, Mode of VB
1	1	4							\$	A
		1								
		PrEj								
3	2	2	4	6					\$	B
		1	2	2						
		Gravity pour								
2	3	1	1						\$ C	Not Applicable
		1	/1							
		Gross failure of bottom head								

Zion SCET Binning - 12 Characteristics (continued)

2	4	4	6						\$ D
		2	1						
		Alpha mode							
2	5	1	1						\$ E Not Applicable
		1	/1						
		Rocket failure							
1	6	2							\$ F
		2							
		No VB							
3	3	A	B	C					\$ Char. 6, SGTR
2	1	1	1						\$ A Not Applicable
		1	/1						
		SGTR w/no SORVs							
2	2	1	1						\$ B Not Applicable
		1	/1						
		SGTR w/ SORVs							
2	3	1	1						\$ C SGTR never occurs for LOCA PDS
		1+2							
		No SGTR							
4	4	A	B	C	D				\$ Char. 7, Amt. of core in CCI
6	1	4	8		4	5	6	8	\$ A
		1	1	+	(2	2	2	1)	
		70-100% of core in CCI							
5	2	5	8		4	5	6		\$ B
		1	1	+	(2	2	1)		
		30-70% of core in CCI							
2	3	1	1						\$ C Not Applicable
		1	/1						
		0-30% of core in CCI							
1	4	8							\$ D
		2							
		No CCI							
2	2	A	B						\$ Char. 8, Zr oxidation
2	1	1	1						\$ A Not Applicable
		1	/1						
		Low Zr oxidation in-vessel							
2	2	1	1						\$ B
		1+2							
		Hi Zr oxidation in-vessel							
4	4	A	B	C	D				\$ Char. 9, HPME
1	1	4							\$ A
		1							
		>40% of core							
2	2	1	1						\$ B Not Applicable
		1	/1						
		Moderate fraction (20-40%)							
2	3	1	1						\$ C Not Applicable
		1	/1						
		Low fraction (<20%)							
1	4	4							\$ D
		2							
		No HPME							

Zion SCET Binning - 12 Characteristics (continued)

	7	7	A	B	C	D	E	F	G			
	2	1	1	1						\$	Char. 10, Type of cont. failure	
			1	/1							A Not Applicable	
			Catastrophic rupture									
	2	2	4	6						\$	B	
			2	1								
			Rupture									
	4	3	1	4	6	9				\$	C	
			1+(1 1)+1									
			Leak									
	2	4	1	1						\$	D Not Applicable	
			1	/1								
			Shear									
	1	5	10							\$	E	
			1									
			BMT									
	2	6	1	1						\$	F Not Applicable	
			1	/1								
			Bypass (SGTR)									
	4	7	1	6	9	10				\$	G	
			2 * 2 * 2 * 2									
			No CF									
	2	2	A	B						\$	Char. 11, No. of holes in RCS	
	2	1	1	1						\$	A Not Applicable	
			1	/1								
			1 hole									
	2	2	1	1						\$	B	
			1	2								
			2 holes									
	2	2	A	B						\$	Char. 12, Time of core damage	
	2	1	1	1						\$	A Not Applicable	
			1	/1								
			Early CD									
	2	2	1	1						\$	B	
			1	2								
			Late CD									
	0											

Comparing this file to the APET binning file in Appendix C, it is clear that not all of the characters could be used. The reason for this is that some details of the accident progression are unavoidably lost in the transition from the full 72-question APET to the decidedly smaller SCET (the mapping of end states to accident progression bins is *into* rather than *onto*). Examples where a reduction sometimes had to be made in the number of characters used to describe a vector dimension are the time of core damage, the number of holes in the reactor coolant system (RCS), the amount of zirconium oxidized in-vessel, and the

amount of CCI. Wherever possible in these cases, characters were eliminated based on their conditional probability of occurrence in the sampled frequency output file.

Once the binning file is constructed, the EVNTRE code is run, with the file describing the SCET as input, along with the binning file needed to assign character vectors to the SCET end states. The result of this is an output file that lists each accident progression bin (identified by a unique 12-dimensional character vector) and its conditional probability of occurrence.

This file forms the input for the ZISOR source term calculation. The ZISOR base case LOCA input file is shown as follows:

SCET LOCA Accident Progression Bins

Conditional Probability	Vector
2.8272E-05	CDADBCABDCBB
8.4000E-08	CDCAFCDDBCBB
1.4417E-04	CDCCACDBACBB
1.3967E-03	CDCDBCDBDCBB
1.4417E-04	CDDCACABACBB
2.0919E-03	CDDDBCABDCBB
1.1591E-03	CDDDBCBBDCBB
4.5599E-05	DAADDCBBDBBB
7.0949E-03	DAFDDCBBDBBB
7.1725E-04	DDCCACDBACBB
7.1725E-04	DDDCACABACBB
5.0634E-06	EDADBCABDCBB
2.5176E-05	EDDCACABACBB
3.7465E-04	EDDDBCABDCBB
2.0759E-04	EDDDBCBBDCBB
2.2484E-03	FDADBCABDEBB
5.5895E-03	FDDCACABAEBB
1.6636E-01	FDDDBCABDEBB
6.9134E-02	FDDDBCBBDEBB
3.3726E-03	GDADBCABDGBB
1.6716E-05	GDCAFCDBDGBB
2.7973E-02	GDCCACDBAGBB
2.7794E-01	GDCDBCDBDGBB
2.2358E-02	GDDCACABAGBB
2.4954E-01	GDDDBCABDGBB
1.6131E-01	GDDDBCBBDGBB

A description for the first of these vectors, CDADBCABDCBB, is provided here for reference. Further discussion can be found in Reference 4.

Vector Character	Description
C	Containment failure before vessel breach
D	Containment sprays available throughout sequence
A	Prompt core-concrete interaction occurs in dry reactor cavity
D	RCS pressure low (<200 psig) at time of vessel failure
B	Gravity pour of debris from failed reactor vessel
C	No SGTR

Vector Character	Description
A	70-100% of core participates in core-concrete interaction
B	Large amount of zirconium oxidation occurs in-vessel
D	No high pressure melt ejection from the failed reactor vessel
C	Containment rupture
B	Two holes in the RCS
B	Core damage occurs late.

The end state binning process was enhanced near the end of the project when the PSTEVNT post-processor code became available.⁸ The use of this code to automatically map the end states of the SCET into ZISOR accident progression bins is described in Section 4.

2.2 Source Term Generation

A source term was calculated for each SCET accident progression bin using the ZISOR parametric source term generation code.⁴ As mentioned earlier, the input to this code is the file listing the accident progression bins and their conditional probabilities of occurrence (these probabilities are not actually used in the ZISOR code). As in the previous case of EVNTRE, only point estimates of the source terms could be obtained from ZISOR, since the necessary post-processor (the PARTITION code)⁹ was not available. This limitation is discussed further in the paragraphs which follow.

Figures 3.5-1 through 3.5-4 in Reference 4 show a comparison of the ZISOR results with those from the Source Term Code Package (STCP) for four accident sequences. As these figures illustrate, the point estimate isotopic release fractions calculated by ZISOR can differ markedly from the mean release fractions determined with ZISOR when using a Latin Hypercube Sampling (LHS) routine. In most cases, the point estimate is lower than the sampling mean, sometimes by several orders of magnitude. Because of this, the conditional consequences calculated with the MACCS code, using point estimate inputs from ZISOR, tend to be lower than the consequence results published in Reference 4. This is an inescapable problem, though not a severe one, that should be recognized when comparing the CPI results in this report to the Zion results published in the June 1989 draft of NUREG-1150.

For each accident progression bin, ZISOR calculates the characteristics of the release and the isotopic release fractions for each of the two allowable release plumes. The ZISOR output for the LOCA SCET accident progression bins is shown below. Following each 12-dimensional character vector are three rows of numbers, which represent the characteristics of the release. The meaning of each of these numbers (from left to right by row) is listed here for easy reference.

- Row 1: 1) Warning time in seconds; usually the time of core collapse.
 2) Time of beginning of early release in seconds.
 3) Duration of early release in seconds.
 4) Time of beginning of late release in seconds.
 5) Duration of late release in seconds.
 6) Elevation of the release in meters.

- Row 2/3: 1) Energy release rate of first/second plume in watts.
 2) Fraction of Xe and Kr released in the first/second plume.
 3) Fraction of iodine released in the first/second plume.
 4) Fraction of cesium released in the first/second plume.
 5) Fraction of tellurium released in the first/second plume.
 6) Fraction of strontium released in the first/second plume.
 7) Fraction of ruthenium released in the first/second plume.
 8) Fraction of lanthanum released in the first/second plume.
 9) Fraction of cerium released in the first/second plume.
 10) Fraction of barium released in the first/second plume.

ZISOR Point Estimate Results for SCET LOCA Accident Progression Bins

1 CDADBCABDCBB	1.440E+04	1.800E+04	1.800E+03	1.980E+04	3.600E+04	1.000E+01														
	3.111E+05	1.000E+00	8.653E-03	8.613E-03	6.972E-03	1.001E-03	7.700E-09	7.700E-10	7.700E-10	1.001E-03										
	1.600E+06	0.000E+00	0.000E+00	0.000E+00	2.132E-04	3.864E-04	4.037E-08	2.180E-06	2.180E-06	3.864E-04										
1 CDCAFCDDBDCBB	1.440E+04	1.800E+04	1.800E+03	1.980E+04	3.600E+04	1.000E+01														
	3.111E+05	1.000E+00	1.188E-03	9.900E-04	6.300E-04	7.150E-05	5.000E-10	5.000E-11	5.000E-11	7.150E-05										
	1.600E+06	0.000E+00	0.000E+00	0.000E+00	0.000E+00	0.000E+00	0.000E+00	0.000E+00	0.000E+00	0.000E+00										
1 CDCCACDBACBB	1.440E+04	1.800E+04	1.800E+03	1.980E+04	3.600E+04	1.000E+01														
	3.111E+05	1.000E+00	4.950E-03	4.950E-03	1.680E-04	4.420E-04	3.400E-09	3.400E-10	3.400E-10	4.420E-04										
	1.600E+06	0.000E+00	0.000E+00	0.000E+00	0.000E+00	0.000E+00	0.000E+00	0.000E+00	0.000E+00	0.000E+00										
1 CDCDBCDBDCBB	1.440E+04	1.800E+04	1.800E+03	1.980E+04	3.600E+04	1.000E+01														
	3.111E+05	1.000E+00	8.613E-03	8.613E-03	6.972E-03	1.001E-03	7.700E-09	7.700E-10	7.700E-10	1.001E-03										
	1.600E+06	0.000E+00	0.000E+00	0.000E+00	0.000E+00	0.000E+00	0.000E+00	0.000E+00	0.000E+00	0.000E+00										
1 CDDCACABACBB	1.440E+04	1.800E+04	1.800E+03	1.980E+04	3.600E+04	1.000E+01														
	3.111E+05	1.000E+00	4.974E-03	4.974E-03	1.680E-04	4.420E-04	3.400E-09	3.400E-10	3.400E-10	4.420E-04										
	1.600E+06	0.000E+00	0.000E+00	0.000E+00	1.283E-04	2.326E-04	2.431E-08	1.313E-06	1.313E-06	2.326E-04										
1 CDDDBCABDCBB	1.440E+04	1.800E+04	1.800E+03	1.980E+04	3.600E+04	1.000E+01														
	3.111E+05	1.000E+00	8.653E-03	8.653E-03	6.972E-03	1.001E-03	7.700E-09	7.700E-10	7.700E-10	1.001E-03										
	1.600E+06	0.000E+00	0.000E+00	0.000E+00	2.132E-04	3.864E-04	4.037E-08	2.180E-06	2.180E-06	3.864E-04										
1 CDDDBCBBDCBB	1.440E+04	1.800E+04	1.800E+03	1.980E+04	3.600E+04	1.000E+01														
	3.111E+05	1.000E+00	8.637E-03	8.637E-03	6.972E-03	1.001E-03	7.700E-09	7.700E-10	7.700E-10	1.001E-03										
	1.600E+06	0.000E+00	0.000E+00	0.000E+00	1.254E-04	2.273E-04	2.375E-08	1.282E-06	1.282E-06	2.273E-04										
1 DAADDCBBDBBB	1.440E+04	1.800E+04	9.000E+02	1.890E+04	3.600E+04	1.000E+01														
	6.222E+05	1.000E+00	1.735E-02	1.735E-02	1.239E-02	1.780E-03	1.369E-08	1.369E-09	1.369E-09	1.780E-03										
	1.600E+06	0.000E+00	0.000E+00	0.000E+00	1.078E-02	1.955E-02	2.042E-06	1.103E-04	1.103E-04	1.955E-02										
1 DAFDDCBDBBB	1.440E+04	1.800E+04	9.000E+02	1.890E+04	3.600E+04	1.000E+01														
	6.222E+05	1.000E+00	1.531E-02	1.531E-02	1.239E-02	1.780E-03	1.369E-08	1.369E-09	1.369E-09	1.780E-03										
	1.600E+06	0.000E+00	2.042E-03	2.042E-03	1.078E-02	1.955E-02	2.042E-06	1.103E-04	1.103E-04	1.955E-02										
1 DDCCACDBACBB	1.440E+04	1.800E+04	1.800E+03	1.980E+04	3.600E+04	1.000E+01														
	3.111E+05	1.000E+00	1.100E-03	1.100E-03	3.733E-05	9.822E-05	7.556E-10	7.556E-11	7.556E-11	9.822E-05										
	1.600E+06	0.000E+00	0.000E+00	0.000E+00	0.000E+00	0.000E+00	0.000E+00	0.000E+00	0.000E+00	0.000E+00										

ZISOR Point Estimate Results for SCET LOCA Accident Progression Bins (continued)

1 DDDCACABACBB										
1.440E+04	1.800E+04	1.800E+03	1.980E+04	3.600E+04	1.000E+01					
3.111E+05	1.000E+00	1.124E-03	1.124E-03	3.733E-05	9.822E-05	7.556E-10	7.556E-11	7.556E-11	9.822E-05	
1.600E+06	0.000E+00	0.000E+00	0.000E+00	1.283E-04	2.326E-04	2.431E-08	1.313E-06	1.313E-06	2.326E-04	
1 EDADBCABDCBB										
1.440E+04	4.320E+04	1.800E+03	4.500E+04	3.600E+04	1.000E+01					
1.944E+04	0.000E+00	0.000E+00	0.000E+00	0.000E+00	0.000E+00	0.000E+00	0.000E+00	0.000E+00	0.000E+00	0.000E+00
1.944E+05	1.000E+00	1.969E-06	1.969E-06	2.140E-04	8.899E-06	4.038E-08	4.955E-08	4.955E-08	8.899E-06	
1 EDDCACABACBB										
1.440E+04	4.320E+04	1.800E+03	4.500E+04	3.600E+04	1.000E+01					
1.944E+04	0.000E+00	0.000E+00	0.000E+00	0.000E+00	0.000E+00	0.000E+00	0.000E+00	0.000E+00	0.000E+00	0.000E+00
1.944E+05	1.000E+00	1.158E-06	1.158E-06	1.284E-04	5.338E-06	2.431E-08	2.983E-08	2.983E-08	5.338E-06	
1 EDDDBCABDCBB										
1.440E+04	4.320E+04	1.800E+03	4.500E+04	3.600E+04	1.000E+01					
1.944E+04	0.000E+00	0.000E+00	0.000E+00	0.000E+00	0.000E+00	0.000E+00	0.000E+00	0.000E+00	0.000E+00	0.000E+00
1.944E+05	1.000E+00	1.969E-06	1.969E-06	2.140E-04	8.899E-06	4.038E-08	4.955E-08	4.955E-08	8.899E-06	
1 EDDDBCBBDCBB										
1.440E+04	4.320E+04	1.800E+03	4.500E+04	3.600E+04	1.000E+01					
1.944E+04	0.000E+00	0.000E+00	0.000E+00	0.000E+00	0.000E+00	0.000E+00	0.000E+00	0.000E+00	0.000E+00	0.000E+00
1.944E+05	1.000E+00	1.573E-06	1.573E-06	1.262E-04	5.283E-06	2.375E-08	2.915E-08	2.915E-08	5.283E-06	
1 FDADBCABDEBB										
1.440E+04	8.640E+04	1.800E+03	8.820E+04	3.600E+04	0.000E+00					
0.000E+00	0.000E+00	0.000E+00	0.000E+00	0.000E+00	0.000E+00	0.000E+00	0.000E+00	0.000E+00	0.000E+00	0.000E+00
0.000E+00	5.000E-03	1.941E-09	1.941E-09	1.691E-09	4.800E-10	2.863E-14	1.454E-12	1.454E-12	4.800E-10	
1 FDCCACABAEBB										
1.440E+04	8.640E+04	1.800E+03	8.820E+04	3.600E+04	0.000E+00					
0.000E+00	0.000E+00	0.000E+00	0.000E+00	0.000E+00	0.000E+00	0.000E+00	0.000E+00	0.000E+00	0.000E+00	0.000E+00
0.000E+00	5.000E-03	1.116E-09	1.116E-09	1.229E-10	2.533E-10	1.696E-14	8.751E-13	8.751E-13	2.533E-10	
1 FDDDBCABDEBB										
1.440E+04	8.640E+04	1.800E+03	8.820E+04	3.600E+04	0.000E+00					
0.000E+00	0.000E+00	0.000E+00	0.000E+00	0.000E+00	0.000E+00	0.000E+00	0.000E+00	0.000E+00	0.000E+00	0.000E+00
0.000E+00	5.000E-03	1.941E-09	1.941E-09	1.691E-09	4.800E-10	2.863E-14	1.454E-12	1.454E-12	4.800E-10	
1 FDDDBCBBDEBB										
1.440E+04	8.640E+04	1.800E+03	8.820E+04	3.600E+04	0.000E+00					
0.000E+00	0.000E+00	0.000E+00	0.000E+00	0.000E+00	0.000E+00	0.000E+00	0.000E+00	0.000E+00	0.000E+00	0.000E+00
0.000E+00	5.000E-03	1.930E-09	1.930E-09	1.633E-09	3.740E-10	1.754E-14	8.552E-13	8.552E-13	3.740E-10	
1 GDADBCABDGEBB										
1.250E+03	3.650E+03	1.800E+03	1.025E+04	2.160E+04	0.000E+00					
3.700E+06	0.000E+00	0.000E+00	0.000E+00	0.000E+00	0.000E+00	0.000E+00	0.000E+00	0.000E+00	0.000E+00	0.000E+00
1.700E+05	5.000E-03	1.941E-09	1.941E-09	1.691E-09	4.800E-10	2.863E-14	1.454E-12	1.454E-12	4.800E-10	
1 GDCAFCDBDGEBB										
1.250E+03	3.650E+03	1.800E+03	1.025E+04	2.160E+04	0.000E+00					
3.700E+06	0.000E+00	0.000E+00	0.000E+00	0.000E+00	0.000E+00	0.000E+00	0.000E+00	0.000E+00	0.000E+00	0.000E+00
1.700E+05	5.000E-03	2.640E-10	2.200E-10	1.400E-10	1.589E-11	1.111E-16	1.111E-17	1.111E-17	1.589E-11	
1 GDCCACDBAGBB										
1.250E+03	3.650E+03	1.800E+03	1.025E+04	2.160E+04	0.000E+00					
3.700E+06	0.000E+00	0.000E+00	0.000E+00	0.000E+00	0.000E+00	0.000E+00	0.000E+00	0.000E+00	0.000E+00	0.000E+00
1.700E+05	5.000E-03	1.100E-09	1.100E-09	3.733E-11	9.822E-11	7.556E-16	7.556E-17	7.556E-17	9.822E-11	
1 GDCCBCBBDGEBB										
1.250E+03	3.650E+03	1.800E+03	1.025E+04	2.160E+04	0.000E+00					
3.700E+06	0.000E+00	0.000E+00	0.000E+00	0.000E+00	0.000E+00	0.000E+00	0.000E+00	0.000E+00	0.000E+00	0.000E+00
1.700E+05	5.000E-03	1.914E-09	1.914E-09	1.549E-09	2.224E-10	1.711E-15	1.711E-16	1.711E-16	2.224E-10	
1 GDDCACABAGBB										
1.250E+03	3.650E+03	1.800E+03	1.025E+04	2.160E+04	0.000E+00					
3.700E+06	0.000E+00	0.000E+00	0.000E+00	0.000E+00	0.000E+00	0.000E+00	0.000E+00	0.000E+00	0.000E+00	0.000E+00
1.700E+05	5.000E-03	1.116E-09	1.116E-09	1.229E-10	2.533E-10	1.696E-14	8.751E-13	8.751E-13	2.533E-10	
1 GDDDBCABDGEBB										
1.250E+03	3.650E+03	1.800E+03	1.025E+04	2.160E+04	0.000E+00					
3.700E+06	0.000E+00	0.000E+00	0.000E+00	0.000E+00	0.000E+00	0.000E+00	0.000E+00	0.000E+00	0.000E+00	0.000E+00
1.700E+05	5.000E-03	1.941E-09	1.941E-09	1.691E-09	4.800E-10	2.863E-14	1.454E-12	1.454E-12	4.800E-10	
1 GDDDBCBBDGEBB										
1.250E+03	3.650E+03	1.800E+03	1.025E+04	2.160E+04	0.000E+00					
3.700E+06	0.000E+00	0.000E+00	0.000E+00	0.000E+00	0.000E+00	0.000E+00	0.000E+00	0.000E+00	0.000E+00	0.000E+00
1.700E+05	5.000E-03	1.930E-09	1.930E-09	1.633E-09	3.740E-10	1.754E-14	8.552E-13	8.552E-13	3.740E-10	

2.3 Consequence Calculations

A PC-based version of the MACCS code¹⁰ was used to calculate conditional offsite consequences for the source terms generated by ZISOR. In order to limit the computer time required for the consequence analysis to approximately 12 hours for each PDS, accident progression bins with similar source terms were grouped together before running MACCS.

The MACCS code is composed of three modules: ATMOS, EARLY, and CHRONC, which are exercised in sequence. This set of modules has been developed for the purpose of evaluating the severe accident consequences at commercial LWR power plants. MACCS 1.5.11 incorporates several improvements over earlier versions of MACCS and codes like CRAC2 in the treatment of variable and long-term releases, deposition modeling, dosimetry, emergency response, long-term mitigative actions, radiological health effects, and economic impacts.

ATMOS treats the atmospheric transport and deposition of material. EARLY models the effects on the surrounding area during the emergency action period, which can be up to one week in length. CHRONC models the effects of the accident in the time following the end of the emergency action period. Atmospheric transport is modeled in ATMOS using a straight line Gaussian treatment of the plume. Plume depletion occurs during transport as a result of radioactive decay and deposition onto the ground. Wet and dry deposition are treated as independent processes in ATMOS. The same initial seed was used to generate the pseudorandom weather sampling for all of the MACCS calculations performed for this analysis. Thus, the weather pattern imposed by MACCS is the same for each release.

As mentioned earlier, five offsite consequences are used as risk measures in this report: (1) the mean number of early fatalities, (2) the mean number of latent fatalities, (3 and 4) the mean population dose over 50 and 1000 miles, and (5) the mean offsite costs. Each of these risk measures is reported per reactor-year of operation, as calculated using Equation 1.1. The conditional consequences calculated by MACCS form one of the inputs to this equation. The actual calculation is done with a PC-based spreadsheet, using Lotus 1-2-3 software.¹¹

2.4 Evaluation of Potential Containment Improvements

There remain two goals of the CPI program that affected the way in which potential containment improvements were evaluated. The first is simplicity: the SCET for each PDS should be small enough to allow the reader to visualize each path through the tree and to perform desired sensitivity calculations more quickly and easily than could be done with the full APET. The second is a desire that the results of the SCET analysis be as close as possible to the published draft NUREG-1150 results. The purpose for the first goal is clear enough. The purpose of the second is to lend a measure of credibility to the CPI analysis, since it is by nature a "simplified" evaluation of a very complex and uncertain problem. However, difficulties can arise in trying to meet both of these goals simultaneously. One such problem, the limitations of point estimate calculations of source terms, has already been discussed. Another closely related problem arises when attempting to determine the risk benefit of a potential containment improvement.

As an example, the authors considered the suggestion of intentional operator depressurization of the reactor to prevent DCH. Because this issue is examined more thoroughly in a later section of the report, it will only be briefly summarized here. The simple approach in evaluating this issue would be to merely eliminate high pressure melt ejection from the SCET by setting the conditional probability of the lower branch of event HPME to 1.0 for all cases. By doing this, vessel failure at other than low pressure is eliminated and early containment failure as a result of DCH is eliminated. The conditional probability of early containment failure from in-vessel steam explosion (α mode failure) increases, because α mode failure is judged by most experts to be more likely at low vessel pressure. However, the complete elimination of DCH by depressurization through the pressurizer pilot-operated relief valves (PORVs) is generally not predicted by the Zion APET constructed for the draft NUREG-1150 analysis. The reason for this is that opening the PORVs to depressurize the reactor vessel does not always reduce the vessel pressure sufficiently to prevent DCH. There must also be an accompanying break in the reactor coolant system (RCS) equivalent in size to a medium LOCA (S2 break), such as an S2 initiating event or induced failure of the pressurizer surge line. Another result predicted by the APET is that induced steam generator

tube rupture (SGTR) is eliminated by operator depressurization. In addition, opening the PORVs can significantly affect the conditional probability of vessel breach for some PDS groups.

The first means of coping with discrepancies between the SCET results and those predicted by the APET is once again the frequency output file showing the realized split fractions for each event in the APET, which is obtained by evaluating the APET in the sampling mode. For operator depressurization, Question 16 in the Zion APET was modified as follows (refer to Appendix A for the base case APET). First, the split fraction for this event was changed to allow a constant conditional probability of 1.0 for depressurization. Secondly, the sampling input file was modified so that Question 16 was not sampled

(refer to Reference 5 for the details of how to modify the sampling input file). This modification was necessary to prevent the sampling routine from overriding the split fraction specified in the APET. The modified APET was then evaluated with EVNTRE in the sampling mode (mode 3). The frequency output file generated by this evaluation was then used to adjust the split fractions in the SCET so that the results matched those from the APET evaluation as closely as possible. For example, in the LOCA PDS, the APET evaluation predicted that depressurization during core melt would eliminate induced SGTR. The split fraction for event SGTR in the SCET used to evaluate vessel depressurization was appropriately modified to reflect this. When the PSTEVT post-processor code⁸ became available, it provided a second, more automated, approach for handling this problem. The use of PSTEVT is discussed further in Section 4.

3. LOSS-OF-COOLANT ACCIDENTS

The LOCA PDS group at Zion is generally characterized by a loss of RCS integrity prior to the time of core uncover.⁴ However, two sequences initiated by a station blackout (SBO) are included in the SBO PDS even though they lead to induced failure of the reactor coolant pump (RCP) seals. Not all of the breaks depressurize the RCS enough to allow the low pressure injection system (LPIS) to inject. Therefore, some LOCA sequences will involve vessel failure at a pressure above the shutoff head of the LPIS pumps unless a temperature-induced break of the RCS occurs or the RCS is intentionally depressurized.

3.1 LOCA Core Damage Frequency

For this analysis, we have used the LOCA core damage frequency of 2.6×10^{-4} per reactor-year reported in Table 2.2-3 of Reference 4.

3.2 LOCA SCET Results

The base case LOCA SCET is shown in Figure 3.1. This SCET was constructed using the methodology outlined in Section 2. Table 3.1 shows the containment failure probabilities, conditional on the occurrence of the LOCA PDS, calculated from this SCET and compares them with the probabilities from Reference 4.

As this table shows, the conditional containment failure probabilities predicted by the SCET agree quite well with the published Zion results. Note that the most probable end states of the SCET involve either no containment failure or basemat melt-through (BMT). The conditional probability of early containment failure (at or near the time of reactor vessel breach) is low, with the majority of early failures caused by α mode steam explosions.

3.3 Base Case LOCA Consequence Results

The SCET end states were binned into accident progression bins and the ZISOR code was used to generate source terms for these bins as described in Section 2. The ZISOR source terms were then further binned in order to reduce the required number of MACCS calculations. Conditional consequences were then obtained for each accident progression bin group using the MACCS code. The Zion site data and

meteorological files were used for these calculations. The conditional consequences for each accident progression bin are shown in Table 3.2.

These conditional consequences are now used in Equation 1.1 to obtain the annual LOCA risk for Zion. Table 3.3 compares the calculated annual risk from the SCETs with the published values in Reference 4.

As shown in Table 3.3, the risk calculated from the SCET is significantly less than that obtained in Reference 4. Recall, however, that the conditional containment failure probabilities from the SCET do not differ greatly from the published values. Therefore, the reason(s) for the difference lie in one or more of the following areas. First, there could be a loss of information in binning the SCET end states into accident progression bins. Secondly, ZISOR could be underestimating the source terms. Finally, a later revision of the MACCS code was used for this report which differed from that used for Reference 4. This disparity could possibly lead to lower values for the chosen risk measures. Each of these possible causes is examined below.

The possibility that crucial source term information was lost in the binning process was examined by performing sensitivity studies on the cases where a choice had to be made from among allowed values of the binning parameters. This choice eliminated some of the parameters used in Reference 4. An example would be the time of core damage (dimension 12). In the Reference 4 binning scheme, this dimension had two allowable parameter values: early or late core damage (A or B, respectively). Since the SCET does not model the time of core damage, a choice had to be made between these two allowed values for this report. The decision was made to assign all end states a value of B in dimension 12 in the vector describing the accident progression bin. In the sensitivity study for this case, the opposite choice was made; all end states were binned with a value of A in dimension 12. The difference in binning for this case, as well as the others that were run, was found to have little or no effect on the annual LOCA risk. Thus, the limitations of the SCET binning scheme do not appear to significantly influence the annual risk for the LOCA PDS.

The next possibility is that ZISOR could be underestimating the source term for the dominant accident progression bins. As discussed in Section 2.2, only point estimates of the source terms could be obtained from ZISOR. This leads to the use of source terms that

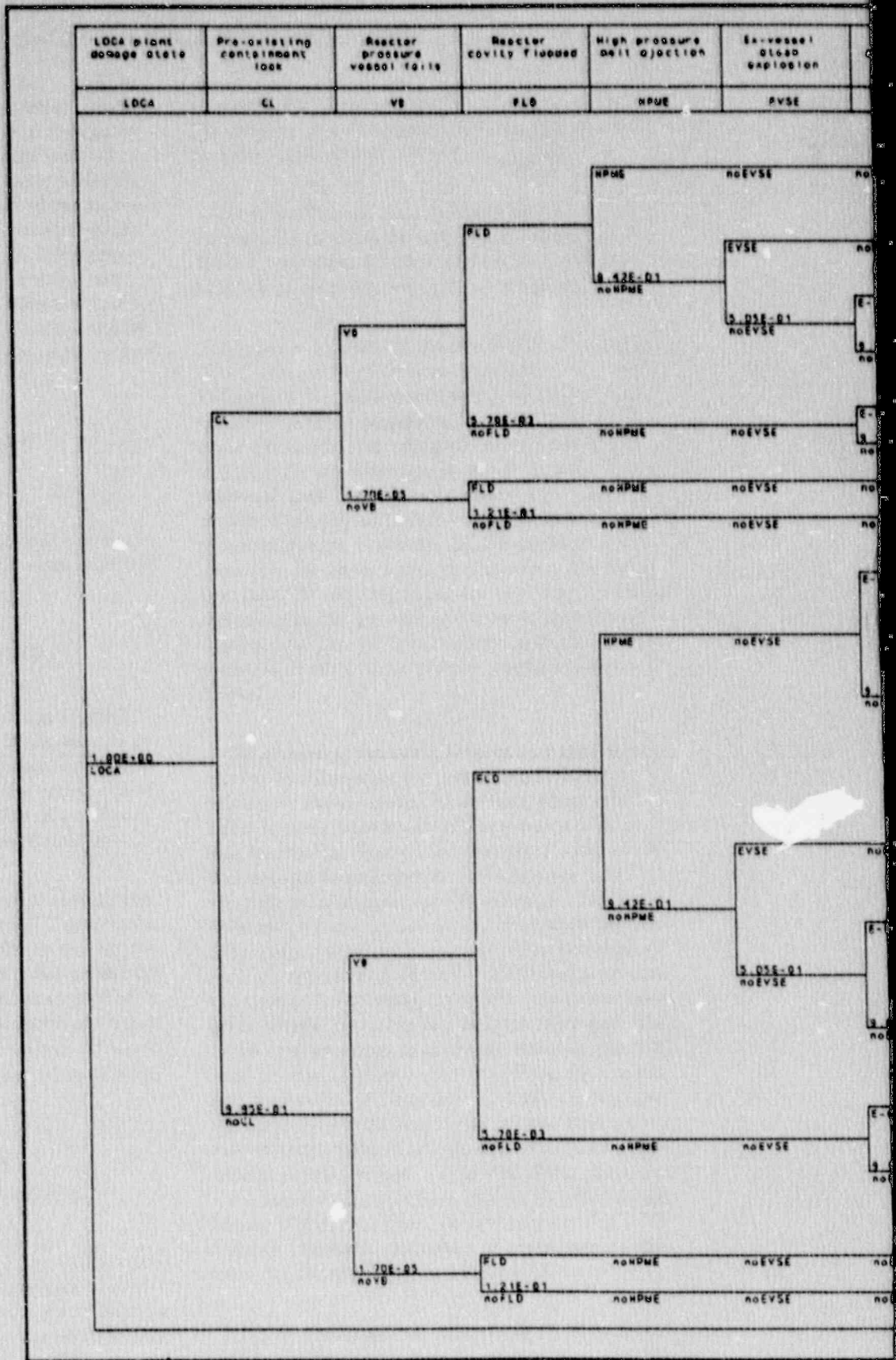


Figure 3.1. Base case LOCA SCET.

Early Failure	Sprays after residual breach	Core-concrete interaction	Late overpressure containment failure	Backmat melt- through	SEQUENCE PROB.	SEQUENCE CLASS	SEQUENCE DESCRIPTION
B-CF	SPRY	CCI	L-OP	NOBT			
		CCI	NOL-OP	NOBMT	1.44E-04	ECF-CL-D	EDDCACBACBB
	SPRY	5.00E-01 NOCCI	NOL-OP	NOBMT	1.44E-04	ECF-CL-D	EDCCACDBACBB
		CCI	NOL-OP	NOBMT	1.16E-03	ECF-CL	EDDBCBDDCBB
	SPRY	5.00E-01 NOCCI	NOL-OP	NOBMT	1.16E-03	ECF-CL	EDCDBCDDBCB
	ROSPRY	CCI	NOL-OP	NOBMT	3.55E-05	ECF-CL-A	DAFDCCBDDBB
		CCI	NOL-OP	NOBMT	2.09E-03	ECF-CL	EDDBCBADCB
	SPRY	1.02E-01 NOCCI	NOL-OP	NOBMT	2.38E-04	ECF-CL	EDCDBCDDBCB
	ROSPRY	CCI	NOL-OP	NOBMT	2.28E-07	ECF-CL-A	DAADCCBDDBB
		CCI	NOL-OP	NOBMT	2.63E-05	ECF-CL	EDADCBADCB
	SPRY	NOCCI	NOL-OP	NOBMT	7.47E-08	ECF-CL	EDCAFCDDBCB
	SPRY	NOCCI	NOL-OP	NOBMT	1.03E-08	ECF-CL	EDCAFCDDBCB
		CCI	NOL-OP	NOBMT	7.17E-04	ECF-B	EDDCACBACBB
	SPRY	5.00E-01 NOCCI	NOL-OP	NOBMT	7.17E-04	ECF-B	EDCCACDBACBB
			L-OP	NOBMT	2.52E-05	LCF	EDDCACBACBB
		CCI		BMT	5.59E-03	BMT	FDDCACBAEBB
			9.99E-01 NOL-OP	NOBMT	2.24E-02	NCF	GDDCACABGBB
				8.00E-01 NOBMT			
		5.00E-01 NOCCI	NOL-OP	NOBMT	2.80E-02	NCF	GDDCACBAGBB
			L-OP	NOBMT	2.08E-04	LCF	EDDBCBDDCBB
		CCI		BMT	8.21E-02	BMT	FDDCBDBEBB
			9.99E-01 NOL-OP	NOBMT	1.61E-01	NCF	GDDCBDBDGBB
				7.00E-01 NOBMT			
		5.00E-01 NOCCI	NOL-OP	NOBMT	2.31E-01	NCF	GDDCBDBDGBB
	ROSPRY	CCI	NOL-OP	NOBMT	7.06E-03	ECF-A	DAFDCCBDDBB
			L-OP	NOBMT	3.75E-04	LCF	EDDBCBADCB
		CCI		BMT	1.66E-01	BMT	FDDCBADDEBB
			9.99E-01 NOL-OP	NOBMT	2.50E-01	NCF	GDDCBADDEBB
				6.00E-01 NOBMT			
		1.02E-01 NOCCI	NOL-OP	NOBMT	4.73E-02	NCF	GDDCBADDEBB
	ROSPRY	CCI	NOL-OP	NOBMT	4.54E-05	ECF-A	DAADCCBDDBB
			L-OP	NOBMT	5.06E-06	LCF	EDADCBADCB
		CCI		BMT	2.25E-03	BMT	FDAADCBDEBB
			9.99E-01 NOL-OP	NOBMT	3.37E-03	NCF	GDAADCBDEBB
				6.00E-01 NOBMT			
	SPRY	NOCCI	NOL-OP	NOBMT	1.49E-05	NCF	GDCAFCDDBGB
	SPRY	NOCCI	NOL-OP	NOBMT	7.05E-06	NCF	GDCAFCDDBGB

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Table 3.1. Conditional containment failure probabilities for the LOCA PDS

Containment Failure Mode	Conditional Probability (SCET)	Conditional Probability (Reference 4)
No containment failure	7.43E-01	7.37E-01
DCH	1.43E-03	1.71E-03
α mode	7.15E-03	8.15E-03
Bypass ^a	4.97E-03	5.00E-03
Late overpressure	6.13E-04	8.78E-04
BMT	2.43E-01	2.46E-01

a. Bypass failures involve pre-existing leakage.

Table 3.2. Conditional consequences for the LOCA accident progression bins

Accident Progression Bin	Mean Early Fatalities	Mean Latent Fatalities	Mean 50-Mile Dose (person-rem)	Mean 1000-Mile Dose (person-rem)	Mean Offsite Costs (\$)
CDADBCABDCBB	1.1E+00	6.0E+02	1.7E+06	3.51E+06	6.3E+08
CDCAFCDBDCBB	6.3E-01	1.2E+02	3.9E+05	6.63E+05	6.2E+07
CDCCACDBACBB	7.3E-01	4.0E+02	1.1E+06	2.31E+06	3.1E+08
CDCDBCDBDCBB	1.1E+00	6.0E+02	1.7E+06	3.51E+06	6.3E+08
CDDCACABACBB	7.3E-01	4.0E+02	1.1E+06	2.31E+06	3.1E+08
CDDDBCABDCBB	1.1E+00	6.0E+02	1.7E+06	3.51E+06	6.3E+08
CDDDBCBBDCBB	1.1E+00	6.0E+02	1.7E+06	3.51E+06	6.3E+08
DAADDCBBDBBB	1.4E+00	1.4E+03	3.6E+06	9.09E+06	3.2E+09
DAFDDCBBDBBB	1.4E+00	1.5E+03	3.7E+06	9.28E+06	3.1E+09
DDCCACDBACBB	6.2E-01	1.4E+02	4.3E+05	7.77E+05	8.0E+07
DDDCACABACBB	6.2E-01	1.4E+02	4.3E+05	7.77E+05	8.0E+07
EDADBCABDCBB	0.0E+00	3.1E+00	1.0E+04	1.91E+04	2.9E+05
EDDCACABACBB	0.0E+00	3.1E+00	1.0E+04	1.91E+04	2.9E+05
EDDDBCABDCBB	0.0E+00	3.1E+00	1.0E+04	1.91E+04	2.9E+05
EDDDBCBBDCBB	0.0E+00	3.1E+00	1.0E+04	1.91E+04	2.9E+05
FDADBCABDEBB	0.0E+00	8.6E-03	2.3E+01	5.17E+01	0.0E+00
FDDCACABAEBB	0.0E+00	8.6E-03	2.3E+01	5.17E+01	0.0E+00
FDDDBCABDEBB	0.0E+00	8.6E-03	2.3E+01	5.17E+01	0.0E+00
FDDDBCBBDEBB	0.0E+00	8.6E-03	2.3E+01	5.17E+01	0.0E+00
GDADBCABDGGB	0.0E+00	2.6E-02	9.7E+01	1.38E+02	0.0E+00
GDCAFCBBDGGB	0.0E+00	2.6E-02	9.7E+01	1.38E+02	0.0E+00
GDCCACDBAGGB	0.0E+00	2.6E-02	9.7E+01	1.38E+02	0.0E+00
GDCDBCBDGGB	0.0E+00	2.6E-02	9.7E+01	1.38E+02	0.0E+00
GDDCACABAGGB	0.0E+00	2.6E-02	9.7E+01	1.38E+02	0.0E+00
GDDDBCABDGGB	0.0E+00	2.6E-02	9.7E+01	1.38E+02	0.0E+00
GDDDBCBBDGGB	0.0E+00	2.6E-02	9.7E+01	1.38E+02	0.0E+00

Table 3.3. Annual base case LOCA risk

	Mean Early Fatalities	Mean Latent Fatalities	Mean 50-Mile Dose (person-rem)	Mean 1000-Mile Dose (person-rem)	Mean Offsite Costs (\$)
SCET	4E-06	4E-03	9	22	6.48E+03
Reference 4	9E-05	2E-02	41	98	1.31E+05
Relative change	22.5	5	4.6	4.5	20.2

are in many cases one or more orders of magnitude less than the mean source terms calculated in Reference 4. However, there is also the possibility that errors could exist in the version of ZISOR used for the SCET analysis in this report. To check this possibility, ZISOR was run on the four sequences for which Reference 4 provides source term distributions. The results matched the point estimate ZISOR source terms shown in Reference 4. Thus, the version of ZISOR used for this report was considered to be working correctly.

The final possibility for the risk discrepancy is that the version of MACCS used for this analysis gives significantly different results than the version used for Reference 4. MACCS 1.5.11 was used for this analysis, whereas Reference 4 used MACCS 1.5.5. This possibility was discussed with an individual involved with verification of the MACCS code. This individual, who was familiar with the changes made to the code in progressing from version 1.5.5 to version 1.5.11, thought that the results between the two versions might differ by 5-10%, but was not certain how the magnitude of the results would change.

Based on these findings, the authors' conclusion is that the majority of the difference in magnitude between the SCET risk results and those published in Reference 4 is due to the limitations of using a point estimate source term rather than a mean value calculated from a distribution.

The contribution of each containment failure mode to the offsite population dose is listed in Table 3.4. As this table shows, α mode failure and bypass are the largest contributors to offsite dose for the LOCA PDS. DCH and late failures are insignificant in comparison.

3.4 Risk Benefit of Potential Improvements

Reference 2 identified several improvements that have the potential to enhance the performance of the large dry PWR containment during a severe accident. These improvements are as follows: (1) enhanced reactor vessel depressurization capability, (2) cavity flooding, (3) prevention of hydrogen burns in containment, (4) containment venting, and (5) modifications to reduce the frequency of the interfacing systems LOCA (V sequence). The risk benefit of the first four of these improvements is evaluated in this section for the LOCA PDS. The fifth improvement is not evaluated, as it affects only the V sequence, not the conventional LOCA. No effort was made to evaluate the feasibility of any of the potential improvements, because this was beyond the scope of the project.

3.4.1 Enhanced Reactor Vessel Depressurization Capability. The ability to depressurize the reactor vessel during a severe core damage accident is desirable from the standpoint of reducing the threat to containment integrity presented by DCH at the time of vessel failure. Intentional depressurization was modeled in the LOCA SCET by eliminating event HPME from consideration; all sequences in which core damage is not arrested involve vessel failure at low pressure (<200 psig) only. In the Zion APET, vessel failure at low pressure does not present a DCH challenge to the containment. Figure 3.2 shows the SCET that models intentional depressurization.

Table 3.5 shows the change in the conditional containment failure probabilities effected by intentional depressurization.

**Table 3.4. Containment failure mode contribution to offsite dose for the LOCA PDS
(from SCET)**

Containment Failure Mode	Contribution to 50-Mile Dose (%)	Contribution to 1000-Mile Dose (%)
DCH	1.7	1.3
α	74.5	78.3
Bypass ^a	23.6	20.2
Late overpressure	ε	ε
BMT	ε	ε

a. Bypass failures involve pre-existing containment leakage.

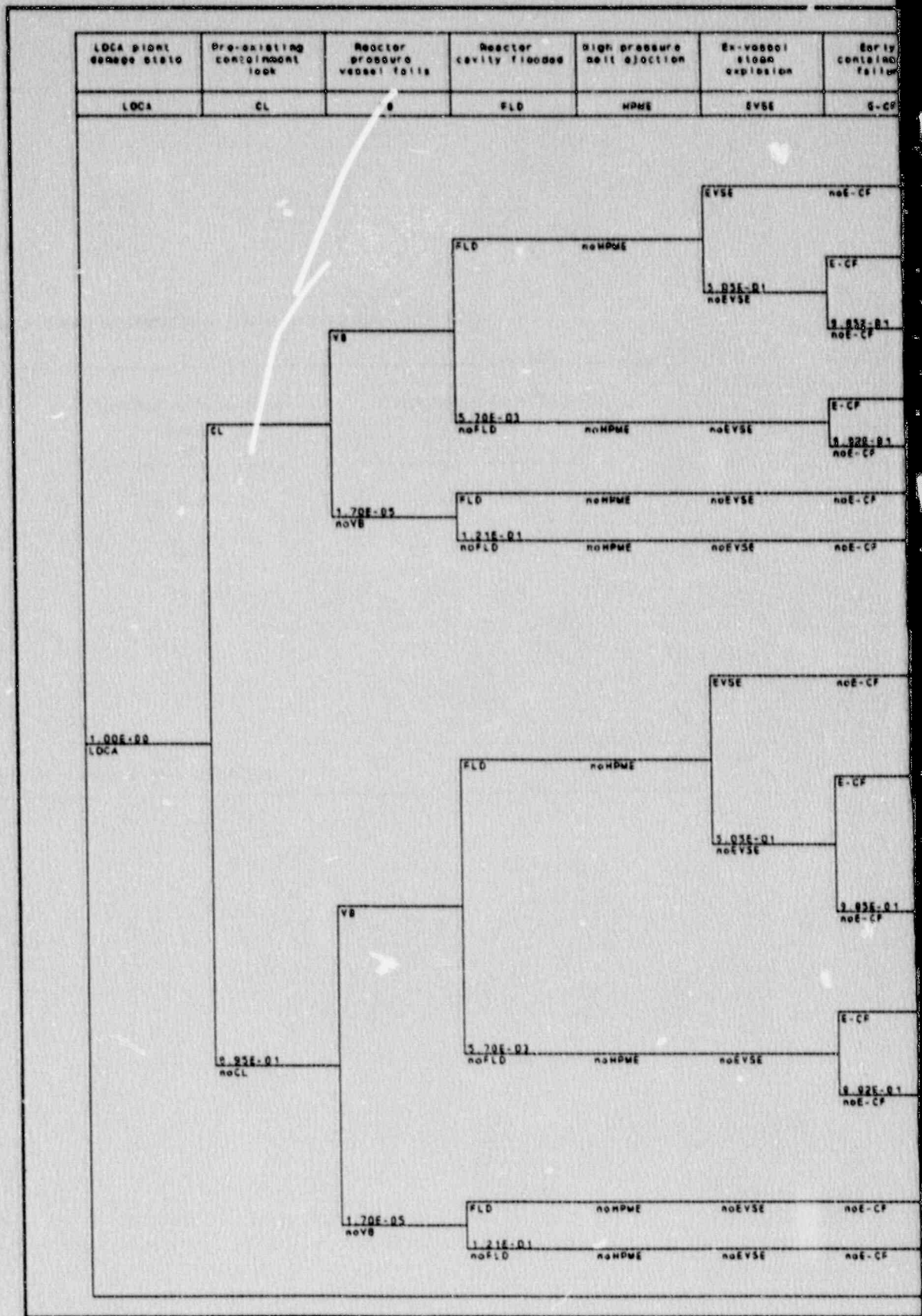


Figure 3.2. LOCA SCET for intentional depressurization.

SPRT	Sprays after vesical breach	Core-concrete interaction	Load over exposure containment failure	Basemat Goit-Through	SEQUENCE PROB.	SEQUENCE CLASS	SEQUENCE DESCRIPTION
	SPRT	CCI	L-OP	NOBT			
		CCI	NOL-OP	NOBT	1.23E-03	ECF-CL	CDDDBCBUCBB
SPRT	3.02E-01 NOCCI		NOL-OP	NOBT	1.23E-03	ECF-CL	CDCDBCBUCBB
NOSPRY	CCI		NOL-OP	NOBT	3.77E-05	ECF-CL-A	DAFDCCBUCBB
		CCI	NOL-OP	NOBT	2.22E-03	ECF-CL	CDDDBCBUCBB
SPRT	1.02E-01 NOCCI		NOL-OP	NOBT	2.52E-04	ECF-CL	CDCDBCBUCBB
NOSPRY	CCI		NOL-OP	NOBT	2.78E-07	ECF-CL-A	DAADCCBUCBB
SPRT	CCI		NOL-OP	NOBT	2.83E-05	ECF-CL	CDADCCBUCBB
SPRT	NOCCI		NOL-OP	NOBT	7.47E-08	ECF-CL	GDCAFCCBUCBB
SPRT	NOCCI		NOL-OP	NOBT	1.03E-08	ECF-CL	GDCAFCCBUCBB
			L-OP	NOBT	2.20E-04	LCF	EEDDBCBUCBB
		CCI	3.88E-01 NOL-OP	NOBT	7.34E-02	BMT	FDDDBCBUCBB
SPRT				3.02E-01 NOBT	1.71E-01	NCF	GDDDBCBUCBB
		CCI	3.02E-01 NOCCI		2.45E-01	NCF	GDCDBCBUCBB
NOSPRY	CCI		NOL-OP	NOBT	7.49E-03	ECF-A	DAFDCCBUCBB
			L-OP	NOBT	3.98E-04	LCF	EEDDBCBUCBB
		CCI	3.88E-01 NOL-OP	NOBT	1.77E-01	BMT	FDDDBCBUCBB
SPRT				3.02E-01 NOBT	2.63E-01	NCF	GDDDBCBUCBB
		CCI	3.02E-01 NOCCI		3.02E-02	NCF	GDCDBCBUCBB
NOSPRY	CCI		NOL-OP	NOBT	4.54E-05	ECF-A	DAADCCBUCBB
			L-OP	NOBT	5.06E-06	LCF	EDADCCBUCBB
SPRT	CCI			NOBT	2.25E-03	BMT	FADDBCBUCBB
			3.88E-01 NOL-OP	NOBT	3.37E-03	NCF	GDADCCBUCBB
SPRT	NOCCI		NOL-OP	NOBT	1.49E-05	NCF	GDCAFCCBUCBB
SPRT	NOCCI		NOL-OP	NOBT	2.05E-06	NCF	GDCAFCCBUCBB

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As shown in Table 3.5, depressurization eliminates DCH failures, but leads to a slight increase in the conditional probability of α mode failure. This occurs because the mean probability of α mode failure used in the Zion APET is a factor of ten higher at low vessel pressure (8×10^{-3}) than it is at intermediate or higher pressures (8×10^{-4}). The effect of this change in containment failure probability on risk is shown in Table 3.6.

Because of the increase in the conditional probability of α mode containment failure brought about by depressurization, there is a small increase in all risk measures apart from the mean number of early fatalities, which decrease very slightly. To reiterate, this result hinges upon the pressure-dependent probabilities assigned to α mode failure in the Zion APET (Question 34). These probabilities are highly uncertain; therefore, the effect

of this dependence on vessel pressure was examined in a sensitivity case in which the conditional probability of α mode containment failure, given vessel breach, was taken to be 8.0×10^{-4} (the mean value with the RCS at high pressure), regardless of vessel pressure at the time of vessel failure. This revised APET was evaluated with no sampling on Question 34 (α mode containment failure) and the split fractions in the LOCA SCET with depressurization were adjusted to fit the results of the frequency output file generated by EVNTRE. This revised SCET is shown in Figure 3.3.

The revised conditional containment failure probabilities from this SCET are shown in Table 3.7.

As expected, the conditional probability of α mode failure is significantly reduced (by one order of magnitude) from the base case probability. The calculated risk for this case is shown in Table 3.8.

Table 3.5. Conditional containment failure probabilities for the LOCA PDS with operator depressurization

Containment Failure Mode	Conditional Probability (SCET)	Base Case Conditional Probability
No containment failure	7.35E-01	7.43E-01
DCH	0.00	1.43E-03
α mode	7.58E-03	7.15E-03
Bypass ^a	4.96E-03	4.97E-03
Late overpressure	6.23E-04	6.13E-04
BMT	2.53E-01	2.43E-01

a. Bypass failures involve pre-existing containment leakage.

Table 3.6. Annual LOCA risk with operator depressurization

	Mean Early Fatalities	Mean Latent Fatalities	Mean 50-Mile Dose (person-rem)	Mean 1000-Mile Dose (person-rem)	Mean Offsite Costs (\$)
Base case	4.11E-06	3.47E-03	9.22	21.74	6.48E+03
SCET	4.06E-06	3.59E-03	9.52	22.58	6.82E+03
% change	-1.22	3.46	3.25	3.86	5.25

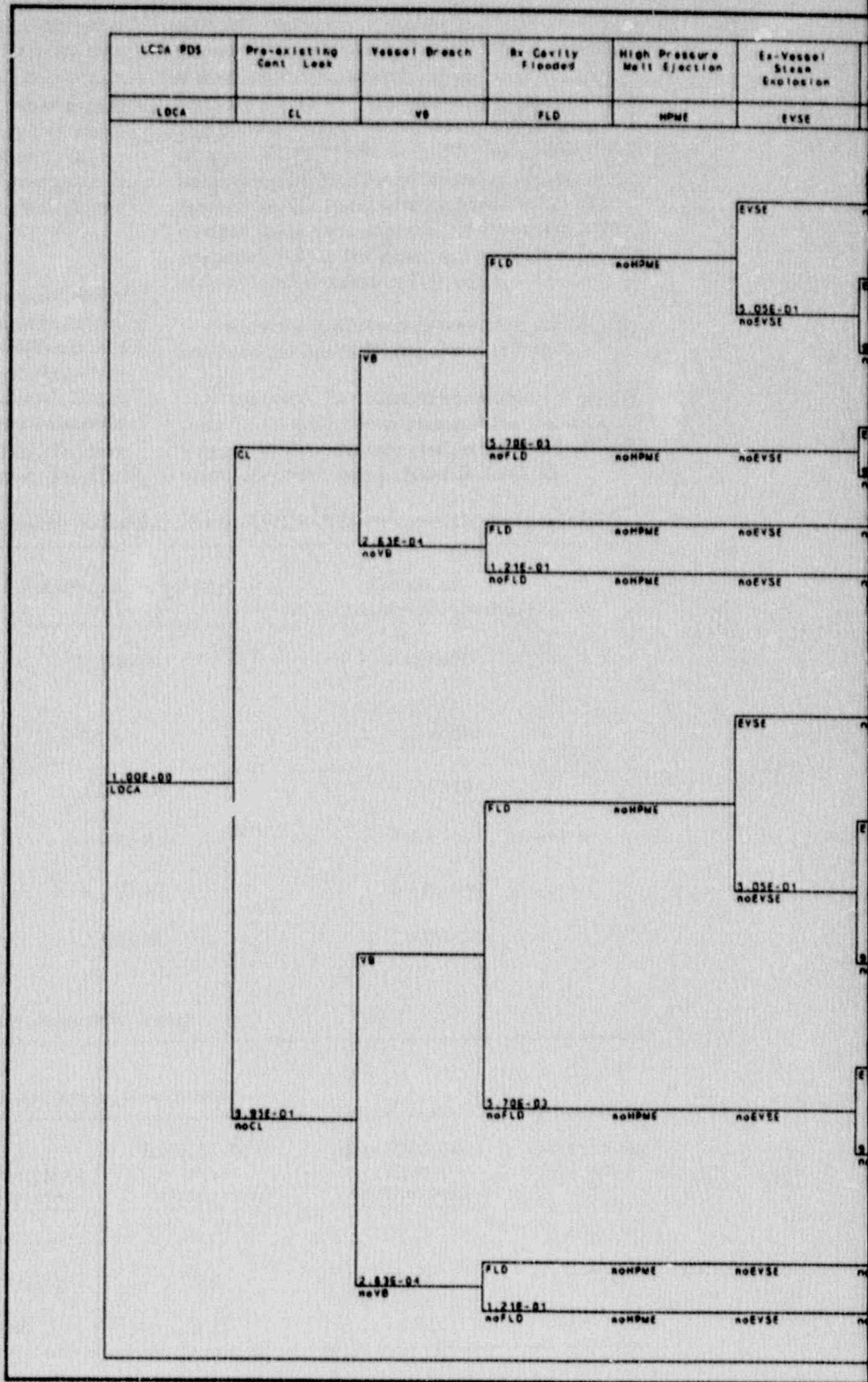


Figure 3.3. LOCA SCET with depressurization and no increase in α mode failure.

Early Cont Failure	Cont. Sprays after VB	Core-Concrete Interaction	Late OP Cont. Failure	Basemat Bolt-Through	SEQUENCE PROB	SEQUENCE CLASS	SEQUENCE DESCRIPTION
E-CF	SPRY	CCI	L-OP	BMT			
		CCI	NOL-OP	NOBMT	1.23E-03	ECF-CL	CDDBCB00C00
E-LF	SPRY	5.00E-01 NOCCI	NOL-OP	NOBMT	1.23E-03	ECF-CL	CDDBCB00C00
CF	NOSPRY	CCI	NOL-OP	NOBMT	3.77E-06	ECF-CL-A	DAFDCC00000
		CCI	NOL-OP	NOBMT	2.25E-03	ECF-CL	CDDBCB00C00
BBE-D1 E-CF	SPRY	1.02E-01 NOCCI	NOL-OP	NOBMT	2.56E-04	ECF-CL	CDDBCB00C00
CF	NOSPRY	CCI	NOL-OP	NOBMT	2.20E-08	ECF-CL-A	DAADCC00000
BBE-D1 E-CF	SPRY	CCI	NOL-OP	NOBMT	2.85E-05	ECF-CL	CDADCB00C00
E-CF	SPRY	NOCCI	NOL-OP	NOBMT	1.16E-08	ECF-CL	CDCAFCD00C00
E-CF	SPRY	NOCCI	NOL-OP	NOBMT	1.59E-07	ECF-CL	CDCAFCD00C00
			L-OP	NOBMT	2.20E-04	LCF	EDDBCB00C00
		CCI		BMT	7.34E-02	BMT	FDDDBCB00E00
E-CF	SPRY		9.00E-01 NOL-OP	7.00E-01 NOBMT	1.71E-01	NCF	GDDDBCB00G00
		5.00E-01 NOCCI	NOL-OP	NOBMT	2.45E-01	NCF	GDDDBCB00G00
CF	NOSPRY	CCI	NOL-OP	NOBMT	7.49E-04	ECF-A	DAFDCC00000
			L-OP	NOBMT	4.03E-04	LCF	EDDDBCB00C00
		CCI		BMT	1.79E-01	BMT	FDDDBCB00E00
BBE-D1 E-CF	SPRY		9.00E-01 NOL-OP	8.00E-01 NOBMT	2.88E-01	NCF	GDDDBCB00G00
		1.02E-01 NOCCI	NOL-OP	NOBMT	5.09E-01	NCF	GDDDBCB00G00
CF	NOSPRY	CCI	NOL-OP	NOBMT	4.54E-06	ECF-A	DAADCC00000
			L-OP	NOBMT	5.10E-04	LCF	EDADCB00C00
BBE-D1 E-CF	SPRY	CCI		BMT	2.26E-03	BMT	FDADCB00E00
			3.23E-01 NOL-OP	8.00E-01 NOBMT	3.40E-03	NCF	GDADCB00G00
E-CF	SPRY	NOCCI	NOL-OP	NOBMT	2.30E-04	NCF	GDCAFCD00G00
E-CF	SPRY	NOCCI	NOL-OP	NOBMT	3.17E-05	NCF	GDCAFCD00G00

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Table 3.7. Conditional containment failure probabilities for the LOCA PDS with operator depressurization and pressure-independent point estimate probability of α mode containment failure

<u>Containment Failure Mode</u>	<u>Revised Conditional Probability</u>	<u>Base Case Conditional Probability</u>
No containment failure	7.47E-01	7.43E-01
DCH	1.43E-03	1.43E-03
α mode	7.15E-04	7.15E-03
Bypass ^a	5.00E-03	4.97E-03
Late overpressure	6.18E-04	6.13E-04
BMT	2.46E-01	2.43E-01

a. Bypass failures involve pre-existing containment leakage.

Table 3.8. Annual LOCA risk with depressurization and pressure-independent point estimate probability of α mode failure

	<u>Mean Early Fatalities</u>	<u>Mean Latent Fatalities</u>	<u>Mean 50-Mile Dose (person-rem)</u>	<u>Mean 1000-Mile Dose (person-rem)</u>	<u>Mean Offsite Costs (\$)</u>
Base case	4.11E-06	3.47E-03	9.22	21.74	6.48E+03
SCET	1.70E-06	1.06E-03	2.98	6.35	1.41E+03
% change	-58.6	-69.6	-67.6	-70.8	-78.2

The risk reduction associated with the decreased probability of α mode failure is significant. Because α mode failure is so dominant in the base case, this issue was investigated in a sensitivity case, where the effects of depressurization were isolated from the uncertainty in the α mode failure probability. This investigation was conducted by rerunning the base case with the conditional probability of α mode failure in the APET set at the point estimate value of 8.0×10^{-4} , with no sampling on Question 34. (It is possible, of course, that the probabilities of α mode failure in NUREG-1150 are too low, in which case α mode failure would be an even larger contributor to offsite risk.) The conditional containment failure probabilities for this sensitivity are shown in Table 3.9.

The annual offsite risk for this sensitivity case is shown in Table 3.10.

3.4.2 Cavity Flooding. The purpose of a reactor cavity flooding system would be to provide a flow of water to core debris released into the cavity at the time of vessel failure. This water could quench the debris and maintain a coolable configuration, reducing CCI and mitigating any gradual overpressurization of con-

tainment caused by the production of noncondensable gases during CCI. The flooded cavity could cause an initial steam spike at the time of vessel failure, but would not be expected to threaten containment integrity.² In analyzing the cavity flooding improvement, the base case probabilities of having a coolable debris geometry were retained. These probabilities are listed here for reference: (1) given α mode or rocket failure of the containment, the probability is 0.7 that the debris is coolable, (2) given KPME or EVSE, there is a 50% chance that the debris is coolable, (3) if there is a gravity pour with no EVSE, there is only a 10% chance of having a coolable debris geometry.

The revised SCET used to analyze the additional cavity flooding system is shown in Figure 3.4. Table 3.11 shows the conditional containment failure probabilities calculated from this tree.

As Table 3.11 shows, the addition of a cavity flooding system does not significantly affect the conditional probabilities of containment failure. The reason for this is that most of the base case LOCA sequences already have a wet cavity at the time of vessel breach; therefore, there is little effect from the addition of the flooding system. The effects on risk are shown in Table 3.12.

Table 3.9. Conditional containment failure probabilities for the base case LOCA PDS with pressure-independent point estimate probability of α mode containment failure

Containment Failure Mode	Revised Conditional Probability	Base Case Conditional Probability
No containment failure	7.39E-01	7.35E-01
DCH	0.00	0.00
α mode	7.58E-04	7.58E-03
Bypass ^a	5.00E-03	4.96E-03
Late overpressure	6.28E-04	6.23E-04
BMT	2.55E-01	2.53E-01

a. Bypass failures involve pre-existing containment leakage.

Table 3.10. Annual base case LOCA risk with pressure-independent point estimate probability of α mode failure

	Mean Early Fatalities	Mean Latent Fatalities	Mean 50-Mile Dose (person-rem)	Mean 1000-Mile Dose (person-rem)	Mean Offsite Costs (\$)
Base case	4.11E-06	3.47E-03	9.22	21.74	6.48E+03
SCET	1.88E-06	1.08E-03	3.05	6.44	1.38E+03
% change	-54.3	-68.9	-66.9	-70.4	-78.7

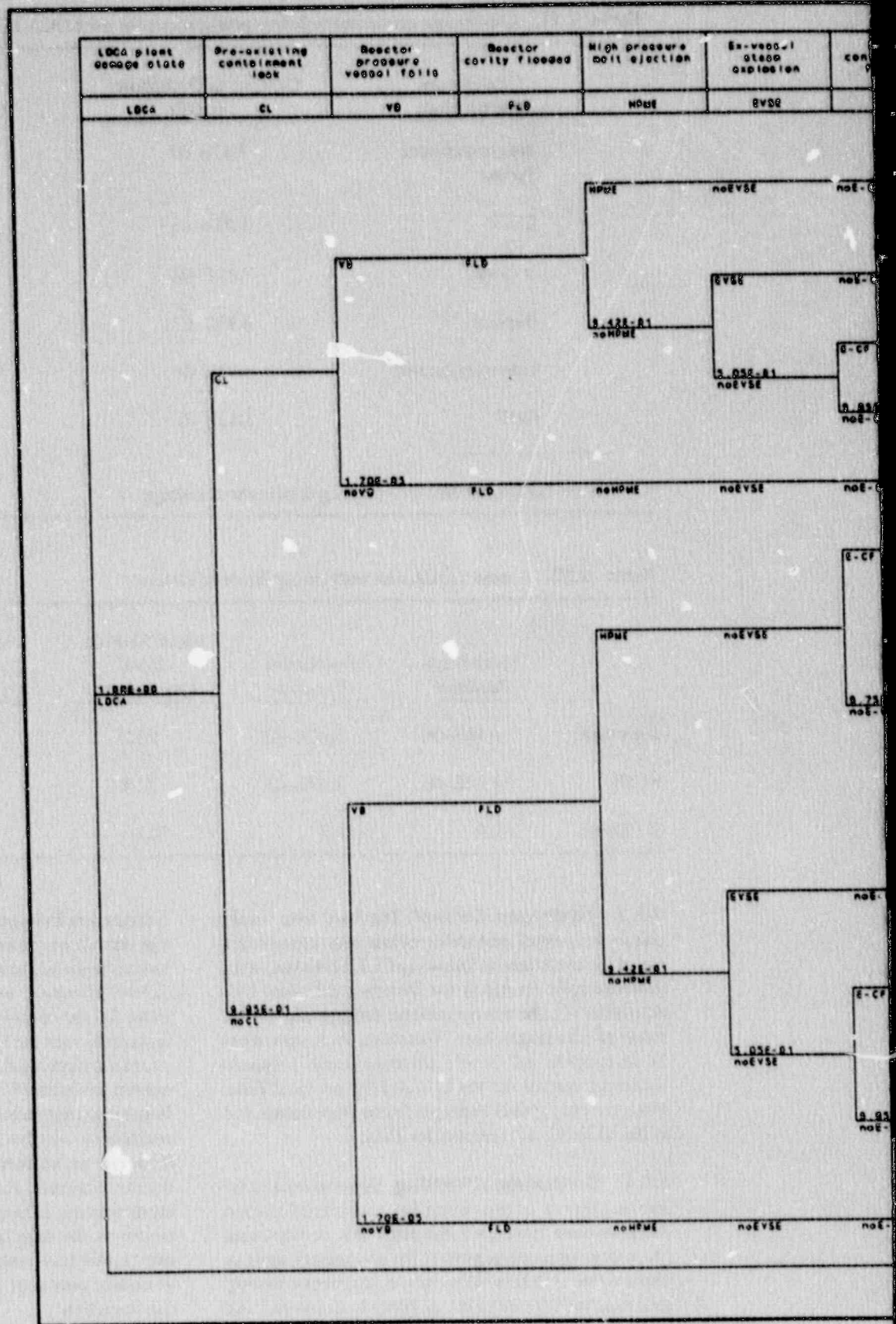


Figure 3.4. LOCA SCET with addition of cavity flooding system.

SPRY	Sequence after spray break	Zero-concrete interaction	Late pressure concurrent failure	Excess water- through	SEQUENCE NO.	SEQUENCE CLASS	SEQUENCE DESCRIPTION
CP	SPRY	ZEI	L-OP	DMT			
SPRY	CCI	NOL-OP	ROBMT		1.45E-04	ECF-CL-B	CDCCACBACB0
	3.0DE-B1 NOCCI	NOL-OP	ROBMT		1.45E-04	ECF-CL-B	CDCCACBACB0
SPRY	CCI	NOL-OP	ROBMT		1.17E-03	ECF-CL	CDCCBDDDC00
	3.0DE-B1 NOCCI	NOL-OP	ROBMT		1.17E-03	ECF-CL	CDCCBDDDC00
ROSPRY	CCI	NOL-OP	ROBMT		3.57E-03	ECF-CL-A	DADDDCBDD00
SPRY	CCI	NOL-OP	ROBMT		2.10E-03	ECF-CL	CDDBCAQDC00
	1.0DE-B1 NOCCI	NOL-OP	ROBMT		2.39E-04	ECF-CL	CDDBCAQDC00
SPRY	NOCCI	NOL-OP	ROBMT		0.50E-00	ECF-CL	CDCAFCD0000
SPRY	CCI	NOL-OP	ROBMT		7.21E-04	ECF-B	DDCCACBACB0
	5.0DE-B1 NOCCI	NOL-OP	ROBMT		7.21E-04	ECF-B	DDCCACBACB0
SPRY	CCI	NOL-OP	L-OP	ROBMT	2.53E-03	LCP	EDDCACAD000
			8.85E-B1 NOL-OP	DMT	5.62E-03	DMT	FDDCACB0000
SPRY	CCI	NOL-OP	ROBMT	8.0DE-B1 NOCCI	2.25E-02	NCF	GDCCACB0000
				5.0DE-B1 NOCCI	2.81E-02	NCF	GDCCACB0000
SPRY	CCI	NOL-OP	L-OP	ROBMT	2.09E-04	LCP	EDDBCBDD000
			8.85E-B1 NOL-OP	DMT	6.85E-02	DMT	FDDDBCB0000
SPRY	CCI	NOL-OP	ROBMT	7.0DE-B1 NOCCI	1.67E-01	NCF	GDDBCBDD000
				5.0DE-B1 NOCCI	2.52E-01	NCF	GDDBCBDD000
ROSPRY	CCI	NOL-OP	ROBMT		7.10E-03	ECF-A	DADDDCBDD00
SPRY	CCI	NOL-OP	L-OP	ROBMT	3.77E-04	LCP	EDDBCBDD000
			8.85E-B1 NOL-OP	DMT	1.67E-01	DMT	FDDDBCB0000
SPRY	CCI	NOL-OP	ROBMT	2.0DE-B1 NOCCI	2.51E-01	NCF	GDDBCBDD000
				1.0DE-B1 NOCCI	4.76E-02	NCF	GDDBCBDD000
SPRY	NOCCI	NOL-OP	ROBMT		1.69E-05	NCF	GDCAFCD00000

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Table 3.11. Conditional containment failure probabilities for the LOCA PDS with cavity flooding system

Containment Failure Mode	Conditional Probability (SCET)	Base Case Conditional Probability
No containment failure	7.43E-01	7.43E-01
DCH	1.43E-03	1.44E-03
α mode	7.15E-04	7.14E-03
Bypass ^a	4.97E-03	4.97E-03
Late overpressure	6.13E-04	6.11E-04
BMT	2.43E-01	2.42E-01

a. Bypass failures involve pre-existing containment leakage.

Table 3.12. Annual LOCA risk with cavity flooding system

	Mean Early Fatalities	Mean Latent Fatalities	Mean 50-Mile Dose (person-rem)	Mean 1000-Mile Dose (person-rem)	Mean Offsite Costs (\$)
Base case	4.11E-06	3.47E-03	9.22	21.74	6.48E+03
SCET	4.03E-06	2.47E-03	7.18	14.56	2.92E+03
% change	-1.9	-28.7	-22.0	-33.0	-55.0

3.4.3 Hydrogen Control. The base case results show a very small probability of late overpressure failure of the containment following CCI. However, as the realized split fractions for Question 62 show (see Appendix B), these overpressure failures are not the result of a hydrogen burn. Therefore, there appears to be no tangible risk benefit associated with enhanced hydrogen control for the LOCA PDS group at Zion. Note that only global hydrogen burns were considered in the NUREG-1150 model for Zion.

3.4.4 Containment Venting. Because there is no internal means of providing for a scrubbed release when venting from the large dry PWR containment (such as a suppression pool or ice condenser), there is probably no risk benefit to early containment venting unless some type of external filter is employed. As

discussed in Reference 2, late venting may have potential benefit in preventing an uncontrolled release of radioactivity as a result of gross containment structural failure. However, because late overpressure containment failure contributes only marginally to the risk associated with the LOCA PDS, there is essentially no benefit to late unfiltered venting. In addition, the current revision of the Westinghouse Owners Group Emergency Procedure Guidelines does not direct the operator to vent the containment, because of the concern over an unfiltered release of fission products to the environment. An evaluation of filtered containment venting is beyond the scope of this analysis. However, the high cost associated with installing an external filtered containment venting system is likely to greatly outweigh any potential benefit in terms of risk reduction.

4. TRANSIENTS AND ANTICIPATED TRANSIENTS WITHOUT SCRAM (ATWS)

The transient + ATWS PDS group at Zion is made up of sequences in which the RCS is intact at the time of core uncover and AC power is available.

4.1 Transient + ATWS Core Damage Frequency

The transient + ATWS PDS group is composed of 11 individual plant damage states, with a combined frequency of 1.18×10^{-5} per reactor-year, as reported in Table 2.2-3 of Reference 4.

4.2 Transient + ATWS SCET Results

The base case transient + ATWS SCET is shown in Figure 4.1. This SCET, like the LOCA SCET, was constructed using the methodology outlined in Section 2. Table 4.1 shows the containment failure probabilities, conditional on the occurrence of the transient + ATWS PDS, calculated from this SCET and compares them with the probabilities from Reference 4.

The conditional containment failure probabilities calculated from the SCET agree well with those published in Reference 4. As in the LOCA PDS, early containment failure is not likely; however, in the transient + ATWS PDS, DCH is slightly more dominant than α mode failure. Late overpressure failures as a result of hydrogen burns are not likely and the probability of eventual overpressure failure from the buildup of steam and noncondensable gases is vanishingly small, as it was for LOCAs. Again, there is a significant chance of basemat melt-through. But as for LOCAs, the most probable end states involve a containment that is structurally intact.

4.3 Base Case Transient + ATWS Consequences

The SCET end states were binned into accident progression bins and the ZISOR code was again used to generate source terms for these bins (see description in Section 2). Conditional consequences were then calculated for each accident progression bin with the MACCS code. As for LOCAs, the Zion site data and meteorological files were used for these calculations. The conditional consequences for each accident progression bin are shown in Table 4.2.

These conditional consequences are used in Equation 1.1 to obtain the annual transient + ATWS risk for Zion. Table 4.3 compares the calculated annual risk from the SCETs with the published values in Reference 4.

As in the LOCA PDS, the risk calculated with the SCET is lower than the published values in Reference 4, although the differences are less in the transient + ATWS case. Again, this is to be expected when calculating risk based on point estimate source terms generated by ZISOR.

Table 4.4 lists the contribution of each containment failure mode to the offsite dose. DCH failures are slightly more significant than in the LOCA PDS. However, the largest contributor to offsite dose is now containment bypass instead of α mode failure, although the contribution from the latter is still significant. The contribution of α mode failure is down because the RCS is more likely to be at high pressure at the time of vessel breach than in the LOCA PDS group. Late containment failures are again an insignificant contributor to offsite dose.

4.4 Risk Benefit of Potential Improvements

The risk benefit of the potential containment performance improvements identified in Reference 2 is calculated in this section.

4.4.1 Enhanced Reactor Vessel Depressurization Capability. With no accompanying break equivalent in size to an S2 LOCA, opening the pressurizer PORVs at Zion has not been shown to reduce RCS pressure below the DCH cutoff used in Reference 4. However, if intentionally opening the PORVs induces a failure of the pressurizer surge line, then RCS pressure will be reduced to <200 psig at the time of vessel failure and DCH will not be a threat to containment integrity. To model intentional depressurization via the PORVs, an additional event to represent intentional opening of the PORVs could be added to the base case SCET and the conditional probability for the lower branch of this event could be changed to 0.0 to represent opening of the PORVs 100% of the time. Tests indicated that this approach was too simplistic because it did not model the dependencies between depressurization and vessel breach that exist in the APET. A number of attempts were made to

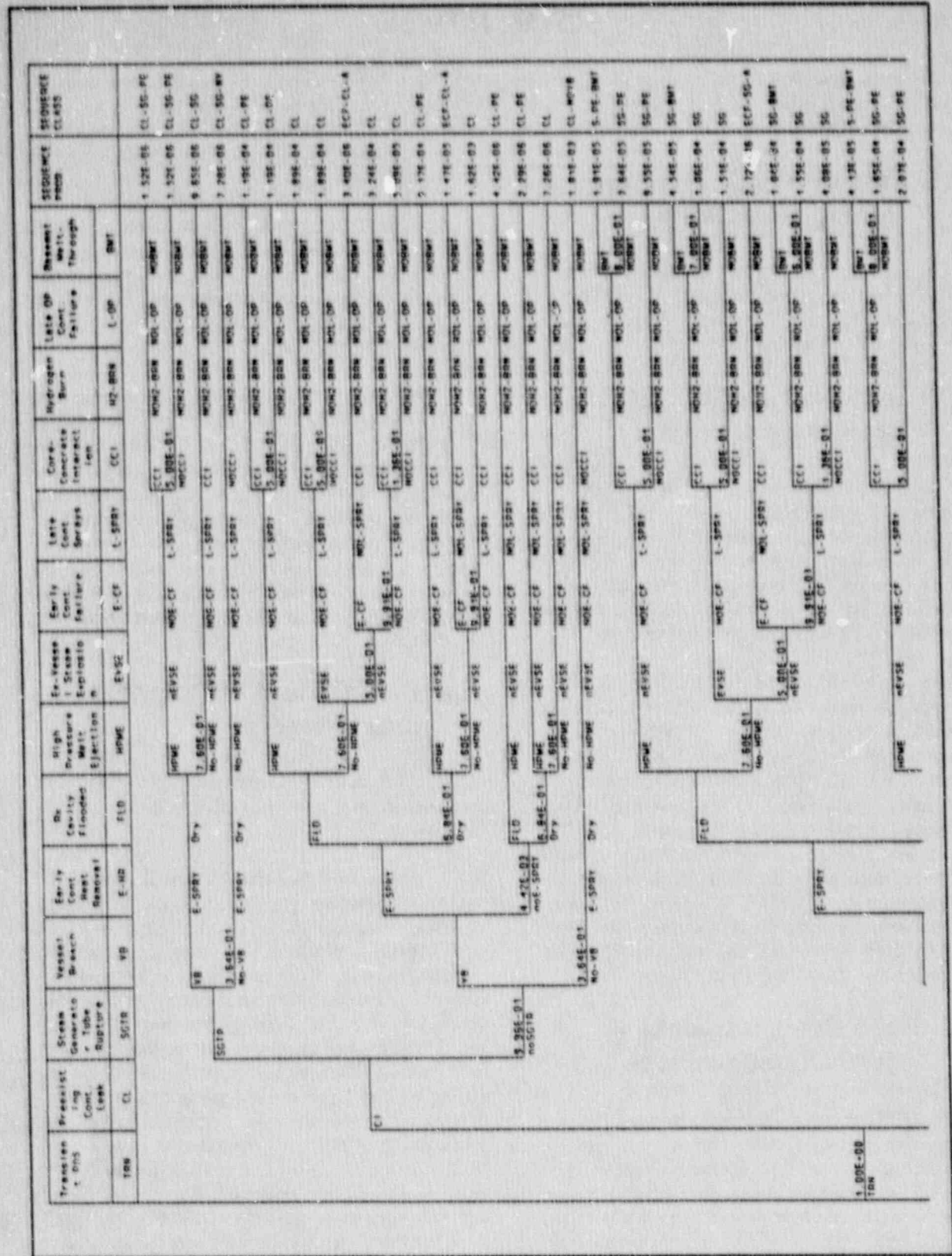


Figure 4.1. Base case transient + ATWS SCET.

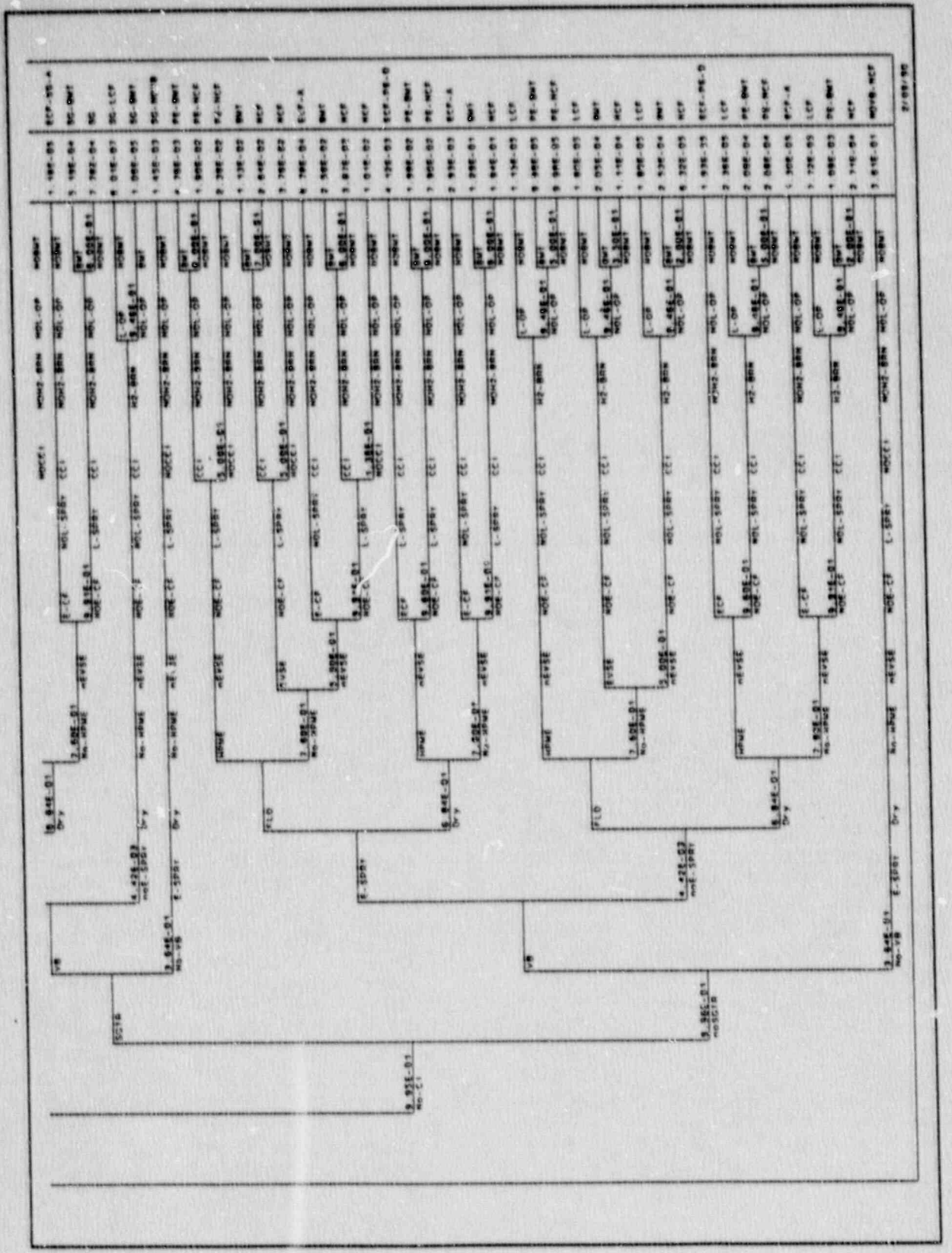


Figure 4.1. (continued).

Table 4.1. Conditional containment failure probabilities for the transient + ATWS PDS

Containment Failure Mode	Conditional Probability (SCET)	Conditional Probability (Reference 4)
No containment failure	7.90E-01	7.90E-01
DCH	4.14E-03	5.19E-03
α mode	3.66E-03	3.51E-03
Bypass ^a	8.94E-03	9.15E-03
Late overpressure	1.49E-04	1.48E-04
BMT	1.93E-01	1.93E-01

a. Bypass failures include pre-existing leakage and induced SGTR.

Table 4.2. Conditional consequences for the transient + ATWS accident progression bins

Accident Progression Bin	Mean Early Fatalities	Mean Latent Fatalities	Mean 50-Mile Dose (person-rem)	Mean 1000-Mile Dose (person-rem)	Mean Offsite Costs (\$)
CDACAAABACBB	9.55E+00	4.24E+03	8.21E+06	2.49E+07	1.51E+10
CDACACABACBB	7.30E-01	3.98E+02	1.13E+06	2.31E+06	3.08E+08
CDADBAABDCBB	9.55E+00	4.24E+03	8.21E+06	2.49E+07	1.51E+10
CDADBCABDCBB	1.12E+00	5.98E+02	1.74E+06	3.51E+06	6.30E+08
CDCCAADBACBB	9.55E+00	4.24E+03	8.21E+06	2.49E+07	1.51E+10
CDCCACDBACBB	7.30E-01	3.98E+02	1.13E+06	2.31E+06	3.08E+08
CDCDBCDBDCBB	1.12E+00	5.98E+02	1.74E+06	3.51E+06	6.30E+08
CDCDFADBCBB	9.54E+00	4.23E+03	8.18E+06	2.48E+07	1.51E+10
CDCDFCDBDCBB	7.30E-01	3.98E+02	1.13E+06	2.31E+06	3.08E+08
CDDCACABACBB	7.30E-01	3.98E+02	1.13E+06	2.31E+06	3.08E+08
CDDDBCABDCBB	1.12E+00	5.98E+02	1.74E+06	3.51E+06	6.30E+08
CHACACABACBB	1.47E+01	4.41E+03	8.06E+06	2.65E+07	1.90E+10
CHADBCABDCBB	1.26E+02	7.04E+03	1.35E+07	4.12E+07	2.79E+10
CHFCACABACBB	1.47E+01	4.41E+03	8.06E+06	2.65E+07	1.90E+10
DAADDABDBBB	1.01E+01	4.50E+03	8.91E+06	2.71E+07	1.70E+10
DAADDDBDBBB	1.45E+00	1.42E+03	3.64E+06	9.09E+06	3.20E+09
DAFDDABDBBB	1.01E+01	4.50E+03	8.91E+06	2.71E+07	1.70E+10
DAFDDDBDBBB	1.35E+00	1.45E+03	3.73E+06	9.28E+06	3.09E+09
DDACACABACBB	6.17E-01	1.38E+02	4.32E+05	7.77E+05	7.97E+07
DHACACABACBB	6.43E-01	2.11E+03	4.55E+06	1.28E+07	5.95E+09
DHADDDBDBBB	4.37E+02	9.56E+03	1.94E+07	5.67E+07	4.41E+10
EHACACABACBB	0.00E+00	1.46E+02	4.75E+05	8.75E+05	1.01E+08
EHADBCABDCBB	8.38E-06	2.39E+02	7.51E+05	1.44E+06	1.78E+08
EHFCACABACBB	8.38E-06	2.39E+02	7.51E+05	1.44E+06	1.78E+08
EHFDBCABDCBB	8.82E-04	5.99E+02	1.54E+06	4.06E+06	8.59E+08
FDACACABAEBB	0.00E+00	8.63E-03	2.28E+01	5.17E+01	0.00E+00

Table 4.2. (continued)

Accident Progression Bin	Mean Early Fatalities	Mean Latent Fatalities	Mean 50-Mile Dose (person-rem)	Mean 1000-Mile Dose (person-rem)	Mean Offsite Costs (\$)
FDADBCABDEBB	0.00E+00	8.63E-03	2.28E+01	5.17E+01	0.00E+00
FDACACABAEBB	0.00E+00	8.63E-03	2.28E+01	5.17E+01	0.00E+00
FODDBCABDEBB	0.00E+00	8.63E-03	2.28E+01	5.17E+01	0.00E+00
FHACACABAEBB	0.00E+00	7.84E-02	3.18E+02	4.61E+02	1.10E+03
FHADBCARDEBB	0.00E+00	1.31E-01	5.38E+02	7.72E+02	3.42E+03
FHFCACABAEBB	0.00E+00	7.84E-02	3.18E+02	4.61E+02	1.10E+03
FHFDBCABDEBB	0.00E+00	1.31E-01	5.38E+02	7.72E+02	3.42E+03
GDACAAABAEBB	8.42E+00	4.23E+03	8.19E+06	2.48E+07	1.51E+10
GDACAAABAFBB	8.42E+00	4.23E+03	8.19E+06	2.48E+07	1.51E+10
GDACACABAGBB	0.00E+00	2.63E-02	9.67E+01	1.38E+02	0.00E+00
GDADBAABDEBB	8.42E+00	4.23E+03	8.19E+06	2.48E+07	1.51E+10
GDADBAABDFBB	8.42E+00	4.23E+03	8.19E+06	2.48E+07	1.51E+10
GDADBCABDGBB	0.00E+00	2.63E-02	9.67E+01	1.38E+02	0.00E+00
GDCCAADBAFBB	8.42E+00	4.23E+03	8.19E+06	2.48E+07	1.51E+10
GDCCACDBAGBB	0.00E+00	2.63E-02	9.67E+01	1.38E+02	0.00E+00
GDCDBADBDFFBB	8.42E+00	4.23E+03	8.19E+06	2.48E+07	1.51E+10
GDCDBCDBDGBB	0.00E+00	2.63E-02	9.67E+01	1.38E+02	0.00E+00
GDCDFADBDFFBB	8.42E+00	4.23E+03	8.19E+06	2.48E+07	1.51E+10
GDCDFCDBDGBB	0.00E+00	2.63E-02	9.67E+01	1.38E+02	0.00E+00
GDDCAAABAEBB	8.42E+00	4.23E+03	8.19E+06	2.48E+07	1.51E+10
GDDCAAABAFBB	8.42E+00	4.23E+03	8.19E+06	2.48E+07	1.51E+10
GDDCACABAGBB	0.00E+00	2.63E-02	9.67E+01	1.38E+02	0.00E+00
GDDDBAABDEBB	8.42E+00	4.23E+03	8.19E+06	2.48E+07	1.51E+10
GDDDBAABDFBB	8.42E+00	4.23E+03	8.19E+06	2.48E+07	1.51E+10
GDDDBCABDGBB	0.00E+00	2.63E-02	9.67E+01	1.38E+02	0.00E+00
GHACACABAGBB	0.00E+00	1.52E-01	6.17E+02	8.77E+02	3.80E+03
GHADBAABDCBB	8.42E+00	4.23E+03	8.19E+06	2.48E+07	1.51E+10
GHADBAABDEBB	8.42E+00	4.23E+03	8.19E+06	2.48E+07	1.51E+10
GHADBCABDGBB	0.00E+00	1.52E-01	6.17E+02	8.77E+02	3.80E+03
GHFCACABAGBB	0.00E+00	1.52E-01	6.17E+02	8.77E+02	3.80E+03
GHFDBCABDGBB	0.00E+00	1.52E-01	6.17E+02	8.77E+02	3.80E+03

Table 4.3. Annual base case transient + ATWS risk

	Mean Early Fatalities	Mean Latent Fatalities	Mean 50-Mile Dose (person-rem)	Mean 1000-Mile Dose (person-rem)	Mean Offsite Costs (\$)
SCET	6E-07	3E-04	0.7	1.8	9E+02
Reference 4	4E-06	5E-04	1.2	2.7	3E+03
Relative change	6.7	1.7	1.7	1.5	3.3

Table 4.4. Containment failure mode contribution to offsite dose for the transient + ATWS PDS (from SCET)

Containment Failure Mode	Contribution to 50-Mile Dose (%)	Contribution to 1000-Mile Dose (%)
DCH	3.4	2.3
α	24.6	22.6
Bypass ^a	71.8	74.9
Late overpressure	ε	ε
BMT	ε	ε

a. Bypass failures include pre-existing containment leaks and induced SGTR.

approximate this dependency by including more top events in the SCET. However, none of these attempts proved successful; the SCET results still did not agree very well with those predicted by the APET.

This modeling problem appears to be a limitation of the SCET methodology. Vessel breach is a critical top event in the SCET and its complicated dependencies on earlier events in the APET cannot be completely modeled without making the SCET excessively large and complicated. At this point, a choice had to be made between simplicity and accuracy. The decision was made to give up some of the simplicity of analyzing depressurization on the SCET in order to accurately model the dependencies in the APET. SCETs were still produced to show the revised pathways through the APET, but depressurization itself was modeled by adjusting split fractions in the APET.

This decision was influenced by the fortuitous availability of the PSTEVNT post-processor code at this point in the project.⁸ PSTEVNT allowed processing of output from EVNTRE runs made in the sampling mode (mode 3), as was done for draft NUREG-1150. Use of PSTEVNT also automated the binning of SCET end states into the ZISOR accident progression bins. The steps in the revised process are as follows:

1. Run EVNTRE in the sampling mode (mode 3) to analyze improvements modeled in the full 72-question APET.
2. Use the binning output file from EVNTRE as input to PSTEVNT.

3. Run PSTEVNT to generate composite sorted output from which the revised SCET is constructed.

4. Run PSTEVNT again to generate the ZISOR accident progression bins and their conditional probabilities of occurrence.

The remaining steps in the risk calculation are the same as were described in Section 2.

Intentional opening of the PORVs was modeled in the APET by adjusting the split fraction for Question 16 to 0.0/1.0 and turning off sampling for this question. The resulting SCET is shown in Figure 4.2. The revised conditional containment failure probabilities are shown in Table 4.5.

As Table 4.5 shows, opening the PORVs significantly reduces the probability of DCH-induced early containment failure. Also, the probability of bypass failure is reduced by the elimination of temperature-induced SGTR. However, there is a slight increase in the probability of late failure—the increase in late overpressure failure is probably attributable to a larger amount of hydrogen being generated in the case where the PORVs are intentionally opened during core degradation. The reason for the slight increase in the probability of BMT is not known. The effects of intentional RCS depressurization on offsite risk are shown in Table 4.6.

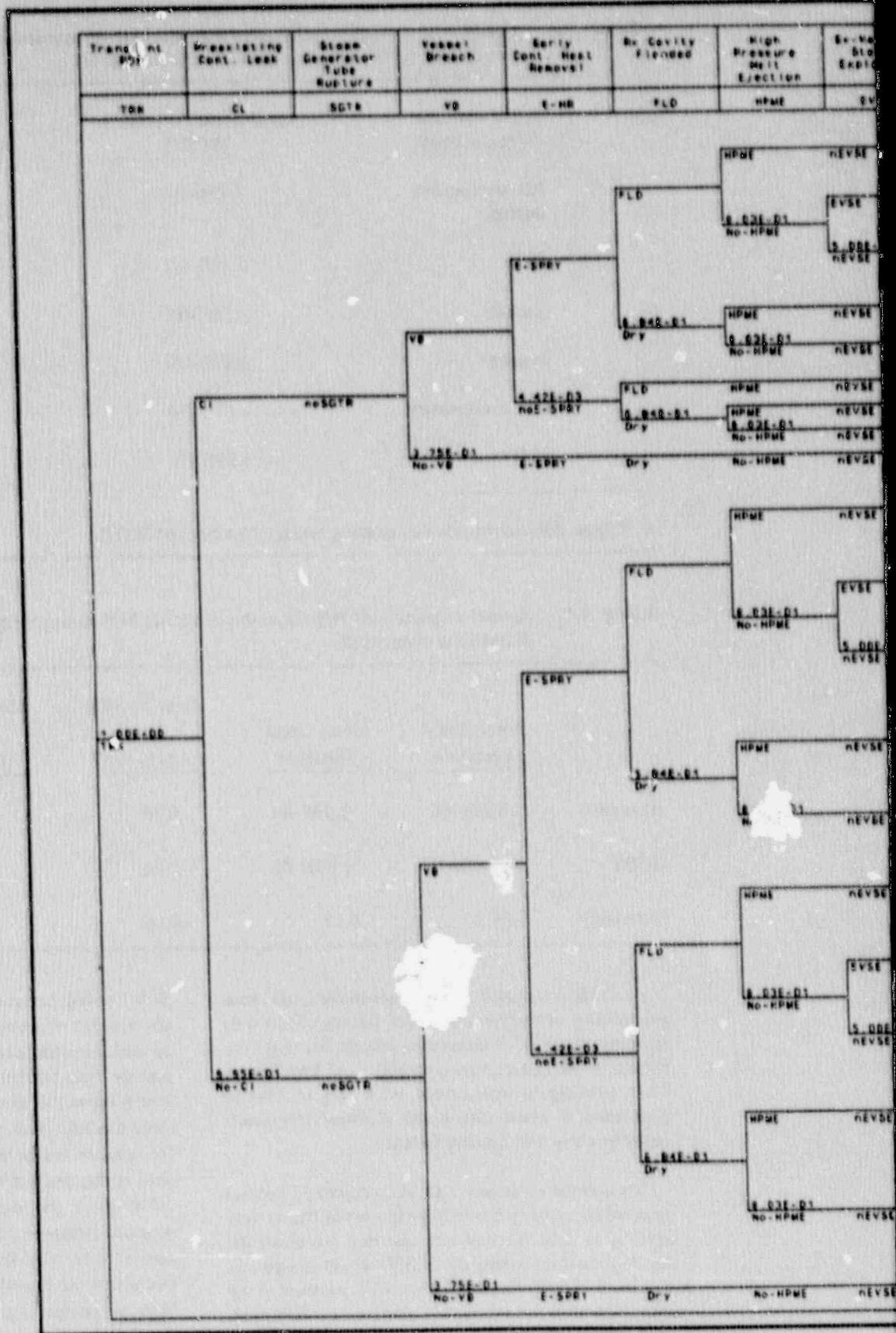


Figure 4.2. Transient + ATWS SCET with intentional depressurization.

Q-1 Station	Early Cont. Fallout	Late Cont. Sprays	Core-Concrete Interaction	Hydrogen Burn	Late DP Cont. Fallout	Personal Dose Through	SEQUENCE PROB	SEQUENCE CLASS
	Q-CF	L-SPRY	CCI	H2-BRN	L-OP	BMT		
	NDE-CF	L-SPRY	CCI 5.0DE-D1 NOCCI	NDH2-BRN	NOL-OP	NDBMT	8.87E-03	CL-PE
	NDE-CF	L-SPRY	CCI 5.0DE-D1 NOCCI	NDH2-BRN	NOL-OP	NDBMT	9.87E-03	CL-PE
	NDE-CF	L-SPRY	CCI 3.0DE-D1 NOCCI	NDH2-BRN	NOL-OP	NDBMT	1.97E-04	CL
	NDE-CF	L-SPRY	CCI 3.0DE-D1 NOCCI	NDH2-BRN	NOL-OP	NDBMT	1.97E-04	CL
	S-CF	NOL-SPRY	CCI	NDH2-BRN	NOL-OP	NDBMT	3.37E-06	ECF-CL-A
	8.81E-D1 NDE-CF	L-SPRY	CCI 1.0DE-D1 NOCCI	NDH2-BRN	NOL-OP	NDBMT	3.81E-04	CL
	NDE-CF	L-SPRY	CCI	NDH2-BRN	NOL-OP	NDBMT	4.18E-04	CL-PE
	E-CF	NOL-SPRY	CCI	NDH2-BRN	NOL-OP	NDBMT	1.46E-05	ECF-CL-A
	8.81E-D1 NDE-CF	L-SPRY	CCI	NDH2-BRN	NOL-OP	NDBMT	1.89E-03	CL
	NDE-CF	NOL-SPRY	CCI	NDH2-BRN	NOL-OP	NDBMT	4.36E-06	CL-PE
	NDE-CF	NOL-SPRY	CCI	NDH2-BRN	NOL-OP	NDBMT	1.88E-08	CL-PE
	NDE-CF	NOL-SPRY	CCI	NDH2-BRN	NOL-OP	NDBMT	7.58E-06	CL
	NDS-CF	NOL-SPRY	NOCCI	NDH2-BRN	NOL-OP	NDBMT	1.87E-03	CL-NOVB
	NDE-CF	L-SPRY	CCI	NDH2-BRN	NOL-OP	BMT 8.0DE-D1 NOBMT	3.85E-03	PE-DMT
	NDE-CF	L-SPRY	CCI 5.0DE-D1 NOCCI	NDH2-BRN	NOL-OP	NDBMT	1.54E-02	PE-NCF
	NDE-CF	L-SPRY	CCI 3.0DE-D1 NOCCI	NDH2-BRN	NOL-OP	NDBMT	1.83E-02	PE-NCF
	NDE-CF	L-SPRY	CCI	NDH2-BRN	NOL-OP	BMT 7.0DE-D1 NOBMT	1.18E-02	BMT
	NDE-CF	L-SPRY	CCI 3.0DE-D1 NOCCI	NDH2-BRN	NOL-OP	NDBMT	2.75E-02	NCF
	E-CF	NOL-SPRY	CCI	NDH2-BRN	NOL-OP	NDBMT	3.83E-02	NCF
	8.81E-D1 NDE-CF	L-SPRY	CCI	NDH2-BRN	NOL-OP	BMT 8.0DE-D1 NOBMT	6.71E-04	ECF-A
	NDE-CF	L-SPRY	CCI	NDH2-BRN	NOL-OP	NDBMT	3.11E-02	BMT
	NDE-CF	L-SPRY	CCI 1.0DE-D1 NOCCI	NDH2-BRN	NOL-OP	NDBMT	4.87E-02	NCF
	NDE-CF	L-SPRY	CCI 1.0DE-D1 NOCCI	NDH2-BRN	NOL-OP	NDBMT	7.78E-05	NCF
	ECF	L-SPRY	CCI	NDH2-BRN	NOL-OP	NDBMT	2.20E-03	ECF-PE-D
	9.74E-D1 NDE-CF	L-SPRY	CCI	NDH2-BRN	NOL-OP	BMT 8.0DE-D1 NOBMT	1.82E-02	PE-DMT
	E-CF	NOL-SPRY	CCI	NDH2-BRN	NOL-OP	NDBMT	8.50E-02	PE-NCF
	8.81E-D1 NDE-CF	L-SPRY	CCI	NDH2-BRN	NOL-OP	BMT 8.0DE-D1 NOBMT	2.80E-03	ECF-A
	NDE-CF	L-SPRY	CCI	NDH2-BRN	NOL-OP	BMT 8.0DE-D1 NOBMT	1.35E-01	BMT
	NDE-CF	NOL-SPRY	CCI	H2-BRN	L-OP	NDBMT	2.02E-01	NCF
	NDE-CF	NOL-SPRY	CCI	H2-BRN	L-OP	NDBMT	1.04E-05	LCF
	NDE-CF	NOL-SPRY	CCI	H2-BRN	9.39E-D1 NOL-OP	BMT 3.0DE-D1 NOBMT	1.04E-05	LCF
	NDE-CF	NOL-SPRY	CCI	H2-BRN	9.39E-D1 NOL-OP	BMT 3.0DE-D1 NOBMT	6.03E-05	PE-DMT
	NDE-CF	NOL-SPRY	CCI	H2-BRN	9.39E-D1 NOL-OP	BMT 3.0DE-D1 NOBMT	6.03E-05	PE-NCF
	NDE-CF	NOL-SPRY	CCI	H2-BRN	L-OP	NDBMT	2.13E-05	LCF
	NDE-CF	NOL-SPRY	CCI	H2-BRN	9.39E-D1 NOL-OP	BMT 3.0DE-D1 NOBMT	2.13E-04	DMT
	NDE-CF	NOL-SPRY	CCI	H2-BRN	9.39E-D1 NOL-OP	BMT 3.0DE-D1 NOBMT	1.15E-04	NCF
	NDE-CF	NOL-SPRY	CCI	H2-BRN	L-OP	NDBMT	2.13E-05	LCF
	NDE-CF	NOL-SPRY	CCI	H2-BRN	9.39E-D1 NOL-OP	BMT 2.0DE-D1 NOBMT	2.62E-04	DMT
	NDE-CF	NOL-SPRY	CCI	H2-BRN	9.39E-D1 NOL-OP	BMT 3.0DE-D1 NOBMT	6.54E-05	NCF
	ECF	NOL-SPRY	CCI	NDH2-BRN	NOL-OP	NDBMT	8.76E-06	ECF-PE-D
	9.74E-D1 NDE-CF	NOL-SPRY	CCI	H2-BRN	L-OP	NDBMT	2.20E-05	LCF
	NDE-CF	NOL-SPRY	CCI	H2-BRN	9.39E-D1 NOL-OP	BMT 3.0DE-D1 NOBMT	1.68E-04	PE-DMT
	NDE-CF	NOL-SPRY	CCI	H2-BRN	9.39E-D1 NOL-OP	BMT 3.0DE-D1 NOBMT	1.68E-04	PE-NCF
	E-CF	NOL-SPRY	CCI	NDH2-BRN	NOL-OP	NDBMT	1.28E-05	ECF-A
	9.81E-D1 NDE-CF	NOL-SPRY	CCI	H2-BRN	L-OP	NDBMT	8.15E-05	LCF
	NDE-CF	NOL-SPRY	CCI	H2-BRN	9.39E-D1 NOL-OP	BMT 2.0DE-D1 NOBMT	1.12E-03	PE-DMT
	NDE-CF	L-SPRY	NOCCI	NDH2-BRN	NOL-OP	NDBMT	2.81E-04	NCF
	NDE-CF	L-SPRY	NOCCI	NDH2-BRN	NOL-OP	NDBMT	3.73E-01	NOVB-NCF

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Table 4.5. Conditional containment failure probabilities for the transient + ATWS PDS with intentional depressurization (HPME not eliminated)

Containment Failure Mode	Conditional Probability (SCET)	Base Case Conditional Probability
No containment failure	7.89E-01	7.90E-01
DCH	2.21E-03	4.14E-03
α mode	3.58E-03	3.66E-03
Bypass ^a	4.97E-03	8.94E-03
Late overpressure	1.67E-04	1.49E-04
BMT	1.99E-01	1.93E-01

a. Bypass failures include pre-existing leakage and induced SGTR.

Table 4.6. Annual transient + ATWS risk with intentional RCS depressurization (HPME not eliminated)

	Mean Early Fatalities	Mean Latent Fatalities	Mean 50-Mile Dose (person-rem)	Mean 1000-Mile Dose (person-rem)	Mean Offsite Costs (\$)
Base case	6.22E-07	3.00E-04	0.66	1.79	892
SCET	2.10E-07	9.67E-05	0.26	0.60	176
% change	-66.3	-67.8	-60.6	-66.6	-80.3

As Table 4.6 shows, the decrease in the conditional probability of early containment failure effected by opening the PORVs more than offsets the slight increase in the conditional probability of late failure. This finding is consistent with the results in Reference 4, where offsite risk is completely dominated by early containment failure.

Because the reduction in DCH containment failures effected by opening the PORVs is a result that is very specific to Zion, a sensitivity case was run under the assumption that opening the PORVs would completely eliminate HPME, that is, that the RCS pressure at the time of vessel breach would always be <200 psig. However, this case could not be modeled simply by changing the probability of event HPME in the base case

SCET to 0.0, because reducing vessel pressure affects the conditional probability of vessel breach (event VB) as well. Accordingly, a modified APET was evaluated and the split fractions in the SCET were adjusted to match those calculated with the APET. Also, as discussed in Section 3, the Zion APET uses a probability for α mode failure that is dependent upon RCS pressure at the time of vessel breach. If the pressure is >200 psig, the mean conditional probability of α mode failure is 8.0×10^{-4} . However, if RCS pressure is <200 psig, the mean probability is 8.0×10^{-3} . Because some experts may question the use of such a high probability of α mode failure, a sensitivity case for full depressurization to <200 psig was also run with a pressure-independent α mode failure

probability of 8.0×10^{-4} , with no sampling of Question 34 in the APET. The revised SCETs for these cases are shown in Figures 4.3 and 4.4. The containment failure probabilities calculated for these cases are shown in Tables 4.7 and 4.8.

Eliminating HPME eliminates DCH failures, as expected. The conditional probability of α mode failure increases if the pressure-dependent probability distribution from draft NUREG-1150 is used but is decreased significantly if a pressure-independent point estimate is used. Table 4.9 shows the annual offsite risk for the fully depressurized case with the α mode probabilities from draft NUREG-1150 and Table 4.10 shows the risk for the fully depressurized case with the pressure-independent point estimate probability of α mode failure.

Note that the risk reduction for both of the fully depressurized cases is quite large, although it is less where a pressure-dependent α mode probability is used. This is attributable to several factors. First, depressurizing allows injection from low pressure systems, because AC power is available. This reduces the conditional probability of vessel breach. Secondly, depressurization eliminates temperature-induced SGTR, which is a significant contributor to the base case transient + ATWS risk, as shown in Table 4.4. Thirdly, the use of a point estimate probability of α mode failure that is independent of the RCS pressure at the time of vessel breach eliminates failures produced by sampling from the "tail" of the α mode failure distribution. In summary, the primary benefits of depressurization for the transient + ATWS PDS group are the elimination of temperature-induced SGTR and the increased probability of recovering the sequence in-vessel. These benefits are diminished

somewhat by the use of the pressure-dependent probability distribution for α mode failure developed for draft NUREG-1150.

4.4.2 Cavity Flooding. The addition of a cavity flooding system was analyzed by appropriately modifying the split fraction of Question 31 in the Zion APET, turning off sampling for this question, and running EVNTPE and PSTEVNT to generate the revised SCET and ZISOR accident progression bins. Figure 4.5 shows the revised SCET for transient + ATWS with the addition of a cavity flooding system. Table 4.11 shows the revised conditional containment failure probabilities.

Cavity flooding leads to an increase in the probability of DCH failure. There is an expected reduction in the probability of BMT. Table 4.12 shows the effect of cavity flooding on offsite risk. For further details, see Section 5.4.2.

4.4.3 Hydrogen Control. Improved hydrogen control was modeled in the SCET by eliminating late overpressure containment failure, that is, by setting the probability of the lower branch of event L-OP to 1.0 for all sequences. The new conditional containment failure probabilities are shown in Table 4.13.

Elimination of hydrogen burns produced no reduction in any of the risk measures used in this report. This finding is consistent with Reference 4, where late overpressure containment failure was a negligible contributor to risk.

4.4.4 Containment Venting. As in the LOCA PDS, containment venting is not evaluated for transient + ATWS. See the discussion of containment venting in Section 3.4.4.

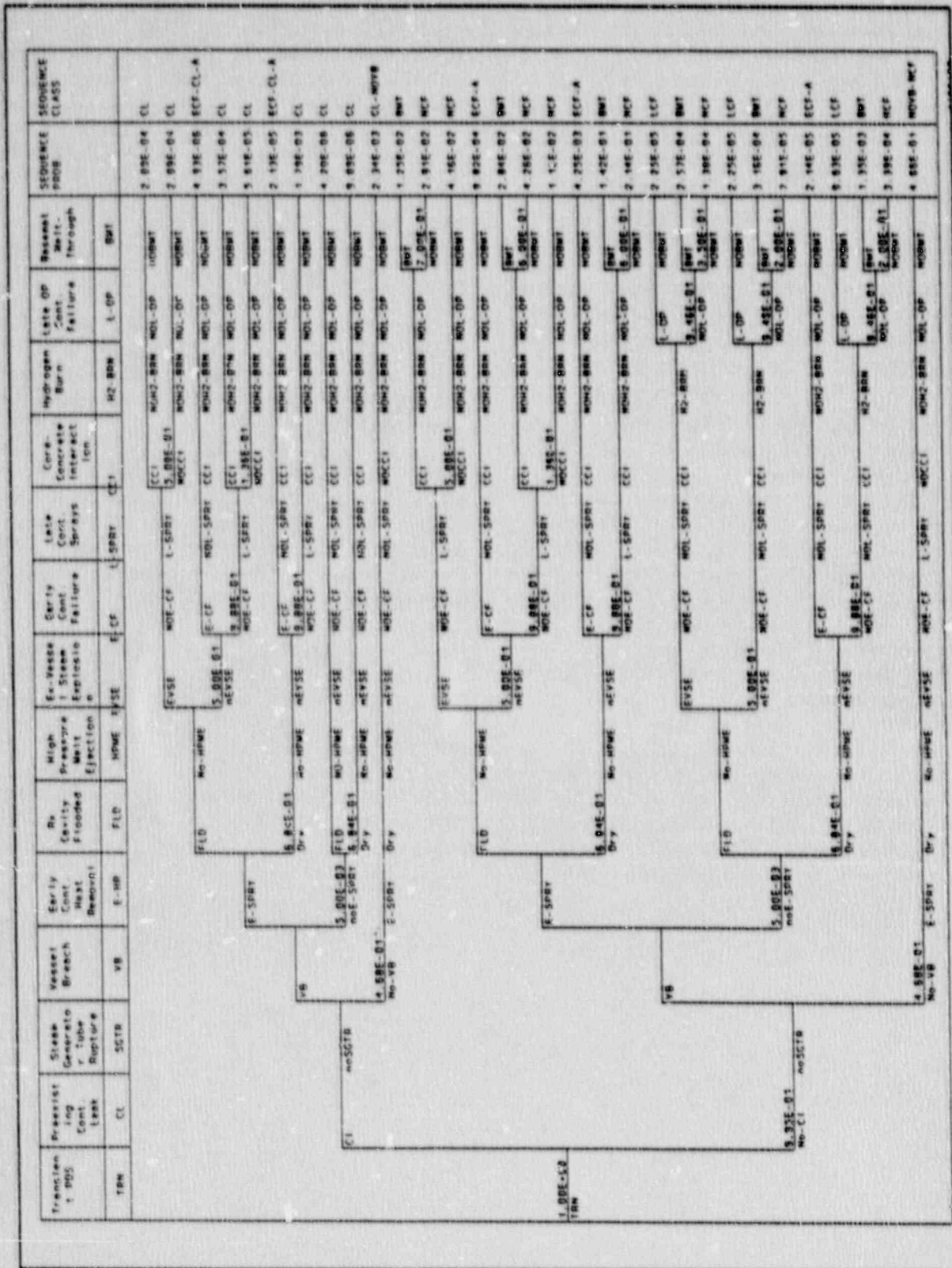


Figure 4.3 Transient + ATWS SCET with full depressurization (no HPME) and pressure-dependent α mode probability.

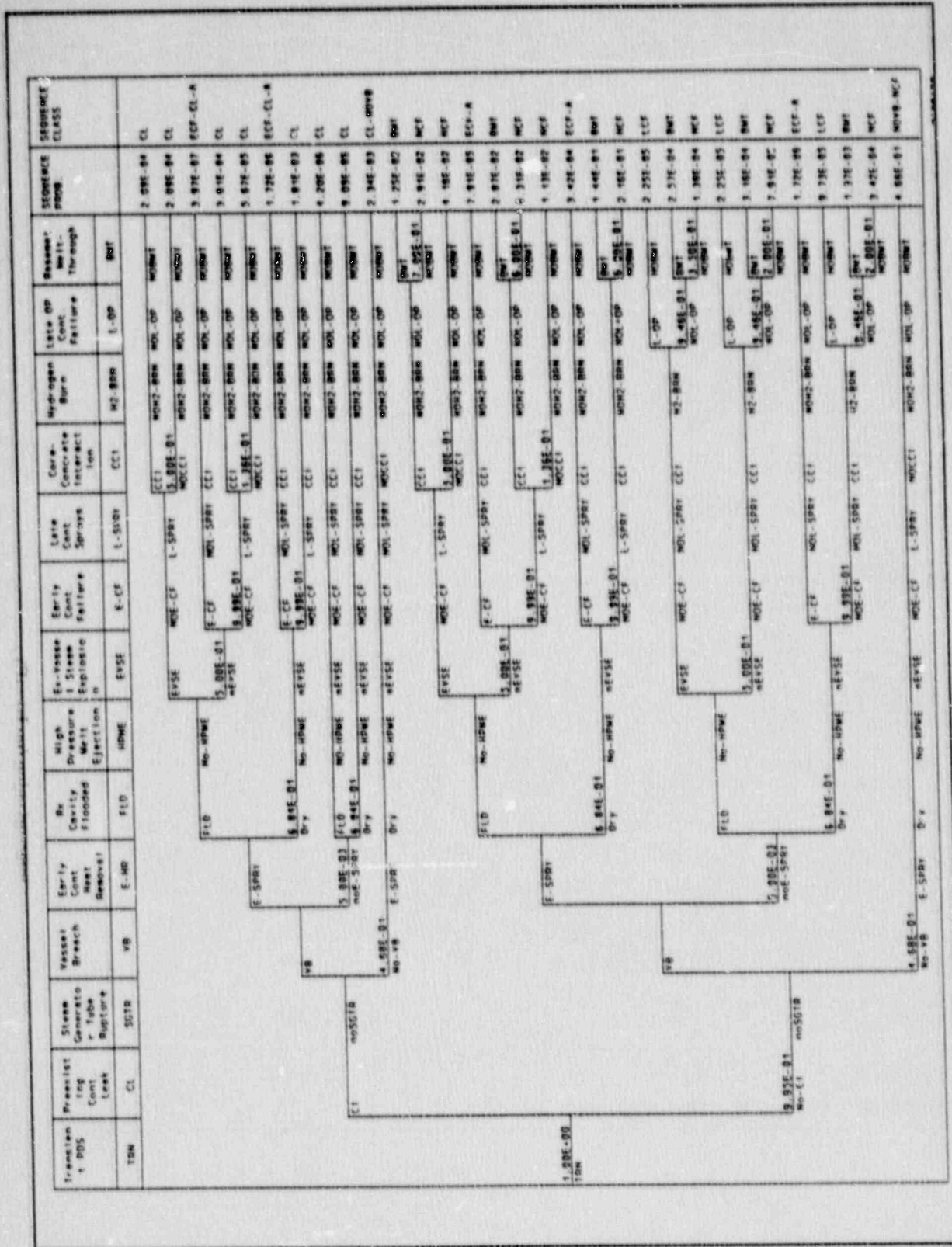


Figure 4.4. Transient + ATWS SCET with intentional depressurization to <200 psig (no HPME) and pressure-independent point estimate α mode probability.

Table 4.7. Conditional containment failure probabilities for the transient + ATWS PDS with operator depressurization and pressure-dependent α mode probability (no HPME)

Containment Failure Mode	Conditional Probability (SCET)	Base Case Conditional Probability
No containment failure	8.05E-01	7.90E-01
DCH	0.00	4.14E-03
α mode	5.28E-03	3.66E-03
Bypass ^a	4.97E-03	8.94E-03
Late overpressure	1.41E-04	1.49E-04
BMT	1.85E-01	1.93E-01

a. Bypass failures involve pre-existing containment leakage.

Table 4.8. Conditional containment failure probabilities for the transient + ATWS PDS with operator depressurization and pressure-independent point estimate α mode probability (no HPME)

Containment Failure Mode	Conditional Probability (SCET)	Base Case Conditional Probability
No containment failure	8.08E-01	7.90E-01
DCH	0.00	4.14E-03
α mode	4.25E-04	3.66E-03
Bypass ^a	5.00E-03	8.94E-03
Late overpressure	1.42E-04	1.49E-04
BMT	1.87E-01	1.93E-01

a. Bypass failures involve pre-existing containment leakage.

Table 4.9. Annual transient + ATWS risk with full RCS depressurization and pressure-dependent α mode probability (No HPME)

	<u>Mean Early Fatalities</u>	<u>Mean Latent Fatalities</u>	<u>Mean 50-Mile Dose (Person-rem)</u>	<u>Mean 1000-Mile Dose (Person-rem)</u>	<u>Mean Offsite Costs (\$)</u>
Base case	6.22E-07	3.00E-04	0.66	1.79	892
SCET	2.67E-07	1.22E-04	0.32	0.76	249
% change	-57.1	-59.4	-51.1	-57.4	-73.1

Table 4.10. Annual transient + ATWS risk with full RCS depressurization and pressure-independent point estimate α mode probability (no HPME)

	<u>Mean Early Fatalities</u>	<u>Mean Latent Fatalities</u>	<u>Mean 50-Mile Dose (Person-rem)</u>	<u>Mean 1000-Mile Dose (Person-rem)</u>	<u>Mean Offsite Costs (\$)</u>
Base case	6.22E-07	3.00E-04	0.66	1.79	892
SCET	8.44E-08	3.85E-05	0.11	0.23	48.0
% change	-86.4	-87.2	-83.5	-87.2	-94.6

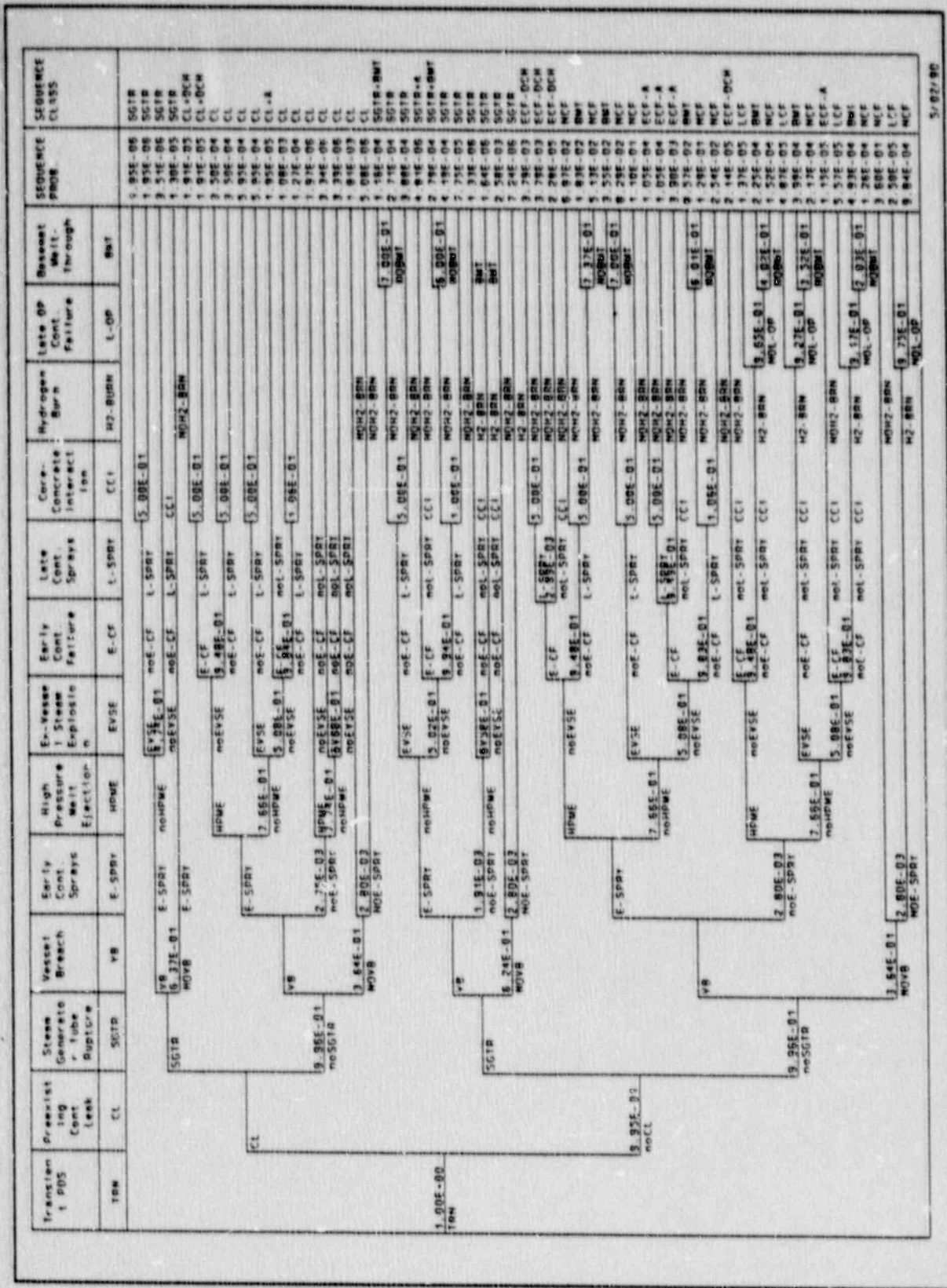


Figure 4.5. Transient + ATWS SCET with cavity flooding system.

Table 4.11. Conditional containment failure probabilities for the transient + ATWS PDS with cavity flooding system

Containment Failure Mode	Conditional Probability (SCET)	Base Case Conditional Probability
No containment failure	8.38E-01	7.90E-01
DCH	7.62E-03	4.14E-03
α mode	4.14E-03	3.66E-03
Bypass ^a	9.12E-03	8.94E-03
Late overpressure	1.43E-04	1.49E-04
BMT	1.41E-01	1.93E-01

a. Bypass failures involve pre-existing containment leakage.

Table 4.12. Annual transient + ATWS risk with addition of cavity flooding system

	Mean Early Fatalities	Mean Latent Fatalities	Mean 50-Mile Dose (Person-rem)	Mean 1000-Mile Dose (Person-rem)	Mean Offsite Costs (\$)
Base case	6.22E-07	3.00E-04	0.66	1.79	892
SCET	6.58E-07	2.97E-04	0.66	1.74	844
% change	5.8	-1.1	0.0	-2.9	-5.4

Table 4.13. Conditional containment failure probabilities for the transient + ATWS PDS with improved hydrogen control

Containment Failure Mode	Conditional Probability (SCET)	Base Case Conditional Probability
No containment failure	7.90E-01	7.90E-01
DCH	4.14E-03	4.14E-03
α mode	3.66E-03	3.66E-03
Bypass ^a	8.94E-03	8.94E-03
Late overpressure	0.00	1.48E-04
BMT	1.93E-01	1.93E-01

a. Bypass failures include pre-existing leakage and induced SGTR.

5. STATION BLACKOUT

Plant damage states in the SBO group involve a loss of offsite power followed by failure of the emergency diesel generators (EDGs) to start and run. The turbine-driven auxiliary feedwater system (AFWS) operates until the station batteries are depleted after an assumed interval of four hours. The RCS is intact at the time of core uncover in one of the SBO PDSs. In the other two, failure of the RCP seals results in an RCS break equivalent in size to a medium LOCA (S2 break) at the time of core uncover.

5.1 Station Blackout Core Damage Frequency

The SBO group is composed of three PDSs with a total mean frequency of 5.21×10^{-6} per reactor-year from Table 2.2-3 in Reference 4.

5.2 Station Blackout SCET Results

The base case SBO SCET is shown in Figure 5.1. This SCET, like those for the previous PDS groups, was constructed using the methodology outlined in Section 2. Note its similarity to the SCET constructed for the transient + ATWS PDS group in Section 4. Table 5.1 shows the containment failure probabilities, conditional on the occurrence of the SBO PDS, calculated from this SCET and compares them to the probabilities from Reference 4.

The conditional containment failure probabilities calculated from the SCET agree well with those published in Reference 4. Again, the most likely end state is one in which the containment maintains its structural integrity. The most likely mode of early containment failure is DCH. This differs from the LOCA PDS, where α mode failure was more likely than DCH.

5.3 Base Case SBO Consequences

The SCET end states were binned into accident progression bins and the ZISOR code was again used to generate source terms for these bins as described in Section 2. Because of the large number of accident progression bins, the source terms were manually combined into groups in order to reduce the required number of MACCS calculations. Conditional consequences were then calculated for each accident

progression bin group with the MACCS code. As for the previous PDS groups, the Zion site data and meteorological files were used for these calculations. The conditional consequences for each accident progression bin are shown in Table 5.2.

Table 5.3 shows the annual risk for the SBO PDS group calculated from Equation 1.1.

Table 5.4 lists the contribution of each containment failure mode to the offsite population dose.

Again, the only significant contributors to offsite dose are early containment failures, with α mode failure being the most dominant. DCH contributes a larger fraction than it did in either the LOCA or transient + ATWS PDS groups. The contribution from containment bypass is also significant.

5.4 Risk Benefit of Potential Improvements

The risk benefit of the potential containment performance improvements identified in Reference 2 is calculated in this section.

5.4.1 Enhanced Reactor Vessel Depressurization Capability. Similar to the transient + ATWS PDS group, intentional depressurization via the pressurizer PORVs could not be accurately modeled with an SCET. Therefore, depressurization was modeled in the APET using the revised methodology discussed in Section 4.4.1. The revised SCET is shown in Figure 5.2. The new conditional probabilities of containment failure thus obtained are shown in Table 5.5.

As Table 5.5 shows, opening the PORVs during SBO does not significantly affect the threat of DCH. Because the APET upon which the model of depressurization is based is specific to the Zion plant, and PORV capacity is a plant-specific parameter,¹² two sensitivity cases were run in which opening the PORVs was assumed to eliminate HPME by reducing vessel pressure below the DCH cutoff of 200 psig. However, just as for the analogous case in the transient + ATWS PDS, this case could not be modeled by simply setting the probability of event HPME in the base case SCET to 0.0, because reducing vessel pressure affects the conditional probability of vessel breach (event VB) as well. Accordingly, a modified APET was evaluated and the split fractions in the SCET were adjusted to match those calculated with the APET. Similar to the transient + ATWS PDS, the

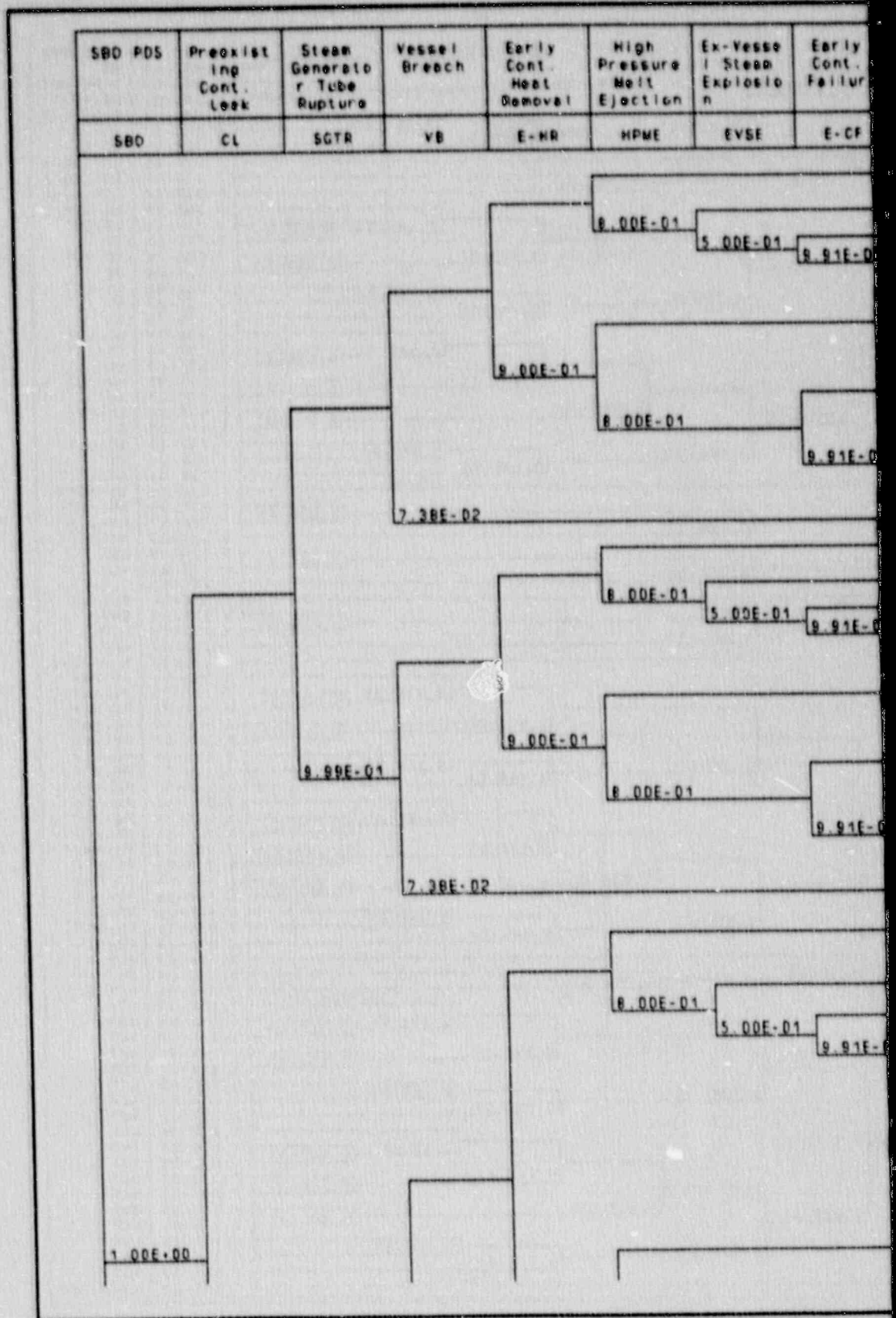


Figure 5.1. Base case SBO SCET.

Late Cont. Heat Removal	Core-Concrete Interaction	Very Late Cont. Heat Removal	Hydrogen Burn	Late OP Cont. Failure	Basemat Melt-Through	SEQUENCE PROB.	SEQUENCE CLASS
L-NR	CCI	VL-NR	H2-BURN	L-OP	BMT		
	5.00E-01					6.48E-08	CL-SG-PE
	5.00E-01					6.48E-08	CL-SG-PE
	5.00E-01					1.30E-07	CL-SG
	5.00E-01					1.30E-07	CL-SG
	5.00E-01					2.26E-09	ECF-CL-A
	5.00E-01					2.31E-07	CL-SG
	5.00E-01					2.57E-08	CL-SG
	5.00E-01					5.25E-07	CL-SG-PE
	5.00E-01					5.25E-07	CL-SG-PE
	5.00E-01					5.25E-08	CL-SG-PE
1.00E-01	5.00E-01	1.00E-01				5.83E-09	CL-SG-PE
1.00E-01	5.00E-01	1.00E-01				5.83E-08	CL-SG-PE
1.00E-01	5.00E-01	1.00E-01				3.65E-08	ECF-CL-A
1.00E-01	5.00E-01	1.00E-01				3.65E-09	ECF-CL-A
1.00E-01	5.00E-01	1.00E-01				4.06E-10	ECF-CL-A
1.00E-01	5.00E-01	1.00E-01				4.10E-06	CL-SG
1.00E-01	5.00E-01	1.00E-01				6.25E-08	CL-SG
1.00E-01	5.00E-01	1.00E-01				4.10E-07	CL-SG
1.00E-01	5.00E-01	1.00E-01				4.56E-08	CL-SG
1.00E-01	5.00E-01	1.00E-01				6.84E-09	CL-SG
1.00E-01	5.00E-01	1.00E-01				5.17E-07	CL-SG-NV
1.00E-01	5.00E-01	1.00E-01				4.62E-05	CL-PE
1.00E-01	5.00E-01	1.00E-01				4.62E-05	CL-PE
1.00E-01	5.00E-01	1.00E-01				9.25E-05	CL
1.00E-01	5.00E-01	1.00E-01				9.25E-05	CL
1.00E-01	5.00E-01	1.00E-01				1.61E-06	ECF-CL-A
1.00E-01	5.00E-01	1.00E-01				1.65E-04	CL
1.00E-01	5.00E-01	1.00E-01				1.83E-05	CL
1.00E-01	5.00E-01	1.00E-01				5.99E-04	CL-PE
1.00E-01	5.00E-01	1.00E-01				1.50E-04	CL-PE
1.00E-01	5.00E-01	1.00E-01				5.99E-05	CL-PE
1.00E-01	5.00E-01	1.00E-01				6.66E-06	CL-PE
1.00E-01	5.00E-01	1.00E-01				1.66E-05	CL-PE
1.00E-01	5.00E-01	1.00E-01				2.61E-05	ECF-CL-A
1.00E-01	5.00E-01	1.00E-01				2.61E-06	ECF-CL-A
1.00E-01	5.00E-01	1.00E-01				2.90E-07	ECF-CL-A
1.00E-01	5.00E-01	1.00E-01				2.85E-03	CL
1.00E-01	5.00E-01	1.00E-01				1.19E-04	CL
1.00E-01	5.00E-01	1.00E-01				2.88E-04	CL
1.00E-01	5.00E-01	1.00E-01				3.20E-05	CL
1.00E-01	5.00E-01	1.00E-01				9.90E-06	CL
1.00E-01	5.00E-01	1.00E-01				3.69E-04	CL-NOVB
1.00E-01	5.00E-01	1.00E-01				2.58E-06	S-PE-BMT
1.00E-01	5.00E-01	1.00E-01			8.00E-01	1.03E-05	SG-PE
1.00E-01	5.00E-01	1.00E-01			8.00E-01	1.29E-05	SG-PE
1.00E-01	5.00E-01	1.00E-01			7.00E-01	7.74E-06	SG-BMT
1.00E-01	5.00E-01	1.00E-01			5.00E-01	1.81E-05	SG
1.00E-01	5.00E-01	1.00E-01			5.00E-01	2.59E-05	SG
1.00E-01	5.00E-01	1.00E-01			6.00E-01	4.48E-07	ECF-SG-A
1.00E-01	5.00E-01	1.00E-01			6.00E-01	1.84E-05	SG-BMT
1.00E-01	5.00E-01	1.00E-01			6.00E-01	2.76E-05	SG
1.00E-01	5.00E-01	1.00E-01			6.00E-01	5.12E-06	SG
1.00E-01	5.00E-01	1.00E-01			9.48E-01	3.04E-07	S-PE-LCF
1.00E-01	5.00E-01	1.00E-01			8.00E-01	1.11E-06	S-PE-BMT
1.00E-01	5.00E-01	1.00E-01			8.44E-01	4.44E-06	SG-PE
1.00E-01	5.00E-01	1.00E-01			8.00E-01	1.97E-05	S-PE-BMT
1.00E-01	5.00E-01	1.00E-01			8.00E-01	7.89E-05	SG-PE
1.00E-01	5.00E-01	1.00E-01			8.48E-01	3.04E-07	S-PE-LCF
1.00E-01	5.00E-01	1.00E-01			8.44E-01	5.55E-06	SG-PE
1.00E-01	5.00E-01	1.00E-01			8.44E-01	9.86E-05	SG-PE
1.00E-01	5.00E-01	1.00E-01			8.48E-01	7.04E-08	S-PE-LCF
1.00E-01	5.00E-01	1.00E-01			8.48E-01	1.11E-07	S-PE-BMT
1.00E-01	5.00E-01	1.00E-01			8.44E-01	4.44E-07	SG-PE

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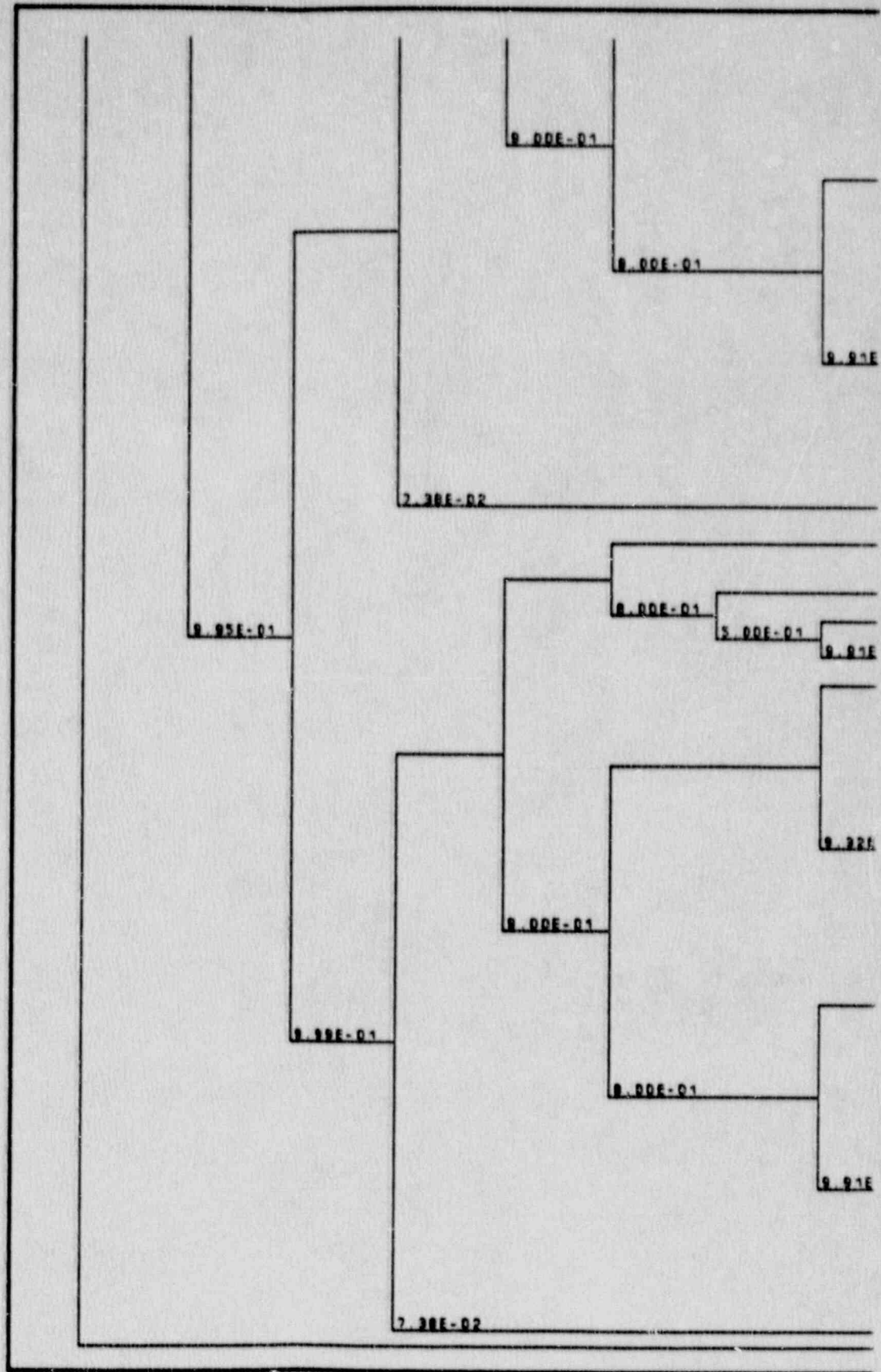
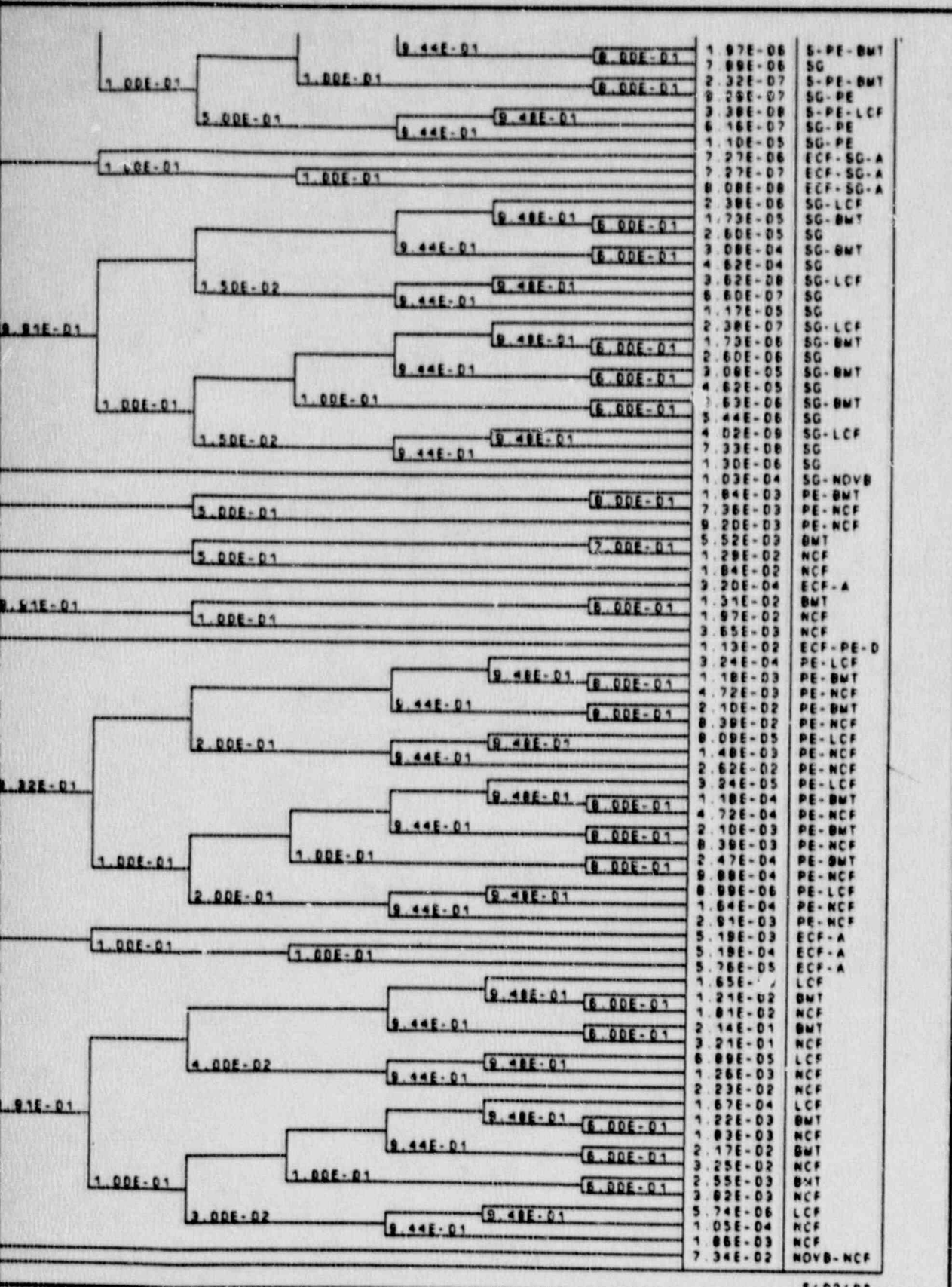


Figure 5.1. (continued).



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Table 5.1. Conditional containment failure probabilities for the SBO PDS

Containment Failure Mode	Conditional Probability (SCET)	Conditional Probability (Reference 4)
No containment failure	6.76E-01	6.57E-01
DCH	1.13E-02	1.66E-02
α mode	6.13E-03	6.15E-03
Bypass ^a	6.35E-03	6.42E-03
Late overpressure	2.34E-03	2.58E-03
BMT	2.97E-01	3.11E-01

a. Bypass failures include pre-existing leakage and induced SGTR.

Table 5.2. Conditional consequences for the SBO accident progression bins

Accident Progression Bin	Mean Early Fatalities	Mean Latent Fatalities	Mean 50-Mile Dose (person-rem)	Mean 1000-Mile Dose (person-rem)	Mean Offsite Costs (\$)
CDCCAADBACBB	9.55E+00	4.24E+03	8.21E+06	2.49E+07	1.51E+10
CDCCACDBACBB	7.30E-01	3.98E+02	1.13E+06	2.31E+06	3.08E+08
CDCDBADBD CBB	9.55E+00	4.24E+03	8.21E+06	2.49E+07	1.51E+10
CDCDBCD BDCBB	1.12E+00	5.98E+02	1.74E+06	3.51E+06	6.30E+08
CDCDFAD BDCBB	9.54E+00	4.23E+03	8.18E+06	2.48E+07	1.51E+10
CDCDFCD BDCBB	7.30E-01	3.98E+02	1.13E+06	2.31E+06	3.08E+08
CDDCAAABACBB	9.55E+00	4.24E+03	8.21E+06	2.49E+07	1.51E+10
CDDCACABACBB	7.30E-01	3.98E+02	1.13E+06	2.31E+06	3.08E+08
CDDDBAABDCBB	9.55E+00	4.24E+03	8.21E+06	2.49E+07	1.51E+10
CDDDBCABDCBB	1.12E+00	5.98E+02	1.74E+06	3.51E+06	6.30E+08
CFACAAABACBA	1.67E+01	5.18E+03	9.65E+06	3.07E+07	1.99E+10
CFACACABACBA	2.96E+01	4.45E+03	8.23E+06	2.67E+07	1.90E+10
CFADBAABDCBA	1.67E+01	5.18E+03	9.65E+06	3.07E+07	1.99E+10
CFADBCABDCBA	1.72E+02	6.86E+03	1.33E+07	3.98E+07	2.69E+10
CFCCAADBACBA	1.67E+01	5.18E+03	9.65E+06	3.07E+07	1.99E+10
CFCCACDBACBA	2.96E+01	4.45E+03	8.23E+06	2.67E+07	1.90E+10
CFCDBAD BDCBA	1.67E+01	5.18E+03	9.65E+06	3.07E+07	1.99E+10
CFCDBCD BDCBA	1.72E+02	6.86E+03	1.33E+07	3.98E+07	2.69E+10
CGACAAABACBA	1.67E+01	5.18E+03	9.65E+06	3.07E+07	1.99E+10
CGACACABACBA	2.96E+01	4.45E+03	8.23E+06	2.67E+07	1.90E+10
CGADBAABDCBA	1.67E+01	5.18E+03	9.65E+06	3.07E+07	1.99E+10
CGADBCABDCBA	1.72E+02	6.86E+03	1.33E+07	3.98E+07	2.69E+10
CGCCAADBACBA	1.67E+01	5.18E+03	9.65E+06	3.07E+07	1.99E+10
CGCCACDBACBA	2.96E+01	4.45E+03	8.23E+06	2.67E+07	1.90E+10
CGCDBAD BDCBA	1.67E+01	5.18E+03	9.65E+06	3.07E+07	1.99E+10
CGCDBCD BDCBA	1.72E+02	6.86E+03	1.33E+07	3.98E+07	2.69E+10
CHACAAABACBA	1.67E+01	5.18E+03	9.65E+06	3.07E+07	1.99E+10
CHACACABACBA	2.96E+01	4.45E+03	8.23E+06	2.67E+07	1.90E+10

Table 5.2. (continued)

Accident Progression Bin	Mean Early Fatalities	Mean Latent Fatalities	Mean 50-Mile Dose (person-rem)	Mean 1000-Mile Dose (person-rem)	Mean Offsite Costs (\$)
CHADBAABDCBA	1.67E+01	5.18E+03	9.65E+06	3.07E+07	1.99E+10
CHADBCABDCBA	1.72E+02	7.09E+03	1.38E+07	4.15E+07	2.79E+10
DADDDABDBBB	1.01E+01	4.50E+03	8.91E+06	2.71E+07	1.70E+10
DADDDCBBDBBB	1.35E+00	1.45E+03	3.73E+06	9.28E+06	3.09E+09
DHACACABACBA	3.09E+00	2.14E+03	4.63E+06	1.29E+07	5.95E+09
DHADDABDBBBA	2.28E+01	5.80E+03	1.07E+07	3.45E+07	2.27E+10
DHADDCCBDBBBA	5.47E+02	9.58E+03	1.98E+07	5.72E+07	4.41E+10
EFACACABACBA	0.00E+00	3.33E+00	1.10E+04	2.00E+04	2.86E+05
EFADBCABDCBA	0.00E+00	3.33E+00	1.10E+04	2.00E+04	2.86E+05
EFCCACDBACBA	0.00E+00	3.33E+00	1.10E+04	2.00E+04	2.86E+05
EFCDCCBDCBA	0.00E+00	3.33E+00	1.10E+04	2.00E+04	2.86E+05
EGACACABACBA	4.67E-07	1.46E+02	4.75E+05	8.77E+05	1.01E+08
EGADBCABDCBA	1.60E-05	2.39E+02	7.53E+05	1.44E+06	1.79E+08
EGCCACDBACBA	4.67E-07	1.46E+02	4.75E+05	8.77E+05	1.01E+08
EGCDBCDBDCBA	1.60E-05	2.39E+02	7.53E+05	1.44E+06	1.79E+08
FDDCACABAEBB	0.00E+00	8.63E-03	2.28E+01	5.17E+01	0.00E+00
FDDDBCABDEBB	0.00E+00	8.63E-03	2.28E+01	5.17E+01	0.00E+00
FFACACABAEBB	0.00E+00	8.63E-03	2.28E+01	5.17E+01	0.00E+00
FFADBCABDEBA	0.00E+00	8.41E-03	2.18E+01	5.04E+01	0.00E+00
FGACACABAEBB	0.00E+00	8.63E-03	2.28E+01	5.17E+01	0.00E+00
FGADBCABDEBA	0.00E+00	8.41E-03	2.18E+01	5.04E+01	0.00E+00
FHACACABAEBB	0.00E+00	3.67E-02	1.43E+02	2.15E+02	1.32E+02
FHADBCABDEBA	0.00E+00	3.67E-02	1.43E+02	2.15E+02	1.32E+02
GDCCAADBAFBB	8.42E+00	4.23E+03	8.19E+06	2.48E+07	1.51E+10
GDCCACDBAGBB	0.00E+00	2.63E-02	9.67E+01	1.38E+02	0.00E+00
GDCDBADBDFFB	8.42E+00	4.23E+03	8.19E+06	2.48E+07	1.51E+10
GDCDBCDBDGBB	0.00E+00	2.63E-02	9.67E+01	1.38E+02	0.00E+00
GDCDFADBDFFB	8.42E+00	4.23E+03	8.19E+06	2.48E+07	1.51E+10
GDCDFCDBDGBB	0.00E+00	2.63E-02	9.67E+01	1.38E+02	0.00E+00
GDDCAAABAEBB	8.42E+00	4.23E+03	8.19E+06	2.48E+07	1.51E+10
GDDCAAABAFBB	8.42E+00	4.23E+03	8.19E+06	2.48E+07	1.51E+10
GDDCACABAGBB	0.00E+00	2.63E-02	9.67E+01	1.38E+02	0.00E+00
GDDDBAABDEBB	8.42E+00	4.23E+03	8.19E+06	2.48E+07	1.51E+10
GDDDBAABDFBB	8.42E+00	4.23E+03	8.19E+06	2.48E+07	1.51E+10
GDDDBCABDGBE	0.00E+00	2.63E-02	9.67E+01	1.38E+02	0.00E+00
GFACAAABACBA	8.42E+00	4.23E+03	8.19E+06	2.48E+07	1.51E+10
GFACAAABAEBB	8.42E+00	4.23E+03	8.19E+06	2.48E+07	1.51E+10
GFACAAABAFBA	8.42E+00	4.23E+03	8.19E+06	2.48E+07	1.51E+10
GFACACABAGBA	0.00E+00	2.63E-02	9.67E+01	1.38E+02	0.00E+00
GFADBAABDCBA	8.42E+00	4.23E+03	8.19E+06	2.48E+07	1.51E+10
GFADBAABDEBA	8.42E+00	4.23E+03	8.19E+06	2.48E+07	1.51E+10
GFADBAABDFBA	8.42E+00	4.23E+03	8.19E+06	2.48E+07	1.51E+10
GFADBCABDGBA	0.00E+00	2.63E-02	9.67E+01	1.38E+02	0.00E+00
GFCCAADBAFBA	8.42E+00	4.23E+03	8.19E+06	2.48E+07	1.51E+10
GFCCAADBAFBA	8.42E+00	4.23E+03	8.19E+06	2.48E+07	1.51E+10
GFCCACDBAGBA	0.00E+00	2.63E-02	9.67E+01	1.38E+02	0.00E+00
GFCDBADBDCBA	8.42E+00	4.23E+03	8.19E+06	2.48E+07	1.51E+10
GFCDBADBDFBA	8.42E+00	4.23E+03	8.19E+06	2.48E+07	1.51E+10
GFCDBCDBDGBA	0.00E+00	2.63E-02	9.67E+01	1.38E+02	0.00E+00
GGACAAABACBA	8.42E+00	4.23E+03	8.19E+06	2.48E+07	1.51E+10
GGACAAABAEBB	8.42E+00	4.23E+03	8.19E+06	2.48E+07	1.51E+10
GGACAAABAFBA	8.42E+00	4.23E+03	8.19E+06	2.48E+07	1.51E+10
GGACACABAGBA	0.00E+00	2.63E-02	9.67E+01	1.38E+02	0.00E+00

Table 5.2. (continued)

Accident Progression Bin	Mean Early Fatalities	Mean Latent Fatalities	Mean 50-Mile Dose (person-rem)	Mean 1000-Mile Dose (person-rem)	Mean Offsite Costs (\$)
GGADBAABDCBA	8.42E+00	4.23E+03	8.19E+06	2.48E+07	1.51E+10
GGADBAABDEBA	8.42E+00	4.23E+03	8.19E+06	2.48E+07	1.51E+10
GGADBAABDFBA	8.42E+00	4.23E+03	8.19E+06	2.48E+07	1.51E+10
GGADBCABDGBA	0.00E+00	2.63E-02	9.67E+01	1.38E+02	0.00E+00
GGCCAADBACBA	8.42E+00	4.23E+03	8.19E+06	2.48E+07	1.51E+10
GGCCAADB AFBA	8.42E+00	4.23E+03	8.19E+06	2.48E+07	1.51E+10
GGCCACDBAGBA	0.00E+00	2.63E-02	9.67E+01	1.38E+02	0.00E+00
GGCDBADBD CBA	8.42E+00	4.23E+03	8.19E+06	2.48E+07	1.51E+10
GGCDBADBD FBA	8.42E+00	4.23E+03	8.19E+06	2.48E+07	1.51E+10
GGCDBCD BDGBA	0.00E+00	2.63E-02	9.67E+01	1.38E+02	0.00E+00
GHACAAABA EBA	8.42E+00	4.23E+03	8.19E+06	2.48E+07	1.51E+10
GHACAAABA FBA	8.42E+00	4.23E+03	8.19E+06	2.48E+07	1.51E+10
GHACACABAGBA	0.00E+00	1.52E-01	6.17E+02	8.77E+02	3.80E+03
GHADBAABDEBA	8.42E+00	4.23E+03	8.19E+06	2.48E+07	1.51E+10
GHADBAABDFBA	8.42E+00	4.23E+03	8.19E+06	2.48E+07	1.51E+10
GHADBCABDGBA	0.00E+00	1.52E-01	6.17E+02	8.77E+02	3.80E+03

Table 5.3. Annual base case SBO risk

	Mean Early Fatalities	Mean Latent Fatalities	Mean 50-Mile Dose (person-rem)	Mean 1000-Mile Dose (person-rem)	Mean Offsite Costs (\$)
Reference 4	1E-05	9E-04	2.1	4.8	6E+03
SCET	2E-05	6E-04	1.2	3.5	2E+03
Relative change	0.5	1.5	1.8	1.4	3.0

Table 5.4. Containment failure mode contribution to offsite dose for the SBO PDS (from SCET)

Containment Failure Mode	Contribution to 50-Mile Dose (%)	Contribution to 1000-Mile Dose (%)
DCH	22.5	21.7
α mode	49.8	49.9
Bypass ^a	27.3	28.5
Late overpressure	ε	ε
BMT	ε	ε

a. Bypass failures include pre-existing leakage and induced SGTR.

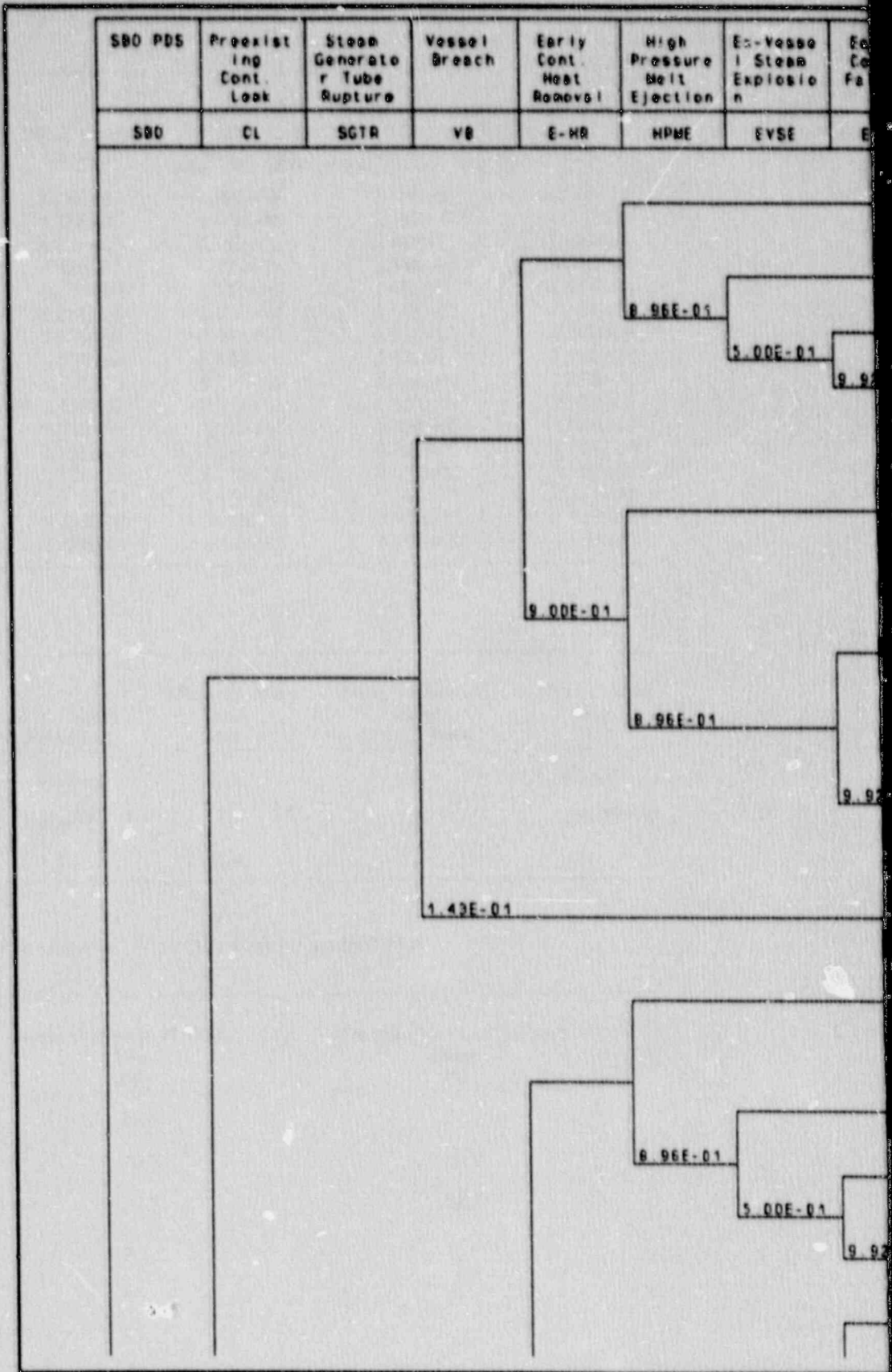


Figure 5.2. SBO SCET with intentional depressurization via the PORVs.

ly t. lure	late Cont. Heat Removal	Core- Concrete Interact ion	Very Lat e Cont. Heat Rem oval	Hydrogen Burn	Late OP Cont. Failure	Basemat Melt- Through	SEQUENCE PROB.	SEQUENCE CLASS
CF	L-HR	CCI	VL-HR	H2-BURN	L-OP	BMT		
		5.00E-01					2.23E-05	CL-PE
							2.23E-05	CL-PE
		5.00E-01					9.60E-05	CL
							9.60E-05	CL
							1.54E-06	ECF-CL-A
E-01		1.00E-01					1.71E-04	CL
							1.90E-05	CL
		2.00E-01					2.89E-04	CL-PE
							7.22E-05	CL-PE
	1.00E-01		1.00E-01				2.89E-05	CL-PE
		2.00E-01					3.21E-06	CL-PE
							8.02E-06	CL-PE
	1.00E-01						2.49E-05	ECF-CL-A
			1.00E-01				2.49E-06	ECF-CL-A
							2.76E-07	ECF-CL-A
							2.96E-03	CL
E-01		4.00E-02					1.23E-04	CL
	1.00E-01		1.00E-01				2.99E-04	CL
		3.00E-02					3.32E-05	CL
							1.03E-05	CL
							7.15E-04	CL-NOVB
						8.00E-01	8.87E-04	PE-BMT
		5.00E-01					3.55E-03	PE-NCF
							4.43E-03	PE-NCF
						7.00E-01	5.73E-03	BMT
		5.00E-01					1.34E-02	NCF
							1.91E-02	NCF
							3.06E-04	ECF-A
						6.00E-01	1.36E-02	BMT
E-01		1.00E-01					2.05E-02	NCF
							3.79E-03	NCF
							1.13E-02	ECF-PE-D

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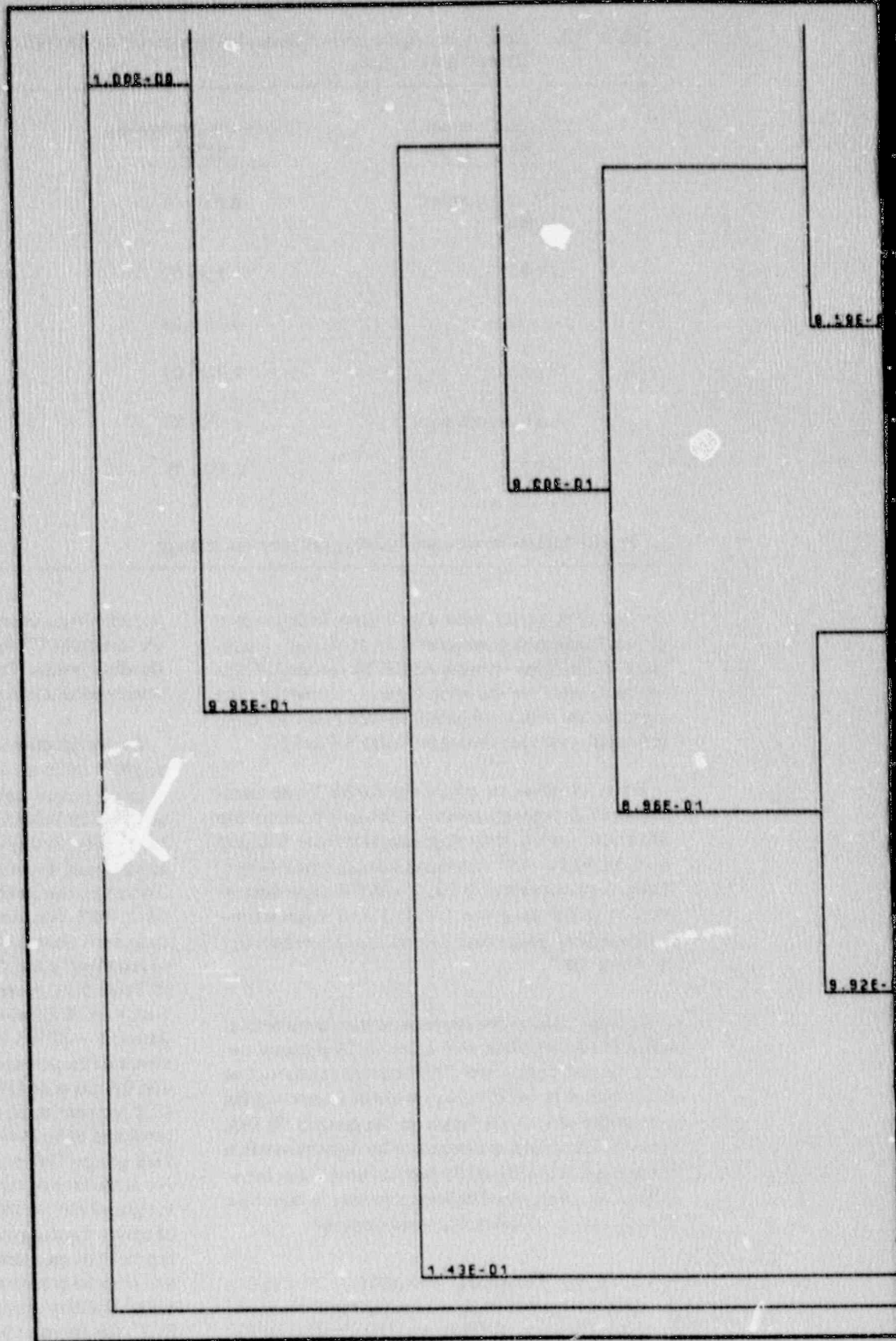
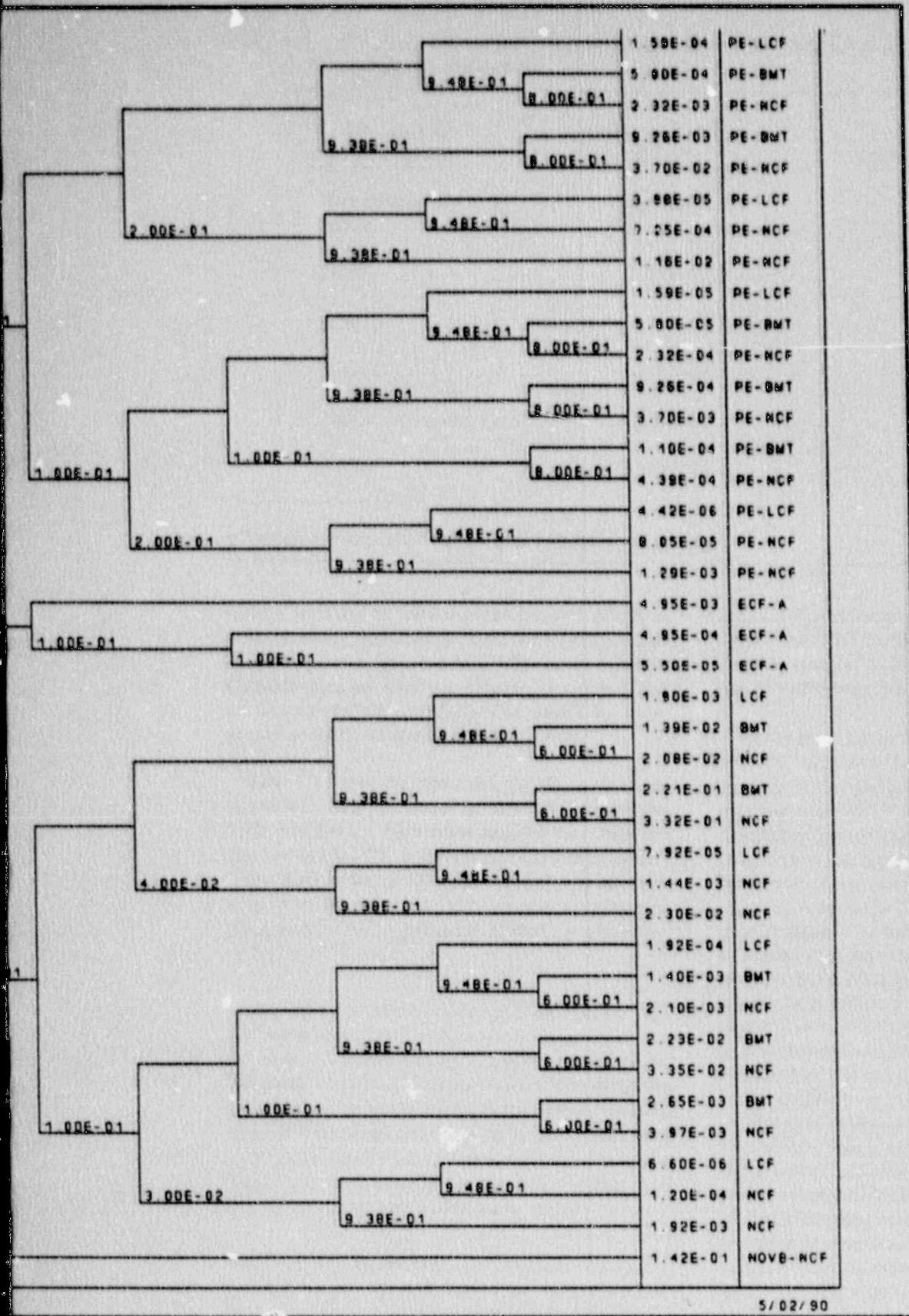


Figure 5.2. (continued).



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Table 5.5. Conditional containment failure probabilities for the SBO PDS with operator depressurization (HPME not eliminated)

Containment Failure Mode	Conditional Probability (SCET)	Base Case Conditional Probability
No containment failure	6.83E-01	6.76E-01
DCH	1.13E-02	1.13E-02
α mode	5.84E-03	6.13E-03
Bypass ^a	4.97E-03	6.35E-03
Late overpressure	2.40E-03	2.34E-03
BMT	2.93E-01	2.97E-01

a. Bypass failures involve pre-existing containment leakage.

second sensitivity used a pressure-independent α mode failure probability of 8.0×10^{-4} , with no sampling on Question 34 in the APET. The revised SCETs for these cases are shown in Figures 5.3 and 5.4. The containment failure probabilities calculated for these sensitivity cases are shown in Tables 5.6 and 5.7.

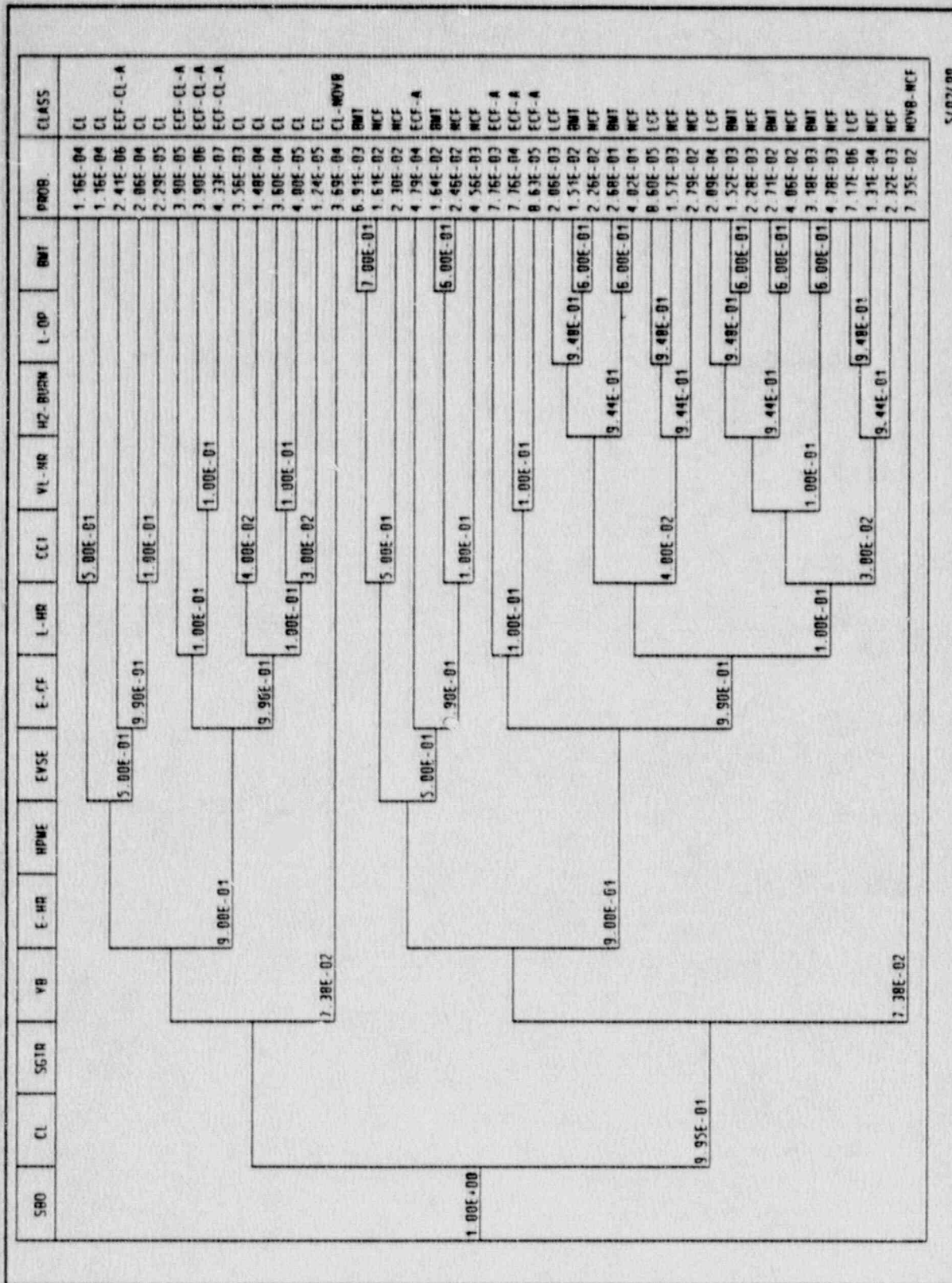
Table 5.8 shows the offsite risk for SBO with intentional RCS depressurization. Table 5.9 shows the SBO risk for the fully depressurized case with the draft NUREG-1150 distribution for α mode failure. Table 5.10 shows the SBO risk with full depressurization to <200 psig (no HPME) and a pressure-independent point estimate α mode probability of 8.0×10^{-4} .

As these tables show, depressurization does not significantly lower SBO risk unless RCS pressure can be reduced below the DCH cutoff (assumed in Reference 4 to be 200 psig) without increasing the probability of α mode failure in the process. In fact, some risk measures are increased by depressurization if the draft NUREG-1150 probabilities of α mode failure are used, since depressurization in this case increases the probability of an early release.

5.4.2 Cavity Flooding. The addition of a cavity flooding system was analyzed by appropriately modifying the split fraction of Question 31 in the Zion APET, turning off sampling for this question, and running EVNTRE and PSTEVNT to generate the revised SCET

and ZISOR accident progression bins. Figure 5.5 shows the revised SCET for SBO with the addition of a cavity flooding system. Table 5.11 shows the revised conditional containment failure probabilities.

Cavity flooding for SBO reduced DCH failures, but slightly increased the conditional probability of α mode failure and late overpressurization. The output from the PSTEVNT runs also showed a very slight probability of early containment failure as the result of an ex-vessel steam explosion following vessel breach. However, the probability of this event was only $\sim 4 \times 10^{-6}$. Because EVNTRE is a single-precision code, such a low probability could simply be the result of round-off error; therefore, it has not been included in Table 5.11. Note that the decrease in the conditional DCH failure probability is in contrast to the transient + ATWS PDS group, where cavity flooding increased the probability of DCH failure. The explanation for this is as follows. In the SBO PDS group, the RCS pressure at the time of vessel breach is generally predicted to be lower than in the transient + ATWS PDS group. This leads to a lower predicted pressure rise at the time of vessel breach, which in turn reduces the probability of DCH failure. Therefore, the effects of cavity flooding on the probability of DCH failure appear to be pressure-dependent; if the RCS is at system set point pressure at the time of vessel breach, then cavity flooding appears to exacerbate the threat from DCH. On the other hand, if RCS pressure is less than system set point pressure, then cavity flooding may offer some benefit in mitigating DCH.



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Figure 5.3. SBO SCET with despressurization to <200 psig (no HP'4E) and pressure-dependent alpha mode probability.

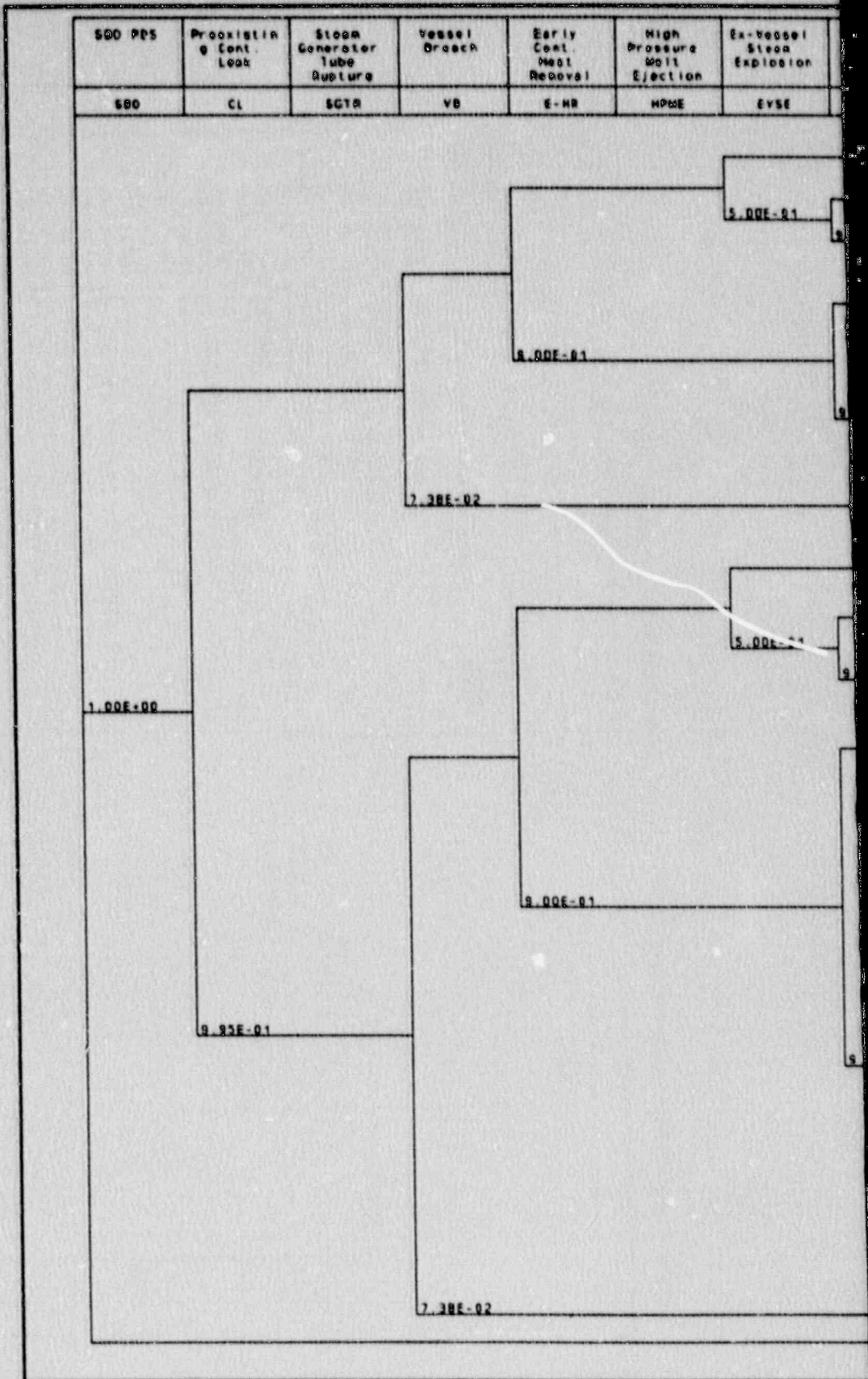


Figure 5.4. SBO SCET with depressurization to <200 psig (no HPME) and pressure-independent point estimate α mo

Early Cont. Failure	Late Cont. Mgmt Removal	Core-Concrete Interaction	Very Late Cont. Mgmt Removal	Hydrogen Burn	Late DP Cont. Failure	Reagent Melt-Through	SEQUENCE PROB.	SEQUENCE CLASS
E-CF	L-NR	CCI	VL-NR	H2-BURN	L-OP	DNT		
		3.00E-01					1.16E-04	CL
							1.16E-04	CL
							1.95E-07	ECF-CL-A
89E-01		1.00E-01					2.08E-04	CL
							2.31E-05	CL
							3.18E-08	ECF-CL-A
	1.00E-01		1.88E-01				3.16E-07	ECF-CL-A
							3.51E-08	ECF-CL-A
		4.80E-02					3.60E-03	CL
89E-01							1.50E-04	CL
	1.00E-01		1.00E-01				3.64E-04	CL
		3.00E-02					4.04E-05	CL
							1.25E-05	CL
							3.89E-04	CL-NOVB
						7.00E-01	6.81E-03	BMT
		5.00E-01					1.81E-02	NCF
							2.30E-02	NCF
							3.08E-05	ECF-A
89E-01						8.00E-01	1.68E-02	BMT
	1.00E-01						2.49E-02	NCF
							4.80E-03	NCF
							5.28E-04	ECF-A
	1.00E-01		1.00E-01				6.28E-05	ECF-A
							6.98E-06	ECF-A
							2.81E-03	LCP
						9.35E-01	1.50E-02	BMT
						6.00E-01	2.25E-02	NCF
			9.44E-01				2.70E-01	BMT
						6.00E-01	4.06E-01	NCF
		4.00E-02				9.35E-01	1.09E-04	LCP
			9.44E-01				1.56E-03	NCF
							2.82E-02	NCF
89E-01						9.35E-01	2.83E-04	LCP
						6.00E-01	1.52E-03	BMT
						9.44E-01	2.27E-03	NCF
			1.00E-01			6.00E-01	2.73E-02	BMT
						6.00E-01	4.10E-02	NCF
	1.00E-01					6.00E-01	3.22E-03	BMT
							4.82E-03	NCF
		3.00E-02				9.35E-01	9.05E-06	LCP
			9.44E-01				1.30E-04	NCF
							2.35E-03	NCF
							7.35E-02	NOVB-NCF

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Probability.

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Table 5.6. Conditional containment failure probabilities for the SBO PDS with operator depressurization and pressure-dependent α mode probability (no HPME)

<u>Containment Failure Mode</u>	<u>Conditional Probability (SCET)</u>	<u>Base Case Conditional Probability</u>
No containment failure	6.46E-01	6.76E-01
DCH	0.00	1.13E-02
α mode	9.15E-03	6.13E-03
Bypass ^a	4.95E-03	6.35E-03
Late overpressure	2.36E-03	2.34E-03
BMT	3.38E-01	2.97E-01

a. Bypass failures involve pre-existing containment leakage.

Table 5.7. Conditional containment failure probabilities for the SBO PDS with operator depressurization and pressure-independent point estimate α mode probability (no HPME)

<u>Containment Failure Mode</u>	<u>Conditional Probability (SCET)</u>	<u>Base Case Conditional Probability</u>
No containment failure	6.51E-01	6.76E-01
DCH	0.00	1.13E-02
α mode	7.41E-04	6.13E-03
Bypass ^a	5.00E-03	6.35E-03
Late overpressure	2.99E-03	2.34E-03
BMT	3.41E-01	2.97E-01

a. Bypass failures involve pre-existing containment leakage.

Table 5.8. Annual SBO risk with intentional RCS depressurization (HPME not eliminated)

	Mean Early Fatalities	Mean Latent Fatalities	Mean 50-Mile Dose (person-rem)	Mean 1000-Mile Dose (person-rem)	Mean Offsite Costs (\$)
Base case	1.99E-05	5.87E-04	1.21	3.49	2.34E+03
SCET	1.91E-05	5.38E-04	1.11	3.20	2.15E+03
% change	-4.1	-8.5	-8.2	-8.6	-8.5

Table 5.9. Annual SBO risk with RCS depressurization <200 psig and pressure-dependent α mode probability (no HPME)

	Mean Early Fatalities	Mean Latent Fatalities	Mean 50-Mile Dose (person-rem)	Mean 1000-Mile Dose (person-rem)	Mean Offsite Costs (\$)
Base case	1.99E-05	5.87E-04	1.21	3.49	2.34E+03
SCET	2.84E-05	5.85E-04	1.20	3.47	2.58E+03
% change	43.2	-0.3	-0.9	-0.6	-10.1

Table 5.10. Annual SBO risk with RCS depressurization <200 psig and pressure-independent point estimate α mode probability (no HPME)

	Mean Early Fatalities	Mean Latent Fatalities	Mean 50-Mile Dose (person-rem)	Mean 1000-Mile Dose (person-rem)	Mean Offsite Costs (\$)
Base case	1.99E-05	5.87E-04	1.21	3.49	2.34E+03
SCET	5.74E-06	1.87E-04	3.70E-01	1.09	7.48E+02
% change	-71.2	-68.2	-69.4	-68.8	-68.1

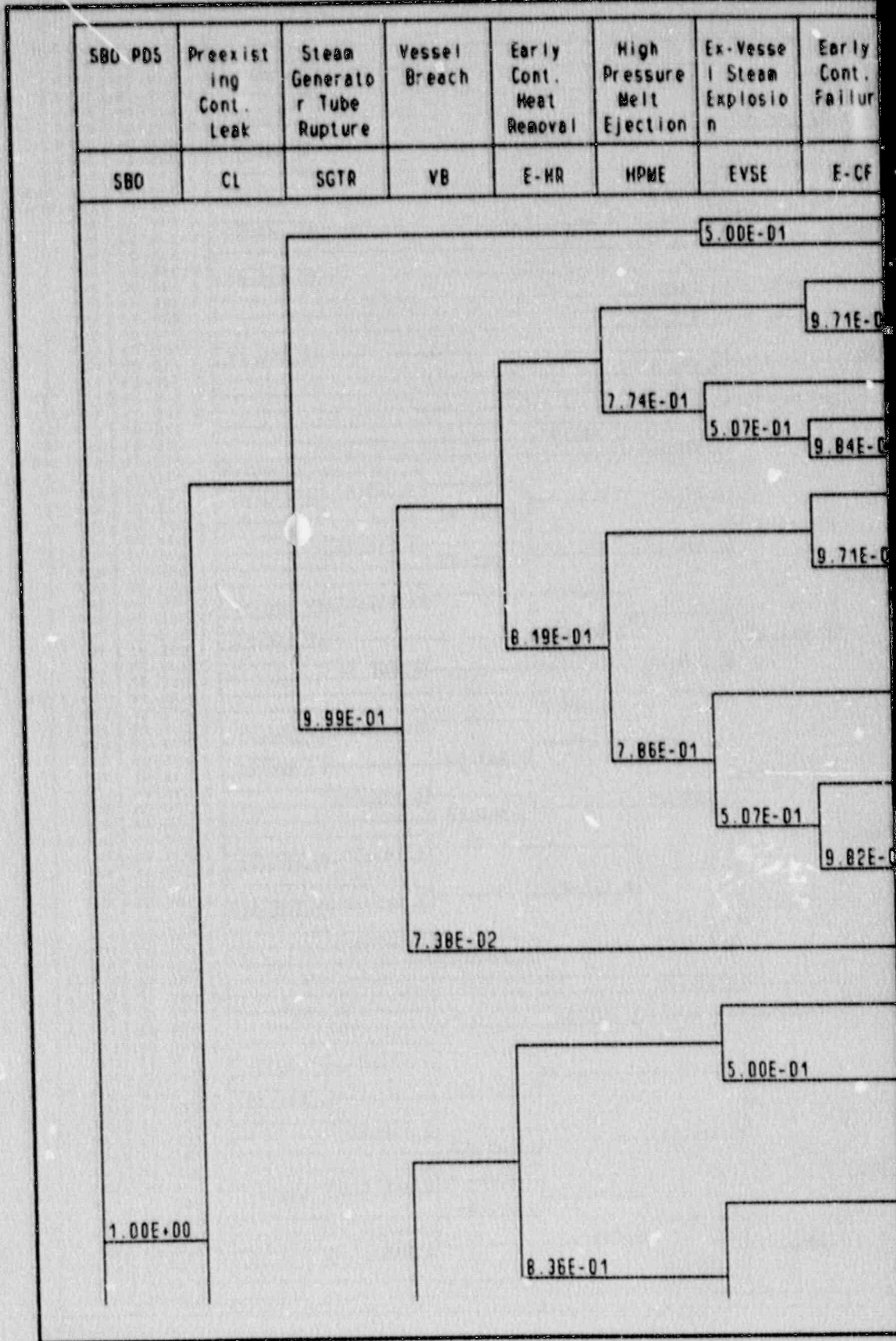


Figure 5.5. SBO SCET with cavity flooding system.

Late Cont. Heat Removal	Core-Concrete Interaction	Very Late Cont. Heat Removal	Hydrogen Burn	Late OP Cont. Failure	Basemat Melt-Through	SEQUENCE PROB.	SEQUENCE CLASS
L-HR	CCI	VL-HR	H2-BRN	L-OP	BMT		
						2.49E-06	SGTR
						2.49E-06	SGTR
	5.00E-01					2.77E-06	CL-DCH
						2.77E-06	CL-DCH
	5.00E-01					9.19E-05	CL
						9.19E-05	CL
	5.00E-01					1.60E-04	CL
						1.60E-04	CL
						5.30E-06	CL-A
	1.05E-01					2.89E-04	CL
						3.38E-05	CL
						2.35E-05	CL-DCH
	5.00E-01					1.95E-05	CL
						1.95E-05	CL
9.51E-01		9.91E-01				6.53E-06	CL
	8.72E-03					7.36E-04	CL
						6.53E-06	CL
	5.00E-01					3.68E-05	CL
						3.68E-05	CL
9.50E-01		9.95E-01				6.79E-06	CL
	4.87E-03					1.38E-03	CL
						6.79E-06	CL
9.47E-01						1.40E-06	CL-A
						2.49E-05	CL-A
	1.04E-01					6.66E-05	CL
						7.75E-06	CL
9.50E-01		9.91E-01				1.23E-05	CL
	1.06E-03					1.39E-03	CL
						1.49E-06	CL
						3.69E-04	CL-NOVB
					7.00E-01	1.57E-05	SG-BMT
	5.00E-01					3.67E-05	SGTR
						5.24E-05	SGTR
					6.00E-01	3.77E-05	SGTR
	1.00E-01					5.66E-05	SGTR
						1.05E-05	SGTR
					7.00E-01	4.55E-06	SGTR
	5.00E-01					1.06E-05	SGTR
						1.52E-05	SGTR
						1.77E-06	SGTR
9.43E-01		9.96E-01			7.00E-01	1.49E-04	SGTR
	5.04E-03					3.47E-04	SGTR
						2.52E-06	SGTR

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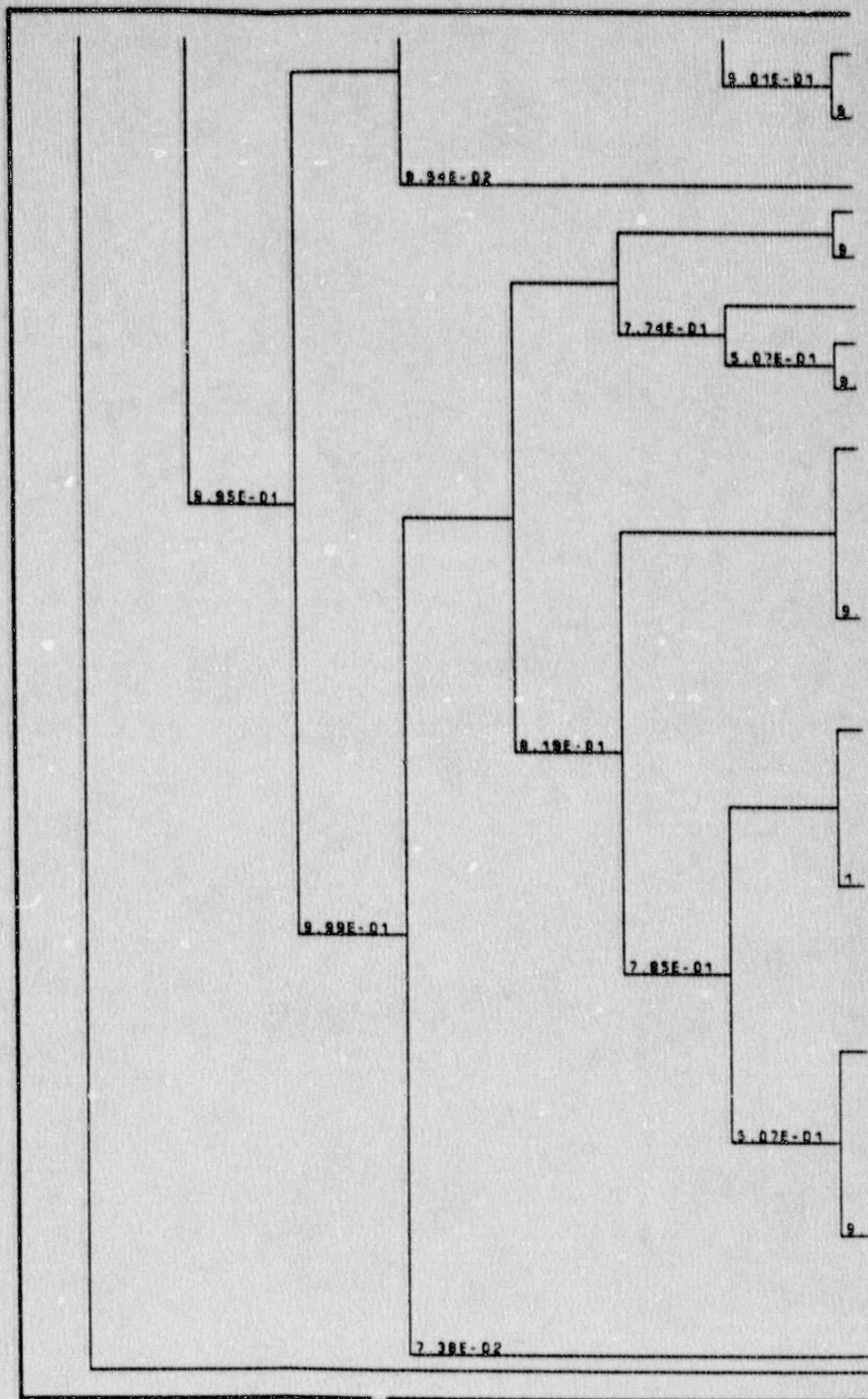
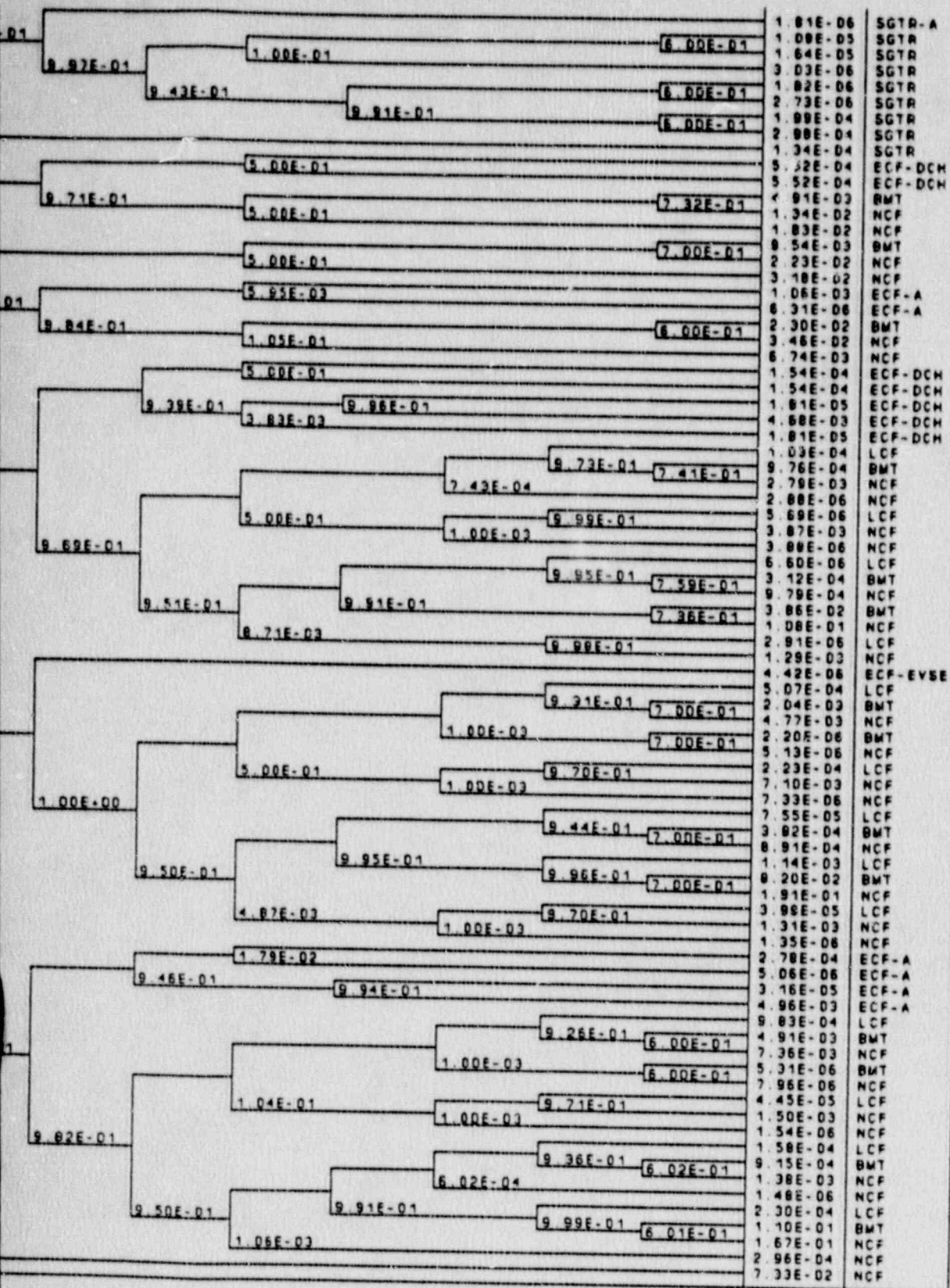


Figure 5.5. (continued).



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Table 5.12 shows the SBO risk with the addition of the cavity flooding system.

5.4.3 Hydrogen Control. Improved hydrogen control was modeled in the SCET by eliminating late overpressure containment failure by setting the probability for the lower branch of event L-OP to 1.0 for all sequences. The new conditional containment failure probabilities are shown in Table 5.13.

Elimination of hydrogen-induced overpressure failures provided no reduction in any of the risk measures used in this report. This finding is consistent with Reference 4, where late overpressure failure was a negligible contributor to risk.

5.4.4 Containment Venting. As in the LOCA PDS, containment venting is not evaluated for SBO. See the discussion of containment venting in Section 3.4.4.

Table 5.11. Conditional containment failure probabilities for the SBO PDS with cavity flooding system

Containment Failure Mode	Conditional Probability (SCET)	Base Case Conditional Probability
No containment failure	7.00E-01	6.76E-01
DCH	6.13E-03	1.13E-02
α mode	6.37E-03	6.13E-03
Bypass ^a	6.37E-03	6.35E-03
Late overpressure	3.52E-03	2.34E-03
BMT	2.78E-01	2.97E-01

a. Bypass failures include pre-existing leakage and induced SGTR.

Table 5.12. Annual SBO risk with cavity flooding system

	Mean Early Fatalities	Mean Latent Fatalities	Mean 50-Mile Dose (person-rem)	Mean 1000-Mile Dose (person-rem)	Mean Offsite Costs (\$)
Base case	1.99E-05	5.87E-04	1.21	3.49	2.34E+03
SCET	1.81E-05	4.88E-04	1.00	2.89	1.99E+03
% change	-9.2	-17.8	-17.1	-17.3	-15.1

Table 5.13. Conditional containment failure probabilities for the SBO PDS with improved hydrogen control

<u>Containment Failure Mode</u>	<u>Conditional Probability (SCET)</u>	<u>Base Case Conditional Probability</u>
No containment failure	6.78E-01	6.76E-01
DCH	1.13E-02	1.13E-02
α mode	6.13E-03	6.13E-03
Bypass ^a	6.35E-03	6.35E-03
Late overpressure	0.00	2.34E-03
BMT	2.98E-01	2.97E-01

a. Bypass failures include pre-existing leakage and induced SGTR.

6. CONTAINMENT BYPASS SEQUENCES

Reference 4 splits sequences that involve containment bypass into two PDS groups. The first group, group 4, is made up of four PDSs in which the initiating event is SGTR. In one of these PDSs, the secondary relief valves stick open, resulting in a release that bypasses both primary containment and the secondary systems. In the other three PDSs, the secondary relief valves do not stick open. The second group, group 5, is made up of a single PDS, initiated by an interfacing systems LOCA (V sequence). This is assumed to be a large break in the low pressure RCS piping outside primary containment in the auxiliary building.

6.1 Bypass Core Damage Frequency

For the purposes of the accident progression analysis in this report, the convention adopted in Reference 4 of grouping together the V sequence and the three group 4 PDSs involving SGTR with no stuck-open secondary relief valves is followed. This grouping will be referred to as the V + SGTR PDS in this report. The remaining bypass sequence, which involves an initial SGTR with the secondary relief valves stuck open, will be referred to as the SGTR PDS. The annual frequencies of these two PDSs are taken from Table 2.2-3 in Reference 4 and are listed here for convenience.

V + SGTR: 1.69×10^{-7} per reactor-year

SGTR: 1.30×10^{-6} per reactor-year

6.2 Bypass SCET Results

One base case SCET was developed for each of the two bypass PDSs. Figure 6.1 shows the base case SCET for the V + SGTR PDS, Figure 6.2 the SGTR PDS alone.

Table 6.1 shows the conditional containment failure probabilities for the V + SGTR PDS.

Table 6.2 shows the analogous results for the SGTR PDS.

6.3 Base Case Bypass Consequences

The SCET end states were binned into accident progression bins and the ZISOR code was again used to generate source terms for these bins as described in Section 2. Conditional consequences were then calculated for each accident progression bin group with the MACCS code. As for the previous PDS groups, the Zion site data and meteorological files were used for these calculations. The conditional consequences for each accident progression bin are shown in Tables 6.3 and 6.4 for the V + SGTR and SGTR PDSs, respectively.

Tables 6.5 and 6.6 show the annual risk calculated for the V + SGTR and SGTR PDSs. The risk from Reference 4 is a combination of the risk from the V + SGTR and the SGTR PDS groups.

Calculations for this report slightly overestimate all risk measures for the SGTR PDS, except early fatalities, which are slightly underestimated. The higher values are very likely attributable to the use of point estimate source terms from ZISOR and the use of a later version of MACCS than was used for Reference 4. Most of the risk for the bypass sequences comes from the SGTR PDS. This result cannot be ascertained from Reference 4 but is plausible for two reasons. First, as Table 6.4 shows, the conditional consequences are generally higher than for the V + SGTR PDS group. Secondly, the core damage frequency of the SGTR PDS is significantly greater than that of the V + SGTR group. Recall that the SGTR PDS involves an initiating SGTR with the secondary relief valves stuck open. This provides a direct release path to the environment. Sequences in the V + SGTR group involve a V break or SGTR with no stuck-open secondary relief valves. Therefore, the release path is through the auxiliary building, providing some decontamination of the release.

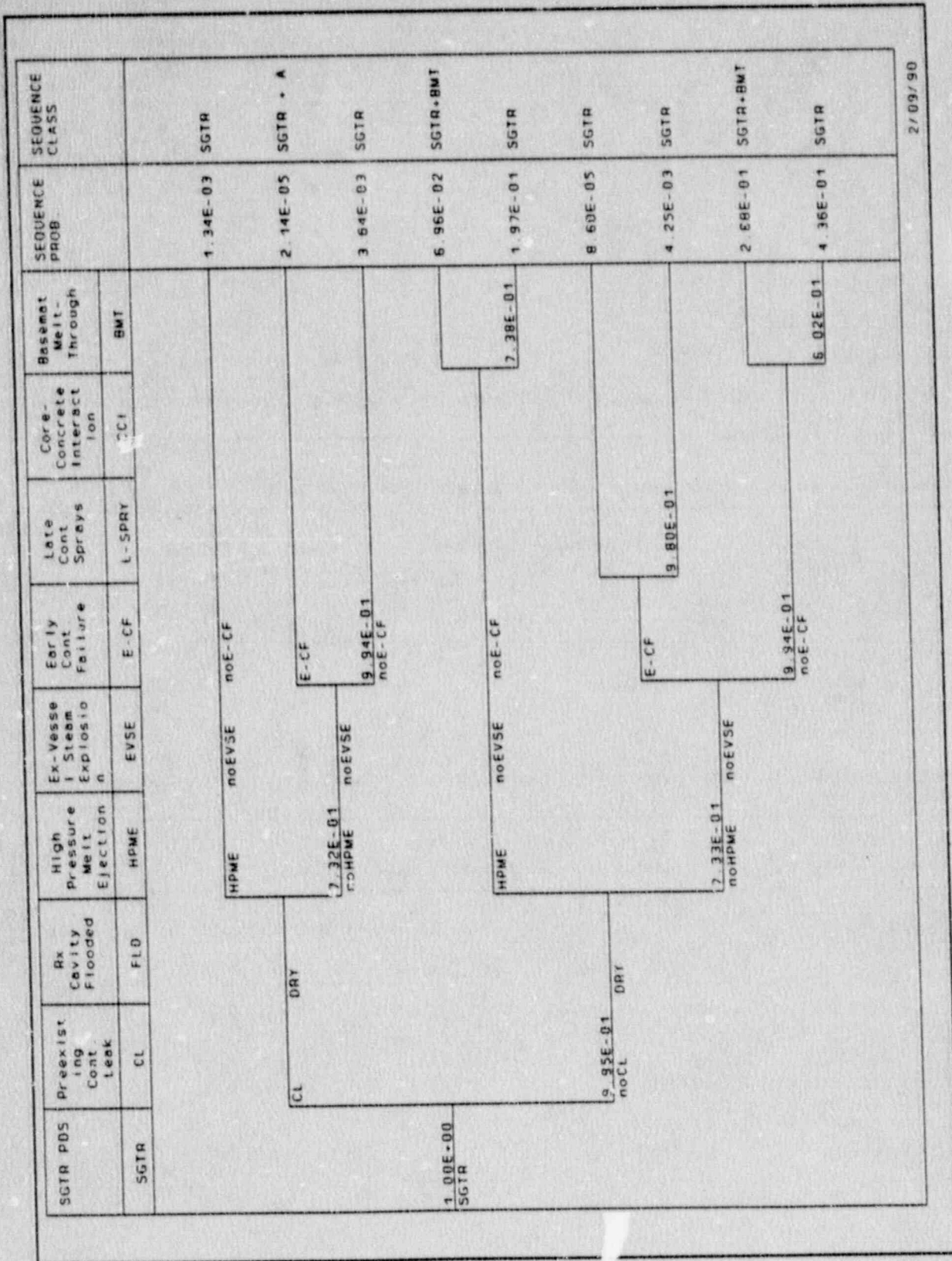
6.4 Risk Benefits of Potential Improvements

The risk benefit of the potential containment performance improvements identified in Reference 2 is calculated in this section.

V+SGTR PDS	V Break	Pre-existing Cont Leak	V Break Submerged	Dr Cavity Flooded	High Pressure Melt Ejection	Evap Steam Explosion	Early Cont Failure	Late Cont Sprays	Core-Concrete Interaction	Basemat Melt Through	SEQUENCE PROB.	SEQUENCE CLASS
V	BYPASS	CL	V-LDC	FLD	HPME	EVSE	E-CF	L-SPRT	CCI	BMT	8 98E-06	V+A
			V-WET	DRY	noHPME	noEVSE	9.94E-01 noE-CF				1 62E-03	V
			5.00E-01 V-DRY	DRY	noHPME	noEVSE	E-CF				2 39E-05	V+A
							9.95E-01 noCL				1 50E-03	V
	B-V		V-WET	DRY	noHPME	noEVSE	E-CF				1 79E-03	V+A
			5.00E-01 V-DRY	DRY	noHPME	noEVSE	9.94E-01 noE-CF			6.00E-01	1 29E-01	V+BMT
1 00E+00							E-CF				1 93E-01	V
			5.00E-01 V-DRY	DRY	noHPME	noEVSE	9.95E-01 noE-CF				4 76E-03	V+A
							E-CF				1 27E-01	V+BMT
							9.95E-01 noE-CF			6.00E-01	1 91E-01	V
			V-DRY	DRY	HPME	noEVSE	noE-CF				4 68E-04	SGTR
					7.32E-01 noHPME	noEVSE	E-CF				7 47E-06	SGTR+A
							9.94E-01 noE-CF				1 27E-03	SGTR
	3.50E-01 B-SGTR										2 44E-02	SGTR+BMT
			9.95E-01 noCL		HPME	noEVSE	noE-CF			7.38E-01	6 88E-02	SGTR
							E-CF				1 52E-03	SGTR+A
			V-DRY	DRY	7.32E-01 noHPME	noEVSE	9.94E-01 noE-CF				1 01E-01	SGTR+BMT
							E-CF				1 53E-01	SGTR
							9.94E-01 noE-CF			6.00E-01		

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Figure 6.1. Base case SCET for the V + SGTR PDS.



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Figure 6.2. Base case SCET for the SGTR PDS.

Table 6.1. Conditional containment failure probabilities for the V + SGTR PDS

Containment Failure Mode	Conditional Probability (SCET)	Conditional Probability (Reference 4)
Event V only	3.87E-01	1.00 ^a
Event V + α	6.58E-03	0.00 ^a
Event V + BMT	2.56E-01	0.00 ^a
SGTR only	2.24E-01	0.00 ^a
SGTR + α	1.53E-03	0.00 ^a
SGTR + BMT	1.25E-01	0.00 ^a

a. Reference 4 does not subdivide the containment failure modes for the bypass sequences; all end states are grouped into the bypass category.

Table 6.2. Conditional containment failure probabilities for the SGTR PDS

Containment Failure Mode	Conditional Probability (SCET)	Conditional Probability (Reference 4)
SGTR only	6.42E-01	1.00 ^a
SGTR + α	2.14E-05	0.00 ^a
SGTR + BMT	3.58E-01	0.00 ^a

a. Reference 4 does not subdivide the containment failure modes for the bypass sequences; all end states are grouped into the bypass category.

Table 6.3. Conditional consequences for the V + SGTR accident progression bins

Accident Progression Bin	Mean Early Fatalities	Mean Latent Fatalities	Mean 50-Mile Dose (person-rem)	Mean 1000-Mile Dose (person-rem)	Mean Offsite Costs (\$)
AAADDCBADBBB	2.94E+01	3.83E+03	7.33E+06	2.30E+07	1.54E+10
ADADBCAADCB	2.94E+01	3.83E+03	7.33E+06	2.30E+07	1.54E+10
ADADBCAADFB	3.01E+01	4.24E+03	8.23E+06	2.58E+07	1.71E+10
BAADDCBADBBB	1.69E+01	1.43E+03	3.63E+06	8.44E+06	3.42E+09
BDADBCAADCB	1.69E+01	1.43E+03	3.63E+06	8.44E+06	3.42E+09
BDADBCAADFB	1.70E+01	1.56E+03	3.91E+06	9.39E+06	3.67E+09
CDACAAAABCBB	2.95E+00	3.94E+03	7.30E+06	2.35E+07	1.51E+10
CDADBAAADCB	2.95E+00	3.94E+03	7.30E+06	2.35E+07	1.51E+10
DAADDABADBBB	3.53E+00	4.25E+03	8.18E+06	2.59E+07	1.70E+10
DDADDABADBBB	2.95E+00	3.94E+03	7.30E+06	2.35E+07	1.51E+10
GDACAAAABFB	2.45E+00	3.91E+03	7.23E+06	2.34E+07	1.50E+10
GDADBAAADFBB	2.45E+00	3.91E+03	7.23E+06	2.34E+07	1.50E+10

Table 6.4. Conditional consequences for the SGTR accident progression bins

Accident Progression Bin	Mean Early Fatalities	Mean Latent Fatalities	Mean 50-Mile Dose (person-rem)	Mean 1000-Mile Dose (person-rem)	Mean Offsite Costs (\$)
CDACABBABCBB	4.40E+00	6.95E+03	1.10E+07	4.18E+07	3.70E+10
CDADBBAAADCBB	4.41E+00	6.97E+03	1.10E+07	4.19E+07	3.70E+10
DAADDBBADDBBB	4.82E+00	7.34E+03	1.16E+07	4.45E+07	3.91E+10
DDADDBBADDBBB	4.41E+00	6.97E+03	1.10E+07	4.19E+07	3.70E+10
GDACABBABFBB	4.40E+00	6.95E+03	1.09E+07	4.17E+07	3.70E+10
GDADBBAAADFBB	4.40E+00	6.95E+03	1.09E+07	4.17E+07	3.70E+10

Table 6.5. Annual base case V + SGTR risk

	Mean Early Fatalities	Mean Latent Fatalities	Mean 50-Mile Dose (person-rem)	Mean 1000-Mile Dose (person-rem)	Mean Offsite Costs (\$)
Reference 4	6E-06	5E-03	10.4	30.1	3.7E+04
SCET	3E-06	6E-04	1.1	3.3	2E+03
Relative change	2.0	8.3	9.5	9.1	18.5

Table 6.6. Annual base case SGTR risk

	Mean Early Fatalities	Mean Latent Fatalities	Mean 50-Mile Dose (person-rem)	Mean 1000-Mile Dose (person-rem)	Mean Offsite Costs (\$)
Reference 4	6E-06	5E-03	10.4	30.1	4E+04
SCET	6E-06	9E-03	14.2	54.2	5E+04
Relative change	0.0	0.6	0.7	0.6	0.8

6.4.1 Enhanced Reactor Vessel Depressurization Capability. As was the case for the transient + ATWS and SBO PDS groups, intentional depressurization via the pressurizer PORVs could not be accurately modeled with an SCET. Therefore, depressurization was modeled in the APET using the revised methodology discussed in Section 4.4.1. The revised SCETs are shown in Figures 6.3 and 6.4. The new conditional probabilities of containment failure are shown in Tables 6.7 and 6.8.

Intentionally opening the PORVs does not change the overall probability of containment bypass, as the containment was bypassed by the initiating event. It can, however, alter the conditional probabilities of other accompanying modes of containment failure. The most significant of these is the conditional probability of α mode failure given reactor vessel breach, which increases in the depressurized case. The effects of this change on risk are shown in Tables 6.9 and 6.10.

Not surprisingly, depressurization has no effect on risk resulting from bypass sequences, even though the conditional probability of α mode failure is higher in the depressurized case. The fact that the release bypasses containment is more important in terms of offsite risk than the additional containment breach that results from α mode failure. In other words, α mode failure is a higher order contributor to risk in the bypass PDS group.

6.4.2 Cavity Flooding. As for the case of SBO, the addition of a cavity flooding system was analyzed by appropriately modifying the split fraction of Question 1 in the Zion APET, turning off sampling for this question, and running EVNTRE and PSTEVNT to

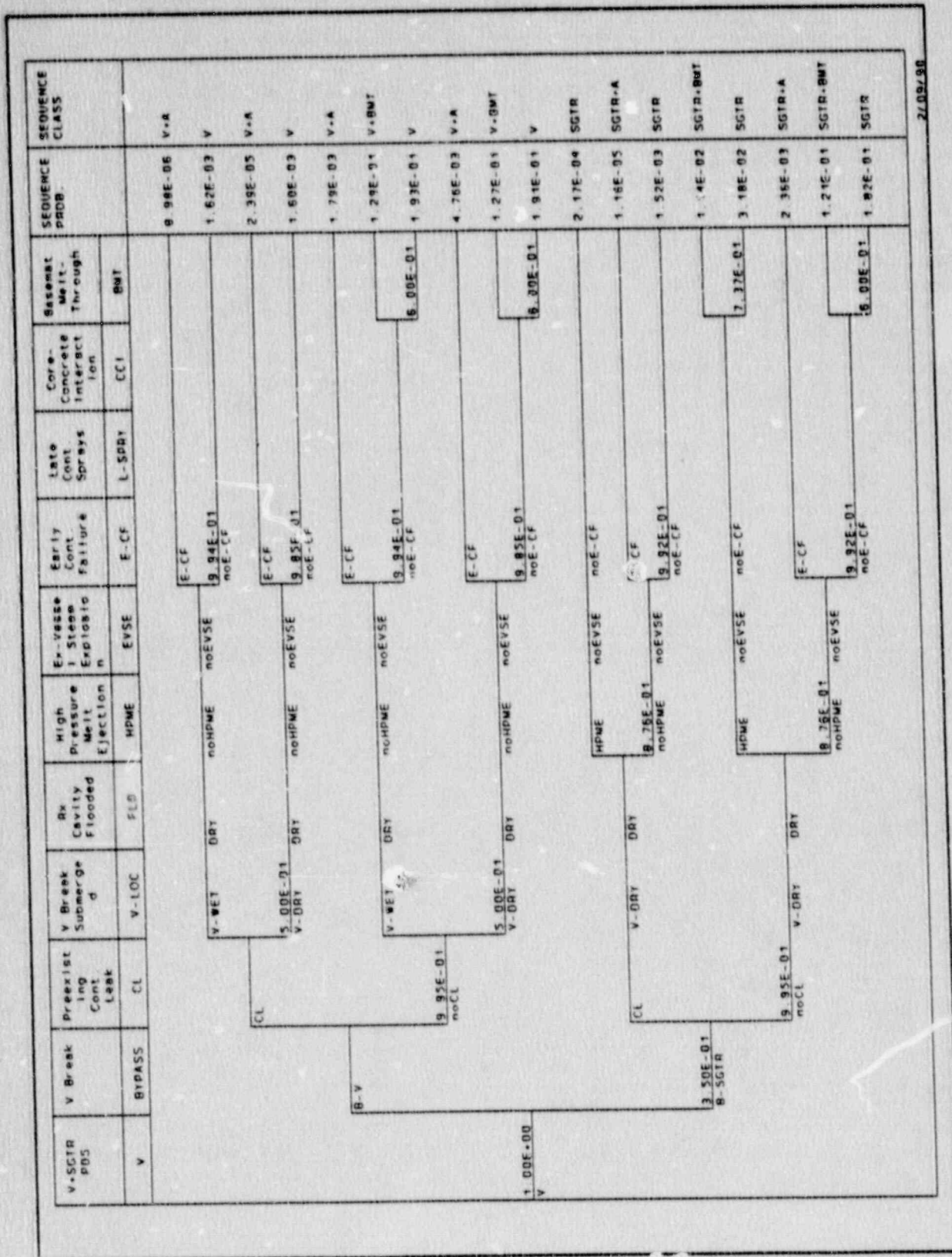
generate the revised SCET and ZISOR accident progression bins. Figures 6.5 and 6.6 show the revised SCETs for the bypass PDS groups with the addition of a cavity flooding system.

Tables 6.11 and 6.12 show the conditional containment failure probabilities with the addition of the cavity flooding system.

The preceding two tables show an interesting result, namely the occurrence of DCH failures as a result of having a flooded cavity at the time of vessel breach. This is a result that is very specific to the modeling assumptions in the Zion APET. Reference 2 discusses the effects on DCH of having a flooded reactor cavity but no firm conclusions were drawn, because some calculations showed a beneficial effect, while others showed detrimental effects. The effects on risk are shown in Tables 6.13 and 6.14.

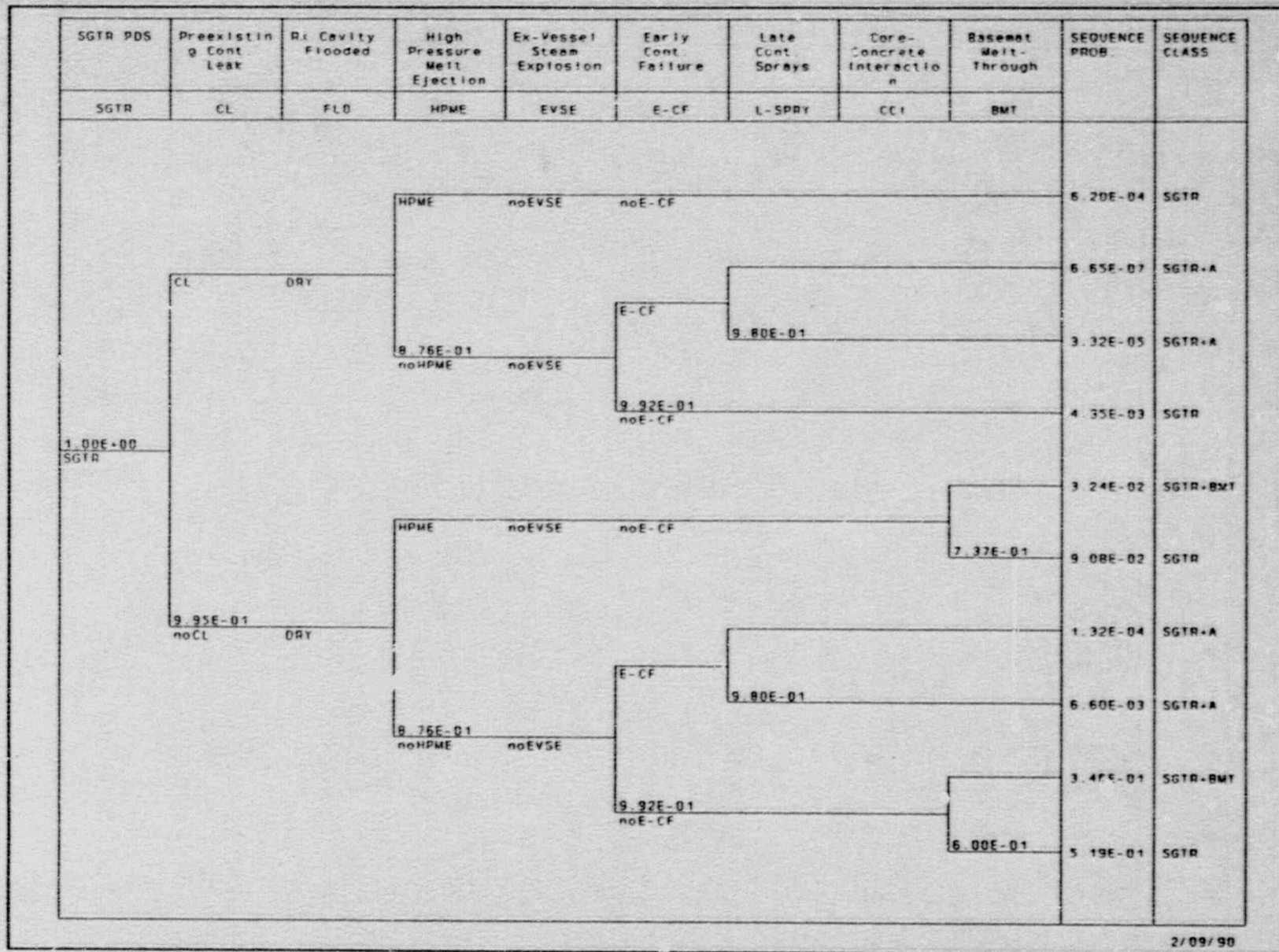
As was the case for depressurization, the addition of a cavity flooding system has an insignificant effect on bypass risk. The slight reduction in risk for the V + SGTR group is probably attributable to scrubbing of releases from containment after vessel breach by the water assumed to be present in the reactor cavity.

6.4.3 Hydrogen Control and Containment Venting. Improvements in the hydrogen control system and containment venting were not analyzed for bypass sequences. An improved hydrogen control system was not evaluated as no hydrogen burns were predicted in the base case APET run. Containment venting was not analyzed because the release bypasses containment, rendering containment venting ineffective as a mitigation strategy.



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Figure 6.3. V + SGTR SCE7 with intentional depressurization via the PORVs.



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Figure 6.4. SGTR SCET with intentional depressurization via the PORVs.

Table 6.7. Conditional containment failure probabilities for the V + SGTR PDS with intentional depressurization via the PORVs

<u>Containment Failure Mode</u>	<u>Conditional Probability (SCET)</u>	<u>Base Case Conditional Probability</u>
Event V only	3.87E-01	3.87E-01
Event V + α	6.58E-03	6.58E-03
Event V + BMT	2.56E-01	2.56E-01
SGTR only	2.16E-01	2.24E-01
SGTR + α	2.37E-03	1.53E-03
SGTR + BMT	1.32E-01	1.25E-01

Table 6.8. Conditional containment failure probabilities for the SGTR PDS with intentional depressurization via the PORVs

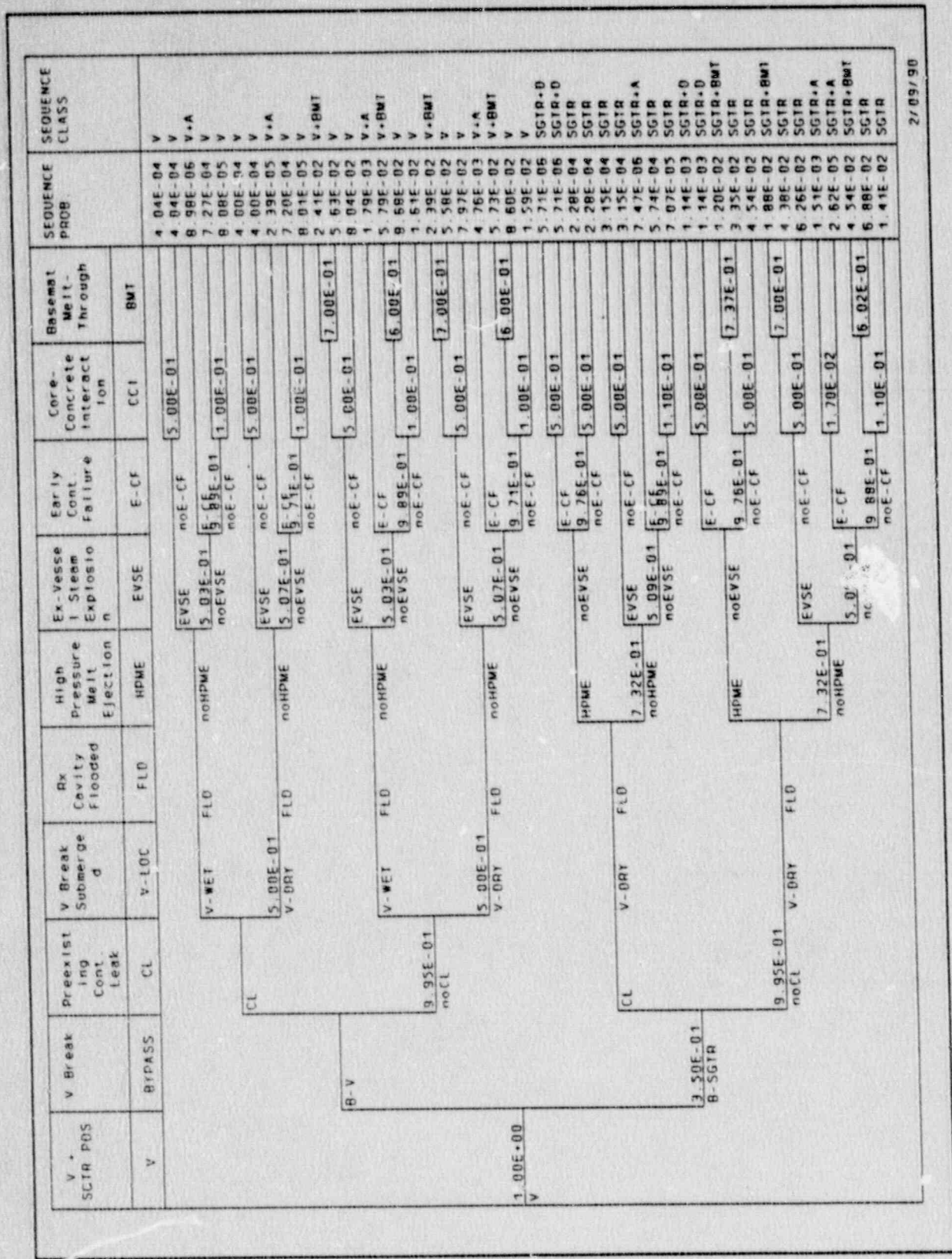
<u>Containment Failure Mode</u>	<u>Conditional Probability (SCET)</u>	<u>Base Case Conditional Probability</u>
SGTR only	6.15E-01	6.42E-01
SGTR + α	6.77E-03	2.14E-05
SGTR + BMT	3.78E-01	3.58E-01

Table 6.9. Annual V + SGTR risk with intentional depressurization via the PORVs

	<u>Mean Early Fatalities</u>	<u>Mean Latent Fatalities</u>	<u>Mean 50-Mile Dose (person-rem)</u>	<u>Mean 1000-Mile Dose (person-rem)</u>	<u>Mean Offsite Costs (\$)</u>
Base case	2.74E-06	5.50E-04	1.10	3.32	2.03E+03
SCET	2.74E-06	5.50E-04	1.10	3.32	2.03E+03
% change	0.00	0.00	0.00	0.00	0.00

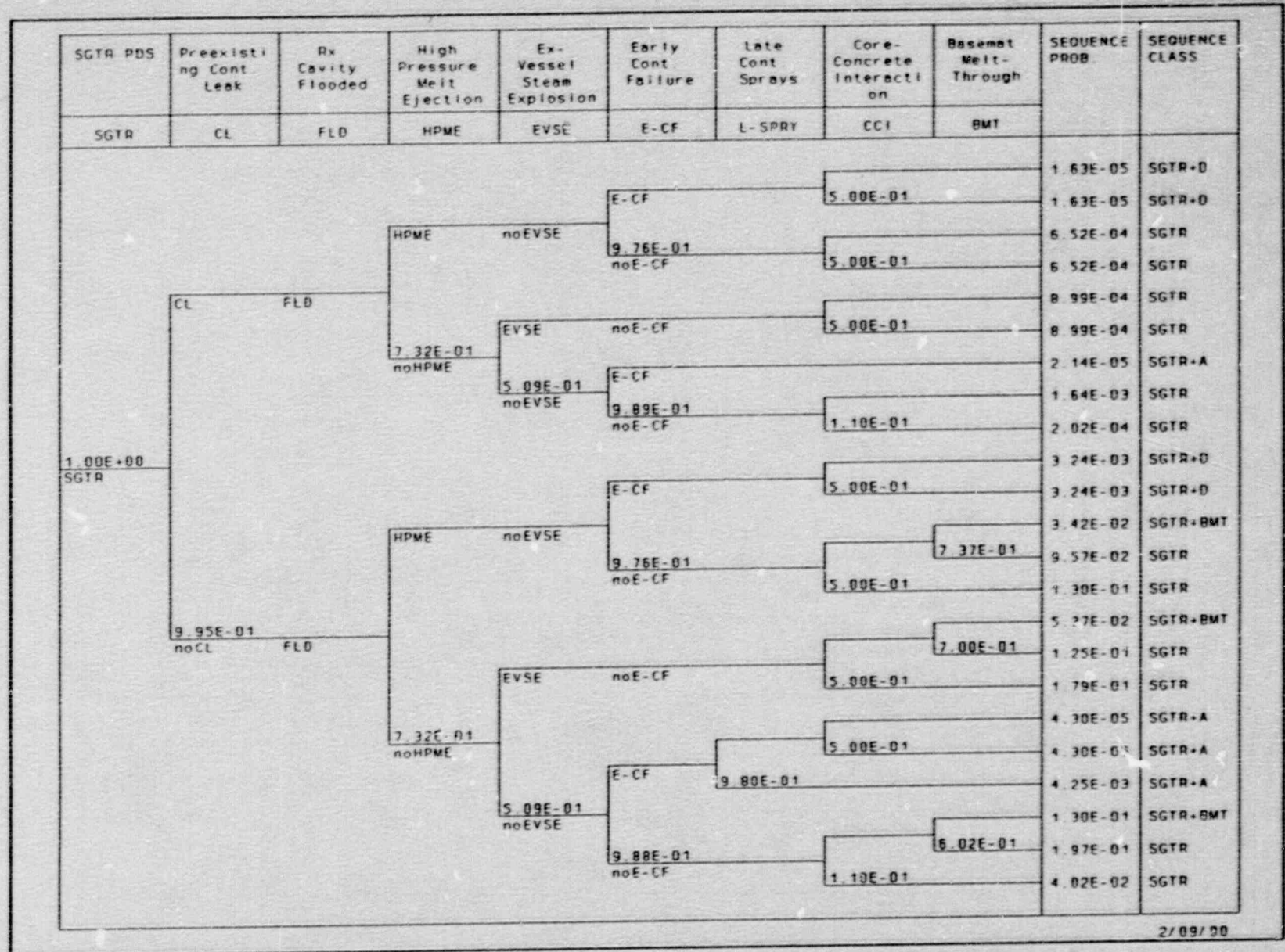
Table 6.10. Annual SGTR risk with intentional depressurization via the PORVs

	<u>Mean Early Fatalities</u>	<u>Mean Latent Fatalities</u>	<u>Mean 50-Mile Dose (person-rem)</u>	<u>Mean 1000-Mile Dose (person-rem)</u>	<u>Mean Offsite Costs (\$)</u>
Base case	5.72E-06	9.04E-03	14.17	54.23	4.81E+04
SCET	5.72E-06	9.04E-03	14.18	54.24	4.81E+04
% change	0.00	0.00	0.00	0.00	0.00



27/09/90

Figure 6.5. V + SGTR SCET with addition of cavity flooding system.



2/09/00

Figure 6.6. SGTR SCET with addition of cavity flooding system.

Table 6.11. Conditional containment failure probabilities for the V + SGTR PDS with addition of cavity flooding system

<u>Containment Failure Mode</u>	<u>Conditional Probability (SCET)</u>	<u>Base Case Conditional Probability</u>
Event V only	4.80E-01	3.87E-01
Event V + α	6.58E-03	6.58E-03
Event V + BMT	1.63E-01	2.56E-01
SGTR only	2.70E-01	2.24E-01
SGTR + α	1.54E-03	1.53E-03
SGTR + BMT	7.62E-02	1.25E-01
SGTR + DCH	2.29E-03	0.00

Table 6.12. Conditional containment failure probabilities for the SGTR PDS with addition of cavity flooding system

<u>Containment Failure Mode</u>	<u>Conditional Probability (SCET)</u>	<u>Base Case Conditional Probability</u>
SGTR only	7.72E-01	6.42E-01
SGTR + α	4.36E-03	2.14E-05
SGTR + BMT	2.18E-01	3.58E-01
SGTR + DCH	6.51E-03	0.00

Table 6.13. Annual V + SGTR risk with addition of cavity flooding system

	<u>Mean Early Fatalities</u>	<u>Mean Latent Fatalities</u>	<u>Mean 50-Mile Dose (person-rem)</u>	<u>Mean 1000-Mile Dose (person-rem)</u>	<u>Mean Offsite Costs (\$)</u>
Base case	2.74E-06	5.50E-04	1.10	3.32	2.03E+03
SCET	2.69E-06	5.16E-04	1.02	3.08	1.90E+03
% change	-1.7	-6.3	-7.0	-7.3	-6.2

Table 6.14. Annual SGTR risk with addition of cavity flooding system

	<u>Mean Early Fatalities</u>	<u>Mean Latent Fatalities</u>	<u>Mean 50-Mile Dose (person-rem)</u>	<u>Mean 1000-Mile Dose (person-rem)</u>	<u>Mean Offsite Costs (\$)</u>
Base case	5.72E-06	9.04E-03	14.17	54.23	4.81E+04
SCET	5.72E-06	9.04E-03	14.17	54.21	4.81E+04
% change	0.00	0.00	0.00	0.00	0.00

7. SUMMARY OF TECHNICAL FINDINGS

This section begins with a summation of the results from the previous sections. Table 7.1 presents the composite risk results for the base case and the improvements and sensitivities, summed over all plant damage state groups.

Several conclusions can be drawn concerning the benefits of the potential improvements that have been examined in this report. However, the authors must preface these conclusions with a very important caveat, one that was stated earlier but bears repeating: *the analysis of improvements performed for this report is both plant-specific and site-specific to Zion.* These results should not be applied to other large dry PWR containments without further analysis, with due consideration given to plant-specific and site-specific features that can affect the results. This having been said, we present the following conclusions.

The benefits to risk of intentional operator depressurization cannot be judged conclusively. There appear to be both positive and negative effects to vessel depressurization. First, vessel breach may be prevented in some sequences by intentional depressurization, since depressurization may allow injection from available low pressure systems. This is the case, for example, in the transient + ATWS PDS group. Depressurization also eliminates temperature-induced SGTR, which by-

passes the containment. However, depressurization reduces but does not eliminate DCH failures for Zion, because the capacity of the pressurizer PORVs has not been shown to be sufficient to fully depressurize the RCS. Finally, if the conditional probabilities of α mode failure developed for draft NUREG-1150 are used, then the benefits of depressurization may be offset by an increased probability of early containment failure, because draft NUREG-1150 has judged α mode failure to be more likely at low RCS pressures.

The addition of a cavity flooding system yields a slight reduction in risk but may increase the probability of DCH failure in some sequences. The effect of a flooded cavity on the threat from DCH is not conclusively known. The discussion in Reference 2 appears to indicate that the effect may be plant-specific, enhancing the threat at some plants while mitigating it at others. At any rate, the risk reduction is not significant for Zion. The benefit in reducing DCH was also found to be at least somewhat pressure-dependent. In those sequences where the RCS is likely to be at system set point pressure at the time of vessel breach, cavity flooding was found to exacerbate the threat from DCH. On the other hand, in sequences such as SBO, where the RCS pressure at the time of vessel breach was likely to be less than system set point pressure, cavity

Table 7.1. Composite annual risk results

	Mean Early Fatalities	Mean Latent Fatalities	Mean 50-Mile Dose (person-rem)	Mean 1000-Mile Dose (person-rem)	Mean Offsite Costs (\$)
Base case	3.31E-05	1.39E-02	26.3	84.6	5.99E+04
Depressurization via PORVs	3.18E-05	1.38E-02	26.2	83.9	5.93E+04
Full depressurization #1 (no HPME)	4.12E-05	1.39E-02	26.3	84.4	5.98E+04
Full depressurization #2 (no HPME) ^a	1.60E-05	1.09E-02	18.7	65.2	5.23E+04
Cavity flooding	3.12E-05	1.28E-02	24.0	76.5	5.58E+04
H ₂ control	3.31E-05	1.39E-02	26.3	84.6	5.99E+04

a. This case includes the use of a point estimate α mode failure probability of 8.0×10^{-4} .

flooding was found to be of some benefit in mitigating DCH. Flooding the cavity does increase the conditional probability of an ex-vessel steam explosion; however, because ex-vessel steam explosions are an insignificant threat to containment integrity at Zion, this effect does not increase offsite risk.

Improvements in the hydrogen control systems are of no benefit in terms of risk. This result is very Zion-specific. Other plants, particularly these with smaller subatmospheric containments, might realize a more significant risk reduction from hydrogen control improvements.

Improvements to reduce the frequency of containment bypass sequences would provide the greatest tangible risk reduction benefit. For Zion, containment bypass sequences, both the interfacing systems LOCA and SGTR, contribute very significantly to the annual offsite risk. "Front-end" improvements to reduce the incidence of bypass initiators were not analyzed. However, any reductions in bypass frequency would provide a corresponding reduction in all risk measures. Based on a plant visit to Surry, D. C. Williams^a mentions two possible mitigative strategies. The first of these is to attempt to ensure that the break location in

a. D. C. Williams, "PWR Dry Containment Parameters: CONTAIN Calculations," draft letter report dated November 27, 1989.

the V sequence is submerged. The second is a suggestion to reflood the steam generators in the case of SGTR, in order to ensure that the release from the RCS is scrubbed through a volume of water. Neither of these suggestions has been evaluated in this report, because the means to do so were unavailable. The improvements for Surry were evaluated and a significant reduction was found in the early and latent fatality risk (no doses or offsite costs were calculated).^b Both may have the potential to generically reduce the bypass sequence risk. However, the efficacy and cost-effectiveness of these strategies would best be determined on a plant-specific basis.

Gradual overpressurization by noncondensable gases (including steam) is not a threat to containment integrity for Zion. Containment failure by eventual overpressurization (time scale of one or more days) was predicted only in the APET runs made for the LOCA plant damage state group. Even in these cases, the conditional probability of eventual overpressurization was very small. Again, this result is Zion-specific. The conditional probability of late containment failure as the result of BMT was considerable; however, these failures are negligible contributors to offsite risk.

b. J. J. Gregory, "PWR Dry Containment Parameters: NUREG-1150 Sensitivity Studies for the Surry Plant," draft letter report dated December 14, 1989.

8. REFERENCES

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APPENDIX A
ZION ACCIDENT PROGRESSION
EVENT TREE

APPENDIX A

Zion Accident Progression
Event Tree

The listing of the full 72-question APET for the Zion plant is provided here for reference. This is the APET used in the NUREG-1150 analysis of the LOCA PDS group for Zion. The APETs for the other PDS groups are similar.

Zion LOCA Accident Progression Event Tree

ZION APET, Rev 6, 8 MAR 89 - 72 Questions - PDS-2, LOCAS

72										
NQuest										
1	1.000									
'PDS-2, LHS'	PInit									
1 Size and Location of RCS Break when the Core Uncovers?						\$ PDS - 1st Letter	(T = PORV)			
6 Brk-A	Brk-S2	Brk-S3	Brk-V	B-SGTR	B-PORV	\$	RIQ	7	8	9
1	1	2	3	4	5	6	\$	10	13	17
	0.046	0.954	0.000	0.000	0.000	0.000	\$	19	21	22
2 For SGTR, are the Secondary System SRVs Stuck Open?								\$	44	51
2 SSRV-St0	SSRVnSt0							\$ PDS - 1st Letter		
1	1	2						\$	RIQ	7
	0.000	1.000								8
3 Status of ECCS?								\$ PDS - 2nd Letter		
5 B-ECCS	BaECCS	BFECCS	B-LPIS	BECCS				\$	RIQ	16
2	1	2	3	4	5					20
3 Cases										23
1	1							\$ Case 1: Large Break in the RCS		
	1							\$		Used for PDS Group 3
	Brk-A									
	0.000	0.0004	0.1266	0.000	0.873					
2	1							\$ Case 2: Small or Very Small Break in the RCS		
	2							\$		Used for PDS Groups 3 & 6
	Brk-S2									
	0.000	0.000	0.9643	0.0003	0.0354					
	Otherwise							\$ Case 3: S3 breaks etc.		RCS
	0.000	0.000	0.997	0.000	0.003					
4 Status of Sprays?								\$ PDS - 3rd Letter		
6 B-Sp	BaSp	BfSp	noB-SWHX	BASp	9CSp			\$	RIQ	24
2	1	2	3	4	5	6				27
2 Cases										
1	1									
	1									
	Brk-A									
	1.000	0.000	0.000	0.000	0.000	0.000				
	Otherwise									
	1.000	0.000	0.000	0.000	0.000	0.000				
5 Status of Fan Coolers?								\$ PDS - Not used for Surry		
3 B-FC	BaFC	BFFC						\$	RIQ	25
2	1	2	3							
3 Cases										
1	1							\$ Case 1: S3 Breaks		
	3									
	B-S3									
	.5	.005	.495							

1	1			\$ Case 2: S2			
	2						
	B-S2						
	.5343	.0036	.4621				
	Otherwise			\$ case 3 : A/S1			
	0.9961	0.0004	0.0035				
6	Status of AC Power?				\$ PDS - 4th Letter		
3	B-ACP	BaACP	BfACP		\$ RIQ	16	19 20
1	1	2	3				
	1.000	0.000	0.000				
7	RWST injected into Containment?				\$ PDS - 5th Letter		
3	RWST-In	RWSTaIn	RWSTfIn		\$ RWST injected implies that the cont.	\$ RIQ	31
2	1	2	3		\$ sprays operated in the injection mode.		
3	Cases				\$ or a break or leak into the containment.		
1	1			\$ Case 1: A size breaks			
	1						
	B-A						
	0.877	0.000	0.123				
1	1						
	2						
	B-S2			\$ Case 2: S2 size break			
	1.000	0.000	0.000				
	Otherwise			\$ Case 3: S3 breaks			
	0.005	0.000	0.995				
8	Heat Removal from the Steam Generators?				\$ PDS - 6th Letter	\$ RIQ	9
4	SG-HR	SGaHR	SGfHR	SGdHR	\$ SG-HR = operating when core uncovers.	10	13
2	1	2	3	4	\$ SGdHR = operated until batteries	17	28
2	Cases				\$ depleted but not operating when	19	51
1	1				\$ core uncovers	44	
	6				\$ Case 1: B-PORV		
	B-PORV				\$ in PDS Group 1		
	0.000	0.000	1.000	0.000			
	Otherwise				\$ Case 2: All other breaks		
	1.000	0.000	0.000	0.000	\$		
9	Did the Operators Depressurize the Secondary before the Core Uncovers?				\$ PDS - 6th Letter		
2	SecDePr	noScDePr			\$ RIQ	13	19
2	1	2			\$	28	44 51
3	Cases						
1	1			\$ Case 1: B-PORV			
	6			\$ in PDS Group 1			
	B-PORV						
	0.000	1.000					
1	1			\$ Case 2: S2-size break			
	2			\$			
	Brk-S2						
	1.000	0.000					
	Otherwise			\$ Case 3: All other break sizes			
	1.000	0.000		\$ and PDSs not in Group 1			
10	Cooling for RCP Seals?				\$ PDS - 7th Letter		
3	B-PSC	BaPSC	BfPSC		\$ RIQ	15	
2	1	2	3				
3	Cases						
1	1			\$ Case 1: A Breaks			
	1						
	B-A						
	0.993	0.000	0.007				
1	1			\$ Case 2 : S2 Breaks			
	2						
	B-S2						
	.033	.000	.967				
	Otherwise			\$ Case 3: S3 Breaks			
	0.000	0.000	1.000				

11 Initial Containment Leak or Isolation Failure?					\$ PDS - 7th Letter			
2 B-Leak	noB-Leak				\$ Leak = 0.1 sq.ft.	\$ RIQ	56	61
1 1	2				\$ Seabrook data	68	69	71
0.00500	0.99500							
12 Event V - Break Location under Water?					\$ Zion FSAR calculation			
2 V-Wet	V-Dry							
1 1	2							
0.500	0.500							
13 RCS Pressure at the Start of Core Degradation?						\$ RIQ	14	15
4 E-SSPr	E-HiPr	E-ImPr	E-LoPr			\$	23	26
2 1	2	3	4					17
4 Cases								
2 1	1				\$ Case 1: Large Break			
1 1	+	4			\$ - Low Pressure - 200 psia or less.			
Brk-A	or	Brk-V			\$ [Following cases cannot have A-size breaks]			
0.000		0.000	0.000	1.000				
1 1					\$ Case 2: No Break in the RCS			
6					\$ - System Setpoint Pressure - around 2500 psia.			
B-PORV					\$ [Following cases must have S2 or S3 or SGTR]			
1.000	0.000	0.000	0.000	0.000				
1 9					\$ Case 3: Sec. DePr. & (S3 or S2 or SGTR)			
1					\$ - IM Pressure - 200 to 600 psia.			
SecDePr								
0.000	0.000	1.000	0.000		\$ Case 4: S2 or S3 with AFW but noDePr or with no AFW			
Otherwise					\$ - High Pressure - 1000 to 1400 psia.			
0.000	1.000	0.000	0.000	0.000				
14 Do the PORVs or SRVs Stick Open?						\$ RIQ	15	17
2 PORV-StC	PORVnStC					\$	21	22
2 1	2							18
2 Cases								
1 13					\$ Case 1: RCS at setpoint pressure, no breaks -			
1					\$ All the water loss is thru the PORVs.			
E-SSPr								
0.500	0.500				\$ Case 2: RCS not at setpoint pressure -			
Otherwise								
0.000	1.000							
15 Temperature-Induced RCP Seal Failure? (After core uncovering)						\$ RIQ	17	18
2 EB-PSSB	noEB-PSF					\$	22	21
2 1	2				\$ No S2 Seal Breaks -			
4 Cases					\$ only 0.4% were S2 in ASEP.			
1 10					\$ Case 1: Have seal cooling.			
1								
B-PSC					\$ Case 2: RCS at Setpoint Pressure.			
0.000	1.000				\$ Distribution from ASEP special panel.			
2 13	14				\$ Case 3: RCS at High Pressure.			
1 *	2				\$ Case 4: RCS at IM or low pressure.			
E-SSPr & PORVnStC								
0.707	0.293							
1 13								
2								
E-HiPr								
0.650	0.350							
Otherwise								
0.600	0.400							
16 Is the RCS Depressurized before Breach by Opening the PZR PORVs?						\$ RIQ	15	17
2 PrmDePr	noPrDePr					\$	21	22
2 1	2							18
2 Cases								
3 6	3	3			\$ Case 1: Have AC power, and Operators			
1 *	-4	* -1			\$ have not already failed to DePressurize.			
B-ACP & noB-LPIS & noB-ECCS					\$ Case 2: No AC Power.			
0.900	0.100				\$ Opening the PORVs is prohibited			
Otherwise					\$ by procedures .			
0.000	1.000							

17 Temperature-Induced Hot Leg or Surge Line Break?

2	EB-HLA	noEB-HLA			\$ Distribution from	RIQ	18	21
2	1	2			\$ In-Vessel Issue 1.			
3	Cases							
4	13	16	14	15	\$ Case 1: No breaks & no AFW -			
	1	2	2	2	\$ RCS around 2500 psia.			
	E-SSPr	& noPrDePr	& PORVnStG	& noEB-PSF	\$ Hot leg break likely.			
	0.722	0.278						
6	1	1	8	8	16	14		\$ Case 2: S3 break & no Ar
	(3 + 5)	*(2 + 4)	* 2	* 2				\$ - RCS around 2000 psia
	(Brk-S3 or B-SGTR)	&(SGaHR or SGdHR)	& noPrDePr	& PORVnStG				\$ Hot leg break unlikely
	0.357	0.643						
	Otherwise -- noSSPr				\$ Case 3: RCS not at 2000-2500 psia.			
	0.001	0.829						

18 Temperature-Induced SGTR?

2	E-SGTRS3	noE-SGTR			\$ Distribution from	RIQ	22	58
2	1	2			\$ In-Vessel Issue 2. Freq is conditional to q 17			
2	Cases							
4	13	16	14	15	\$ Case 1: No breaks & no AFW -			
	1	2	2	2	\$ RCS at setpoint pressure.			
	E-SSPr	& noPrDePr	& PORVnStG	& noEB-PSF	\$ SGTR very unlikely.			
	0.018	0.982						
	Otherwise				\$ Case 2: RCS not at Setpoint Pressure			
	0.000	1.000			\$ - SGTR not credible			

19 Is AC Power Available Early (Between Uncovering TAF & VB-30 min)?

3	E-ACP	EaACP	EfACP			\$ RIQ	20	24
2	1	2	3			\$	25	44
7	Cases							
1	6				\$ Case 1: Had power initially			
	1				\$ - have power now.			
	B-ACP				\$ B-ACP implies SG-HR.			
	1.000	0.000	0.000					
1	6				\$ Case 2: Power failed initially			
	3				\$ - not recoverable.			
	BfACP				\$ Remaining cases have recoverable power.			
	0.000	0.000	1.000					
2	8	8			\$ Case 3: No initial AFW. (Fast TMLB')			
	2	3			\$ Recovery period = 0.5 to 2 hours.			
	SGaHR or SGfHR				\$ Remaining cases have SGdHR - AFW initially available.			
	0.564	0.436	0.000					
1	1				\$ Case 4: Initial AFW & S2 Break - S2RRR-RDYR & S2RRR-RCYR			
	2				\$ Recovery Period = 1 to 4 hours.			
	Brk-S2				\$ BaACP & SGdHR implied by previous questions.			
	0.736	0.264	0.000					
2	1	9			\$ Case 5: Initial AFW & S3 Break - S3RRR-RCYR			
	3	2			\$ No Depressurization of the Secondary			
	Brk-S3 & noScDePr				\$ Recovery Period = 4 to 5.5 hours			
	0.393	0.607	0.000					
2	1	9			\$ Case 6: Initial AFW & S3 Break - S3RRR-RDYR			
	3	1			\$ Secondary Depressurized			
	Brk-S3 & SecDePr				\$ Recovery Period = 4 to 10 hours			
	0.801	0.199	0.000					
	Otherwise - B-PORV				\$ Case 7: Initial AFW & no Break, SecDePr - TRRR-RDYR & TRRR-RDYY			
	0.675	0.325	0.000		\$ Recovery Period = 7 to 12 hours			

20 After Power Recovery, Is Coolant Injection Re-Established Promptly?

2	E-RECC	noE-RECC				\$ RIQ	23	
2	1	2						
3	Cases							
1	3				\$ Case 0: automatic inj. ECCS operating!			
	1							
	B-ECCS							
	1.000	0.000						

3	6	19	3	\$	Case 1: If electric power is				
	2 *	1 *	2	\$	restored core cooling should				
	BaACP &	E-ACP &	BaECCS	\$	be re-established promptly.				
	0.950	0.050							
	Otherwise			\$	Case 2: Power not restored, or ECCS failed,				
	0.000	1.000		\$	or LPIS has been available all along.				
21	Rate of Blowdown to Containment? [This is blowdown before vessel breach.]								
4	EBD-A	EBD-S2	EBD-S3	noEBD		\$ RIQ	27	22	
2	1	2	3	4					
4	Cases								
2	1	17			\$	Case 1: Large break -			
	1 +	1			\$	initial or induced.			
	Brk-A or	EB-HLA							
	1.000	0.000	0.000	0.000					
1	1				\$	Case 2: V no blowdown			
	4								
	Brk-V								
	0.000	0.000	0.000	1.000					
3	1	14	16		\$	Case 3: S2 break - initial,			
	2 +	1 +	1		\$	induced, or deliberate.			
	Brk-S2 or	PORV-St0	or PrmDePr		\$	Includes stuck-open PORV.			
	0.000	1.000	0.000	0.000					
	Otherwise				\$	Case 4: S3 and some SGTR -			
	0.000	0.000	1.000	0.000					
22	Vessel Pressure just before Breach?								
4	1-SSPr	1-HiPr	1-ImPr	1-LoPr	\$	RIQ	23	28	
2	1	2	3	4	\$		38	39	
4	Cases				\$		48	49	
5	21	1	1	16					
	1 +	4 + (2 * (1 +		\$	Case 1: Large Break or		
	EBD-A or	Brk-V or (Brk-S2 &	(PrmDePr or		\$	S2 with PORVs open.		
	0.000	0.000	0.000	1.000		\$	Low Pressure - < 200 psia.		
1	21					\$	Case 2: S2 Break		
	2					\$	Intermediate Pressure - 200-600 psia.		
	EBD-S2					\$	[No A breaks by Case 1]		
	0.000	0.000	1.000	0.000					
4	15	18	1	1		\$	Case 3: S3 Break		
	1 +	1 +	3 +	5		\$	High Pressure - 1000-2000 psia.		
	EB-PSS3	or E-SGTRS3	or Brk-S3	or B-SGTR		\$	[EBD-S3 includes B-PORV - can't use here]		
	0.000	1.000	0.00	0.000					
	Otherwise - B-PORV					\$	Case 4: RCS Pressure Boundary Intact -		
	1.000	0.000	0.000	0.000		\$	System Setpoint Pressure - 2300 to 2500 psia.		
23	Is Core Damage Arrested?			No Vessel Breach?		\$	RIQ	28	35
2	noVB	VB			\$		39	40	51
2	1	2			\$		57	69	70
9	Cases								
2	19	3			\$	Case 1: No power or no injection			
	-1 +	3			\$	assures vessel breach.			
	noE-ACP or	BFECCS			\$	Rest of cases have electric power before VB.			
	0.000	1.000							
2	1	3			\$	Case 2: Large Initial Break with LPIS available all along.			
	1 *	4			\$	RCS will depressurize before core damage			
	Brk-A &	B-LPIS			\$	has gone very far.			
	0.950	0.050							
4	22	3	22	3	\$	Case 3: Depressurization was either later or			
	(4 *	4)+ (-1 *	1)	\$	slower than in Case 2. Chances of			
	(1-LoPr	B-LPIS	or(no1-SSPr	& B-ECCS)	\$	avoiding VB are less than in Case 2.			
	0.900	0.100							
1	3				\$	Case 4: The remaining cases must have recoverable ECCS.			
	-2				\$	They have electric power by case 1.			
	noBaECCS				\$	E.G., B-LPIS & Hi-Pr goes to VB at this case.			
	0.000	1.000							

2 8 8
 (2 + 3)
 (SGdHR cr SGfHR)
 0.900 0.100
 1 1
 2
 Brk-S2
 0.700 0.300
 2 1 9
 3 * 2
 Brk-S3 & noScDePr
 0.500 0.500
 2 1 9
 3 * 1
 Brk-S3 & SecDePr
 123,5,1 123,5,2
 Otherwise - B-PORV
 123,5,1 123,5,2

24 Early Sprays?
 3 E-Sp EaSp EfSp
 2 1 2 3
 4 Cases
 3 4 4 4
 1 + 5 + 6
 B-Sp or BAsp or BCSp
 1.000 0.000 0.000
 1 4
 3
 BfSp
 0.000 0.000 1.000
 2 4 19
 2 * 1
 BaSp & E-ACP
 1.000 0.000 0.000
 Otherwise
 0.000 1.000 0.000

25 Early Fan Coolers?
 3 E-FC EaFC EffC
 2 1 2 3
 4 Cases
 1 5
 1
 B-FC
 1.000 0.000 0.000
 1 5
 3
 EffC
 0.000 0.000 1.000
 2 5 19
 2 * 1
 BaFC & E-ACP
 1.000 0.000 0.000
 Otherwise
 0.000 1.000 0.000

26 Early Containment Heat Removal?
 2 E-CHR EfCHR
 2 1 2
 3 Cases
 1 25
 1
 E-FC
 1.000 0.000

\$ Case 5: No initial AFW - TRRR-RSR.
 \$ Recovery period = 0.5 to 2 hours.
 \$ Remaining cases have SGdHR - AFW initially available.
 \$ Case 6: Initial AFW & S2 Break - S2RRR-RDR & S2RRR-RCR.
 \$ Recovery Period = 1 to 4 hours.
 \$ BaACP & SGdHR implied by previous questions.
 \$ Case 7: Initial AFW & S3 Break - S3RRR-RCR.
 \$ No Depressurization of the Secondary.
 \$ Recovery Period = 4 to 5.5 hours.
 \$ Case 8: Initial AFW & S3 Break - S3RRR-RDR.
 \$ Secondary Depressurized.
 \$ Recovery Period = 4 to 10 hours.
 \$ Case 9: Initial AFW & no Break, SecDePr - TRRR-RDR & TRRR-RDY.
 \$ Recovery Period = 7 to 12 hours.

\$ RIQ 26 31
 43 58

\$ Case 1: Had sprays on or
 \$ operating on ps demand
 \$ Case 2: Sprays were failed
 \$ - stay failed.
 \$ Case 3: Sprays were available and have power now
 \$ - sprays operate. Even if containment pressure
 \$ never gets high enough for auto actuation, assume
 \$ operator will turn on sprays to cool sump water.
 \$ Case 4: No power
 \$ - sprays remain available.

\$ RIQ 26 46

\$ Case 1: Had fan coolers at start
 \$ - have fan coolers now.
 \$ Case 2: Fan coolers were failed
 \$ - stay failed.
 \$ Case 3: Fan Coolers were available and have power now
 \$ - fan coolers operate. Even if containment
 \$ pressure never gets high enough for auto
 \$ actuation, assume operator will turn on.
 \$ Case 4: No power - fan
 \$ coolers remain available.

\$ RIQ 27

\$ Case 1: Have Fans - Have CHR

2	24	20		\$ Case 2: Have Sprays and ECCS			
	1	*	1	\$ - Have CHR			
	E-Sp	and E-RECC					
	1.000	0.000					
	Otherwise			\$ Case 2: No Sprays, No Fan Coolers			
	0.000	1.000		\$ - No CHR			
27	Baseline Containment Pressure just before VB?				\$ PUIQ	42	
1	IPBase			\$ IPBase - Parameter 1			
4	1						
4	Cases						
2	21	23		\$ Case 1: No blowdown to containment, or			
	4	+	1	\$ no vessel breach. Containment will be			
	noEBD	or	noVB	\$ near normal operating pressure.			
	1.000						
1				\$ pressure in psia			
1	15.00						
2	26	4		\$ Case 2: Have Sprays or Fan Coolers and			
	1	*	-4	\$ Service Water. Containment			
	E-CHR	&	B-SWHX	\$ will be near Ambient Pressure.			
	1.000			\$ See S2D run in BMI-2139			
1							
1	19.00						
2	26	21		\$ Case 3: No CHR and blowdown to			
	2	*	1	\$ containment from a large break.			
	EfCHR	&	EBD-A	\$ Pressure around 44 Psia, S2DCirFir BMI-2139			
	1.000						
1							
1	44.00			\$ Case 4: No sprays and no large break.			
	Otherwise			\$ Pressure around 24-26 psia			
	1.000			\$ See TMLBn BMI-2139			
1							
1	26.00						
28	Time of Accumulator Discharge?				\$ RIQ	29	50 56
3	AcDbCM	AcDdCM	AcDaVB				
2	1	2	3				
3	Cases						
4	13	13	8	9	\$ Case 1: Accumulators Discharge		
	3	+	4	+	4	*	1
	E-ImPr	or E-LoPr	or (SGdHR	&	SecDePr)
	1.000	0.000	0.000				
2	22	22		\$ Case 2: Accumulators Discharge			
	3	+	4	\$ during Core Degradation			
	I-ImPr	or I-LoPr					
	0.000	1.000	0.000				
	Otherwise			\$ Case 3: Accumulators Discharge			
	0.000	0.000	1.000	\$ at Vessel Breach			
29	Fraction of Zr Oxidized In-Vessel during Core Degradation?				\$ PUIQ	30	59
1	ZrOx-InV			\$ ZrOx-InV- Parameter 2			
4	1						
7	Cases						
2	13	28		\$ Case 1: RCS at System Setpoint Pressure (2500 psia)			
	1	*	-2	\$ Accm. dump before or after core melt			
	E-SSPr	&	AcDnCM	\$ In-Vessel #5 - Case 1a/1c			
	1.000						
1							
2	0.44						
2	13	28		\$ Case 2: RCS at System Setpoint Pressure (2500 psia)			
	1	*	2	\$ Accumulator dump during core melt			
	E-SSPr	&	AcDdCM	\$ In-Vessel #5 - Case 1b			
	1.000						
1							
2	0.50						

2 13 28 \$ Case 3: RCS at High Pressure (1000-1400 psia)
 2 * -2 \$ Accum. dump before or after core melt
 E-HiPr & AcDnCM \$ In-Vessel #5 - Case 2a/2c/5
 1.000

1
 2 0.32
 2 13 28 \$ Case 4: RCS at High Pressure (1000-1400 psia)
 2 * 2 \$ Accumulator dump during core melt
 E-HiPr & AcDdCM \$ In-Vessel #5 - Case 2b
 1.000

1
 2 0.38
 2 13 28 \$ Case 5: Intermediate Pressure (200-600 psia)
 3 * -2 \$ Accum. dump before or after core melt
 E-ImPr & AcDnCM \$ In-Vessel #5 - Case 3a
 1.000

1
 2 0.45
 2 13 28 \$ Case 6: Intermediate Pressure (200-600 psia)
 3 * 2 \$ Accumulator dump during core melt
 E-ImPr & AcDdCM \$ In-Vessel #5 - Case 3b
 1.000

1
 2 0.52
 Otherwise - E-LoPr \$ Case 7: Low RCS Pressure (<200 psia)
 1.000 \$ In-Vessel #5 - Case 4

1
 2 0.45
 30 Amount of Zr Oxidized In-Vessel during Core Degradation?
 2 Hi-ZrOx Lo-ZrOx
 5 1 2 \$ Put fraction Zr oxidized
 1 2 \$ into 2 categories -- need
 ZrOx-InV \$ this information for SURSOR
 AND
 GETHRESH 1 0.4
 Fraction of Zr Oxidized In-Vessel

31 Amount of Water in the Reactor Cavity at Vessel Breach?
 2 RC-Wet RC-Dry \$ RIQ 38 39 40
 2 1 2 \$ 50 56 69
 2 Cases
 3 7 7 19 \$ Case 1: RWST not injected only
 3 + (2 * -1) \$ criterion for dry cav.
 RWST or (RWSTa & NoEACP) \$ Time of Accm. Dump irrelevant for DCH & EVSE.
 0.000 1.000 \$ If Dump at VB, it will be after DCH or EVSE.
 Otherwise \$ Case 2: RWST injected or sprays operating -
 1.000 0.000 \$ The Cavity is Full (12,000 ft³ = 340 m³).

32 Fraction of Core Released from Vessel at Breach?
 1 FCorRel \$ PUIQ 33
 3 1 \$ FCorRel- Parameter 3
 1.000 \$ Fraction Released or Expelled Promptly at Breach
 1 \$ Distribution from In-Vessel Issue 6
 3 0.30

33 Amount of Core Released from Vessel at Breach?
 3 Hi-FCoR Md-FCoR Lo-FCoR \$ This question puts the fractions \$ RIQ 38
 5 1 2 3 \$ obtained in the previous question 39 48
 1 3 \$ into a small number of categories.
 FCorRel
 AND
 GETHRESH 2 0.4 0.2
 Fraction of Core Participating in HPME

34	Does an Alpha Mode Event Fail both the Vessel and the Containment?					\$	RIQ	35	38	39
2	Alpha	noAlpha				\$	42	43	48	49
2	1	2								
3	Cases									
2	23	22			\$					
	2 *	4			\$					
	VB &	I-LoPr								
	0.0080	0.9920								
2	23	22			\$					
	2 *	-4			\$					
	VB &	noI-LoPr								
	0.0008	0.9992								
	Otherwise				\$					
	0.0000	1.0000								
35	Type of Vessel Breach?					\$	RIQ	36	37	38
4	PrEj	Pour	BtmHd	noVBoA		\$		40	42	48
2	1	2	3	4		\$		49	58	
5	Cases									
2	23	34			\$					
	1 +	1			\$					
	noVB	or Alpha								
	0.0000	0.0000	0.0000	1.0000						
1	22				\$					
	1				\$					
	I-SSPr				\$					
	0.7900	0.1900	0.0200	0.0000						
1	22				\$					
	2				\$					
	I-HiPr									
	0.6000	0.3800	0.0200	0.0000						
1	22				\$					
	3				\$					
	I-ImPr									
	0.6000	0.3800	0.0200	0.0000						
	Otherwise - I-LoPr				\$					
	0.0000	1.0000	0.0000	0.0000						
36	Does the Vessel become a "Rocket" and Fail the Containment?					\$	RIQ	38	39	42
2	Rocket	noRocket				\$		43	48	49
2	1	2								
2	Cases									
2	35	22			\$					
	3 *	1			\$					
	BtmHd &	I-SSPr			\$					
	0.001	0.999								
	Otherwise				\$					
	0.000	1.000								
37	Size of Hole in Vessel (after Ablation)?									
2	LrgHole	SmHole					\$	RIQ	38	39
2	1	2								
2	Cases									
1	35				\$					
	1									
	PrEj									
	0.100	0.900								
	Otherwise				\$					
	1.000	0.000								
38	Pressure Rise at Vessel Breach? Large Hole Cases									
1	DP-VB						\$	PUIQ	42	
4	1				\$					
16	Cases									

1	23				\$ Case 1: No Vessel Breach	
	1					
	noVB					
	1.000					
1					\$ Pressure rise in psi	
4	0.00					
2	34	36			\$ Case 2: Alpha Mode or Rocket -	
	1 +	1			\$ Very Large Dummy Value used to	
	Alpha or	Rocket			\$ Assure Containment Failure.	
	1.000					
1						
4	777.00				\$ Case 3: Low Pressure in RCS, or Pour.	
2	22	35			\$ Loads Issue 9, Case 4	
	4 +	2				
	1-LoPr	or Pour				
	1.000					
1						
4	24.90				\$ Case 4: Small Hole Cases -	
1	37				\$ Treated in next question	
	2					
	SmHole				\$ The following questions are	
	1.000				\$ thus all large hole cases.	
1						
4	0.00					
3	22	31	33		\$ Case 5: IM Pressure in RCS.	506
	3 *	1 *	1		\$ Cavity Full or Part Full.	507
	1-ImPr &	RC-Wet &	Hi-FCor		\$ High Fraction Ejected	508
	1.000				\$ Loads Issue 9, Case 3 curve 9	
1						510
4	75.70					511
3	22	31	33		\$ Case 6: IM Pressure in RCS.	512
	3 *	1 *	2		\$ Cavity Full or Part Full.	513
	1-ImPr &	RC-Wet &	Md-FCor		\$ Medium Fraction Ejected	
	1.000				\$ Loads Issue 9, Case 3 curve 11	
1						516
4	51.90					517
3	22	31	33		\$ Case 7: IM Pressure in RCS.	512
	3 *	1 *	3		\$ Cavity Full or Part Full.	513
	1-ImPr &	RC-Wet &	Lo-FCor		\$ Low Fraction Ejected	
	1.000				\$ Loads Issue 9, Case 3 curve 11	
1						516
4	51.90					517
3	22	24	33		\$ Case 8: IM Pressure in RCS.	518
	3 *	-1 *	1		\$ no sprays	519
	1-ImPr &	No-Esp &	Hi-FCor		\$ High Fraction Ejected	520
	1.000				\$ Loads Issue 9, Case 3A curve 13	
1						522
4	85.20					523
3	22	24	33		\$ Case 9: IM Pressure.	
	3 *	-1 *	2		\$ no sprays	
	1-ImPr &	No-Esp &	Md-Fcor		\$ int ejection	
	1.000				\$ Case 3A curve 15	
1						
4	57.2					
3	22	24	33		\$ Case 10: IM Pressure.	
	3 *	-1 *	3		\$ no sprays	
	1-ImPr &	No-Esp &	Lo-Fcor		\$ low ejection	
	1.000				\$ Case 3A curve 15	
1						
4	57.2					

4	22	22	31	33	\$ Case 11: SS or HI Pressure in RCS.	524
	(1 + 2) *		2	* 1	\$ Cavity Dry.	525
	(1-SSPr or 1-HiPr) &	RC-Dry	&	Hi-FCor	\$ High Fraction Ejected	526
	1.000				\$ Loads Issue 9, Case 1B/1C curve 5	528
1						529
4	105.00					
4	22	22	31	33	\$ Case 12: SS or HI Pressure in RCS.	531
	(1 + 2) *		2	* 2	\$ Cavity Dry.	532
	(1-SSPr or 1-HiPr) &	RC-Dry	&	Md-FCor	\$ Medium Fraction Ejected	
	1.000				\$ Loads Issue 9, Case 1B/1C curve 7	534
1						535
4	70.70					
4	22	22	31	33	\$ Case 13: SS or HI Pressure in RCS.	531
	(1 + 2) *		2	* 3	\$ Cavity Dry.	532
	(1-SSPr or 1-HiPr) &	RC-Dry	&	Lo-FCor	\$ Low Fraction Ejected	
	1.000				\$ Loads Issue 9, Case 1B/1C curve 7	534
1						535
4	70.70					542
4	22	22	31	33	\$ Case 14: SS or HI Pr & Cavity Wet	
	(1 + 2) *		1	* 1		
	(1-SSPr or 1-HiPr)andRC-Wetand				\$ High Fraction Ejected	
544	1.000				\$ Loads Issue 9, Cases 1 curve 1	545
1						546
4	95.00					547
4	22	22	31	33	\$ Case 15: SS or HI Pr & Cavity Wet	542
	(1 + 2) *		1	* 2		
	(1-SSPr or 1-HiPr)andRC-Wetand				\$ Medium Fraction Ejected	
544	1.000				\$ Loads Issue 9, Cases 1 curve 1	545
1						546
4	95.00					547
	Otherwise				\$ Case 16: SS or HI Pressure & Cavity Wet	554
	1.000					
1					\$ Low Fraction Ejected	556
4	64.70				\$ Loads Issue 9, Cases 1 curve 3	558
39	Pressure Rise at Vessel Breach?				Small Hole Cases	
1	DP-VB					\$ PUIQ 42
4	1				\$ dp2-VB - Parameter 5	560
13	Cases					561
5	37	23	34	22	36	\$ Case 1: Large Hole, or no VB, or Alpha,
	1 + 1 + 1 + 4 + 1					\$ or Rocket, or Low Pressure -
	LrgHole or noVB or Alpha or 1-LoPr or Rocket					\$ Treated in previous question.
	1.000					\$ The following questions are
						\$ thus all small hole cases.
1						566
5	0.00					567
3	22	31	33			\$ Case 2: IM Pressure in RCS.
	3 *	1 *	1			\$ Cavity Full or Part Full.
	1-ImPr & RC-Wet & Hi-FCor					\$ High Fraction Ejected
	1.000					\$ Loads Issue 9, Case 3 curve 10
1						572
5	63.70					573
3	22	31	33			\$ Case 3: IM Pressure in RCS.
	3 *	1 *	2			\$ Cavity Full or Part Full.
	1-ImPr & RC-Wet & Md-FCor					\$ Medium Fraction Ejected
	1.000					\$ Loads Issue 9, Case 3 curve 12
1						578
5	44.50					579

3	22	31	33	\$	Case 4: IM Pressure in RCS.	574	
	3 *	1 *	3	\$	Cavity Full or Part Full.	575	
	1-ImPr &	RC-Wet &	Lo-FCor	\$	Low Fraction Ejected	576	
	1.000			\$	Loads Issue 9, Case 3 curve 12		
1						578	
5	44.50					579	
3	22	24	33	\$	Case 5: IM Pressure in RCS.	580	
	3 *	-1 *	1	\$	no sprays	581	
	1-ImPr &	No-Esp and	Hi-FCor	\$	High Fraction Ejected	582	
	1.000			\$	Loads Issue 9, Case 3 curve 14		
1						584	
5	71.40					585	
3	22	24	33	\$	Case 6: Same as 5 but int ejected		
	3 *	-1 *	2	\$	Loads Issue 9 Case 3 cu 39 Pressure Rise at Vessel Breach?	Small	
Hole Cases							
	1-ImPr and No_esp and Md-FCor						
	1.000						
1							
5	49.50						
3	22	24	33	\$	Case 7: Same as 5 but low ejected		
	3 *	-1 *	3	\$	Loads Issue 9 Case 3 cu 39 Pressure Rise at Vessel Breach?	Small	
Hole Cases							
	1-ImPr and No_esp and Lo-FCor						
	1.000						
1							
5	49.50						
4	22	22	31	33	\$	Case 8: SS or HI Pressure in RCS.	
	(1 +	2) *	2	*	1	\$	Cavity Dry.
	(1-SSPr or	1-HiPr) &	RC-Dry	&	Hi-FCor	\$	High Fraction Ejected
	1.000			\$	Loads Issue 9, Case 1B/1C curve 6		
1						590	
5	95.60					591	
4	22	22	31	33	\$	Case 9: SS or HI Pressure in RCS.	
	(1 +	2) *	2	*	2	\$	Cavity Dry.
	(1-SSPr or	1-HiPr) &	RC-Dry	&	Lo-FCor	\$	Medium Fraction Ejected
	1.000			\$	Loads Issue 9, Case 1B/1C curve 8		
1						592	
5	64.20					597	
4	22	22	31	33	\$	Case 10: SS or HI Pressure in RCS.	
	(1 +	2) *	2	*	3	\$	Cavity Dry.
	(1-SSPr or	1-HiPr) &	RC-Dry	&	Md-FCor	\$	Low Fraction Ejected
	1.000			\$	Loads Issue 9, Case 1B/1C curve 8		
1						596	
5	64.20					597	
4	22	22	31	33	\$	Case 11: SS or HI Pr & Cavity Wet	
	(1 +	2) *	1	*	1	\$	High Fraction Ejected
	(1-SSPr or	1-HiPr) &	RC-Wet	&	Hi-FCor	\$	Loads Issue 9, Case 1 curve 2
	1.000			\$			
1						606	
5	85.80					609	
4	22	22	31	33	\$	Case 12: SS or HI Pr & Cavity Wet	
	(1 +	2) *	1	*	2	\$	Med Fraction Ejected
	(1-SSPr or	1-HiPr) &	RC-Wet	&	Md-FCor	\$	Loads Issue 9, Case 1 curve 2
	1.000			\$			
1						606	
5	85.80					608	
	Otherwise					609	
	1.000					616	
1						\$	Low Fraction Ejected
5	57.60					\$	Loads Issue 9, Case 1 curve 4

40 Does a Significant Ex-Vessel Steam Explosion Occur?						\$ RIQ	42	48
2	EVSE	noEVSE					49	
2	1	2						
2	Cases							
2	3	35			\$ Case 1: Gravity Pour into Pool in Cavity			
	1	2			\$ is the only case where EVSE is possible.			
	RC-Wet & Pour							
	0.500	0.500						
	Otherwise -- No EVSE				\$ Case 2: Alpha Mode, or Rocket, or			
	0.000	1.000			\$ no VB, or HPME, or cavity dry.			
41 Containment Failure Pressure?								
1	CF-Pr				\$ Read Failure Pressure and Random Number for Failure Mode			
3	1				\$ CF-Pr - Parameter 6	\$ Both PUID	42	64
	1.000				\$ RndNum - Parameter 7			
2					\$ Failure Pressure in psig			
6	128.20				\$ Distribution from Structural Issue 2			
7	0.50							
42 Containment Failure, and Type of Containment Failure?						\$ RIQ	43	56
5	ICF-CtRp	ICF-Rupt	ICF-Leak	no-ICF	ICF-Shear		61	64
6	1	2	3	4	5		69	71
2	Cases							
2	34	36			\$ Case 1: Alpha or Rocket -			
	1 +	1			\$ Rupture Assured			
	Alpha or Rocket							
2	1	4						
	IPBase	dp1-VB						
	AND							
	GETHRESH 4	999 3	2	1				
	Dumpr Values to Assure Rupture							
	Otherwise				\$ Case 2:			
5	1	4	5	6	7			
	IPBase	dp1-VB	dp2-VB	CF-Pr	RndNum			
	FUN-ICFFet							
	GETHRESH 4	4 3	2	1				
	User Function for Fast Pressure Rise							
43 Sprays after Vessel Breach?								
3	12-Sp	12aSp	12fSp					
2	1	2	3					
5	Cases						\$ RIQ	45
3	24	34	36		\$ Case 1: Sprays failed - remain failed.			
	3 +	1 +	1		\$ Alpha or Rocket always fail sprays.			
	EfSp or Alpha	or Rocket						
	0.000	0.000	1.000					
2	24	42			\$ Case 2: Sprays available & no Cat. Rupture -			
	2 *	-1			\$ Sprays stay available. (Have not asked			
	EaSp & noICFCRp				\$ power recovery since last spray question.)			
	0.000	1.000	0.000					
2	24	42			\$ Case 3: Sprays operating & no Cat. Rupture			
	1 *	-1			\$ - Sprays stay operating.			
	E-Sp & noICFCRp							
	1.000	0.000	0.000					
2	24	42			\$ Case 4: Cat. Rupture at Vessel Breach -			
	1 *	1			\$ Sprays Operating			
	E-Sp & ICF-CtRp							
	0.900	0.000	0.100					
	Otherwise -- EaSp & ICF-CtRp							
	0.000	143.4,1	143.4,3		\$ Case 5: CR at VB, sprays only available.			

44 Is AC Power Available Late (during CCI)?

3	L-ACP	LaACP	LfACP	\$ RIQ	45	46	51
2	1	2	3				
7	Cases						
1	19						
	1						
	E-ACP						
	1.000	0.000	0.000				
1	19						
	3						
	EfACP						
	0.000	0.000	1.000				
2	8						
	2	+	3				
	SGdHR	or	SGfHR				
	0.888	0.112	0.000				
1	1						
	2						
	Brk-S2						
	0.754	0.246	0.000				
2	1						
	3	*	2				
	Brk-S3 & noScDePr						
	0.601	0.399	0.000				
2	1						
	3	*	1				
	Brk-S3 & SecDePr						
	0.731	0.269	0.000				
	Otherwise - B-PORV						
	0.604	0.396	0.000				

\$ Case 1: Had power initially or recovered it already - have power now.

\$ Case 2: Power failed initially - not recoverable. Remaining cases have recoverable power.

\$ Case 3: No initial AFW. (Fast TMLB') Uncov. at 100; VB at 180 min. Recovery period = 2 to 9 hours. Remaining cases have SGdHR - AFW initially available.

\$ Case 4: Initial AFW & S2 Break - S2RRR-RDYR & S2RRR-RCYR Recovery Period = 4 to 9 hours. BaACP & SGdHR implied by previous questions.

\$ Case 5: Initial AFW & S3 Break - S3RRR-RCYR No Depressurization of the Secondary Recovery Period = 5.5 to 9 hours

\$ Case 6: Initial AFW & S3 Break - S3RRR-RDYR Secondary Depressurized Recovery Period = 10 to 17 hours

\$ Case 7: Initial AFW & no Break, SecDePr - TRRR-RDYR & TRRR-RDYY Recovery Period = 12 to 17 hours

45 Late Sprays? (during CCI)

3	L-Sp	LaSp	LfSp	\$ RIQ	47	50	52
2	1	2	3				
4	Cases						
1	43						
	1						
	I2-Sp						
	1.000	0.000	0.000				
1	43						
	3						
	I2fSp						
	0.000	0.000	1.000				
2	43						
	2	*	1				
	I2aSp & L-ACP						
	1.000	0.000	0.000				
	Otherwise						
	0.000	1.000	0.000				

\$ Case 1: Had sprays after VB - have sprays now.

\$ Case 2: Sprays failed earlier - stay failed.

\$ Case 3: Sprays were available and power has been recovered, so sprays operate.

\$ Case 4: AC power not recovered, so sprays remain available.

46 Late Fan Coolers?

3	L-FC	LaFC	LfFC	\$ RIQ	47	53
2	1	2	3			
4	Cases					
1	25					
	1					
	E-FC					
	1.000	0.000	0.000			
1	25					
	3					
	EfFC					
	0.000	0.000	1.000			

\$ Case 1: Had fan coolers before - have fan coolers now.

\$ Case 2: Fan coolers were failed - stay failed.

2	25	44		\$ Case 3: Fan coolers were available and have power now
	2	*	1	\$ - fan coolers operate.
	EaFC	& L-ACP		
	1.000	0.000	0.000	
	Otherwise			\$ Case 4: No power - fan
	0.000	1.000	0.000	\$ coolers remain available.
47	Late Containment Heat Removal?			
2	L-CHR	LFCHR		
2	1	2		
2	Cases			
1	46			\$ Case 1 : have fan coolers
	1			
	L-FC			
	1.000	0.000		
	Otherwise			\$ Case 2: No Sprays, No Fan Coolers
	0.000	1.000		\$ - No CHR
48	Amount of Core Available for CCI?			
3	Lrg-CCI	Med-CCI	Sm1-CCI	\$ Large means > 70% \$ RIQ 57
2	1	2	3	\$ Medium means > 30% and < 70%
6	Cases			\$ Small means < 30%
2	34	36		\$ Case 1: Alpha Mode or Rocket
	1	1		
	Alpha	or Rocket		
	0.000	1.000	0.000	\$ Case 2: No Vessel Breach
1	35			
	no BJA			
	0.000	0.000	1.000	
4	35	35	22	33 \$ Case 3: HPME and Fr. Ejected Not Small
	(1 + (3 * -4))*		-3	
	(PrEj or (BtmHd & noI-LoPr)) & NoLoFCor			
	0.000	1.000	0.000	
4	35	35	22	33 \$ Case 4: HPME and Fraction Ejected Small
	(1 + (3 * -4))*		3	
	(PrEj or (BtmHd & noI-LoPr)) & Lo-FCor			
	1.000	0.000	0.000	
4	35	35	22	40 \$ Case 5: Gravity Pour and EVSE
	(2 + (3 * 4))*		1	
	(Pour or (BtmHd & I-LoPr)) & EVSE			
	0.500	0.500	0.000	
	Otherwise			\$ Case 6: Gravity Pour, no EVSE
	1.000	0.000	0.000	
49	Is the Debris Bed in a Coolable Configuration?			
2	CDB	noCDB		\$ RIQ 50
2	1	2		
5	Cases			
2	34	36		\$ Case 1: Alpha or Rocket -
	1	1		\$ At least some of the Core Debris will
	Alpha	or Rocket		\$ be widely scattered throughout containment.
	0.700	0.300		
1	35			\$ Case 2: No VB.
	4			
	noVBoA			
	1.000	0.000		
3	35	35	22	\$ Case 3: High Pressure Melt Ejection.
	1 + (3 * -4)			\$ At least some of the Core Debris will
	PrEj or (BtmHd & noI-LoPr)			\$ be widely scattered throughout containment.
	0.500	0.500		

1	40				\$ Case 4: Gravity Pour with EVSE.			
	EVSE				\$ EVSE likely to distribute some debris outside containment.			
	0.500	0.500			\$ But fine particles may make debris in the cavity noncoolable.			
	Otherwise				\$ Case 5: Gravity Pour with no EVSE.			
	0.100	0.900			\$ Debris bed is not likely to be coolable.			
50 Does Prompt CCI Occur?						\$ RIQ	55	56
2 PrmptCCI	noPrmCCI					\$	57	68
2	1	2						
2 Cases								
4	49	31	28	23	\$ Case 1: Coolable Debris with Water, or no VB - no			
	(1 * (1 + 3)) +			1	\$ prompt CCI. Late sprays are not considered			
	(CDB & (RC-Wet or AcDaVB)) or			noVB	\$ because they may start at any time during CCI			
	0.000	1.000			\$ and water is needed from the start.			
	Otherwise -- Not coolable or no water				\$ Case 2: No water in the Reactor Cavity			
	1.000	0.000			\$ or debris not coolable - prompt CCI.			
51 Is AC Power Available Very Late (after CCI)?						\$ RIQ	52	53
3 L2-ACP	L2aACP	L2fACP				\$	58	65
2	1	2	3					66
4 Cases								
1	44				\$ Case 1: Had power initially or			
	1				\$ recovered it already - have power now.			
	L-ACP							
	1.000	0.000	0.000					
1	44				\$ Case 2: Power failed initially			
	3				\$ - not recoverable.			
	LfACP				\$ Remaining case have power recoverable.			
	0.000	0.000	1.000					
4	8	1	1	9	\$ Case 3: Initial AFW & (no Break or S3 with SecDePr)			
	4 * (6 + (3 * 1))				\$ TRRR-RDYR, TRRR-RDYY, or S3RRR-RDYR			
	SGdHR & (B-PORV or (Brk-S3 & SecDePr))				\$ Recovery Period = 17 to 24 hours			
	0.679	0.321	0.000					
	Otherwise				\$ Case 4: All other blackout cases - TRRR-R5YR, S3RRR-RCYR,			
	0.916	0.084	0.000		\$ & S2RRR-RaYR - Recovery Period = 9 to 24 hours			
52 Very Late Sprays?						\$ RIQ	54	55
3 L2-Sp	L2aSp	L2fSp						65
2	1	2	3					
4 Cases								
1	45				\$ Case 1: Had sprays after VB - have sprays now.			
	1							
	L-Sp							
	1.000	0.000	0.000		\$ Case 2: Sprays failed earlier - stay failed.			
1	45							
	3							
	LfSp							
	0.000	0.000	1.000		\$ Case 3: Sprays were available and power has been			
2	45	51			\$ recovered, so sprays operate.			
	2 * 1							
	LaSp & L2-ACP							
	1.000	0.000	0.000		\$ Case 4: AC power not recovered, so			
	Otherwise				\$ sprays remain available.			
	0.000	1.000	0.000					
53 Very Late Fan Coolers?						\$ RIQ	54	66
3 L2-FC	L2aFC	L2fFC						
2	1	2	3					
4 Cases								
1	46				\$ Case 1: Had fan coolers before			
	1				\$ - have fan coolers now.			
	L-FC							
	1.000	0.000	0.000					

1	46				\$ Case 2: Fan coolers were failed		
	3				\$ - stay failed.		
	L1FC						
	0.000	0.000	1.000				
2	46	51			\$ Case 3: Fan coolers were available		
	2	*	1		\$ and have power now		
	L1FC	& L2-ACP			\$ - fan coolers operate.		
	1.000	0.000	0.000				
	Otherwise				\$ Case 4: No power - fan		
	0.000	1.000	0.000		\$ coolers remain available.		
54	Very Late Containment Heat Removal?					\$ RIQ	56 60
2	L2-CHR	L2fCHR					
2	1	2					
2	Cases						
1		53			\$ Case 1: Have Fan Coolers		
		1			\$ - have CHR		
	L2-FC						
	1.000	0.000					
	Otherwise				\$ Case 2: No Sprays, No Fan Coolers		
	0.000	1.000			\$ - No CHR		
55	Does Delayed CCI Occur?					\$ RIQ	56 57
2	DelydCCI	noDldCCI				\$	68 69
2	1	2					
2	Cases						
4	50	52	50	23	\$ Case 1: Did not have CCI promptly (so debris		
	(2 * 1) +	1 +	1 +	1	\$ is coolable), and have water now, or had		
	(noPrmCCI & L2-Sp) or PrmptCCI	or noVB			\$ prompt CCI, or no VB - can't have CCI now.		
	0.000	1.000					
	Otherwise				\$ Case 2: Water boiled off and there are no		
	1.000	0.000			\$ sprays now - delayed CCI occurs.		
56	Baseline Containment Pressure Very Late?					\$ PU10	63 64
1	L2PBase						
4	1				\$ L2PBase - Parameter B		
6	Cases						
4	42	42	42	23	\$ Case 1: Containment already		
	1 +	2 +5 +	1	1	\$ ruptured or no vessel breach.		
	ICF-CtRp or ICF-Rupt or ICF-SH or				\$ noVB		
	1.000						
1							
8	15.00						
3	54	11		42	\$ Case 2: Have CHR or a leak.		
	1 +	1 +		3	\$ No rupture by case 1.		
	L2-CHR	or B-Leak	or ICF-Leak				
	1.000						
1							
8	16.00						
3	50	31		28	\$ Case 3: Prompt CCI with Cavity Dry after VB		
	1 * 2 * -3				\$ - Generate only the non-cond. gases from C/		
	PrmptCCI & RC-Dry	& noAcDaVB			\$ By cases 1 & 2 have no CF & no CHR.		
	1.000				\$ 35 psia from STCP, Letter Rept.. TB & 53B.		
1							
8	35.00						
3	50	31		28	\$ Case 4: Prompt CCI Under Water, Debris Bed is Not Coolable		
	1 * (1 + 3)				\$ - Generate the non-cond. gases from CCI and some steam.		
	PrmptCCI & (RC-Wet	or AcDaVB)			\$ By cases 1 & 2 have no CF & no CHR.		
	1.000						
1							
8	47.00						
3	55	31		28	\$ Case 5: Debris is Coolable, but the Cavity is only 1/4 Full.		
	1 * 2 *			3	\$ The Accumulator Water (1.7E5 lbm) Boils Off in about an		

1	DelydCCI & RC-Dry & AcDeVB	\$	Hour, and then CCI Starts. Have Steam and Non-Cond. Gases.
1	1.000		
8	53.00		
1	Otherwise -- DelydCCI & RC-Full	\$	Case 5: Debris bed is Coolable and the Cavity is Full (7.2E5 lbm)
1	1.000	\$	It takes Many Hours to Boil Off the Water before CCI starts.
8	70.00	\$	Intact Cont. & No CHR. This Case is Not Realizable at Surry.
57	How much H2 and CO2 is Produced during CCI?		
2	CCI noCCI	\$	H2-CCI - Parameter 9 \$ Both PUIQ 59
4	1 2	\$	CO2-CCI - Parameter 10
4	Cases		
2	50 55	\$	Case 1: No CCI.
2	2 * 2		
	noPrmCCI & noDldCCI	\$	H2-CCI = Hydrogen produced by CCI (Kg-moles) in addition to
2	0.000 1.000	\$	that produced by oxidizing the rest of the Zr.
9	0.00 0.00	\$	It includes any CO produced.
10	0.00 0.00	\$	CO2-CCI = Carbon Dioxide produced by CCI (Kg-moles)
1	48	\$	Case 2: A large amount (> 70%) of
1	1	\$	the core is involved in CCI.
	Lrg-CCI		
2	1.000 0.000		
9	220.00 0.00		
10	35.00 0.00		
1	48	\$	Case 3: A moderate amount (30-70%) of
2	2	\$	the core is involved in CCI.
	Med-CCI		
2	1.000 0.000		
9	130.00 0.00		
10	22.00 0.00		
	Otherwise -- Sm1-CCI	\$	Case 4: A small amount (< 30%) of
2	1.000 0.000	\$	the core is involved in CCI.
9	37.00 0.00		
10	7.00 0.00		
1	How much Hydrogen Burns or Leaks Out of Containment?		
1	FrH2-Rem	\$	FrH2-Brn - Parameter 11 \$ Both PUIQ 59
4	1	\$	FrH2-Lk - Parameter 12
5	Cases		
2	1 42	\$	Case 1: Containment has failed -
	4 + -4	\$	Most of the Hydrogen has leaked out. Since the
	Brk-V or ICF	\$	containment is already failed, the fractions
2	1.000	\$	burned and leaked do not matter much.
11	0.00	\$	FrH2-Brn = Fraction of H2 from before VB burned at VB.
12	0.80	\$	FrH2-Lk = Fraction of H2 leaked out of containment.
4	35 35 22 24	\$	Case 2: HPME occurred at VB, and the containment was
	(1 + (3 * -4)) * 1	\$	not steam-inert. Most of the hydrogen from before
	(PrEJ or (BtmHd & notLoPr)) & E-Sp	\$	VB burned.
2	1.000	\$	None of these cases apply when AC power is con-
11	0.90	\$	tinuously available. See Flammability Question.
12	0.00		
4	35 35 22 24	\$	Case 3: HPME occurred at VB, and the cont. could have
	(1 + (3 * -4)) * -1	\$	been steam-inert. Some hydrogen from before VB
	(PrEJ or (BtmHd & notLoPr)) & noE-Sp	\$	could have burned, but the hydrogen produced at
2	1.000	\$	VB is not likely to have burned.


```

L2-H2F & L2-ACP          $ AC Power is Available -
0.999 0.001              $ Ignition is Highly Likely.

2
15 0.9F 0.00
16 0.90 0.00
2 61 51
1 * -1
L2-H2F & noL2-ACP
0.300 0.700

2
15 0.95 0.00
16 0.90 0.00
Otherwise
0.000 1.000

2
15 0.00 0.00
16 0.00 0.00
63 Resulting Pressure Rise?
2 L2-H2Rrn L2nH2BRn          $ P17 UIQ 64
6 1 2
2 Cases
1 62
2
noL2-HB
1 17
dp-L2HB
FUN-NOBURN
'THRESH' 1 999.000
Set DP-L2HB to zero.
Otherwise
6 13 14 15 8 16 17
L2H2-Cnt L2StmCnt L2-ConvR L2PBase dp-Scale dp-L2HB
FUN-LH2BRN
'THRESH' 1 1.000
Calculate pressure rise from combustion
64 Containment Failure, and Type of Containment Failure?
5 L2CF-CRp L2CF-Rp L2CF-Lk L2CF-SHEAR no-L2CF          $ RIQ 65 68
6 1 2 3 4 5
2 Cases
1 42
-4
ICF
1 8
L2PBase
AND
GETHRESH 4 999 888 777 666
Dummy Values to Assure No Failure
Otherwise
4 8 17 6 7          $ Case 2:
L2PBase dp-L2HB CF-Pr RndNum
FUN-LCFFst
GETHRESH 4 4 3 2 1
User Function for Fast Pressure Rise
65 Sprays after Very Late CF?
2 F-Sp noF-Sp          $ RIQ 67 68
2 1 2
3 Cases
2 52 51
3 + 3
L2fSp or L2fACP
0.000 1.000
$ Case 1: Sprays failed or power not recoverable.
$ Assume AC power always recovered by this time,
$ so sprays operate unless damaged by CF
$ in the remaining cases.

```

1	64						\$ Case 2: Catastrophic rupture of containment - \$ spray failure unlikely. \$ Use the same values as in question 42.	
	1							
	L2CF-CRp							
	143,4,1	143,4,3						
	Otherwise							
	1.000	0.000						
66	Fan Coolers after Very Late CF?						\$ RIQ	67
2	F-FC	FfFC						
2	1	2						
2	Cases							
2	53	51						
	3 +	3						
	L2FfC or	L2fACP						
	0.000	1.000						
	Otherwise							
	1.000	0.000						
67	Containment Heat Removal after Very Late CF?						\$ RIQ	69
2	F-CHR	FfCHR						
2	1	2						
2	Cases							
2	65	66						
	1 +	1						
	F-Sp or	F-FC						
	1.000	0.000						
	Otherwise							
	0.000	1.000						
68	Eventual Basemat Melt through?						\$ RIQ	70
3	MTnDePr	MTwDePr	noMT					1145
2	1	2	3					1146
7	Cases							1147
4	11	23	42	64	\$ Case 1: Containment failed already,			1148
	1 +	1 +	-4 +	-5	\$ or no VB - BMT is not of interest.			1149
	B-Leak or	noVB or	ICF or	L2CF				1150
	0.000	0.000	1.000					1151
3	50	55	65		\$ Case 2: Coolable debris bed and sprays operating			1152
	2 *	2 *	1		\$ - no basemat melt-thru. If FCs drained to the			1153
	noPrmCCI &	noDldCCI &	F-Sp		\$ cavity, could use F-CHR instead of F-Sp.			1154
	0.000	0.000	1.000					1155
2	48	65			\$ Case 3: Large fraction of core in CCI, water covered.			1156
	1 *	1			\$ This and the following cases must			1157
	Lrg-CCI &	F-Sp			\$ have CCI by case 2.			1158
	0.300	0.100	0.600					1159
2	48	65			\$ Case 4: Large fraction of core in CCI, dry cavity.			1160
	1 *	2						1161
	Lrg-CCI &	noF-Sp						1162
	0.400	0.400	0.200					1163
2	48	65			\$ Case 5: Medium fraction of core in CCI, water covered			1164
	2 *	1						1165
	Med-CCI &	F-Sp						1166
	0.150	0.050	0.800					1167
2	48	65			\$ Case 6: Medium fraction of core in CCI, dry cavity.			1168
	2 *	2						1169
	Med-CCI &	noF-Sp						1170
	0.250	0.250	0.500					1171
	Otherwise - Sm1-CCI				\$ Case 7: Small fraction of core in CCI, wet or dry.			1172
	0.025	0.025	0.950					1173
69	Eventual Overpressure Failure of Containment?						\$ RIQ	70
2	F-CF-OP	noFCFOP						
2	1	2						
3	Cases							

5	11	23	42	64	67	\$ Case 1: Containment is already		
	1 +	1	+ -4	+ -5	+ 1	\$ failed, or have CHR, or have		
	B-Leak or	noVB	or ICF	or L2CF	or F-CHR	\$ no VB - OP now not credible.		
	0.000	1.000						
2	55	31				\$ Case 2: CDB boiled off a full cavity		
	1 *	1				\$ of water - OP now is at least possible.		
	DehydCCl	& RC-Wet						
	0.050	0.950						
	Otherwise					\$ Case 3: Did not boil off full cavity - noncondensibles alone		
	0.001	0.999				\$ or with boiloff of accumulator water won't result in OP.		
70	Basemat Melt-through before Overpressure Failure?							
3	F-BMT	FCF-Lk	Neither			\$ R10	71	
2	1	2	3			\$ Very Late OP Failure is always leak	1189	
5	Cases						1190	
4	68	69				\$ Case 1: Have eventual BMT, but	1191	
	-3 *	2				\$ do not have eventual OP.	1192	
	BMT &	noFCFOP					1193	
	1.000	0.000	0.000				1194	
2	68	69				\$ Case 2: Have eventual OP, but	1195	
	3 *	1				\$ do not have eventual BMT.	1196	
	noMT &	F-CF-OP					1197	
	0.000	1.000	0.000				1198	
2	68	69				\$ Case 3: Have eventual OP and have	1199	
	1 *	1				\$ BMT which does not depressurize	1200	
	MTnDePr &	F-CF-OP				\$ containment in two hours or less.	1201	
	0.250	0.750	0.000				1202	
2	68	69				\$ Case 4: Have eventual OP and have	1203	
	2 *	1				\$ BMT which does depressurize	1204	
	MTwDePr &	F-CF-OP				\$ containment in two hours or less.	1205	
	0.500	0.500	0.000				1206	
	Otherwise					\$ Case 5: Have neither BMT nor	1207	
	0.000	0.000	1.000			\$ OP, or already have CF.	1208	
71	Final Containment Condition?						1209	
6	F-Ruptr	F-Leak	F-MT	Bypass	noCF	Shear	1210	
2	1	2	3	4	5	6		
6	Cases							
4	42	42	64	64			\$ Case 1: Containment ruptured.	
	1 +	2 +	1 +	2				
	ICF-CtRp or	ICF-Rupt or	L2CF-CRp or	L2CF-Rp				
	1.000	0.000	0.000	0.000	0.000	0.000		
4	11	42	64	70			\$ Case 2: Containment leaks.	
	1 +	3 +	3 +	2				
	B-Leak or	ICF-Leak or	L2CF-Lk or	FCF-Lk				
	0.000	1.000	0.000	0.000	0.000	0.000		
2	42	64						
	5 +	4						
	ICF-Shear or	L2CF-Shear						
	0.000	0.000	0.000	0.000	1.000			
3	1	1	18				\$ Case 3: Containment bypassed.	
	4 +	5 +	1					
	Brk-V or	B-SGTR or	E-SGTRS3					
	0.000	0.000	0.000	1.000	0.500	0.000		
1	70						\$ Case 4: Basemat Melt-Thru.	
	1							
	F-BMT							
	0.000	0.000	1.000	0.000	0.000	0.000		
	Otherwise						\$ Case 6: No Containment Failure.	
	0.000	0.000	0.000	0.000	1.000	0.000		
72	Time of core damage							
2	ECorD	LCorD						
2	1	2						

2	Cases				
4		3	8	5	19
		5	+ 1	+(2	* 1)
	BiECCS or SG-HR or (BaECCS and E-ACP)				
		0.000		1.000	
	Otherwise				
		1.000		0.000	

APPENDIX B

**REALIZED APET SPLIT FRACTIONS
FOR THE ZION LOCA PDS GROUP**

APPENDIX B

Realized APET Split Fractions
for the Zion LOCA PDS Group

This Appendix lists the realized split fractions from a sampling evaluation of the Zion LOCA APET. The evaluation consisted of 150 Latin Hypercube Samples, with the questions sampled according to the input sampling files obtained from BNL.

Realized LOCA split fractions, 150 LHS, base case, no truncation

TREE ID: ZION APET, Rev 6, 8 MAR 89 - 72 Questions - PDS-2, LOCAS
OF QUESTIONS: 72
OBSERVATIONS: 150
FOR SERIES: CET FINAL SAMP
SEQUENCE: PDS-2, LHS

***** QUESTION: 1 Size and Location of RCS Break when the Core Uncovers? \$
Q-TYPE/TIMES ASKED: INDEP. INPUT PROB. 300
BRANCHES: Brk-A Brk-S2 Brk-S3 Brk-V B-SGTR B-PORV
1 2 3 4 5 6
REALIZED SPLIT: 4.600E-02 9.540E-01 0.000E+00 0.000E+00 0.000E+00 0.000E+00

***** QUESTION: 2 For SGTR, are the Secondary System SRVs Stuck Open? \$
Q-TYPE/TIMES ASKED: INDEP. INPUT PROB. 300
BRANCHES: SSRV-St0 SSRVnSt0
1 2
REALIZED SPLIT: 0.000E+00 1.000E+00

***** QUESTION: 3 Status of ECCS? \$
Q-TYPE/TIMES ASKED: DEP. INPUT PROB. 900
BRANCHES: B-ECCS BaECCS BfECCS B-LPIS BIECCS
1 2 3 4 5
REALIZED SPLIT: 0.000E+00 1.840E-05 9.258E-01 2.862E-04 7.393E-02

SUMMARY BY CASE

CASE NUMBER/SPLIT: 1 4.600E-02
DEPENDENCIES: 1
REQ. BRANCHES: 1
DESCRIPTION: Brk-A
CASE/BRANCH SPLIT: 0.000E+00 1.840E-05 5.824E-03 0.000E+00 4.016E-02
CASE NUMBER/SPLIT: 2 9.540E-01
DEPENDENCIES: 1
REQ. BRANCHES: 2
DESCRIPTION: Brk-S2
CASE/BRANCH SPLIT: 0.000E+00 0.000E+00 9.199E-01 2.862E-04 3.377E-02

***** QUESTION: 4 Status of Sprays? \$
Q-TYPE/TIMES ASKED: DEP. INPUT PROB. 900

BRANCHES:	B-Sp	BaSp	BfSp	noB-SWHX	BASp	BCSp
	1	2	3	4	5	6
REALIZED SPLIT:	1.000E+00	0.000E+00	0.000E+00	0.000E+00	0.000E+00	0.000E+00

SUMMARY BY CASE

CASE NUMBER/SPLIT:	1	4.600E-02				
DEPENDENCIES:	1					
REQ. BRANCHES:	1					
DESCRIPTION:	Brk-A					
CASE/BRANCH SPLIT:		4.600E-02	0.000E+00	0.000E+00	0.000E+00	0.000E+00
CASE NUMBER/SPLIT:	2	9.540E-01				
DESCRIPTION:	Otherwise					
CASE/BRANCH SPLIT:		9.540E-01	0.000E+00	0.000E+00	0.000E+00	0.000E+00

***** QUESTION: 5 Status of Fan Coolers? \$

Q-TYPE/TIMES ASKED: DEP. INPUT PROB. 2700

BRANCHES:	B-FC	BaFC	BFFC
	1	2	3
REALIZED SPLIT:	5.555E-01	3.453E-03	4.410E-01

SUMMARY BY CASE

CASE NUMBER/SPLIT:	2	9.540E-01			
DEPENDENCIES:	1				
REQ. BRANCHES:	2				
DESCRIPTION:	Brk-S2				
CASE/BRANCH SPLIT:		5.097E-01	3.434E-03	4.408E-01	
CASE NUMBER/SPLIT:	3	4.600E-02			
DESCRIPTION:	Otherwise			\$ case 3 : A/S1	
CASE/BRANCH SPLIT:		4.582E-02	1.840E-05	1.610E-04	

***** QUESTION: 6 Status of AC Power? \$

Q-TYPE/TIMES ASKED: INDEP. INPUT PROB. 2700

BRANCHES:	B-ACP	BaACP	BfACP
	1	2	3
REALIZED SPLIT:	1.000E+00	0.000E+00	0.000E+00

***** QUESTION: 7 RWST Injected into Containment? \$

Q-TYPE/TIMES ASKED: DEP. INPUT PROB. 4050

BRANCHES:	RWST-In	RWSTaIn	RWSTfIn
	1	2	3
REALIZED SPLIT:	9.943E-01	0.000E+00	5.658E-03

SUMMARY BY CASE

CASE NUMBER/SPLIT:	1	4.600E-02			
DEPENDENCIES:	1				
REQ. BRANCHES:	1				
DESCRIPTION:	Brk-A				
CASE/BRANCH SPLIT:		4.034E-02	0.000E+00	5.658E-03	
CASE NUMBER/SPLIT:	2	9.540E-01			
DEPENDENCIES:	1				
REQ. BRANCHES:	2				
DESCRIPTION:	Brk-S2				
CASE/BRANCH SPLIT:		9.540E-01	0.000E+00	0.000E+00	

***** QUESTION: 8 Heat Removal from the Steam Generators? \$

Q-TYPE/TIMES ASKED: DEP. INPUT PROB. 4050
 BRANCHES: SS-HR SGaHR SGfHR SGdHR
 1 2 3 4
 REALIZED SPLIT: 1.000E+00 0.000E+00 0.000E+00 0.000E+00

SUMMARY BY CASE

CASE NUMBER/SPLIT: 2 1.000E+00
 DESCRIPTION: Otherwise \$ Case 2: All other brea
 CASE/BRANCH SPLIT: 1.000E+00 0.000E+00 0.000E+00 0.000E+00

***** QUESTION: 9 Did the Operators Depressurize the Secondary before the Core Uncovers? \$
 Q-TYPE/TIMES ASKED: DEP. INPUT PROB. 4050
 BRANCHES: SecDePr noScDePr
 1 2
 REALIZED SPLIT: 1.000E+00 0.000E+00

SUMMARY BY CASE

CASE NUMBER/SPLIT: 2 9.540E-01
 DEPENDENCIES: 1
 REQ. BRANCHES: 2
 DESCRIPTION: Brk-S2
 CASE/BRANCH SPLIT: 9.540E-01 0.000E+00

CASE NUMBER/SPLIT: 3 4.600E-02
 DESCRIPTION: Otherwise \$ Case 3: All other break sizes
 CASE/BRANCH SPLIT: 4.600E-02 0.000E+00

***** QUESTION: 10 Cooling for RCP Seals? \$
 Q-TYPE/TIMES ASKED: DEP. INPUT PROB. 8100
 BRANCHES: B-PSC BaPSC BFPSC
 1 2 3
 REALIZED SPLIT: 7.716E-02 0.000E+00 9.228E-01

SUMMARY BY CASE

CASE NUMBER/SPLIT: 1 4.600E-02
 DEPENDENCIES: 1
 REQ. BRANCHES: 1
 DESCRIPTION: Brk-A
 CASE/BRANCH SPLIT: 4.568E-02 0.000E+00 3.220E-04

CASE NUMBER/SPLIT: 2 9.540E-01
 DEPENDENCIES: 1
 REQ. BRANCHES: 2
 DESCRIPTION: Brk-S2
 CASE/BRANCH SPLIT: 3.148E-02 0.000E+00 9.225E-01

***** QUESTION: 11 Initial Containment Leak or Isolation Failure? \$
 Q-TYPE/TIMES ASKED: INDEP. INPUT PROB. 16200
 BRANCHES: B-Leak noB-Leak
 1 2
 REALIZED SPLIT: 5.000E-03 9.950E-01

***** QUESTION: 12 Event V - Break Location under Water? \$
 Q-TYPE/TIMES ASKED: INDEP. INPUT PROB. 16200
 BRANCHES: V-Wet V-Dry
 1 2
 REALIZED SPLIT: 5.000E-01 5.000E-01

***** QUESTION: 13 RCS Pressure at the Start of Core Degradation? \$
 Q-TYPE/TIMES ASKED: DEP. INPUT PROB. 16200
 BRANCHES: E-SSPr E-HiPr E-ImPr E-LoPr
 1 2 3 4
 REALIZED SPLIT: 0.000E+00 0.000E+00 9.540E-01 4.600E-02

SUMMARY BY CASE

CASE NUMBER/SPLIT: 1 4.600E-02
 DEPENDENCIES: 1 1
 REQ. BRANCHES: 1 + 4
 DESCRIPTION: Brk-A Brk-V
 CASE/BRANCH SPLIT: 0.000E+00 0.000E+00 0.000E+00 4.600E-02
 CASE NUMBER/SPLIT: 3 9.540E-01
 DEPENDENCIES: 9
 REQ. BRANCHES: 1
 DESCRIPTION: SecDePr
 CASE/BRANCH SPLIT: 0.000E+00 0.000E+00 9.540E-01 0.000E+00

***** QUESTION: 14 Do the PORVs or SRVs Stick Open? \$
 Q-TYPE/TIMES ASKED: DEP. INPUT PROB. 16200
 BRANCHES: PORV-StD PORVnStD
 1 2
 REALIZED SPLIT: 0.000E+00 1.000E+00

SUMMARY BY CASE

CASE NUMBER/SPLIT: 2 1.000E+00
 DESCRIPTION: Otherwise \$ Case 2: RCS not at setpoint pres
 CASE/BRANCH SPLIT: 0.000E+00 1.000E+00

***** QUESTION: 15 Temperature-Induced RCP Seal Failure? (After core uncovering)
 Q-TYPE/TIMES ASKED: DEP. INPUT PROB. 16200
 BRANCHES: EB-PSS3 noEB-PSF
 1 2
 REALIZED SPLIT: 5.537E-01 4.463E-01

SUMMARY BY CASE

CASE NUMBER/SPLIT: 1 7.716E-02
 DEPENDENCIES: 10
 REQ. BRANCHES: 1
 DESCRIPTION: B-PSC
 CASE/BRANCH SPLIT: 0.000E+00 7.716E-02
 CASE NUMBER/SPLIT: 4 9.228E-01
 DESCRIPTION: Otherwise \$ Case 4: RCS at IM or low pressur
 CASE/BRANCH SPLIT: 5.537E-01 3.691E-01

***** QUESTION: 16 Is the RCS Depressurized before Breach by Opening the PZR PORVs?
 Q-TYPE/TIMES ASKED: DEP. INPUT PROB. 30600
 BRANCHES: PrmDePr PZRDePr
 1 2
 REALIZED SPLIT: 5.000E-01 5.000E-01

SUMMARY BY CASE

CASE NUMBER/SPLIT: 1 9.997E-01
 DEPENDENCIES: 6 3 3
 REQ. BRANCHES: 1 * /4 * /1

DESCRIPTION: B-ACP /B-LPIS /B-ECCS
CASE/BRANCH SPLIT: 5.000E-01 4.997E-01

CASE NUMBER/SPLIT: 2 2.862E-04
DESCRIPTION: Otherwise \$ Opening the PORVs is prohibi
CASE/BRANCH SPLIT: 0.000E+00 2.862E-04

***** QUESTION: 17 Temperature-Induced Hot Leg or Surge Line Break?
Q-TYPE/TIMES ASKED: DEP. INPUT PROB. 61200
BRANCHES: EB-HLA noEB-HLA
1 2
REALIZED SPLIT: 1.000E-03 9.990E-01

SUMMARY BY CASE

CASE NUMBER/SPLIT: 3 1.000E+00
DESCRIPTION: Otherwise -- noSSPr \$ Case 3: RCS not at 2
CASE/BRANCH SPLIT: 1.000E-03 9.990E-01

***** QUESTION: 18 Temperature-Induced SGTR?
Q-TYPE/TIMES ASKED: DEP. INPUT PROB. 61200
BRANCHES: E-SGTRS3 noE-SGTR
1 2
REALIZED SPLIT: 0.000E+00 1.000E+00

SUMMARY BY CASE

CASE NUMBER/SPLIT: 2 1.000E+00
DESCRIPTION: Otherwise \$ Case 2: R
CASE/BRANCH SPLIT: 0.000E+00 1.000E+00

***** QUESTION: 19 Is AC Power Available Early (Between Uncovering TAF & VB-30 min)?
Q-TYPE/TIMES ASKED: DEP. INPUT PROB. 61200
BRANCHES: E-ACP EaACP EfACP
1 2 3
REALIZED SPLIT: 1.000E+00 0.000E+00 0.000E+00

SUMMARY BY CASE

CASE NUMBER/SPLIT: 1 1.000E+00
DEPENDENCIES: 6
REQ. BRANCHES: 1
DESCRIPTION: B-ACP
CASE/BRANCH SPLIT: 1.000E+00 0.000E+00 0.000E+00

***** QUESTION: 20 After Power Recovery, Is Coolant Injection Re-Established Promptly?
Q-TYPE/TIMES ASKED: DEP. INPUT PROB. 61200
BRANCHES: E-RECC noE-RECC
1 2
REALIZED SPLIT: 0.000E+00 1.000E+00

SUMMARY BY CASE

CASE NUMBER/SPLIT: 3 1.000E+00
DESCRIPTION: Otherwise \$ Case 2: Power not restored, or E
CASE/BRANCH SPLIT: 0.000E+00 1.000E+00

***** QUESTION: 21 Rate of Blowdown to Containment? [This is blowdown before vessel bre
Q-TYPE/TIMES ASKED: DEP. INPUT PROB. 61200
BRANCHES: EBD-A EBD-S2 EBD-S3 noEBD

REALIZED SPLIT: 1 2 3 4
 4.695E-02 9.531E-01 0.000E+00 0.000E+00

SUMMARY BY CASE

CASE NUMBER/SPLIT: 1 4.695E-02
 DEPENDENCIES: 1 17
 REQ. BRANCHES: 1 + 1
 DESCRIPTION: Brk-A EB-HLA
 CASE/BRANCH SPLIT: 4.695E-02 0.000E+00 0.000E+00 0.000E+00

CASE NUMBER/SPLIT: 3 9.531E-01
 DEPENDENCIES: 1 14 16
 REQ. BRANCHES: 2 + 1 + 1
 DESCRIPTION: Brk-S2 PORV-StO PrmDePr
 CASE/BRANCH SPLIT: 0.000E+00 9.531E-01 0.000E+00 0.000E+00

***** QUESTION: 22 Vessel Pressure just before Breach?
 Q-TYPE/TIMES ASKED: DEP INPUT PROB. 61200
 BRANCHES: I-SSPr I-HiPr I-ImPr I-LoPr
 1 2 3 4
 REALIZED SPLIT: 0.000E+00 0.000E+00 9.612E-02 9.039E-01

SUMMARY BY CASE

CASE NUMBER/SPLIT: 1 5.235E-01
 DEPENDENCIES: 21 1 1 16 14
 REQ. BRANCHES: 1 + 4 + (2 * (1 + 1))
 DESCRIPTION: EBD-A Brk-V Brk-S2 PrmDePr PORV-StO
 CASE/BRANCH SPLIT: 0.000E+00 0.000E+00 0.000E+00 5.235E-01

CASE NUMBER/SPLIT: 2 4.765E-01
 DEPENDENCIES: 21
 REQ. BRANCHES: 2
 DESCRIPTION: EBD-S2
 CASE/BRANCH SPLIT: 0.000E+00 0.000E+00 9.612E-02 3.804E-01

***** QUESTION: 23 Is Core Damage Arrested? No Vessel Breach?
 Q-TYPE/TIMES ASKED: DEP INPUT PROB. 78840
 BRANCHES: noVB VB
 1 2
 REALIZED SPLIT: 2.227E-04 9.998E-01

SUMMARY BY CASE

CASE NUMBER/SPLIT: 1 9.258E-01
 DEPENDENCIES: 19 3
 REQ. BRANCHES: /1 + 3
 DESCRIPTION: /E-ACP BFECCS
 CASE/BRANCH SPLIT: 0.000E+00 9.258E-01

CASE NUMBER/SPLIT: 3 2.290E-04
 DEPENDENCIES: 22 3 22 3
 REQ. BRANCHES: (4 * 4) + (/1 * 1)
 DESCRIPTION: I-LoPr B-LPIS /I-SSPr B-ECCS
 CASE/BRANCH SPLIT: 2.061E-04 2.290E-05

CASE NUMBER/SPLIT: 4 7.399E-02
 DEPENDENCIES: 3
 REQ. BRANCHES: /2
 DESCRIPTION: /BaECCS
 CASE/BRANCH SPLIT: 0.000E+00 7.399E-02

CASE NUMBER/SPLIT: 9 1.840E-05
DESCRIPTION: Otherwise - B-PORV
CASE/BRANCH SPLIT: 1.656E-05 1.840E-06

\$ Case 9: Initial AFW & no Break,

***** QUESTION: 24 Early Sprays?
Q-TYPE/TIMES ASKED: DEP. INPUT PROB. 78840
BRANCHES: E-Sp EaSp EfSp
1 2 3
REALIZED SPLIT: 1.000E+00 0.000E+00 0.000E+00

SUMMARY BY CASE

CASE NUMBER/SPLIT: 1 1.000E+00
DEPENDENCIES: 4 4 4
REQ. BRANCHES: 1 + 5 + 6
DESCRIPTION: B-Sp BASp BCSp
CASE/BRANCH SPLIT: 1.000E+00 0.000E+00 0.000E+00

***** QUESTION: 25 Early Fan Coolers?
Q-TYPE/TIMES ASKED: DEP. INPUT PROB. 78840
BRANCHES: E-FC EaFC EFFC
1 2 3
REALIZED SPLIT: 5.590E-01 0.000E+00 4.410E-01

SUMMARY BY CASE

CASE NUMBER/SPLIT: 1 5.556E-01
DEPENDENCIES: 5
REQ. BRANCHES: 1
DESCRIPTION: B-FC
CASE/BRANCH SPLIT: 5.556E-01 0.000E+00 0.000E+00
CASE NUMBER/SPLIT: 2 4.410E-01
DEPENDENCIES: 5
REQ. BRANCHES: 3
DESCRIPTION: BfFC
CASE/BRANCH SPLIT: 0.000E+00 0.000E+00 4.410E-01
CASE NUMBER/SPLIT: 3 3.453E-03
DEPENDENCIES: 5 19
REQ. BRANCHES: 2 * 1
DESCRIPTION: BaFC E-ACP
CASE/BRANCH SPLIT: 3.453E-03 0.000E+00 0.000E+00

***** QUESTION: 26 Early Containment Heat Removal?
Q-TYPE/TIMES ASKED: DEP. INPUT PROB. 78840
BRANCHES: E-CHR EfCHR
1 2
REALIZED SPLIT: 5.590E-01 4.410E-01

SUMMARY BY CASE

CASE NUMBER/SPLIT: 1 5.590E-01
DEPENDENCIES: 25
REQ. BRANCHES: 1
DESCRIPTION: E-FC
CASE/BRANCH SPLIT: 5.590E-01 0.000E+00
CASE NUMBER/SPLIT: 3 4.410E-01
DESCRIPTION: Otherwise
CASE/BRANCH SPLIT: 0.000E+00 4.410E-01

\$ Case 2: No Sprays, No Fan Cooler

***** QUESTION: 27 Baseline Containment Pressure just before VB?
 Q-TYPE/TIMES ASKED: DEP. INPUT PROB. INPUT PARM. 78840
 BRANCHES: IPBase
 1
 REALIZED SPLIT: 1.000E+00

SUMMARY BY CASE

CASE NUMBER/SPLIT: 1 2.227E-04
 DEPENDENCIES: 21 23
 REQ. BRANCHES: 4 + 1
 DESCRIPTION: noEBD noVB
 CASE/BRANCH SPLIT: 2.227E-04

CASE NUMBER/SPLIT: 2 5.589E-01
 DEPENDENCIES: 26 4
 REQ. BRANCHES: 1 * /4
 DESCRIPTION: E-CHR /noB-SWX
 CASE/BRANCH SPLIT: 5.589E-01

CASE NUMBER/SPLIT: 3 6.017E-04
 DEPENDENCIES: 26 21
 REQ. BRANCHES: 2 * 1
 DESCRIPTION: EfCHR EBD-A
 CASE/BRANCH SPLIT: 6.017E-04

CASE NUMBER/SPLIT: 4 4.403E-01
 DESCRIPTION: Otherwise
 CASE/BRANCH SPLIT: 4.403E-01

\$ Case 4: No sprays and no large b

***** QUESTION: 28 Time of Accumulator Discharge?
 Q-TYPE/TIMES ASKED: DEP. INPUT PROB. 78840
 BRANCHES: AcDbCM AcDdCM AcDaVB
 1 2 3
 REALIZED SPLIT: 1.000E+00 0.000E+00 0.000E+00

SUMMARY BY CASE

CASE NUMBER/SPLIT: 1 1.000E+00
 DEPENDENCIES: 13 13 8 9
 REQ. BRANCHES: 3 + 4 +(4 * 1)
 DESCRIPTION: E-ImPr E-LoPr SGdHR SecDePr
 CASE/BRANCH SPLIT: 1.000E+00 0.000E+00 0.000E+00

***** QUESTION: 29 Fraction of Zr Oxidized In-Vessel during Core Degradation?
 Q-TYPE/TIMES ASKED: DEP. INPUT PROB. INPUT PARM. 78840
 BRANCHES: ZrOx-InV
 1
 REALIZED SPLIT: 1.000E+00

SUMMARY BY CASE

CASE NUMBER/SPLIT: 5 9.540E-01
 DEPENDENCIES: 13 28
 REQ. BRANCHES: 3 * /2
 DESCRIPTION: E-ImPr /AcDdCM
 CASE/BRANCH SPLIT: 9.540E-01

CASE NUMBER/SPLIT: 7 4.600E-02
 DESCRIPTION: Otherwise - E-LoPr \$ Case 7: Low RCS Pressure (<200 psia)
 CASE/BRANCH SPLIT: 4.600E-02

***** QUESTION: 30 Amount of Zr Oxidized In-Vessel during Core Degradation?
 Q-TYPE/TIMES ASKED: INDEP. CALC. PROB. 78840
 BRANCHES: Hi-ZrOx Lo-ZrOx
 1 2
 REALIZED SPLIT: 4.976E-01 5.025E-01

***** QUESTION: 31 Amount of Water in the Reactor Cavity at Vessel Breach?
 Q-TYPE/TIMES ASKED: DEP. INPUT PROB. 78840
 BRANCHES: RC-Wet RC-Dry
 1 2
 REALIZED SPLIT: 9.944E-01 5.658E-03

SUMMARY BY CASE

CASE NUMBER/SPLIT: 1 5.658E-03
 DEPENDENCIES: 7 7 19
 REQ. BRANCHES: 3 +(2 * /1)
 DESCRIPTION: RWSTfin RWSTaIn /E-ACP
 CASE/BRANCH SPLIT: 0.000E+00 5.658E-03

CASE NUMBER/SPLIT: 2 9.944E-01
 DESCRIPTION: Otherwise \$ Case 2: RWST injected or sprays
 CASE/BRANCH SPLIT: 9.944E-01 0.000E+00

***** QUESTION: 32 Fraction of Core Released from Vessel at Breach?
 Q-TYPE/TIMES ASKED: INDEP. INPUT PROB. INPUT PARM. 78840
 BRANCHES: FCoRel
 1
 REALIZED SPLIT: 1.000E+00

***** QUESTION: 33 Amount of Core Released from Vessel at Breach?
 Q-TYPE/TIMES ASKED: INDEP. CALC. PROB. 78840
 BRANCHES: Hi-FCoR Md-FCoR Lo-FCoR
 1 2 * 3
 REALIZED SPLIT: 2.667E-01 4.467E-01 2.867E-01

***** QUESTION: 34 Does an Alpha Mode Event Fail both the Vessel and the Containment?
 Q-TYPE/TIMES ASKED: DEP. INPUT PROB. 105636
 BRANCHES: Alpha noAlpha
 1 2
 REALIZED SPLIT: 8.146E-03 9.919E-01

SUMMARY BY CASE

CASE NUMBER/SPLIT: 1 9.037E-01
 DEPENDENCIES: 23 22
 REQ. BRANCHES: 2 * 4
 DESCRIPTION: VB I-LoPr
 CASE/BRANCH SPLIT: 8.070E-03 8.956E-01

CASE NUMBER/SPLIT: 2 9.612E-02
 DEPENDENCIES: 23 22
 REQ. BRANCHES: 2 * /4
 DESCRIPTION: VB /I-LoPr
 CASE/BRANCH SPLIT: 7.689E-05 9.604E-02

CASE NUMBER/SPLIT: 3 2.227E-04
 DESCRIPTION: Otherwise \$ Case 3: Core Damage Arrested, no SF
 CASE/BRANCH SPLIT: 0.000E+00 2.227E-04

***** QUESTION: 35 Type of Vessel Breach? \$
 Q-TYPE/TIMES ASKED: DEP. INPUT PROB. 106320
 BRANCHES: PrEj Pour BtmHd noVBoA
 1 2 3 4
 REALIZED SPLIT: 5.153E-02 9.391E-01 1.052E-03 8.369E-03

SUMMARY BY CASE

CASE NUMBER/SPLIT: 1 8.369E-03
 DEPENDENCIES: 23 34
 REQ. BRANCHES: 1 + 1
 DESCRIPTION: noVB Alpha
 CASE/BRANCH SPLIT: 0.000E+00 0.000E+00 0.000E+00 8.369E-03

CASE NUMBER/SPLIT: 4 9.604E-02
 DEPENDENCIES: 22
 REQ. BRANCHES: 3
 DESCRIPTION: I-ImPr
 CASE/BRANCH SPLIT: 5.153E-02 4.346E-02 1.052E-03 0.000E+00

CASE NUMBER/SPLIT: 5 8.956E-01
 DESCRIPTION: Otherwise - I-LoPr \$ Case 5: RCS at Low P
 CASE/BRANCH SPLIT: 0.000E+00 8.956E-01 0.000E+00 0.000E+00

***** QUESTION: 36 Does the Vessel become a "Rocket" and Fail the Containment?
 Q-TYPE/TIMES ASKED: DEP. INPUT PROB. 106320
 BRANCHES: Rocket noRocket
 1 2
 REALIZED SPLIT: 0.000E+00 1.000E+00

SUMMARY BY CASE

CASE NUMBR/SPLIT: 2 1.000E+00
 DESCRIPTION: Otherwise \$ Case 2: Not BtmHd & SSPr - Rocket Not Credib
 CASE/BRANCH SPLIT: 0.000E+00 1.000E+00

***** QUESTION: 37 Size of Hole in Vessel (after Ablation)?
 Q-TYPE/TIMES ASKED: DEP. INPUT PROB. 106320
 BRANCHES: LrgHole SmHole
 1 2
 REALIZED SPLIT: 9.485E-01 5.153E-02

SUMMARY BY CASE

CASE NUMBER/SPLIT: 1 5.153E-02
 DEPENDENCIES: 35
 REQ. BRANCHES: 1
 DESCRIPTION: PrEj
 CASE/BRANCH SPLIT: 0.000E+00 5.153E-02

CASE NUMBER/SPLIT: 2 9.485E-01
 DESCRIPTION: Otherwise \$ Case 2: Not HPME - Large Hole or Irrelevant
 CASE/BRANCH SPLIT: 9.485E-01 0.000E+00

***** QUESTION: 38 Pressure Rise at Vessel Breach? Large Hole Cases
 Q-TYPE/TIMES ASKED: DEP. INPUT PROB. INPUT PARM. 106320
 BRANCHES: DP-VB
 1
 REALIZED SPLIT: 1.000E+00

SUMMARY BY CASE

CASE NUMBER/SPLIT: 1 2.227E-04
 DEPENDENCIES: 23
 REQ. BRANCHES: 1
 DESCRIPTION: noVB
 CASE/BRANCH SPLIT: 2.227E-04

 CASE NUMBER/SPLIT: 2 8.146E-03
 DEPENDENCIES: 34 36
 REQ. BRANCHES: 1 + 1
 DESCRIPTION: Alpha Rocket
 CASE/BRANCH SPLIT: 8.146E-03

 CASE NUMBER/SPLIT: 3 9.391E-01
 DEPENDENCIES: 22 35
 REQ. BRANCHES: 4 + 2
 DESCRIPTION: I-LoPr Pour
 CASE/BRANCH SPLIT: 9.391E-01

 CASE NUMBER/SPLIT: 4 5.153E-02
 DEPENDENCIES: 37
 REQ. BRANCHES: 2
 DESCRIPTION: SmlHole
 CASE/BRANCH SPLIT: 5.153E-02

 CASE NUMBER/SPLIT: 5 3.689E-04
 DEPENDENCIES: 22 31 33
 REQ. BRANCHES: 3 * 1 * 1
 DESCRIPTION: I-ImPr RC-Wet HI-FCoR
 CASE/BRANCH SPLIT: 3.689E-04

 CASE NUMBER/SPLIT: 6 2.977E-04
 DEPENDENCIES: 22 31 33
 REQ. BRANCHES: 3 * 1 * 2
 DESCRIPTION: I-ImPr RC-Wet Md-FCoR
 CASE/BRANCH SPLIT: 2.977E-04

 CASE NUMBER/SPLIT: 7 3.850E-04
 DEPENDENCIES: 22 31 33
 REQ. BRANCHES: 3 * 1 * 3
 DESCRIPTION: I-ImPr RC-Wet Lo-FCoR
 CASE/BRANCH SPLIT: 3.850E-04

***** QUESTION: 39 Pressure Rise at Vessel Breach? Small Hole Cases
 Q-TYPE/TIMES ASKED: DEP. INPUT PROB. INPUT PARM. 106320
 BRANCHES: DP-VB
 1
 REALIZED SPLIT: 1.000E+00

SUMMARY BY CASE

CASE NUMBER/SPLIT: 1 9.485E-01
 DEPENDENCIES: 37 23 34 22 36
 REQ. BRANCHES: 1 + 1 + 1 + 4 + 1
 DESCRIPTION: LrgHole noVB Alpha I-LoPr Rocket
 CASE/BRANCH SPLIT: 9.485E-01

 CASE NUMBER/SPLIT: 2 1.808E-02
 DEPENDENCIES: 22 31 33
 REQ. BRANCHES: 3 * 1 * 1
 DESCRIPTION: I-ImPr RC-Wet HI-FCoR
 CASE/BRANCH SPLIT: 1.808E-02

 CASE NUMBER/SPLIT: 3 1.459E-02

DEPENDENCIES: 22 31 33
 REQ. BRANCHES: 3 * 1 * 2
 DESCRIPTION: I-ImPr RC-Wet Md-FCoR
 CASE/BRANCH SPLIT: 1.459E-02

CASE NUMBER/SPLIT: 4 1.886E-02
 DEPENDENCIES: 22 31 33
 REQ. BRANCHES: 3 * 1 * 3
 DESCRIPTION: I-ImPr RC-Wet Lo-FCoR
 CASE/BRANCH SPLIT: 1.886E-02

***** QUESTION: 40 Does a Significant Ex-Vessel Steam Explosion Occur?
 Q-TYPE/TIMES ASKED: DEP. INPUT PROB. 145236
 BRANCHES: EVSE noEVSE
 1 2
 REALIZED SPLIT: 4.667E-01 5.333E-01

SUMMARY BY CASE

CASE NUMBER/SPLIT: 1 9.335E-01
 DEPENDENCIES: 31 35
 REQ. BRANCHES: 1 * 2
 DESCRIPTION: RC-Wet Pour
 CASE/BRANCH SPLIT: 4.667E-01 4.667E-01

CASE NUMBER/SPLIT: 2 6.655E-02
 DESCRIPTION: Otherwise -- No EVSE \$ Case 2: Alpha Mode, or Rocket, o
 CASE/BRANCH SPLIT: 0.000E+00 6.655E-02

***** QUESTION: 41 Containment Failure Pressure?
 Q-TYPE/TIMES ASKED: INDEP. INPUT PROB. INPUT PARM. 145236
 BRANCHES: CF-Pr
 1
 REALIZED SPLIT: 1.000E+00

***** QUESTION: 42 Containment Failure, and Type of Containment Failure?
 Q-TYPE/TIMES ASKED: DEP. CALC. PROB. 145236
 BRANCHES: ICF-CtRp ICF-Rupt ICF-Leak no-ICF ICF-Shear
 1 2 3 4 5
 REALIZED SPLIT: 0.000E+00 8.147E-03 1.436E-03 9.904E-01 4.691E-05

SUMMARY BY CASE

CASE NUMBER/SPLIT: 1 8.147E-03
 DEPENDENCIES: 34 36
 REQ. BRANCHES: 1 + 1
 DESCRIPTION: Alpha Rocket
 CASE/BRANCH SPLIT: 0.000E+00 8.147E-03 0.000E+00 0.000E+00 0.000E+00

CASE NUMBER/SPLIT: 2 9.919E-01
 DESCRIPTION: Otherwise \$ Case 2:
 CASE/BRANCH SPLIT: 0.000E+00 0.000E+00 1.436E-03 9.904E-01 4.691E-05

***** QUESTION: 43 Sprays after Vessel Breach? \$ (The 5 to 30 minute
 Q-TYPE/TIMES ASKED: DEP. INPUT PROB. 145236
 BRANCHES: I2-Sp I2aSp I2fSp
 1 2 3
 REALIZED SPLIT: 9.919E-01 0.000E+00 8.147E-03

SUMMARY BY CASE

CASE NUMBER/SPLIT: 1 8.147E-03
 DEPENDENCIES: 24 34 36
 REQ. BRANCHES: 3 + 1 + 1
 DESCRIPTION: EfSp Alpha Rocket
 CASE/BRANCH SPLIT: 0.000E+00 0.000E+00 8.147E-03

CASE NUMBER/SPLIT: 3 9.919E-01
 DEPENDENCIES: 24 42
 REQ. BRANCHES: 1 * /1
 DESCRIPTION: E-Sp /ICF-CtRp
 CASE/BRANCH SPLIT: 9.919E-01 0.000E+00 0.000E+00

***** QUESTION: 44 Is AC Power Available Late (during CCI)?
 Q-TYPE/TIMES ASKED: DEP. INPUT PROB. 145236
 BRANCHES: L-ACP LaACP LfACP
 1 2 3
 REALIZED SPLIT: 1.000E+00 0.000E+00 0.000E+00

SUMMARY BY CASE

CASE NUMBER/SPLIT: 1 1.000E+00
 DEPENDENCIES: 19
 REQ. BRANCHES: 1
 DESCRIPTION: E-ACP
 CASE/BRANCH SPLIT: 1.000E+00 0.000E+00 0.000E+00

***** QUESTION: 45 Late Sprays? (during CCI)
 Q-TYPE/TIMES ASKED: DEP. INPUT PROB. 145236
 BRANCHES: L-Sp LaSp LfSp
 1 2 3
 REALIZED SPLIT: 9.919E-01 0.000E+00 8.147E-03

SUMMARY BY CASE

CASE NUMBER/SPLIT: 1 9.919E-01
 DEPENDENCIES: 43
 REQ. BRANCHES: 1
 DESCRIPTION: 12-Sp
 CASE/BRANCH SPLIT: 9.919E-01 0.000E+00 0.000E+00

CASE NUMBER/SPLIT: 2 8.147E-03
 DEPENDENCIES: 43
 REQ. BRANCHES: 3
 DESCRIPTION: 12fSp
 CASE/BRANCH SPLIT: 0.000E+00 0.000E+00 8.147E-03

***** QUESTION: 46 Late Fan Coolers?
 Q-TYPE/TIMES ASKED: DEP. INPUT PROB. 145236
 BRANCHES: L-FC LaFC LFFC
 1 2 3
 REALIZED SPLIT: 5.590E-01 0.000E+00 4.410E-01

SUMMARY BY CASE

CASE NUMBER/SPLIT: 1 5.590E-01
 DEPENDENCIES: 25
 REQ. BRANCHES: 1
 DESCRIPTION: E-FC
 CASE/BRANCH SPLIT: 5.590E-01 0.000E+00 0.000E+00

CASE NUMBER/SPLIT: 2 4.410E-01
 DEPENDENCIES: 25

REQ. BRANCHES: 3
 DESCRIPTION: EFC
 CASE/BRANCH SPLIT: 0.000E+00 0.000E+00 4.410E-01

***** QUESTION: 47 Late Containment Heat Removal? 145236
 Q-TYPE/TIMES ASKED: DEP. INPUT PROB.
 BRANCHES: L-CHR LFCR
 1 2
 REALIZED SPLIT: 5.590E-01 4.410E-01

SUMMARY BY CASE

CASE NUMBER/SPLIT: 1 5.590E-01
 DEPENDENCIES: 46
 REQ. BRANCHES: 1
 DESCRIPTION: L-FC
 CASE/BRANCH SPLIT: 5.590E-01 0.000E+00

CASE NUMBER/SPLIT: 2 4.410E-01
 DESCRIPTION: Otherwise \$ Case 2: No Sprays, No Fan Cooler
 CASE/BRANCH SPLIT: 0.000E+00 4.410E-01

***** QUESTION: 48 Amount of Core Available for CCI? 184152
 Q-TYPE/TIMES ASKED: DEP. INPUT PROB.
 BRANCHES: Lrg-CCI Med-CCI Sm1-CCI
 1 2 3
 REALIZED SPLIT: 7.250E-01 2.749E-01 2.227E-04

SUMMARY BY CASE

CASE NUMBER/SPLIT: 1 8.147E-03
 DEPENDENCIES: 34 36
 REQ. BRANCHES: 1 + 1
 DESCRIPTION: Alpha Rocket
 CASE/BRANCH SPLIT: 0.000E+00 8.147E-03 0.000E+00

CASE NUMBER/SPLIT: 2 2.227E-04
 DEPENDENCIES: 35
 REQ. BRANCHES: 4
 DESCRIPTION: noVBoA
 CASE/BRANCH SPLIT: 0.000E+00 0.000E+00 2.227E-04

CASE NUMBER/SPLIT: 3 3.334E-02
 DEPENDENCIES: 35 35 22 33
 REQ. BRANCHES: (1 + (3 * /4)) * /3
 DESCRIPTION: PrEj BtmHd /I-LoPr /Lo-FCoR
 CASE/BRANCH SPLIT: 0.000E+00 3.334E-02 0.000E+00

CASE NUMBER/SPLIT: 4 1.925E-02
 DEPENDENCIES: 35 35 22 33
 REQ. BRANCHES: (1 + (3 * /4)) * 3
 DESCRIPTION: PrEj BtmHd /I-LoPr Lo-FCoR
 CASE/BRANCH SPLIT: 1.925E-02 0.000E+00 0.000E+00

CASE NUMBER/SPLIT: 5 4.668E-01
 DEPENDENCIES: 35 35 22 40
 REQ. BRANCHES: (2 + (3 * 4)) * 1
 DESCRIPTION: Pour BtmHd I-LoPr EVSE
 CASE/BRANCH SPLIT: 2.334E-01 2.334E-01 0.000E+00

CASE NUMBER/SPLIT: 6 4.724E-01
 DESCRIPTION: Otherwise \$ Case 6:
 CASE/BRANCH SPLIT: 4.724E-01 0.000E+00 0.000E+00

***** QUESTION: 49 Is the Debris Bed in a Coolable Configuration?
 Q-TYPE/TIMES ASKED: DEP. INPUT PROB. 350664
 BRANCHES: CDB noCDB
 1 2
 REALIZED SPLIT: 3.129E-01 6.873E-01

SUMMARY BY CASE

CASE NUMBER/SPLIT: 1 8.148E-03
 DEPENDENCIES: 34 36
 REQ. BRANCHES: 1 + 1
 DESCRIPTION: Alpha Rocket
 CASE/BRANCH SPLIT: 5.703E-03 2.444E-03

CASE NUMBER/SPLIT: 2 2.227E-04
 DEPENDENCIES: 35
 REQ. BRANCHES: 4
 DESCRIPTION: noVBoA
 CASE/BRANCH SPLIT: 2.227E-04 0.000E+00

CASE NUMBER/SPLIT: 3 5.259E-02
 DEPENDENCIES: 35 22
 REQ. BRANCHES: 1 +(3 * /4)
 DESCRIPTION: PrEj BtmHd /I-LoPr
 CASE/BRANCH SPLIT: 2.030E-02 2.630E-02

CASE NUMBER/SPLIT: 4 4.668E-01
 DEPENDENCIES: 40
 REQ. BRANCHES: 1
 DESCRIPTION: EVSE
 CASE/BRANCH SPLIT: 2.334E-01 2.334E-01

CASE NUMBER/SPLIT: 5 4.724E-01
 DESCRIPTION: Otherwise
 CASE/BRANCH SPLIT: 4.724E-02 4.252E-01

\$ Case 5: Gravity Pour with no EVS

***** QUESTION: 50 Does Prompt CCI Occur?
 Q-TYPE/TIMES ASKED: DEP. INPUT PROB. 350664
 BRANCHES: PrmptCCI noPrmCCI
 1 2
 REALIZED SPLIT: 6.879E-01 3.122E-01

SUMMARY BY CASE

CASE NUMBER/SPLIT: 1 3.122E-01
 DEPENDENCIES: 49 31 28 23
 REQ. BRANCHES: (1 *(1 + 3)) + 1
 DESCRIPTION: CDB RC-Wet AcDaVB noVB
 CASE/BRANCH SPLIT: 0.000E+00 3.122E-01

CASE NUMBER/SPLIT: 2 6.879E-01
 DESCRIPTION: Otherwise -- Not coolable or no water
 CASE/BRANCH SPLIT: 6.879E-01 0.000E+00

\$ Case 2: No water in

***** QUESTION: 51 Is AC Power Available Very Late (after CCI)?
 Q-TYPE/TIMES ASKED: DEP. INPUT PROB. 350664
 BRANCHES: L2-ACP L2aACP L2FACP
 1 2 3
 REALIZED SPLIT: 1.000E+00 0.000E+00 0.000E+00

SUMMARY BY CASE

CASE NUMBER/SPLIT: 1 1.000E+00
DEPENDENCIES: 44
REQ. BRANCHES: 1
DESCRIPTION: L-ACP
CASE/BRANCH SPLIT: 1.000E+00 0.000E+00 0.000E+00

***** QUESTION: 52 Very Late Sprays?
Q-TYPE/TIMES ASKED: DEP. INPUT PROB. 350664
BRANCHES: L2-Sp L2aSp L2fSp
1 2 3
REALIZED SPLIT: 9.919E-01 0.000E+00 8.147E-03

SUMMARY BY CASE

CASE NUMBER/SPLIT: 1 9.919E-01
DEPENDENCIES: 45
REQ. BRANCHES: 1
DESCRIPTION: L-Sp
CASE/BRANCH SPLIT: 9.919E-01 0.000E+00 0.000E+00
CASE NUMBER/SPLIT: 2 8.147E-03
DEPENDENCIES: 45
REQ. BRANCHES: 3
DESCRIPTION: LfSp
CASE/BRANCH SPLIT: 0.000E+00 0.000E+00 8.147E-03

***** QUESTION: 53 Very Late Fan Coolers?
Q-TYPE/TIMES ASKED: DEP. INPUT PROB. 350664
BRANCHES: L2-FC L2aFC L2FFC
1 2 3
REALIZED SPLIT: 5.590E-01 0.000E+00 4.411E-01

SUMMARY BY CASE

CASE NUMBER/SPLIT: 1 5.590E-01
DEPENDENCIES: 46
REQ. BRANCHES: 1
DESCRIPTION: L-FC
CASE/BRANCH SPLIT: 5.590E-01 0.000E+00 0.000E+00
CASE NUMBER/SPLIT: 2 4.411E-01
DEPENDENCIES: 46
REQ. BRANCHES: 3
DESCRIPTION: LFFC
CASE/BRANCH SPLIT: 0.000E+00 0.000E+00 4.411E-01

***** QUESTION: 54 Very Late Containment Heat Removal?
Q-TYPE/TIMES ASKED: DEP. INPUT PROB. 350664
BRANCHES: L2-CHR L2fCHR
1 2
REALIZED SPLIT: 5.590E-01 4.411E-01

SUMMARY BY CASE

CASE NUMBER/SPLIT: 1 5.590E-01
DEPENDENCIES: 53
REQ. BRANCHES: 1
DESCRIPTION: L2-FC
CASE/BRANCH SPLIT: 5.590E-01 0.000E+00
CASE NUMBER/SPLIT: 2 4.411E-01

DESCRIPTION: Otherwise
CASE/BRANCH SPLIT: 0.000E+00 4.411E-01

\$ Case 2: No Sprays, No Fan Cooler

***** QUESTION: 55 Does Delayed CCI Occur?
Q-TYPE/TIMES ASKED: DEP. INPUT PROB. 350664
BRANCHES: DelydCCI noDldCCI
1 2
REALIZED SPLIT: 5.663E-03 9.943E-01

SUMMARY BY CASE

CASE NUMBER/SPLIT: 1 9.943E-01
DEPENDENCIES: 50 52 50 23
REQ. BRANCHES: (2 * 1) + 1 + 1
DESCRIPTION: noPrmCCI L2-Sp PrmptCCI noVB
CASE/BRANCH SPLIT: 0.000E+00 9.943E-01

CASE NUMBER/SPLIT: 2 5.663E-03
DESCRIPTION: Otherwise
CASE/BRANCH SPLIT: 5.663E-03 0.000E+00

\$ Case 2: Water boiled

***** QUESTION: 56 Base line Containment Pressure Very Late?
Q-TYPE/TIMES ASKED: DEP. INPUT PROB. INPUT PARM. 350664
BRANCHES: L2PBase
1
REALIZED SPLIT: 1.000E+00

SUMMARY BY CASE

CASE NUMBER/SPLIT: 1 8.417E-03
DEPENDENCIES: 42 42 42 23
REQ. BRANCHES: 1 + 2 + 5 + 1
DESCRIPTION: ICF-CtRp ICF-Rupt ICF-Shear noVB
CASE/BRANCH SPLIT: 8.417E-03

CASE NUMBER/SPLIT: 2 5.571E-01
DEPENDENCIES: 54 11 42
REQ. BRANCHES: 1 + 1 + 3
DESCRIPTION: L2-CHR B-Leak ICF-Leak
CASE/BRANCH SPLIT: 5.571E-01

CASE NUMBER/SPLIT: 3 1.924E-05
DEPENDENCIES: 50 31 28
REQ. BRANCHES: 1 * 2 * /3
DESCRIPTION: PrmptCCI RC-Dry /AcDaVB
CASE/BRANCH SPLIT: 1.924E-05

CASE NUMBER/SPLIT: 4 2.995E-01
DEPENDENCIES: 50 31 28
REQ. BRANCHES: 1 *(1 + 3)
DESCRIPTION: PrmptCCI RC-Wet AcDaVB
CASE/BRANCH SPLIT: 2.995E-01

CASE NUMBER/SPLIT: 6 1.351E-01
DESCRIPTION: Otherwise -- DelydCCI & RC-Full \$ Case 6: Debris bed is Coolable a
CASE/BRANCH SPLIT: 1.351E-01

***** QUESTION: 57 How much H2 and CO2 is Produced during CCI?
Q-TYPE/TIMES ASKED: DEP. INPUT PROB. INPUT PARM. 350664
BRANCHES: CCI noCCI
1 2
REALIZED SPLIT: 6.935E-01 3.066E-01

SUMMARY BY CASE

CASE NUMBER/SPLIT: 1 3.066E-01
 DEPENDENCIES: 50 55
 REQ. BRANCHES: 2 * 2
 DESCRIPTION: noPrmCCI noDldCCI
 CASE/BRANCH SPLIT: 0.000E+00 3.066E-01

CASE NUMBER/SPLIT: 2 5.520E-01
 DEPENDENCIES: 48
 REQ. BRANCHES: 1
 DESCRIPTION: Lrg-CCI
 CASE/BRANCH SPLIT: 5.520E-01 0.000E+00

CASE NUMBER/SPLIT: 3 1.415E-01
 DEPENDENCIES: 48
 REQ. BRANCHES: 2
 DESCRIPTION: Med-CCI
 CASE/BRANCH SPLIT: 1.415E-01 0.000E+00

***** QUESTION: 58 How much Hydrogen Burns or Leaks Out of Containment?
 Q-TYPE/TIMES ASKED: DEP. INPUT PROB. INPUT PARM. 350664
 BRANCHES: FrH2-Rem
 1
 REALIZED SPLIT: 1.000E+00

SUMMARY BY CASE

CASE NUMBER/SPLIT: 1 9.631E-03
 DEPENDENCIES: 1 42
 REQ. BRANCHES: 4 + /4
 DESCRIPTION: Brk-V /no-ICF
 CASE/BRANCH SPLIT: 9.631E-03

CASE NUMBER/SPLIT: 2 5.112E-02
 DEPENDENCIES: 35 35 22 24
 REQ. BRANCHES: (1 +(3 * /4)) * 1
 DESCRIPTION: PrEj BtmHd /I-LoPr E-Sp
 CASE/BRANCH SPLIT: 5.112E-02

CASE NUMBER/SPLIT: 5 9.393E-01
 DESCRIPTION: Otherwise \$ Case 5: Intact Containment and n
 CASE/BRANCH SPLIT: 9.393E-01

***** QUESTION: 59 Add H2 produced by CCI to H2 already in Containment
 Q-TYPE/TIMES ASKED: INDEP. CALC. PROB. 350664
 BRANCHES: L2-H2 L2nH2
 1 2
 REALIZED SPLIT: 6.935E-01 3.066E-01

***** QUESTION: 60 Amount of Steam in Containment after CCI?
 Q-TYPE/TIMES ASKED: DEP. INPUT PROB. INPUT PARM. 350664
 BRANCHES: L2StmCnt
 1
 REALIZED SPLIT: 1.000E+00

SUMMARY BY CASE

CASE NUMBER/SPLIT: 1 5.590E-01
 DEPENDENCIES: 54
 REQ. BRANCHES: 1

DESCRIPTION: L2-CHR
CASE/BRANCH SPLIT: 5.590E-01

CASE NUMBER/SPLIT: 2 4.411E-01
DESCRIPTION: otherwise
CASE/BRANCH SPLIT: 4.411E-01

\$ Case 2: Sprays Not Operating

***** QUESTION: 62 Is the H2 Concentration Flammable? 350664
Q-TYPE/TIMES ASKED: DEP. CALC. PROB.
BRANCHES: L2-H2F noL2-H2F
1 2
REALIZED SPLIT: 0.000E+00 1.000E+00

SUMMARY BY CASE

CASE NUMBER/SPLIT: 1 1.000E+00
DEPENDENCIES: 11 42 19 24
REQ. BRANCHES: 1 + /4 + (1 * 1)
DESCRIPTION: B-Leak /no-ICF E-ACP E-Sp
CASE/BRANCH SPLIT: 0.000E+00 1.000E+00

***** QUESTION: 62 Does Ignition Occur? Conversion Ratio?
Q-TYPE/TIMES ASKED: DEP. INPUT PROB. INPUT PARM. 350664
BRANCHES: L2-HB noL2-HB
1 2
REALIZED SPLIT: 0.000E+00 1.000E+00

SUMMARY BY CASE

CASE NUMBER/SPLIT: 3 1.000E+00
DESCRIPTION: otherwise
CASE/BRANCH SPLIT: 0.000E+00 1.000E+00

\$ Case 3: Concentration not flamma

***** QUESTION: 63 Resulting Pressure Rise? 350664
Q-TYPE/TIMES ASKED: DEP. CALC. PROB.
BRANCHES: L2-H2Brn L2nH2Brn
1 2
REALIZED SPLIT: 0.000E+00 1.000E+00

SUMMARY BY CASE

CASE NUMBER/SPLIT: 1 1.000E+00
DEPENDENCIES: 62
REQ. BRANCHES: 2
DESCRIPTION: noL2-HB
CASE/BRANCH SPLIT: 0.000E+00 1.000E+00

***** QUESTION: 64 Containment Failure, and Type of Containment Failure? 350664
Q-TYPE/TIMES ASKED: DEP. CALC. PROB.
BRANCHES: L2CF-CRp L2CF-Rp L2CF-Lk L2CF-SHEA no-L2CF
1 2 3 4 5
REALIZED SPLIT: 0.000E+00 0.000E+00 8.775E-04 0.000E+00 9.991E-01

SUMMARY BY CASE

CASE NUMBER/SPLIT: 1 9.631E-03
DEPENDENCIES: 42
REQ. BRANCHES: /4
DESCRIPTION: /no-ICF
CASE/BRANCH SPLIT: 0.000E+00 0.000E+00 0.000E+00 0.000E+00 9.631E-03

CASE NUMBER/SPLIT: 2 9.904E-01
 DESCRIPTION: Otherwise \$ Case 2:
 CASE/BRANCH SPLIT: 0.000E+00 0.000E+00 8.775E-04 0.000E+00 9.895E-01

***** QUESTION: 65 Sprays after Very Late CF?
 Q-TYPE/TIMES ASKED: DEP. INPUT PROB. 350664
 BRANCHES: F-Sp noF-Sp
 1 2
 REALIZED SPLIT: 9.919E-01 8.147E-03

SUMMARY BY CASE

CASE NUMBER/SPLIT: 1 8.147E-03
 DEPENDENCIES: 52 51
 REQ. BRANCHES: 3 + 3
 DESCRIPTION: L2fSp L2fACP
 CASE/BRANCH SPLIT: 0.000E+00 8.147E-03

CASE NUMBER/SPLIT: 3 9.919E-01
 DESCRIPTION: Otherwise \$ Case 3: No catastrophic rupture
 CASE/BRANCH SPLIT: 9.919E-01 0.000E+00

***** QUESTION: 66 Fan Coolers after Very Late CF?
 Q-TYPE/TIMES ASKED: DEP. INPUT PROB. 350664
 BRANCHES: F-FC FfCC
 1 2
 REALIZED SPLIT: 5.590E-01 4.411E-01

SUMMARY BY CASE

CASE NUMBER/SPLIT: 1 4.411E-01
 DEPENDENCIES: 53 51
 REQ. BRANCHES: 3 + 3
 DESCRIPTION: L2fFC L2fACP
 CASE/BRANCH SPLIT: 0.000E+00 4.411E-01

CASE NUMBER/SPLIT: 2 5.590E-01
 DESCRIPTION: Otherwise \$ were available and we as
 CASE/BRANCH SPLIT: 5.590E-01 0.000E+00

***** QUESTION: 67 Containment Heat Removal after Very Late CF?
 Q-TYPE/TIMES ASKED: DEP. INPUT PROB. 350664
 BRANCHES: F-CHR FfCHR
 1 2
 REALIZED SPLIT: 9.965E-01 3.551E-03

SUMMARY BY CASE

CASE NUMBER/SPLIT: 1 9.965E-01
 DEPENDENCIES: 65 66
 REQ. BRANCHES: 1 + 1
 DESCRIPTION: F-Sp F-FC
 CASE/BRANCH SPLIT: 9.965E-01 0.000E+00

CASE NUMBER/SPLIT: 2 3.551E-03
 DESCRIPTION: Otherwise \$ Case 2: No Sprays, No Fan Cooler
 CASE/BRANCH SPLIT: 0.000E+00 3.551E-03

***** QUESTION: 68 Eventual Basemat Melt-through?
 Q-TYPE/TIMES ASKED: DEP. INPUT PROB. 511296
 BRANCHES: MTnDePr MTwDePr noMT

REALIZED SPLIT: 1 2 3
 1.846E-01 6.153E-02 7.541E-01

SUMMARY BY CASE

CASE NUMBER/SPLIT: 1 1.568E-02
 DEPENDENCIES: 11 23 42 64
 REQ. BRANCHES: 1 + 1 + /4 + /5
 DESCRIPTION: B-Leak noVB /no-ICF /no-L2CF
 CASE/BRANCH SPLIT: 0.000E+00 0.000E+00 1.568E-02

CASE NUMBER/SPLIT: 2 3.032E-01
 DEPENDENCIES: 50 55 65
 REQ. BRANCHES: 2 * 2 * 1
 DESCRIPTION: noPrmCCI noDldCCI F-Sp
 CASE/BRANCH SPLIT: 0.000E+00 0.000E+00 3.032E-01

CASE NUMBER/SPLIT: 3 5.493E-01
 DEPENDENCIES: 48 65
 REQ. BRANCHES: 1 * 1
 DESCRIPTION: Lrg-CCI F-Sp
 CASE/BRANCH SPLIT: 1.648E-01 5.493E-02 3.296E-01

CASE NUMBER/SPLIT: 5 1.320E-01
 DEPENDENCIES: 48 65
 REQ. BRANCHES: 2 * 1
 DESCRIPTION: Med-CCI F-Sp
 CASE/BRANCH SPLIT: 1.980E-02 6.599E-03 1.056E-01

***** QUESTION: 69 Eventual Overpressure Failure of Containment?
 Q-TYPE/TIMES ASKED: DEP. INPUT PROB. 511296
 BRANCHES: F-CF-OP noFCFOP
 1 2
 REALIZED SPLIT: 0.000E+00 1.000E+00

SUMMARY BY CASE

CASE NUMBER/SPLIT: 1 1.000E+00
 DEPENDENCIES: 11 23 42 64 67
 REQ. BRANCHES: 1 + 1 + /4 + /5 + 1
 DESCRIPTION: B-Leak noVB /no-ICF /no-L2CF F-CHR
 CASE/BRANCH SPLIT: 0.000E+00 1.000E+00

***** QUESTION: 70 Basemat Melt-through before Overpressure Failure?
 Q-TYPE/TIMES ASKED: DEP. INPUT PROB. 511296
 BRANCHES: F-BMT FCF-Lk Neither
 1 2 3
 REALIZED SPLIT: 2.461E-01 0.000E+00 7.540E-01

SUMMARY BY CASE

CASE NUMBER/SPLIT: 1 2.461E-01
 DEPENDENCIES: 68 69
 REQ. BRANCHES: /3 * 2
 DESCRIPTION: /noMT noFCFOP
 CASE/BRANCH SPLIT: 2.461E-01 0.000E+00 0.000E+00

CASE NUMBER/SPLIT: 5 7.540E-01
 DESCRIPTION: Otherwise
 CASE/BRANCH SPLIT: 0.000E+00 0.000E+00 7.540E-01 \$ Case 5: Have neither BMT nor

***** QUESTION: 71 Final Containment Condition?

Q-TYPE/TIMES ASKED: DEP. INPUT PRGB. 511296
 BRANCHES: F-Rubtr F-Leak F-MT Bypass noCF Shear
 1 2 3 4 5 6
 REALIZED SPLIT: 8.148E-03 7.267E-03 2.461E-01 0.000E+00 7.386E-01 4.668E-05

SUMMARY BY CASE

CASE NUMBER/SPLIT: 1 8.148E-03
 DEPENDENCIES: 42 42 64 64
 REQ. BRANCHES: 1 + 2 + 1 + 2
 DESCRIPTION: ICF-CtRp ICF-Rupt L2CF-CRp L2CF-Rp
 CASE/BRANCH SPLIT: 8.148E-03 0.000E+00 0.000E+00 0.000E+00 0.000E+00 0.000E+00

CASE NUMBER/SPLIT: 2 7.267E-03
 DEPENDENCIES: 11 42 64 70
 REQ. BRANCHES: 1 + 3 + 3 + 2
 DESCRIPTION: B-Leak ICF-Leak L2CF-Lk FCF-Lk
 CASE/BRANCH SPLIT: 0.000E+00 7.267E-03 0.000E+00 0.000E+00 0.000E+00 0.000E+00

CASE NUMBER/SPLIT: 3 4.668E-05
 DEPENDENCIES: 42 64
 REQ. BRANCHES: 5 + 4
 DESCRIPTION: ICF-Shear L2CF-SHEA
 CASE/BRANCH SPLIT: 0.000E+00 0.000E+00 0.000E+00 0.000E+00 0.000E+00 4.668E-05

CASE NUMBER/SPLIT: 5 2.461E-01
 DEPENDENCIES: 70
 REQ. BRANCHES: 1
 DESCRIPTION: F-BMT
 CASE/BRANCH SPLIT: 0.000E+00 0.000E+00 2.461E-01 0.000E+00 0.000E+00 0.000E+00

CASE NUMBER/SPLIT: 6 7.386E-01
 DESCRIPTION: Otherwise \$ Case 6: No C
 CASE/BRANCH SPLIT: 0.000E+00 0.000E+00 0.000E+00 0.000E+00 7.386E-01 0.000E+00

***** QUESTION: 72 Time of core damage
 Q-TYPE/TIMES ASKED: DEP. INPUT PRGB. 511296
 BRANCHES: ECorD LCorD
 1 2
 REALIZED SPLIT: 0.000E+00 1.000E+00

SUMMARY BY CASE

CASE NUMBER/SPLIT: 1 1.000E+00
 DEPENDENCIES: 3 8 3 19
 REQ. BRANCHES: 5 + 1 + (2 * 1)
 DESCRIPTION: BIECCS SG-HR BaECCS E-ACP
 CASE/BRANCH SPLIT: 0.000E+00 1.000E+00

APPENDIX C
ZION ACCIDENT PROGRESSION BINNING
FILE FROM DRAFT NUREG/CR-4551

APPENDIX C

Zion Accident Progression Binning
File From Draft NUREG/CR-4551

This Appendix lists the EVNTRE input file used to bin the end states of the Zion APET into accident progression bins for the ZISOR source term analysis in draft NUREG/CR-4551 for Zion.

Zion Binning - Rev. 5 - 6/2/89 - 12 Characteristics

12 CF-Time	Sprays	CCI	RCS-Pres	VB-Mode	SGTR	Amt-CCI					
Zr-Ox	HPME	CF-Size	RCS-Hole	CD-Time							
7	7	V-Dry	V-Wet	Early-CF	CF-at-VB	Late-CF	VLate-CF	No-CF			\$ Char. 1, Containment Failure Time
4	7	71	1	18	71						\$ Case 1, Attr. 7 (G), No CF or No V
		5	+	((5 + 1) *	4)						
		noCF	or((B-SGTR	or E-SGTRS3)	& Bypass)						\$ Case 2, Attr. 1 (A), V-Dry
2	1	1	12								
		4	*	2							
		Brk-V	&	V-Dry							\$ Case 3, Attr. 2 (B), V-Wet
2	2	1	12								
		4	*	1							
		Brk-V	&	V-Wet							\$ Case 4, Attr. 3 (C), CF before Vessel Breach
4	3	11	42	42	42						\$ Case 4, Attr. 3 (C), CF before Vessel Breach
		1	*	-1	* -5	* -2					
		B-Leak	&	noICF-CR	&	noICF-Sh	&	noICF-Rp			\$ Case 5, Attr. 4 (D), CF at Vessel Breach
1	4	42									
		-4									
		I-CF									\$ Case 6, Attr. 5 (E), Very Late CF (after CCI)
1	5	64									
		-5									
		L2-CF									\$ Case 7, Attr. 6 (F), Final CF (about 24 hours after VB)
2	6	70	71								
		2	+	3							
		FCF-Lk	or	F-MT							
8	8	Sp-Early	Sp-E+I	Sp-E+I+L	SpAlways	Sp-Late	Sp-L+VL				\$ Characteristic 2, Sprays
		Sp-VL	Sp-Never	Sp-Final							
4	1	24	43	45	52						\$ Case 1, Attr. 1 (A), Early sprays only
		1	*	-1	*	-1					
		E-Sp	&	noI2-Sp	&	noL-Sp	&	noL2-Sp			\$ Case 2, Attr. 2 (B), Early & Im sprays only
4	2	24	43	45	52						
		1	*	1	*	-1	*	-1			
		E-Sp	&	I2-Sp	&	noL-Sp	&	noL2-Sp			\$ Case 3, Attr. 3 (C), Early, Im & Late sprays
4	3	24	43	45	52						
		1	*	1	*	1	*	-1			
		E-Sp	&	I2-Sp	&	L-Sp	&	noL2-Sp			\$ Case 4, Attr. 4 (D), Sprays always
4	4	24	43	45	52						\$ (Always w/r/t releases)
		1	*	1	*	1	*	1			
		E-Sp	&	I2-Sp	&	L-Sp	&	L2-Sp			\$ Case 5, Attr. 5 (E), Late sprays only
4	5	24	43	45	52						
		-1	*	-1	*	1	*	-1			
		E-CHR	&	noI2-Sp	&	L-Sp	&	noL2-Sp			\$ Case 6, Attr. 6 (F), Late & VL sprays only
4	6	24	43	45	52						
		-1	*	-1	*	1	*	1			
		E-CHR	&	noI2-Sp	&	L-Sp	&	L2-Sp			\$ Case 7, Attr. 7 (G), Very Late sprays only
4	7	24	43	45	52						
		-1	*	-1	*	-1	*	1			
		E-CHR	&	noI2-Sp	&	noL-Sp	&	L2-Sp			

5	8	24	43	45	52	65	\$ Case 8, Attr. 8 (H), Sprays never (Never w/r/t releases)
		(-1 * E-CHR	-1 * & noI2-Sp	-1 * & noI-Sp	-1 * & noL2-Sp	+1 \$ or F-Sp	
6	6	Prompt-Dry	PromptShlw	No-CCI	PromptDeep	SDlyd-Dry	LDlyd-Dry
3	1	50	31	28			\$ Characteristic 3, \$ Core-Concrete Interaction
		1 * PrmptCCI	2 * & RC-Dry	-3 \$ & noAcDaVB			\$ Case 1, Attr. 1 (A), Prompt CCI - Cavity Dry
3	2	50	31	28			\$ Case 2, Attr. 2 (B), Prompt CCI - Shallow Pool Scrubbing
		1 * PrmptCCI	2 * & RC-Dry	3 \$ & AcDaVB			\$ Cavity contains accumulator water only
2	3	50	55				\$ Case 3, Attr. 3 (C), No CCI
		2 * noPrmCCI	2 * & noDldCCI				\$ Coolable with water, or no VB.
2	4	50	31				\$ Case 4, Attr. 4 (D), Prompt CCI - Deep Pool Scrubbing
		1 * PrmptCCI	1 * & RC-Wet				\$ Cavity is full (14 feet)
2	5	55	26				\$ Case 5, Attr. 5 (E), Delayed CCI - Cavity Dry
		1 * DelydCCI	2 * & AcDaVB				\$ Short Delay - Boil off Accumulator water only
2	6	55	31				\$ Case 6, Attr. 6 (F), Delayed CCI - Cavity Dry
		1 * DelydCCI	1 * & RC-Wet				\$ Long Delay - Boil off Full (14 ft) Cavity
4	4	SSPr	HiPr	ImPr	LoPr		\$ Characteristic 4, RCS Pressure before VB
1	1	22					\$ Case 1, Attr. 1 (A), System setpoint pressure
		1 I-SSPr					
1	2	22					\$ Case 2, Attr. 2 (B), High pressure
		2 I-HiPr					
1	3	22					\$ Case 3, Attr. 3 (C), Intermediate pressure
		3 I-ImPr					
1	4	22					\$ Case 4, Attr. 4 (D), Low pressure
		4 I-LoPr					
6	6	VB-HPME	VB-Pour	VB-BtmHd	Alpha	Rocket	No-VB
1	1	35					\$ Characteristic 5, Mode of Vessel Breach
		1 PrEj					\$ Case 1, Attr. 1 (A), Pressurized Ejection (incl. Core Heating)
							\$ Characteristic 5 is Not Used in SRSOR.
							\$ All HPME information is obtained from Char. 9.
1	2	35					\$ Case 2, Attr. 2 (B), Gravity Pour
		2 Pour					
1	5	36					\$ Case 3, Attr. 5 (E), Rocket
		1 Rocket					\$ Has to come before BtmHd since BtmHd required for Rocket
1	3	35					\$ Case 4, Attr. 3 (C), Gross Bottom Head Failure
		3 BtmHd					
1	4	34					\$ Case 5, Attr. 4 (D), Alpha Mode
		1 Alpha					
1	6	23					\$ Case 6, Attr. 6 (F), No Vessel Breach
		1 noVB					
3	3	SGTR	SGTR-SRVD	No-SGTR			\$ Char. 6, Steam Generator Tube Rupture
3	1	1	2	18			\$ Case 1, Attr. 1 (A), SGTR
		(5 * (B-SGTR	2) + & SSRVnStD	1 \$ orE-SGTRS3			\$ Secondary system SRVs are not stuck open
2	2	1	2				\$ Case 2, Attr. 2 (B), SGTR with Stuck-Open SRVs
		5 * B-SGTR	1 * & SSRV-StD				
2	3	1	18				\$ Case 3, Attr. 3 (C), No SGTR
		-5 * noB-SGTR	2 * & noE-SGTR				
4	4	Lrg-CCI	Med-CCI	SmI-CCI	No-CCI		\$ Characteristic 7, Amount of Core in CCI

2	4	50	55							\$ Case 1, Attr. 4 (D), No CCI	
		2	*	2							
1	1	noPrmCCI	& noDldCCI	48	35	35	22			\$ Case 2, Attr. 1 (A), Large Amount of Core in CCI (70-100%)	
		1	+	1	+	(3 * -4)					
1	2	Lrg-CCI		48						\$ Case 3, Attr. 2 (B), Medium Amount of Core in CCI (30-70%)	
		2									
1	3	Med-CCI		48						\$ Case 4, Attr. 3 (C), Small Amount of Core in CCI (0-30%)	
		3									
2	2	Lo-ZrOx	Hi-ZrOx							\$ Characteristic 8, Zr Oxidation	
1	1	30								\$ Case 1, Attr. 1 (A), Lo Zr Oxidation (<40%) In-Vessel	
		2									
1	2	Lo-ZrOx								\$ Case 2, Attr. 2 (B), Hi Zr Oxidation (>40%) In-Vessel	
		30									
		1									
4	4	Hi-HPME	Md-HPME	Lo-HPME	No-HPME					\$ Char. 9, High Pressure Melt Ejection	
4	1	33		35	35		22			\$ Case 1, Attr. 1 (A), High Fraction Ejected (>40%)	
		1	*	(1 + (3 * -4))							
4	2	Hi-FCoR	& (PrEj or (BtmHd & noI-LoPr))	33	35	35	22			\$ Case 2, Attr. 2 (B), Medium Fraction Ejected (20-40%)	
		2	*	(1 + (3 * -4))							
4	3	Md-FCoR	& (PrEj or (BtmHd & noI-LoPr))	33	35	35	22			\$ Case 3, Attr. 3 (C), Low Fraction Ejected (<20%)	
		3	*	(1 + (3 * -4))							
1	4	LJ-FCoR	& (PrEj or (BtmHd & noI-LoPr))	35						\$ Case 4, Attr. 4 (D), No HPME	
		-1									
7	7	noPrEj								\$ 10th Char., Type of Cont. Failure	
1381		Cat-Rupt	Rupture	Leak	SHEAR	BMT	Bypass	No-CF			
1	6	71								\$ Case 1, Attr. 6 (F), Bypass (V or SGTR)	1382
		4									1383
		Bypass									1384
2	1	42		64						\$ Case 2, Attr. 1 (A), Catastrophic Rupture	1385
		1	+	1							1386
		ICF-CtRp	orL2CF-CRp								1387
2	2	42		64						\$ Case 3, Attr. 2 (B), Rupture	1388
		2	+	2							1389
		ICF-Rupt	orL2CF Rp								1390
1	3	71								\$ Case 4, Attr. 3 (C), Leak	1391
		2									1392
		F-Leak									1393
1	4	71								\$ Case 5, Attr. 4 (D), SHEAR	1394
		6									1395
		F-SHEAR									1396
1	5	71								\$ Case 6, Attr. 5 (E), BMT	1397
		3									1398
		F-MT									
1	7	71									
		5									
		NOCF									
2	2	1-Hole	2-Holes							\$ Char. 11, Number of Holes in the RCS	
4	1	21		21		34		36		\$ Case 1, Attr. 1 (A), One Hole	
		-1	*	-2	*	2	*	2		\$ Event V = 1 hole - path too long	
		noEBD-A	& noEBD-S2	& noAlpha	& noRocket						
4	2	21		21		34		36		\$ Case 2, Attr. 2 (B), Two Holes	
		1	+	2	+	1	+	1		\$ S3 Holes are too Small for Natural Circulation	
		EBD-A	or EBD-S2	or Alpha	or Rocket						
2	2	E-CD	L-CD								
1	1	72									
		1									

1 2 ECorD
72
2
LCorD

1
10 1 4 6 8 5 9 3 7 2 10
CF-Time CD-Time RCS-Pres SGTR Zr-Ox VB-Mode HPME CCI
Amt-CCI RCS-Hole Sprays CF-Size

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11. ABSTRACT (200 words or less)

This report calculates the risk benefit associated with potential performance improvements for the large dry pressurized water reactor (PWR) containment. The analysis is based on the June 1989 draft NUREG-1150 results for the Zion commercial nuclear reactor. Simplified containment event trees and the large accident progression event trees from draft NUREG-1150 are used to evaluate the effects of potential improvements on the response of the Zion containment to dominant severe accident sequences. Source terms are generated parametrically using the ZISOR code and offsite consequences are calculated with the MELCOR Accident Consequence Code System (MACCS). These results give point estimates of the risk reduction associated with each containment improvement identified by Brookhaven National Laboratory in their draft Issues Characterization Report.

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