



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

PHILADELPHIA ELECTRIC COMPANY
PUBLIC SERVICE ELECTRIC AND GAS COMPANY
DELMARVA POWER AND LIGHT COMPANY
ATLANTIC CITY ELECTRIC COMPANY

DOCKET NO. 50-277

PEACH BOTTOM ATOMIC POWER STATION, UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 155
License No. DPR-44

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Philadelphia Electric Company, et al. (the licensee) dated July 11, 1989 as supplemented on April 20, 1990, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I.
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health or safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C(2) of Facility Operating License No. DPR-44 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 155, are hereby incorporated in the license. PECO shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

/s/

Walter R. Butler, Director
Project Directorate 1-2
Division of Reactor Projects - I/II

Attachment:
Changes to the Technical
Specifications

Date of Issuance: August 20, 1990

PDI-2/LA
M. Butler
7/29/90

PDI-2/PM
GSuh
7/19/90

OGC
CNU
6/2/90

PDI-2/D
WButler
8/16/90

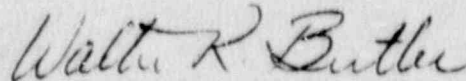
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FOR THE NUCLEAR REGULATORY COMMISSION



Walter R. Butler, Director
Project Directorate I-2
Division of Reactor Projects - I/II

Attachment:
Changes to the Technical
Specifications

Date of Issuance: August 20, 1990

ATTACHMENT TO LICENSE AMENDMENT NO. 155

FACILITY OPERATING LICENSE NO. DPR-44

DOCKET NO. 50-277

Replace the following pages of the Appendix A Technical Specifications with the enclosed pages. The revised areas are indicated by marginal lines.

<u>Remove</u>	<u>Insert</u>
iva	iva
vii	vii
243	243
244	244
245	245
245a	245a
246	246
252	252

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3.5.1.K	DELETED	142g
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3.6.2	Minimum Temperature for Mechanical Heatup of Cooldown following Nuclear Shutdown	164a
3.6.3	Minimum Temperature for Core Operation (Criticality)	164b
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3.6.5	Thermal Power and Core Flow Limits	164d
3.8.1	Site Boundary and Effluent Release Points	216e
6.2-1	Deleted	
6.2-2	Deleted	

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6.0 ADMINISTRATIVE CONTROLS6.1 Responsibility

- 6.1.1 The Plant Manager shall be responsible for overall facility operation. In the absence of the Plant Manager, the Superintendent - Operations (or any other person that the Plant Manager may designate in writing) shall assume the Manager's responsibility for overall facility operation.
- 6.1.2 The Shift Supervisor (or, during his absence from the control room, a designated individual) shall be responsible for the control room command function. A management directive to this effect, signed by the Vice President, Peach Bottom Atomic Power Station shall be reissued to all station personnel on an annual basis.

6.2 Organization6.2.1 Offsite and Onsite Organizations

Onsite and offsite organizations shall be established for unit operation and corporate management, respectively. The onsite and offsite organizations shall include the positions for activities affecting the safety of the nuclear power plant.

- a. Lines of authority, responsibility, and communication shall be established and defined for the highest management levels through intermediate levels to and including all operating organization positions. These relationships shall be documented and updated, as appropriate, in the form of organization charts, functional descriptions of departmental responsibilities and relationships, and job descriptions for key personnel positions, or in equivalent forms of documentation. These requirements shall be documented in the Peach Bottom Quality Assurance Plan.
- b. The Plant Manager shall be responsible for overall unit safe operation and shall have control over those onsite activities necessary for safe operation and maintenance of the plant.
- c. The Vice President, Peach Bottom Atomic Power Station shall have corporate responsibility for overall plant nuclear safety and shall take any measures needed to ensure acceptable performance of the staff in operating, maintaining, and providing technical support to the plant to ensure nuclear safety.
- d. The individuals who train the operating staff and those who carry out health physics and quality assurance functions may report to the appropriate onsite manager; however, they shall have sufficient organizational freedom to ensure their independence from operating pressures.

Continued

6.2.2 Facility Staff

The facility organization shall be subject to the following:

- a. Each on-duty shift shall be composed of at least the minimum shift crew composition shown in Table 6.2.1, except that the shift crew composition may be less than the minimum requirements for a period of time not to exceed 2 hours in order to accommodate unexpected absence of on-duty shift crew members provided immediate action is taken to restore the shift crew composition to within the minimum requirements.
- b. At least one licensed operator shall be in the control room and assigned to each reactor that contains fuel.
- c. At least two licensed operators, excluding the operator on the second unit, shall be present in the control room during reactor startup, scheduled reactor shutdown and during recovery from reactor trips.
- d. An individual qualified in radiation protection procedures shall be onsite when fuel is in the reactor.
- e. All CORE ALTERATIONS shall be directly supervised by either a licensed Senior Reactor Operator or Senior Reactor Operator Limited to Fuel Handling who has no other concurrent responsibility during this operation.
- f. A Fire Brigade of at least 5 members shall be maintained onsite at all times. The Fire Brigade shall not include the minimum shift crew necessary for safe shutdown of the unit(s) (3 members) or any personnel required for other essential functions during a fire emergency.
- g. In addition to the requirements of 6.2.2.a, the following shall hold a senior reactor operator license: 1) Plant Manager or Superintendent-Operations, 2) Assistant Superintendent-Operations and 3) Superintendent-Technical or Engineer-Systems.

Table 6.2.1

Shift Crew Composition - Minimum Requirements

One or Both Units Operating*:	1 Shift Manager (SRO)
	1 Shift Supervisor (SRO)
	3 Reactor Operators (RO)
	5 Operators
	1 Shift Technical Advisor
Both Units not Operating*	1 Shift Manager or Shift Supervisor (SRO)
	2 Reactor Operators (RO)
	5 Operators

SRO - NRC licensed senior reactor operator

RO - NRC licensed reactor operator

*For the purpose of this table, a unit is considered not to be operating when it is in a Cold Condition with the reactor mode switch in shutdown or refuel.

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6.3 Facility Staff Qualifications

- 6.3.1 Each member of the facility staff shall meet or exceed the minimum qualifications of ANSI N18.1 - 1971 for comparable positions, except for (1) Senior - Health Physicist (radiation protection manager) who shall meet or exceed the qualifications of Regulatory Guide 1.8, September 1975 and (2) the Shift Technical Advisor who shall have a bachelor's degree or equivalent in a scientific or engineering discipline with specific training in plant design, and response and analysis of the plant for transients and accidents.

6.4 Training

- 6.4.1 A retraining and replacement training program for the facility staff shall be maintained under the direction of the Superintendent-Training and shall meet the requirements of Section 5.5 of ANSI N18.1-1971 and 10 CFR 55, Appendix A.
- 6.4.2 A training program for the Fire Brigade shall be conducted such that Fire Brigade members complete an instruction program within a two year period. Regularly planned meetings will be held every 3 months.

6.5 Review and Audit

6.5.1 Plant Operations Review Committee (PORC)

6.5.1.1 Function

The Plant Operations Review Committee shall function to advise the Plant Manager on all matters related to nuclear safety.

6.5.1.2 Composition

The Plant Operations Review Committee shall be composed of the:

Superintendent - Operations (Chairman)
 Superintendent - Technical
 Superintendent - Maintenance/Instrumentation and Controls
 Superintendent - Plant Services
 Assistant Superintendent - Operations
 Maintenance Engineer
 Engineer-Systems
 Regulatory Engineer
 Shift Manager

6.5.1.3 Alternates

Alternate members shall be appointed in writing by the PORC Chairman to serve on a temporary basis; however, no more than two alternate members shall be used to satisfy a PORC quorum (See 6.5.1.5).

6.5.2.8 Continued

- e. The Facility Emergency Plan and implementing procedures at least once per year.
- f. The Facility Security Plan and implementing procedures at least once per two years.
- g. The Offsite Dose Calculation Manual and implementing procedures at least once per two years.
- h. The performance of activities required by the Quality Assurance Program regarding the radiological monitoring program to meet the provisions of Regulatory Guide 4.1, Revision 1, April 1975, at least once per calendar year.
- i. Any other area of facility operation considered appropriate by the NRB or the Executive Vice President - Nuclear.

Authority

- 6.5.2.9 The NRB shall report to and advise the Executive Vice President - Nuclear on those areas of responsibility specified in Section 6.5.2.7 and 6.5.2.8.

Records

- 6.5.2.10 Records of NRB activities shall be prepared, approved and distributed as indicated below:
- a. Minutes of each NRB meeting shall be prepared, approved and forwarded to the Executive Vice President - Nuclear within 10 working days following each meeting.
 - b. Reports of reviews encompassed by Section 6.5.2.7.e, f, g and h above shall be prepared, approved and forwarded to the Executive Vice President - Nuclear, within 10 working days following completion of the review.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
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PHILADELPHIA ELECTRIC COMPANY
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ATLANTIC CITY ELECTRIC COMPANY

DOCKET NO. 50-278

PEACH BOTTOM ATOMIC POWER STATION, UNIT NO. 3

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 157
License No. DPR-56

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Philadelphia Electric Company, et al. (the licensee) dated July 11, 1989 as supplemented on April 20, 1990, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I.
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health or safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C(2) of Facility Operating License No. DPR-56 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 157, are hereby incorporated in the license. PECO shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

/s/

Walter R. Butler, Director
Project Directorate I-2
Division of Reactor Projects - I/II

Attachment:
Changes to the Technical
Specifications

Date of Issuance: August 20, 1990

PDI-2/LA
NO/Exten
7/19/90

PDI-2/PMO
GSuh
7/19/90

OGC
CPW
8/2/90

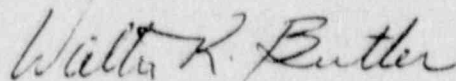
PDI-2/D
WButler
8/16/90 WB

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 157, are hereby incorporated in the license. PECO shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Walter R. Butler, Director
Project Directorate I-2
Division of Reactor Projects - I/II

Attachment:
Changes to the Technical
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Date of Issuance: August 20, 1990

ATTACHMENT TO LICENSE AMENDMENT NO. 157

FACILITY OPERATING LICENSE NO. DPR-56

DOCKET NO. 50-278

Replace the following pages of the Appendix A Technical Specifications with the enclosed pages. The revised areas are indicated by marginal lines.

<u>Remove</u>	<u>Insert</u>
iv	iv
iva	--
vii	vii
243	243
244	244
245	245
245a	245a
246	246
252	252

PBAPS
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3.8.1	Site Boundary and Effluent Release Points	216e
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Continued

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6.5.2.8 Continued

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- g. The Offsite Dose Calculation Manual and implementing procedures at least once per two years.
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- i. Any other area of facility operation considered appropriate by the NRB or the Executive Vice President - Nuclear.

Authority

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