CONNECTICUT YANKEE ATOMIC POWER COMPANY

HADDAM NECK PLANT

RR#1 • BOX 127E • EAST HAMPTON, CT 06424-9341

August 15, 1990
Re: Technical Specification 6.9.1d
Docket No. 50-213

U. S. Nuclear Regulatory Commission
Document Control Desk
Washington, D. C. 20555

Dear Sir:

In accordance with reporting requirements of Technical Specification 6.9.1d, the Connecticut Yankee Haddam Neck Plant Monthly Operating Report 90-07, covering operations for the period July 1, 1990 to July 31, 1990 is hereby forwarded.

Very truly yours,

E. A. DeBarba
Station Director

EAD/jhb

cc: (1) Regional Administrator, Region 1
U. S. Nuclear Regulatory Commission
475 Allendale Road
King of Prussia, PA 19406

(2) John T. Shedlosky
Sr. Resident Inspector
Connecticut Yankee

IE24
1/1

Connecticut Yankee Atomic Power Company

Haddam Neck Plant

Haddam, Connecticut

Monthly Operating Report No. 90-06

For The Month of

July 1990

Plant Operations Summary - July, 1990

The following is a summary of Plant Operations for July, 1990.

The 15th refueling and maintenance outage continued for the month of July with the unit in Cold Shutdown, Mode 5.

July 21, 1990 at 2000 hours the plant achieved Hot Shutdown, Mode 4.

July 23, 1990 at 1507 hours the unit returned to Cold Shutdown, Mode 5.

July 25, 1990 at 733 hours, ascended to Hot Shutdown, Mode 4.

July 27, 1990 at 0113 hours the unit reached Hot Standby Mode 3 and remained there for the remainder of the month.

AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-213
Conn. Yankee
UNIT Haddam Neck

DATE 7/90

COMPLETED BY K. C. Emmons

TELEPHONE (203) 267-3654

MONTH: July 1990

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)	DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1	0	17	0
2		18	
3		19	
4		20	
5		21	
6		22	
7		23	
8		24	
9		25	
10		26	
11		27	
12		28	
13		29	
14		30	
15		31	
16			

INSTRUCTIONS

On this format, list the average daily unit power level in MWe-Net for each day in the reporting month. Complete the nearest whole megawatt.

(9/77)

Maintenance 7/90

SYSTEM OR COMPONENT	MALFUNCTION		EFFECT ON SAFE OPERATION	CORRECTIVE ACTION TAKEN TO PREVENT REPETITION	SPECIAL PRECAUTIONS TAKEN TO PROVIDE FOR REACTOR SAFETY DURING REPAIR
	CAUSE	RESULT			
CH-V-285 PT-105 Root Isolation	Cracked weld in pipe upstream of valve.	Loss of RC spread of contamination.	None	Replaced upstream pipe, repacked valve.	RX shutdown at this time.
P-18-1B (Charging Pump)	Shaft seal leaking	Loss of RC, spread of contamination	None	Replaced with new mech. seal.	RX shutdown at this time.
DH-MOV-562	Valve won't shut all the way. Expected cause is valve disc putin 180° out when bonnet gasket changed.	Unacceptable flow of RC through drain header.	None	Downstream isolation valve added. (DH-V-102). Valve to be repaired or replaced next refueling outage.	RX shutdown at this time.
RCS Hydro prior to Start-up	Numerous packing leaks	Loss of RC, spread of contamination	None	Packings adjusted, looking into routinely repacking valves with history of leakage each refueling.	RX shutdown at this time.

I&C 7/90

SYSTEM OR COMPONENT	MALFUNCTION		EFFECT ON SAFE OPERATION	CORRECTIVE ACTION TAKEN TO PREVENT REPETITION	SPECIAL PRECAUTIONS TAKEN TO PROVIDE FOR REACTOR SAFETY DURING REPAIR
	CAUSE	RESULT			
Containment Isolation Actuation System	Holding spring mechanism for train "A" CIAs relay	Automatic actuation of an engineered safety feature	None-Plant in Mode 6 with core offloaded to the spent fuel pool	A visual inspection of both the HCP relays as well as the safety injection relays will be performed prior to entering Mode 4 during the present outage. Additionally, inspection of other latching relays supplied by G.E. will be evaluated during Mode 1 operation	N/A

CONNECTICUT YANKEE
REACTOR COOLANT DATA
MONTH: JULY 1990

REACTOR COOLANT ANALYSIS	MINIMUM	AVERAGE	MAXIMUM
PH @ 25 DEGREES C	4.54E+00	4.85E+00	5.46E+00
CONDUCTIVITY (UMHOS/CM)	6.93E+00	8.80E+00	1.19E+01
CHLORIDES (PPM)	<5.00E-02	<5.00E-02	<5.00E-02
DISSOLVED OXYGEN (PPB)	<5.00E+00	8.17E+01	7.00E+02
BORON (PPM)	2.18E+03	2.40E+03	2.50E+03
LITHIUM (PPM)	0.00E-01	7.11E-01	9.87E-01
TOTAL GAMMA ACT. (UC/ML)	1.26E-04	1.08E-02	4.20E-02
IODINE-131 ACT. (UC/ML)	0.00E-01	0.00E-01	0.00E-01
I-131/I-133 RATIO	0.00E-01	0.00E-01	0.00E-01
CRUD (MG/LITER)	<1.00E-02	<1.00E-02	<1.00E-02
TRITIUM (UC/ML)	1.72E-03	8.39E-03	1.81E-02
HYDROGEN (CC/KG)	<5.00E+00	<5.00E+00	<5.00E+00

AERATED LIQUID WASTE PROCESSED(GALLONS):	8.68E+04
WASTE LIQUID PROCESSED THROUGH BORON RECOVERY(GALLONS):	7.74E+04
AVERAGE PRIMARY LEAK RATE(GALLONS PER MINUTE):	0.00E-01
PRIMARY TO SECONDARY LEAK RATE(GALLONS PER MINUTE):	0.00E+00

NRC OPERATING STATUS REPORT

Haddam Neck

1. Docket: 50-213
2. Reporting Period: 07/90 Outage + On-line Hours: 744.0 + 0.0 = 744.0
3. Utility Contact: J. Stanford (203) 267-3635
4. Licensed Thermal Power (MWt): 1825
5. Nameplate Rating (Gross MWe): 667 x 0.9 = 600.3
6. Design Electrical Rating (Net MWe): 582
7. Maximum Dependable Capacity (Gross MWe): 591.8
8. Maximum Dependable Capacity (Net MWe): 565
9. If changes occur above since last report, reasons are: NONE
10. Power level to which restricted, if any (Net MWe): N/A
11. Reasons for restriction, if any: N/A

	MONTH	YEAR-TO-DATE	CUMULATIVE
12. Report period hours:	744.0	5,087.0	197,951.0
13. Hours reactor critical:	0.0	0.0	158,249.2
14. Reactor reserve shutdown hours:	0.0	0.0	1,285.0
15. Hours generator on-line:	0.0	0.0	152,087.6
16. Unit reserve shutdown hours:	0.0	0.0	398.0
17. Gross thermal energy generated (MWtH):	0.0	0.0	262,867,318.0 *
18. Gross electrical energy generated (MWeH):	0.0	0.0	86,093,046.0 *
19. Net electrical energy generated (MWeH):	-4,902.3	-17,175.1	81,805,804.9 *
20. Unit service factor:	0.0	0.0	76.8
21. Unit availability factor:	0.0	0.0	77.0
22. Unit capacity factor using MDC net:	0.0	0.0	75.2
23. Unit capacity factor using DER net:	0.0	0.0	71.0
24. Unit forced outage rate:	0.0	0.0	5.6
25. Forced outage hours:	0.0	0.0	8,975.6

26. Shutdowns scheduled over next 6 months (type,date, duration): NONE

27. If currently shutdown, estimated startup date: 8/3/90

* Cumulative values from the first criticality (07/24/67). (The remaining cumulative values are from the first date of commercial operation, 01/01/68).

UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 50-213
UNIT NAME Connecticut Yank
DATE 7/90
COMPLETED BY K. C. Emmons
TELEPHONE 203-267-3654

REPORT MONTH July 1990

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	LER RPT.	System[4] Code	Component[5] Code	Cause & Corrective Action to Prevent Recurrence
89-03	7/1/90	S	744	C	1	N/A	RC	Fuel XX	Continuation of Core XV-XVI Refueling.

1
F Forced
S Scheduled

2
Reason:
A-Equipment Failure (Explain)
B-Maintenance or Test
C-Refueling
D-Regulatory Restriction
E-Operator Training & License Examination
F-Administrative
G-Operational Error (Explain)
H-Other(Explain

3
Method:
1-Manual
2-Manual Scram
3-Automatic Scram
4-Other(Explain)

4
Exhibit G-Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

5
Exhibit 1 Same Source

Refueling Information Request

1. Name of facility

   Haddam Neck

2. Scheduled date for next refueling shutdown.

   September 1, 1991

3. Scheduled date for restart following refueling.

   November 1, 1991

4. (a) Will refueling or resumption of operation thereafter require a technical specification change or other license amendment?
   Yes

   (b) If answer is yes, what, in general, will these be?

   Incorporate the guidance provided in the NRC Generic Letter 88-16. The Generic Letter addresses removing cycle specific parameters from Technical Specifications and transferring them to the technical report supporting cycle operation. The amendment to implement these changes has been issued.

   (c) If answer is no, has the reload fuel design and core configuration been reviewed by your Plant Safety Review Committee to determine whether any unreviewed safety questions are associated with the core reload?
   (Ref. 10 CFR section 50.59)

   n/a

   (d) If no such review has taken place, when is it scheduled?

   n/a

5. Scheduled date(s) for submitting proposed licensing action and supporting information.

   The TSCR was submitted to the NRC on July 28, 1989. The Amendment was issued on April 26, 1990.

6. Important licensing considerations associated with refueling, e.g., new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures.

   No

7. The number of fuel assemblies (a) in the core and (b) in the spent fuel storage pool.

   (a) 157 (b) 701

8. The present licensed spent fuel pool storage capacity and the size of any increase in licensed storage capacity that has been requested or is planned, in number of fuel assemblies.

   1168

9. The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity.

   1996